

Program EVALPLOT  
(Version 2018-1)

by

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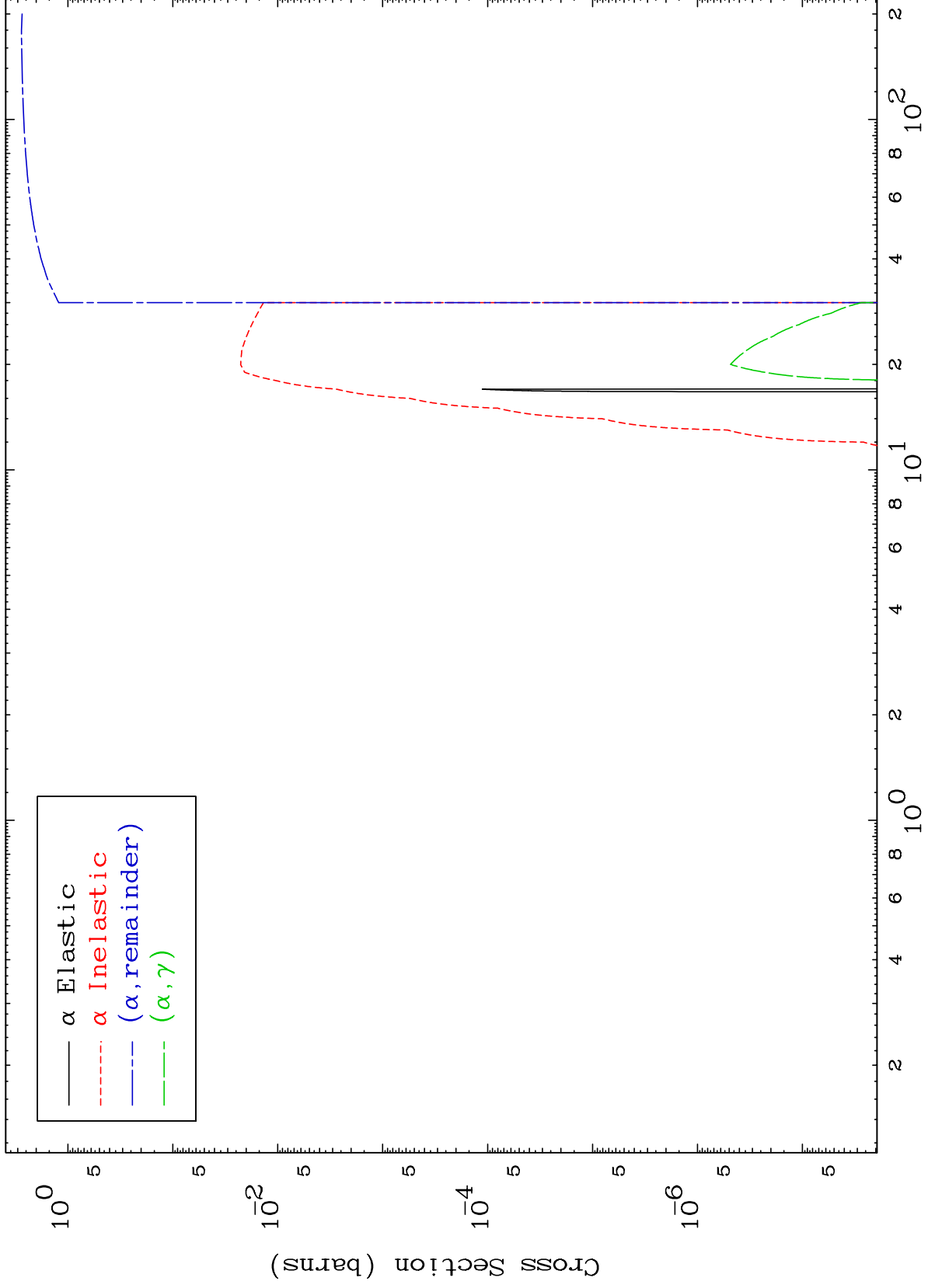
Press Mouse Button to Start

