

Program EVALPLOT
(Version 2018-1)

by

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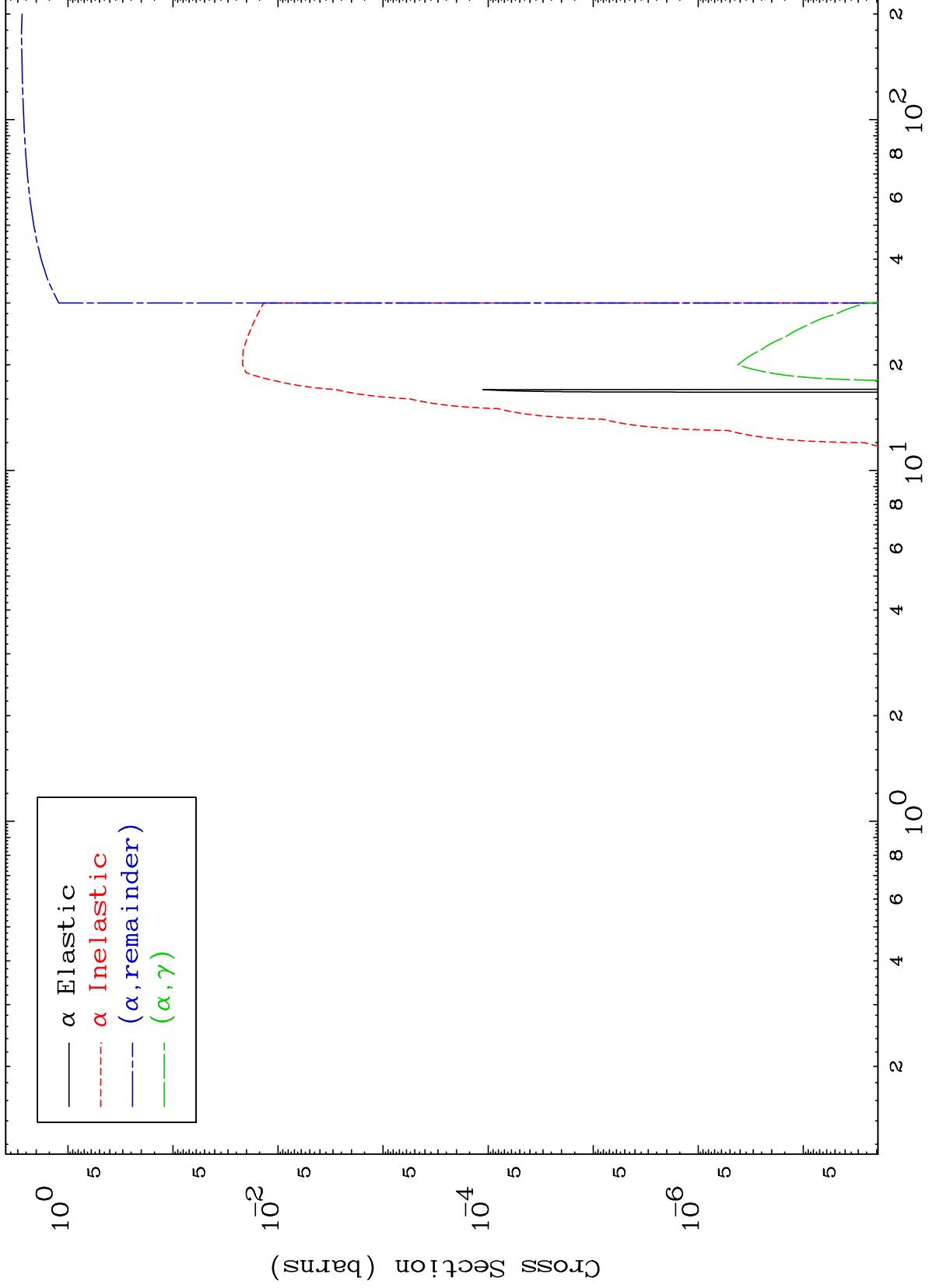
Press Mouse Button to Start

MAT 7743

α Major

77-Ir-197

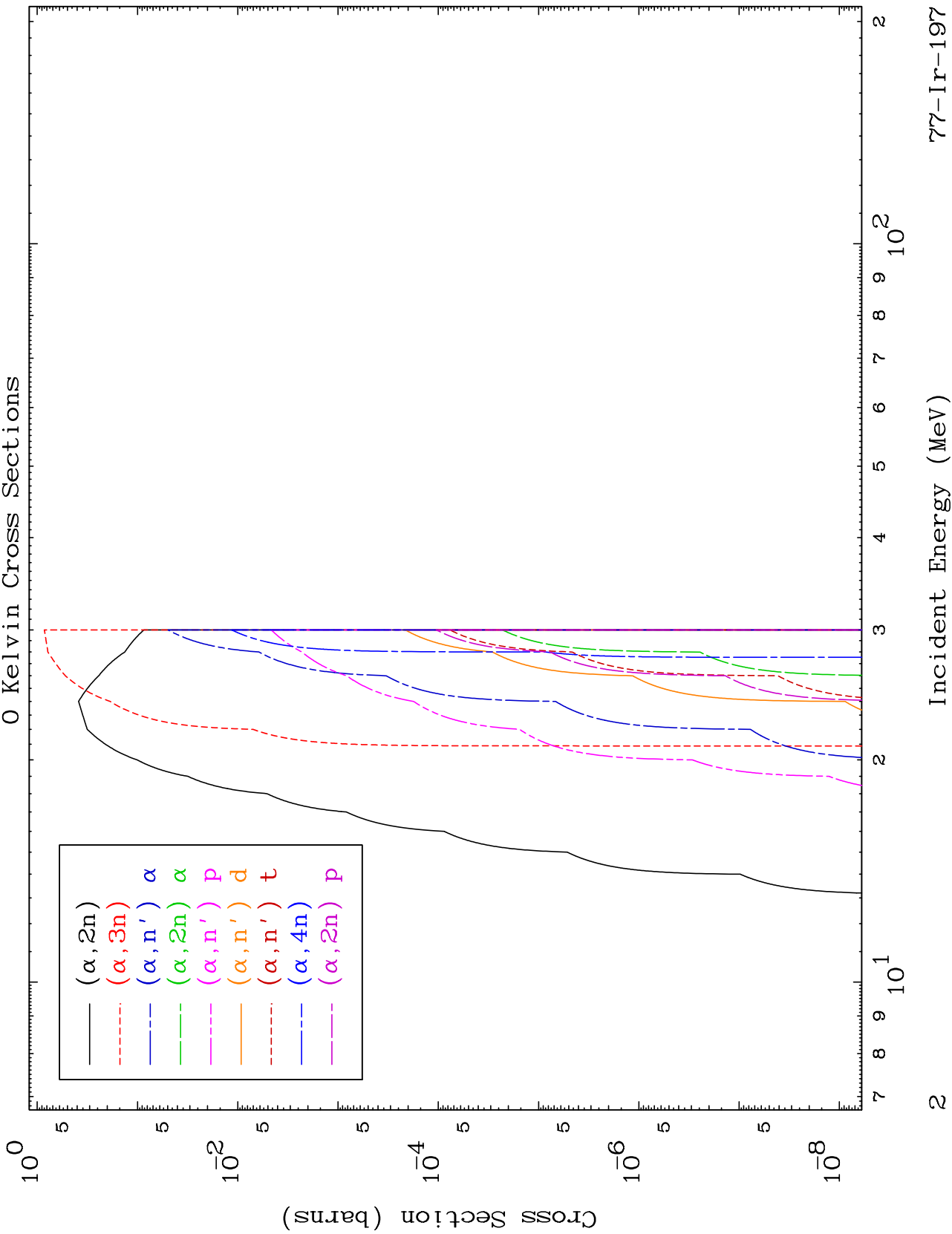
0 Kelvin Cross Sections

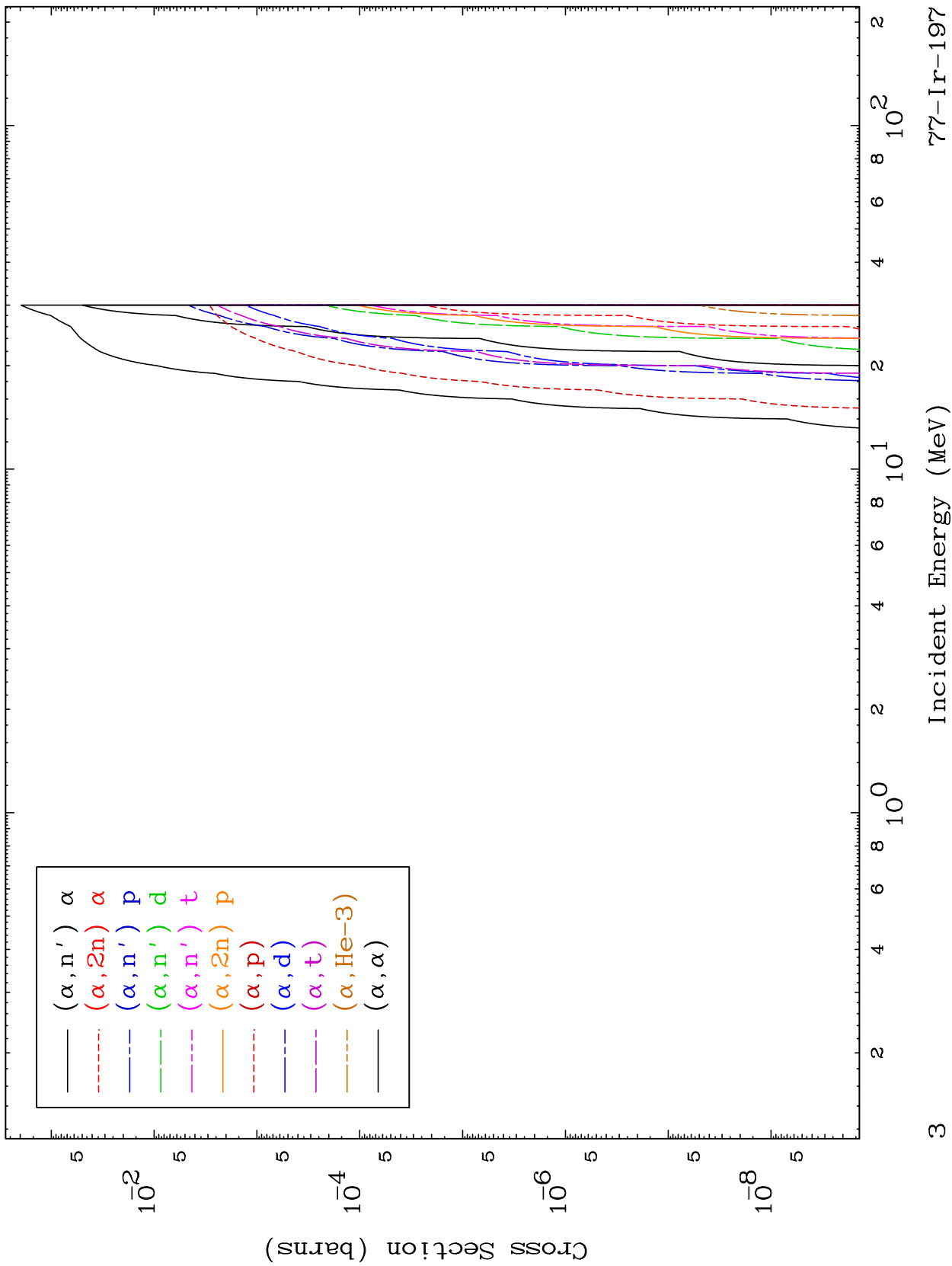


MAT 7743

α Neutron Production
0 Kelvin Cross Sections

77-Ir-197



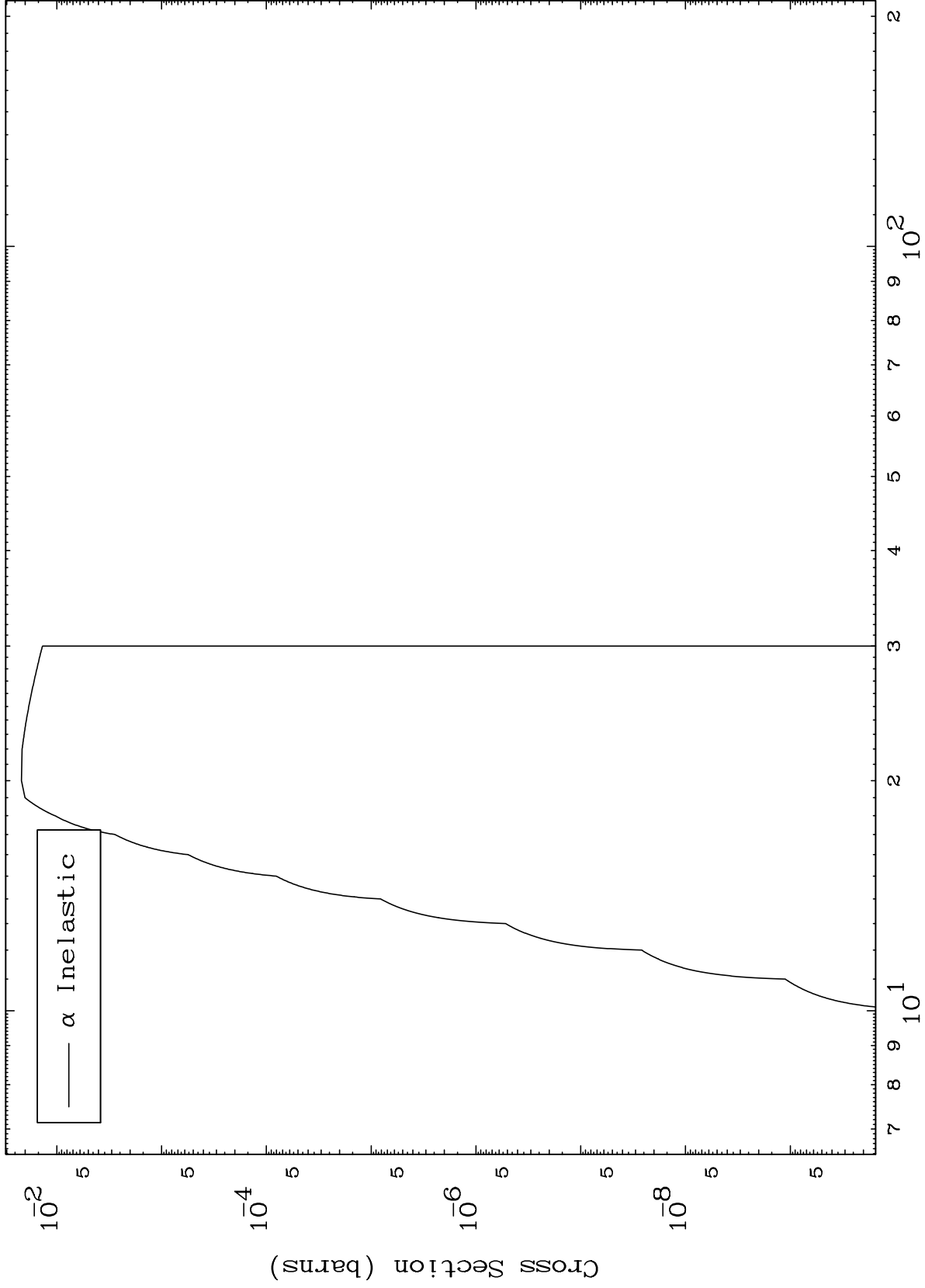


MAT 7743

(α, n') Level

77-Ir-197

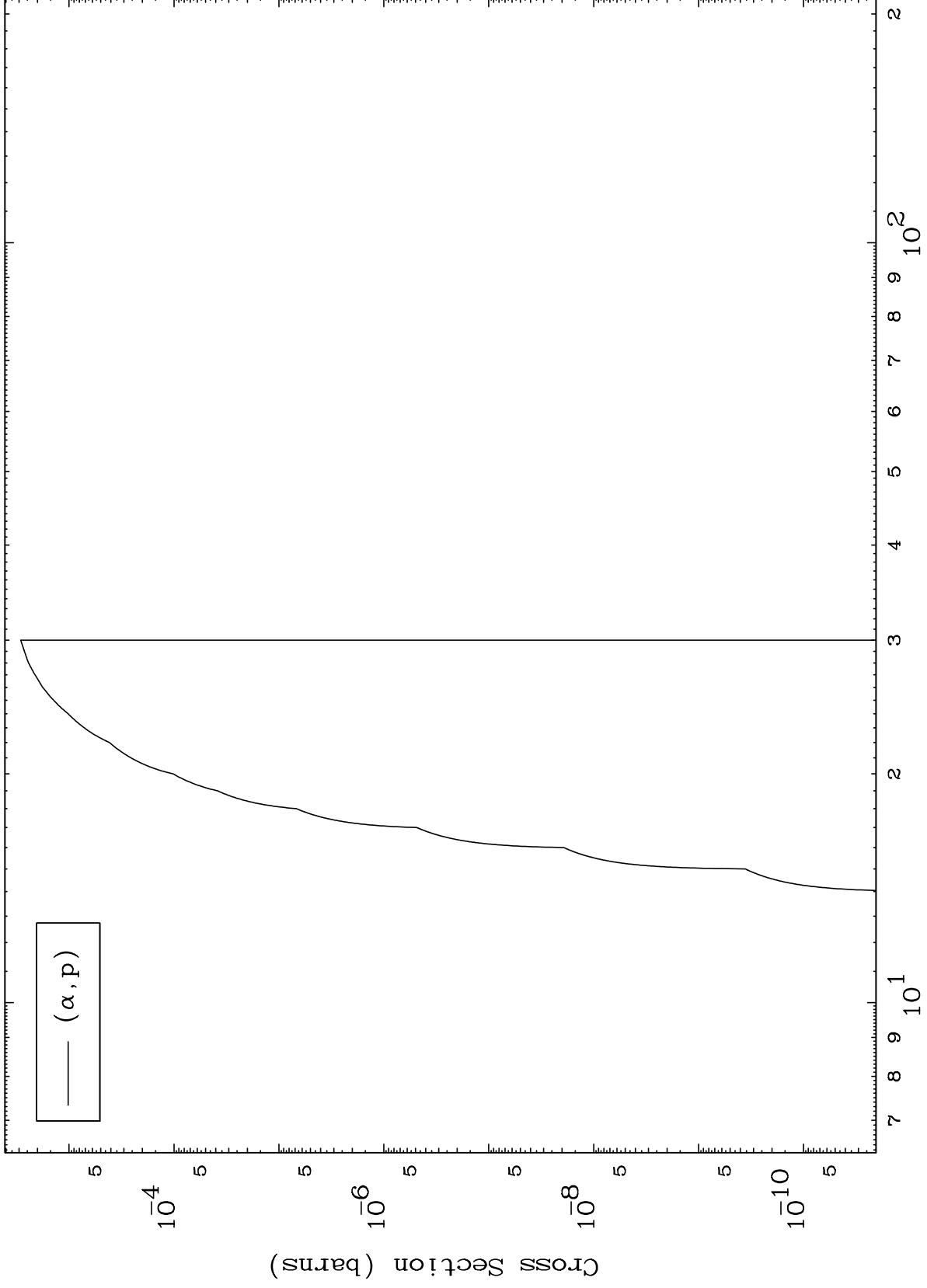
0 Kelvin Cross Sections



MAT 7743

(α, p) Levels
0 Kelvin Cross Sections

77-Ir-197



5

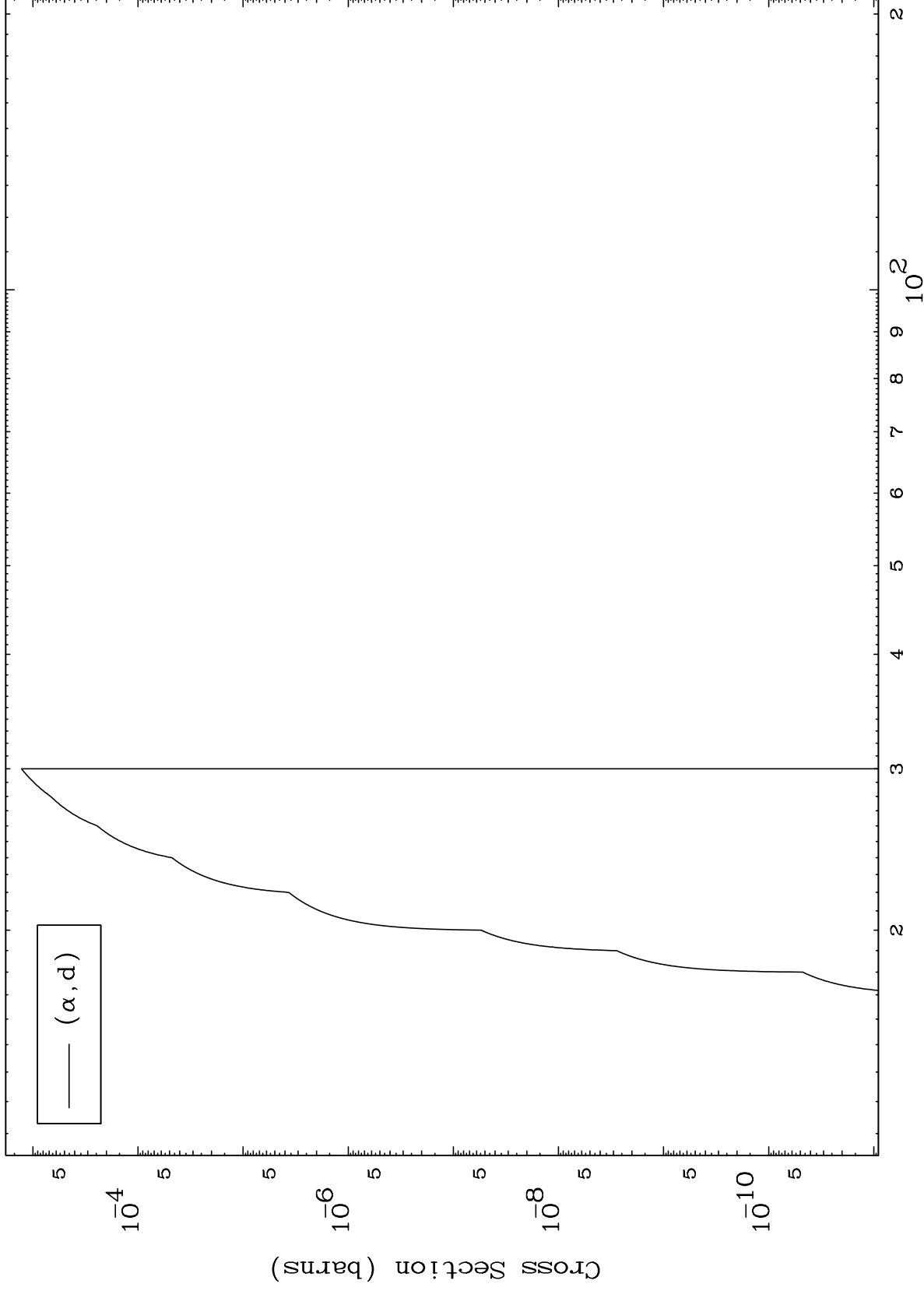
Incident Energy (MeV)

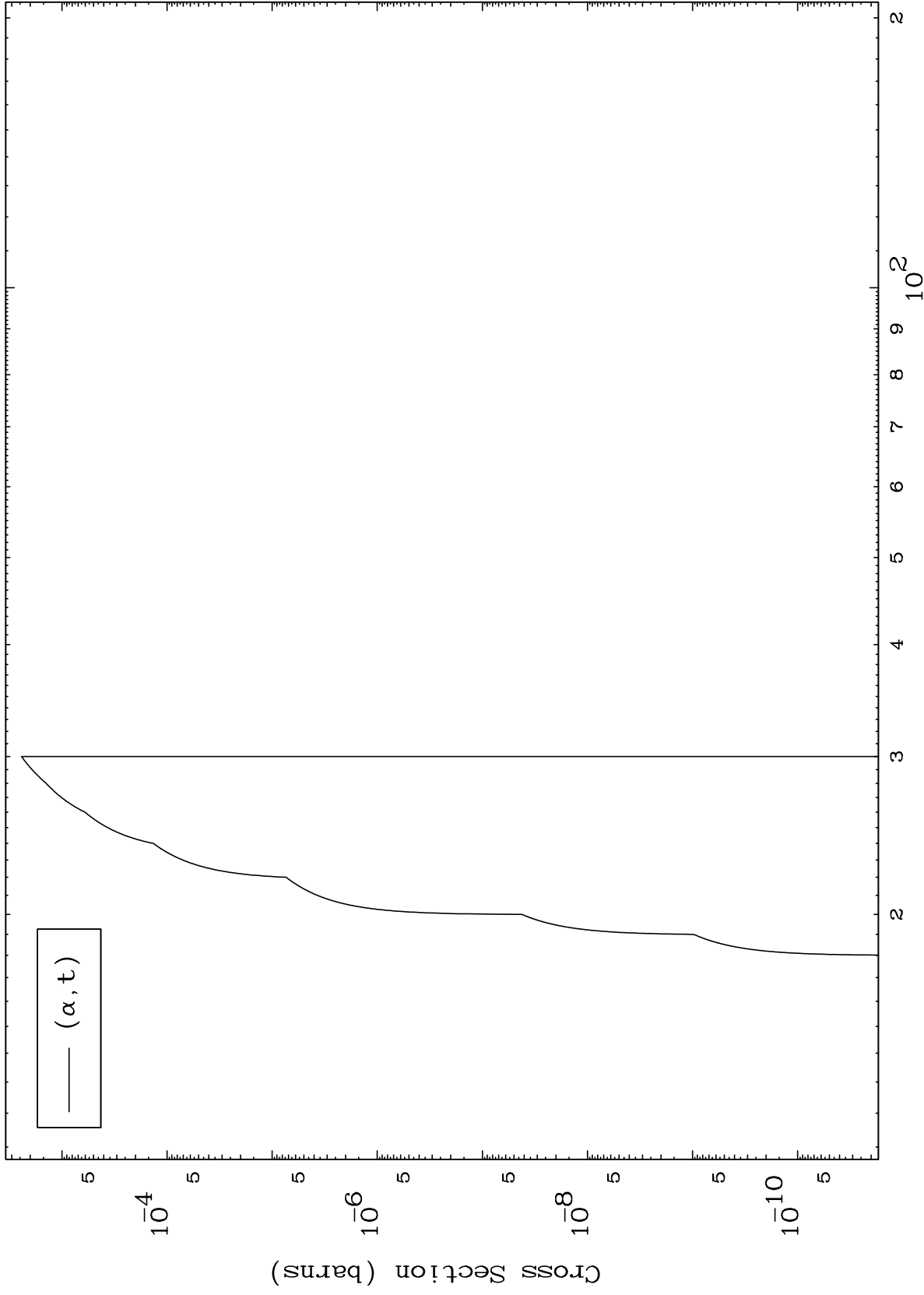
77-Ir-197

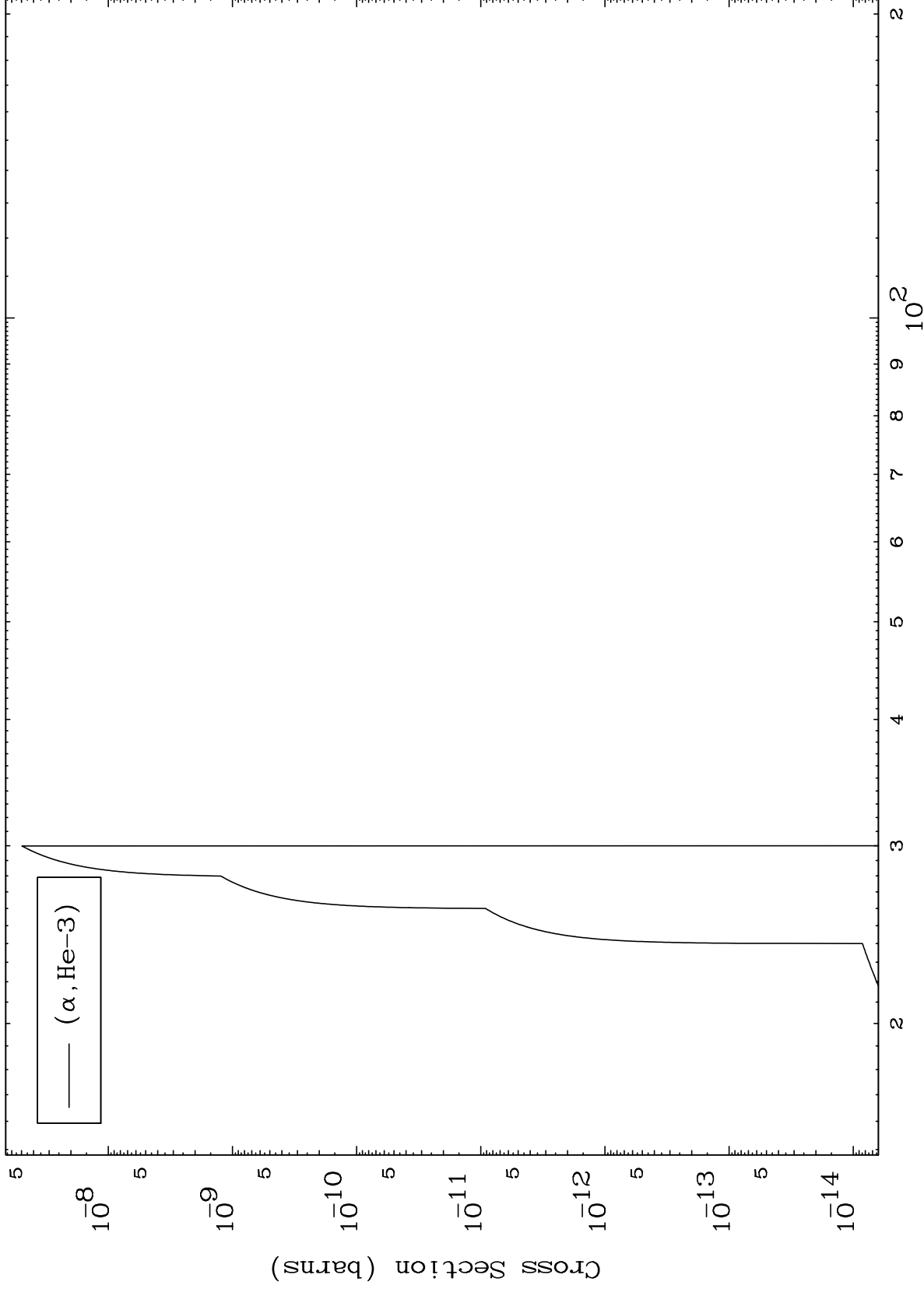
MAT 7743

(α, d) Levels
0 Kelvin Cross Sections

77-Ir-197





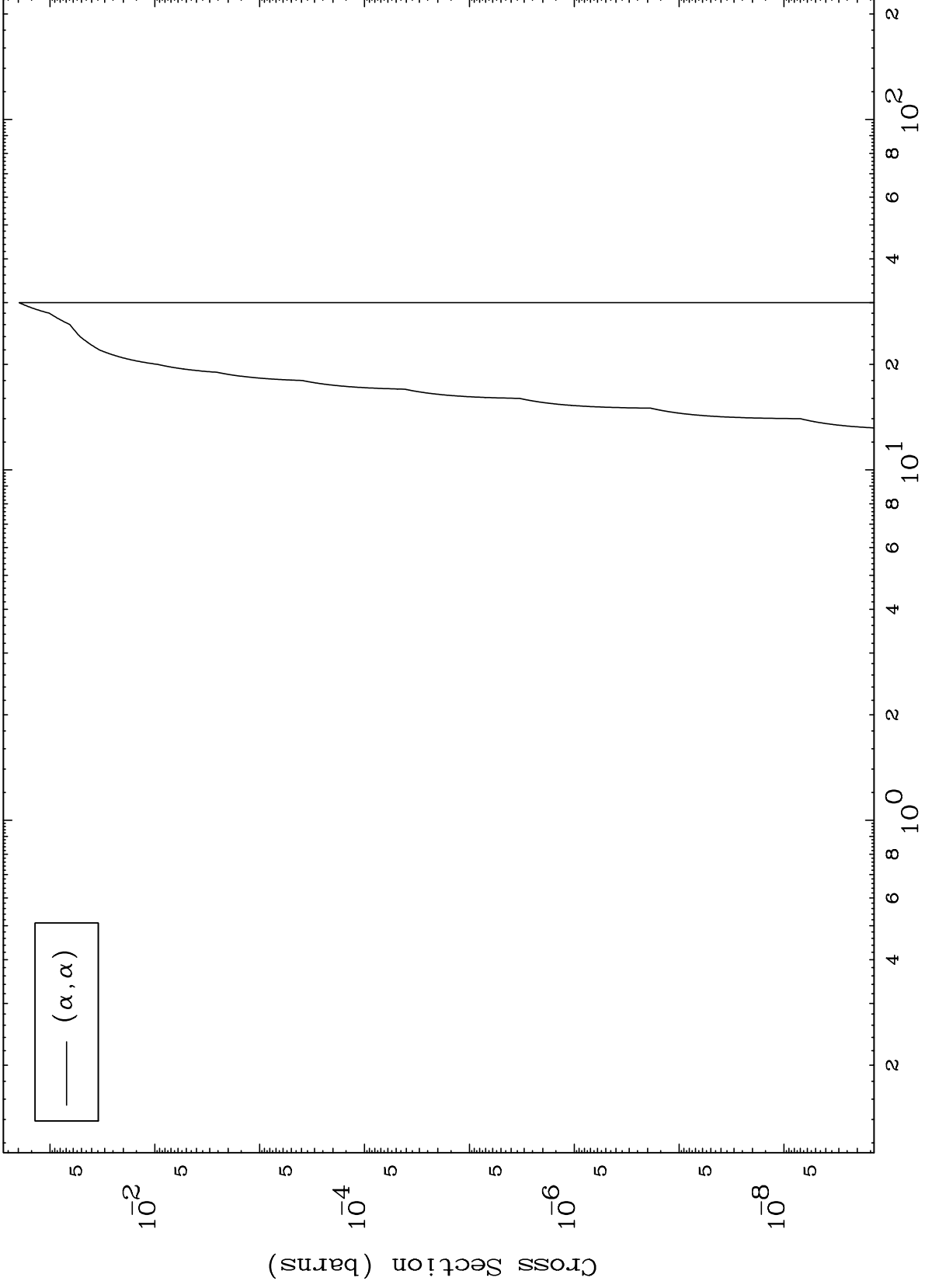


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(α, α) Levels

77-Ir-197

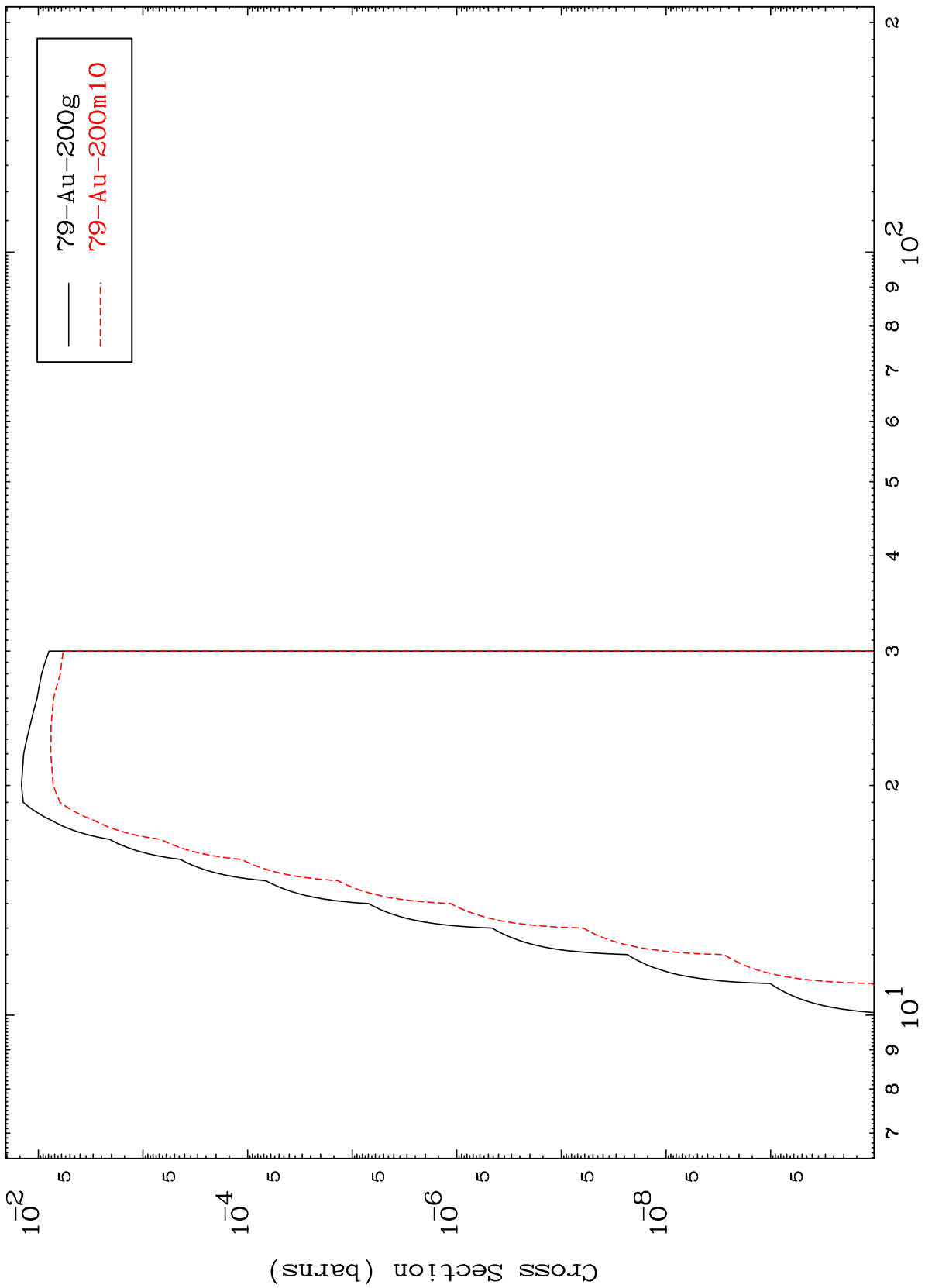
0 Kelvin Cross Sections



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77-Ir-197

α Inelastic
Radionuclide Production Cross Section



10

Incident Energy (MeV)

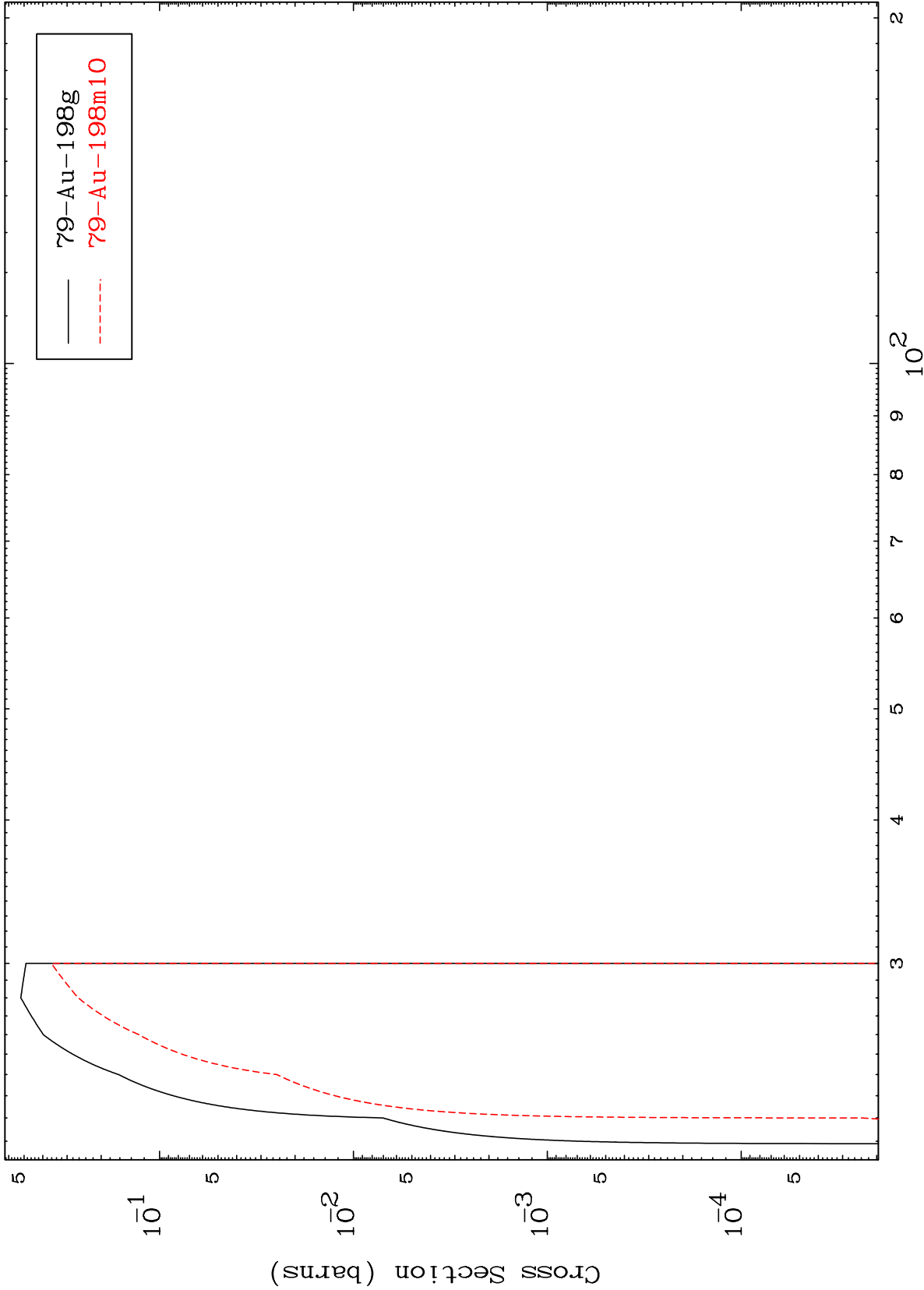
77-Ir-197

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($\alpha, 3n$)

⁷⁷Ir-197

Radionuclide Production Cross Section



79-Au-198g
79-Au-198m10

11

Incident Energy (MeV)

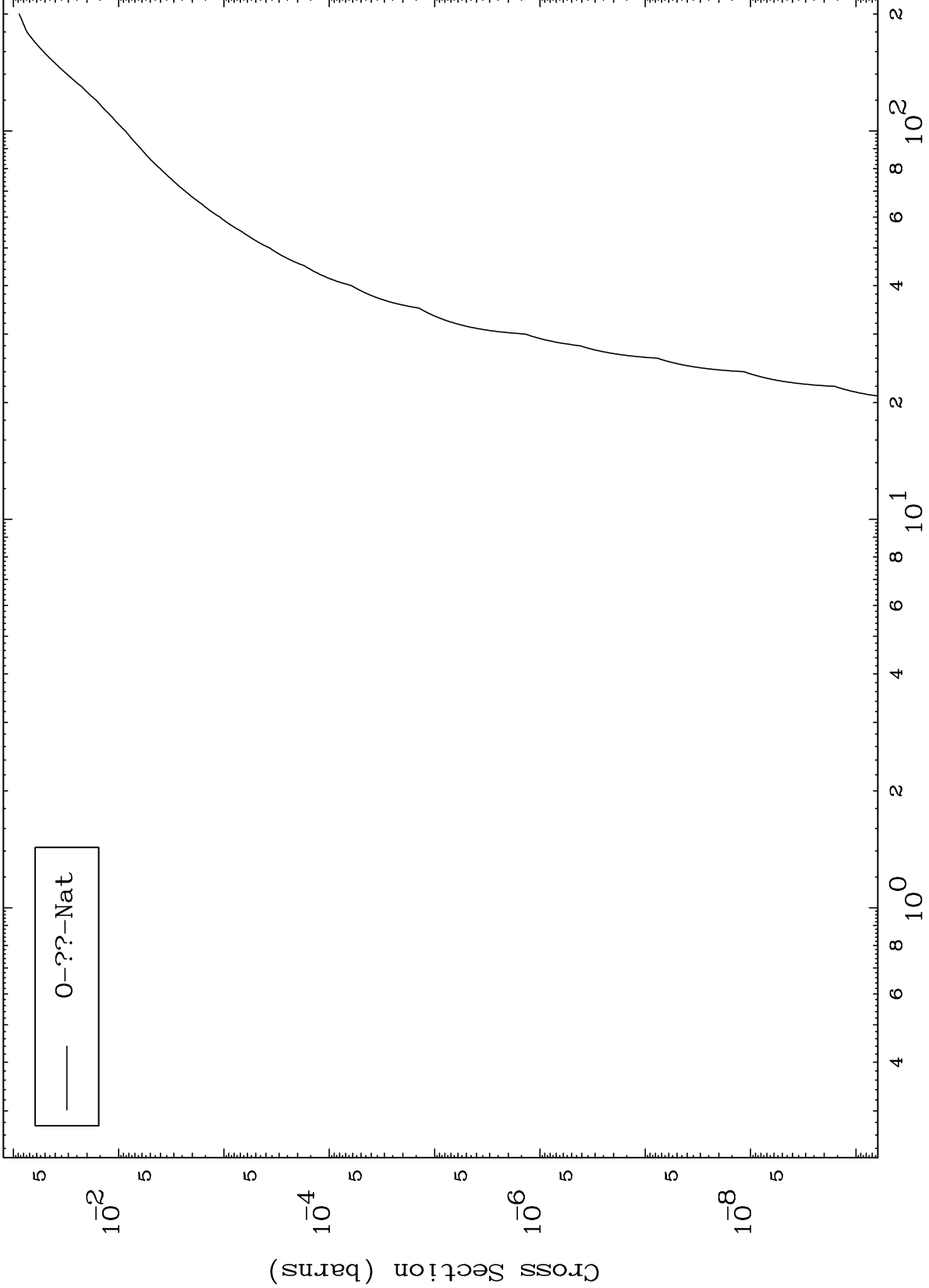
⁷⁷Ir-197

MAT 7743

α Fission

77-Ir-197

Radionuclide Production Cross Section

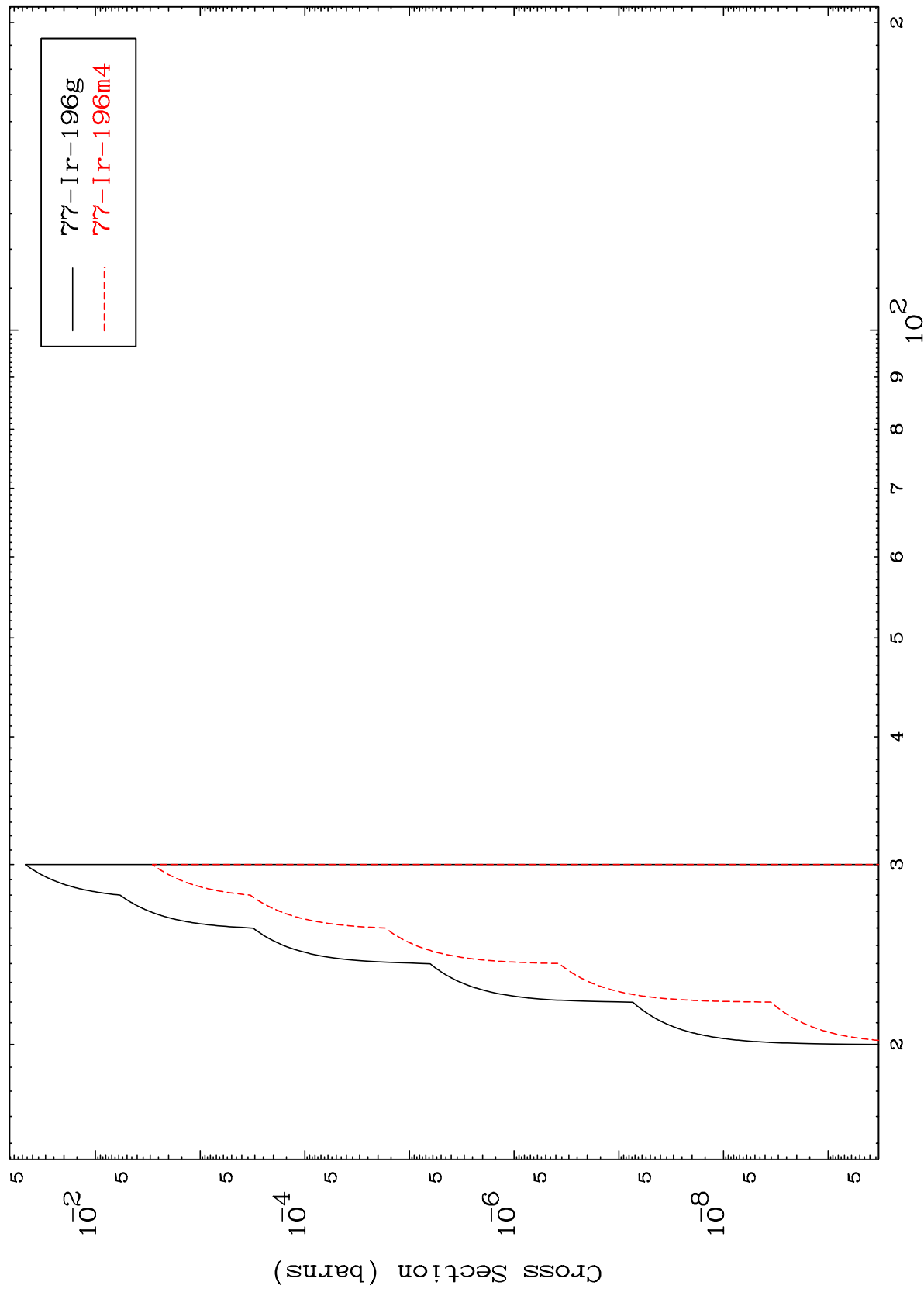


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(α, n') α

77-Ir-197

Radionuclide Production Cross Section



13

Incident Energy (MeV)

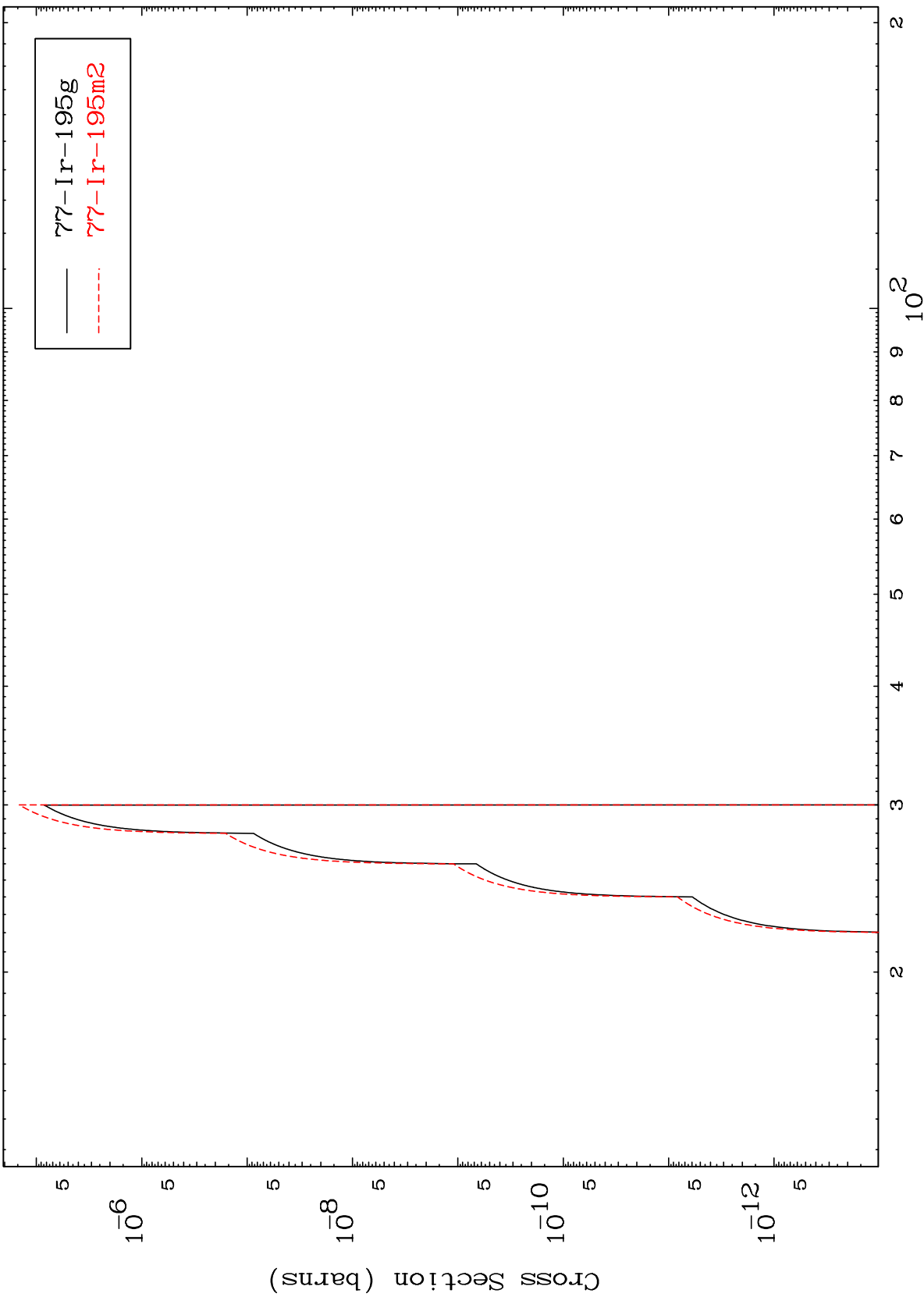
77-Ir-197

MAT 7743

77-Ir-197

$(\alpha, 2n) \alpha$

Radionuclide Production Cross Section

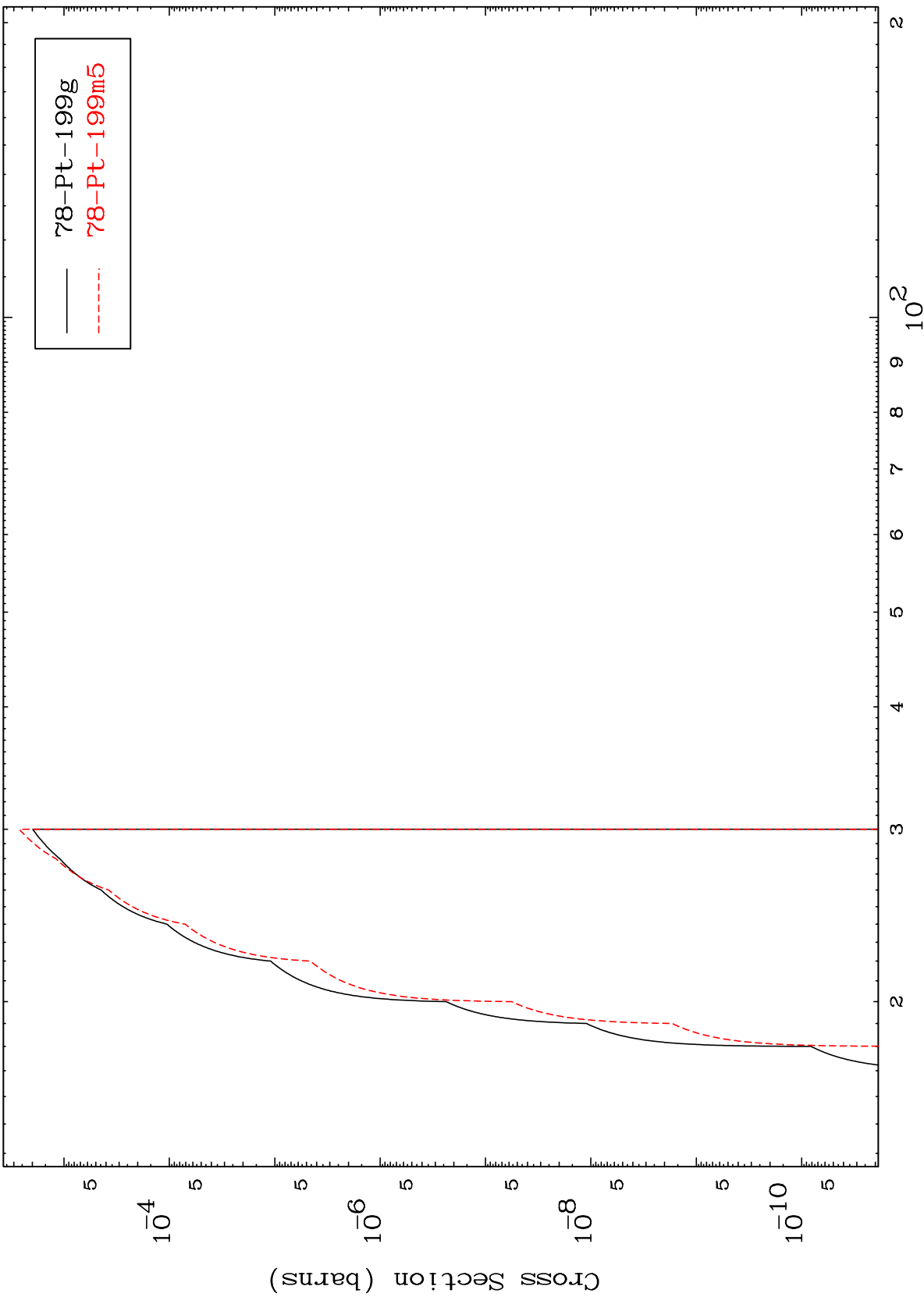


14

Incident Energy (MeV)

77-Ir-197

Radionuclide Production Cross Section



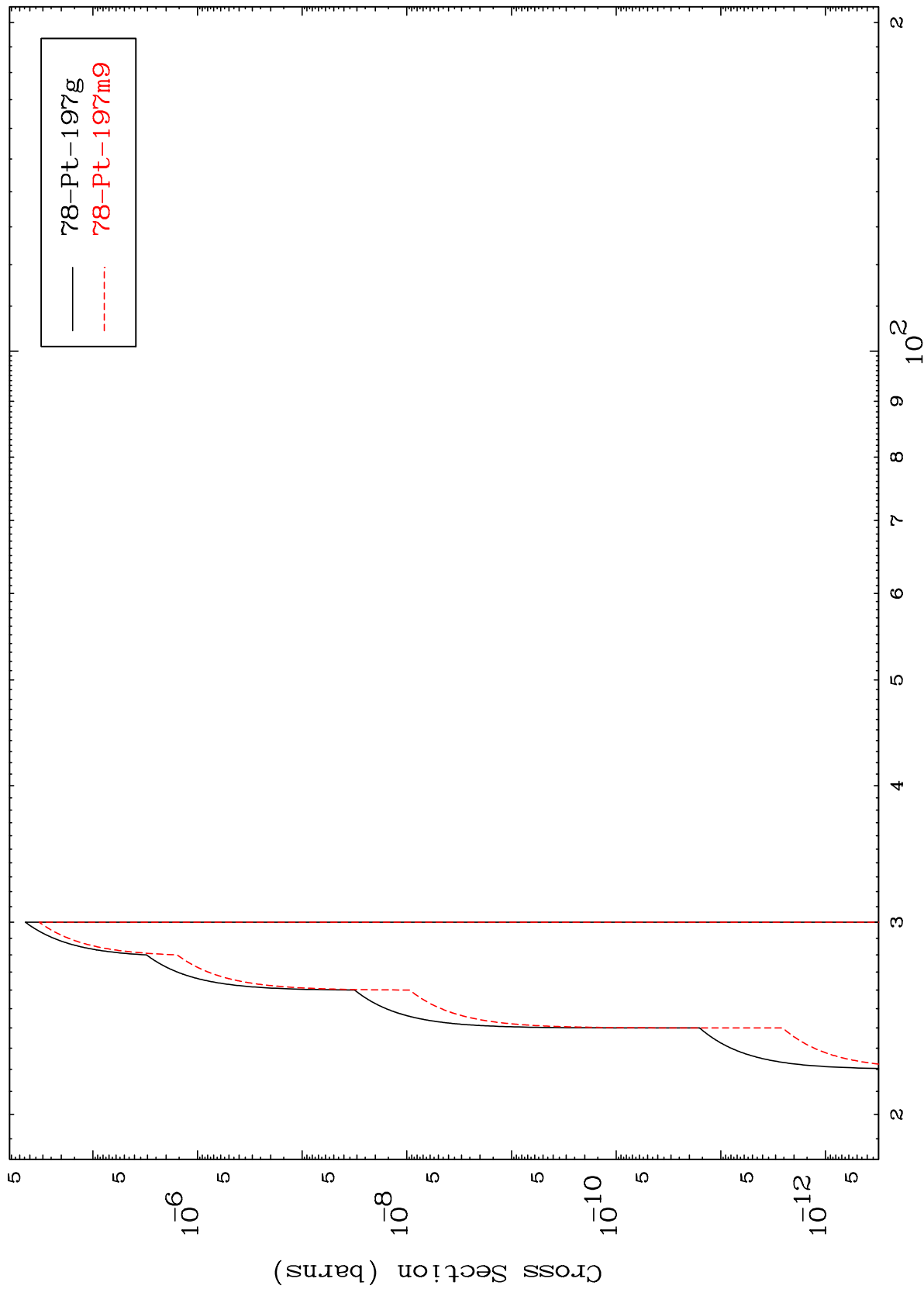
78-Pt-199g
78-Pt-199m5

MAT 7743

(α, n') t

77-Ir-197

Radionuclide Production Cross Section



16

Incident Energy (MeV)

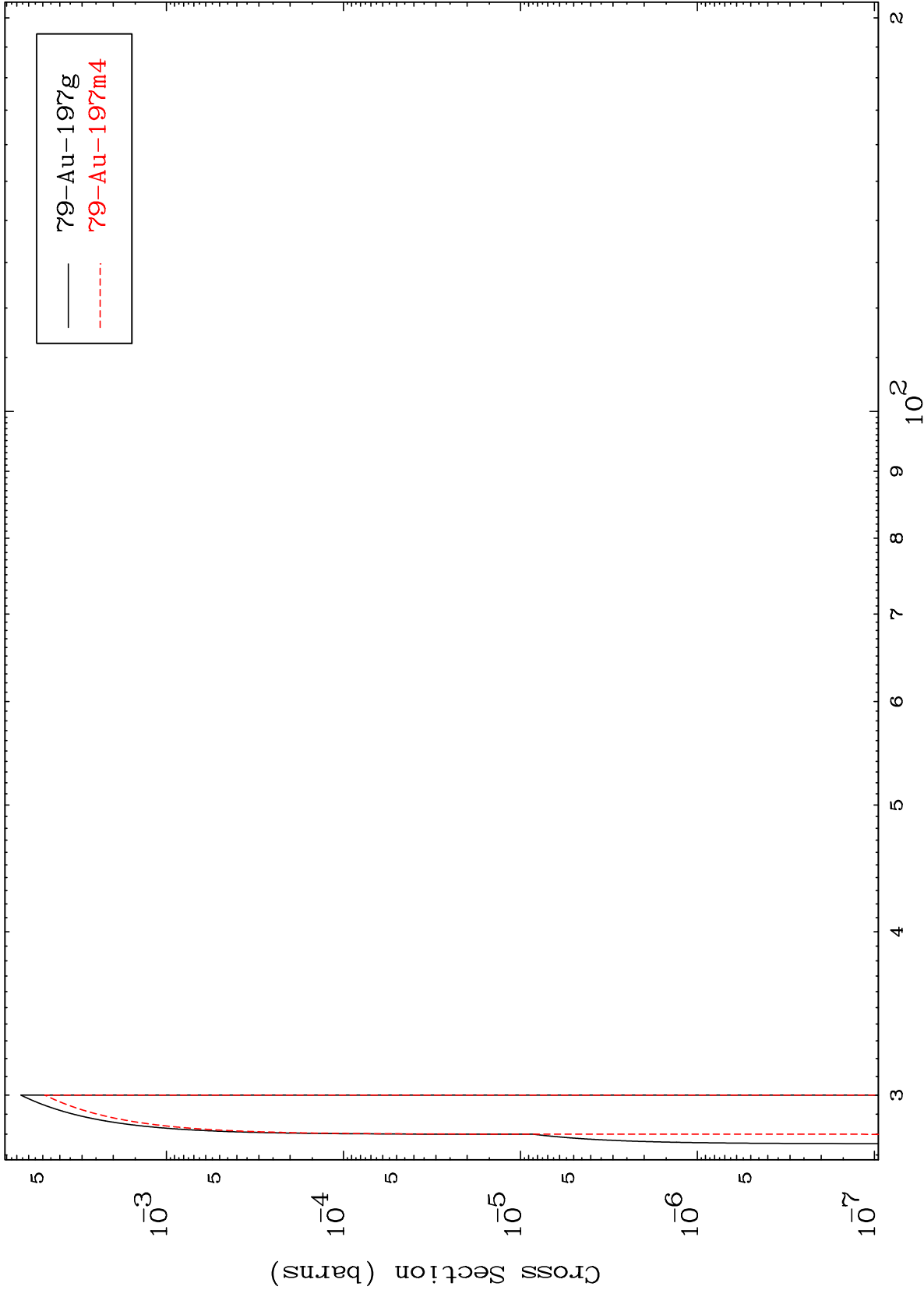
77-Ir-197

MAT 7743

($\alpha, 4n$)

77-Ir-197

Radionuclide Production Cross Section



17

Incident Energy (MeV)

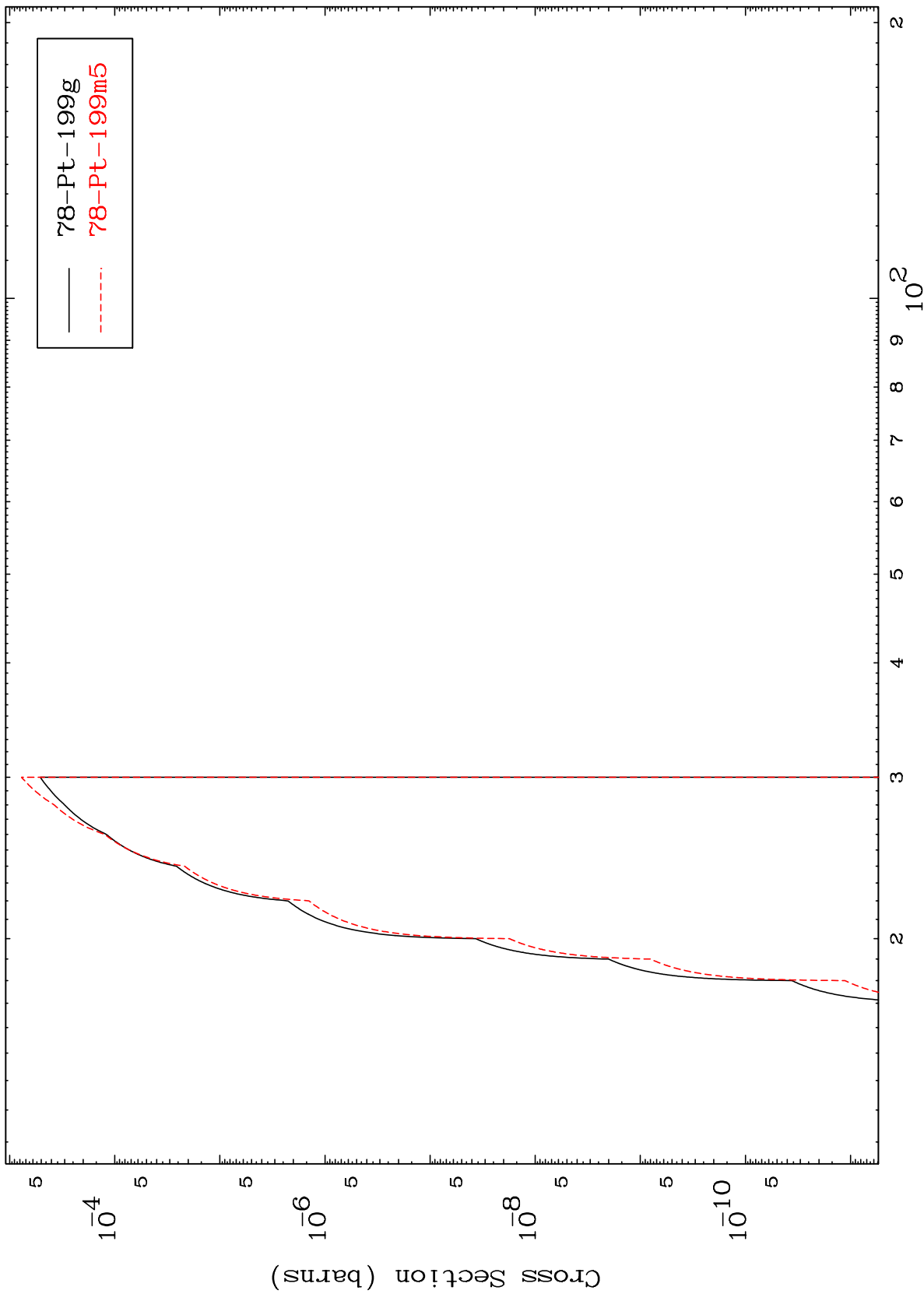
77-Ir-197

MAT 7743

(α, d)

77-Ir-197

Radionuclide Production Cross Section



18

Incident Energy (MeV)

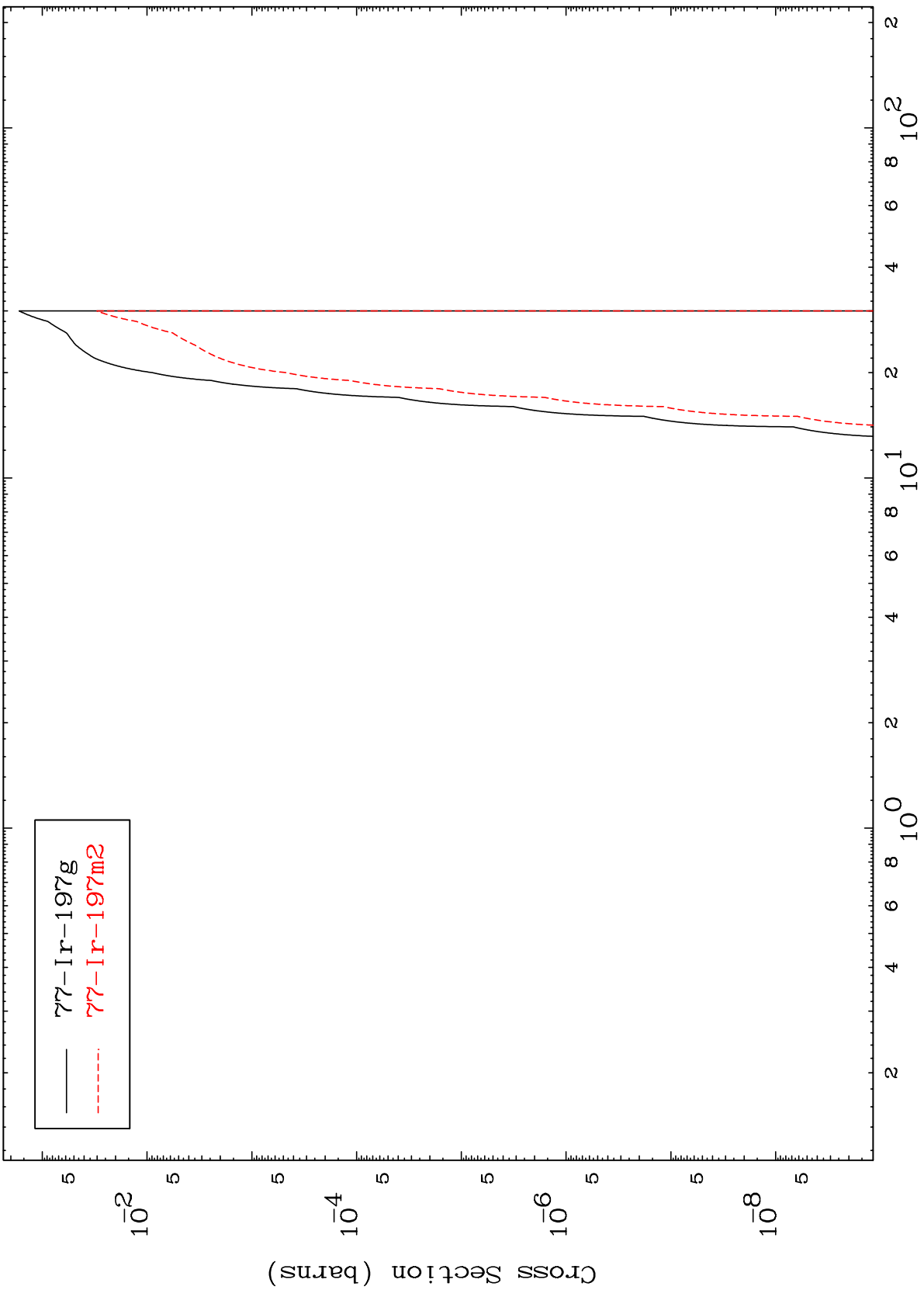
77-Ir-197

MAT 7743

(α, α)

⁷⁷Ir-197

Radionuclide Production Cross Section



— ⁷⁷Ir-197g
- - - ⁷⁷Ir-197m2