MAT 7744

$\alpha$  Major

77-Ir-197

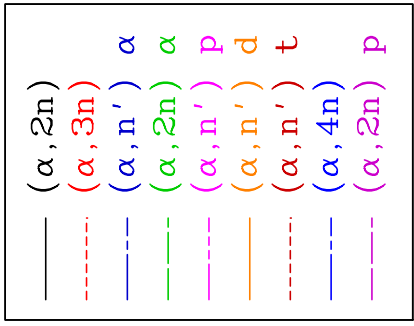
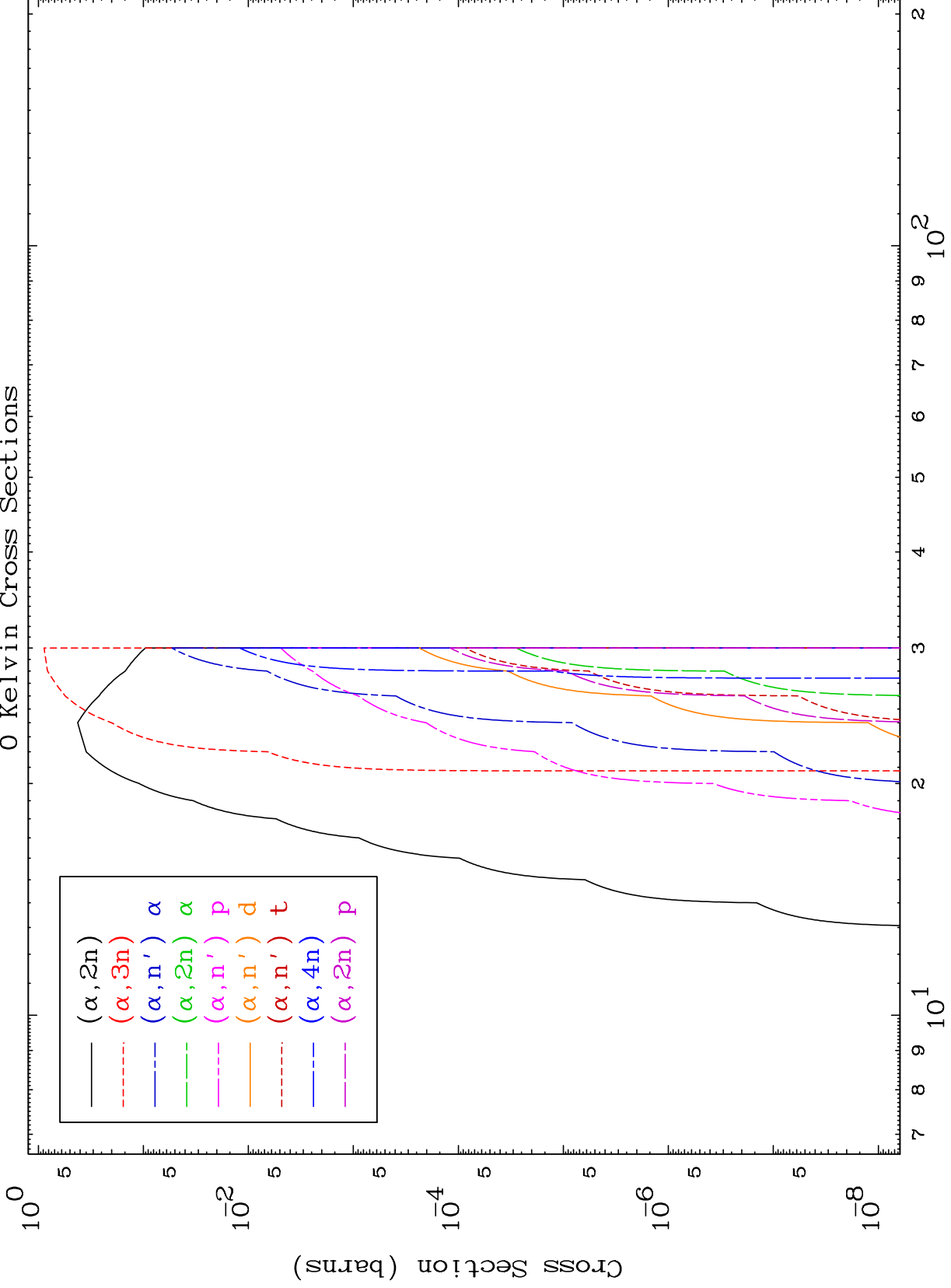
0 Kelvin Cross Sections



MAT 7744

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

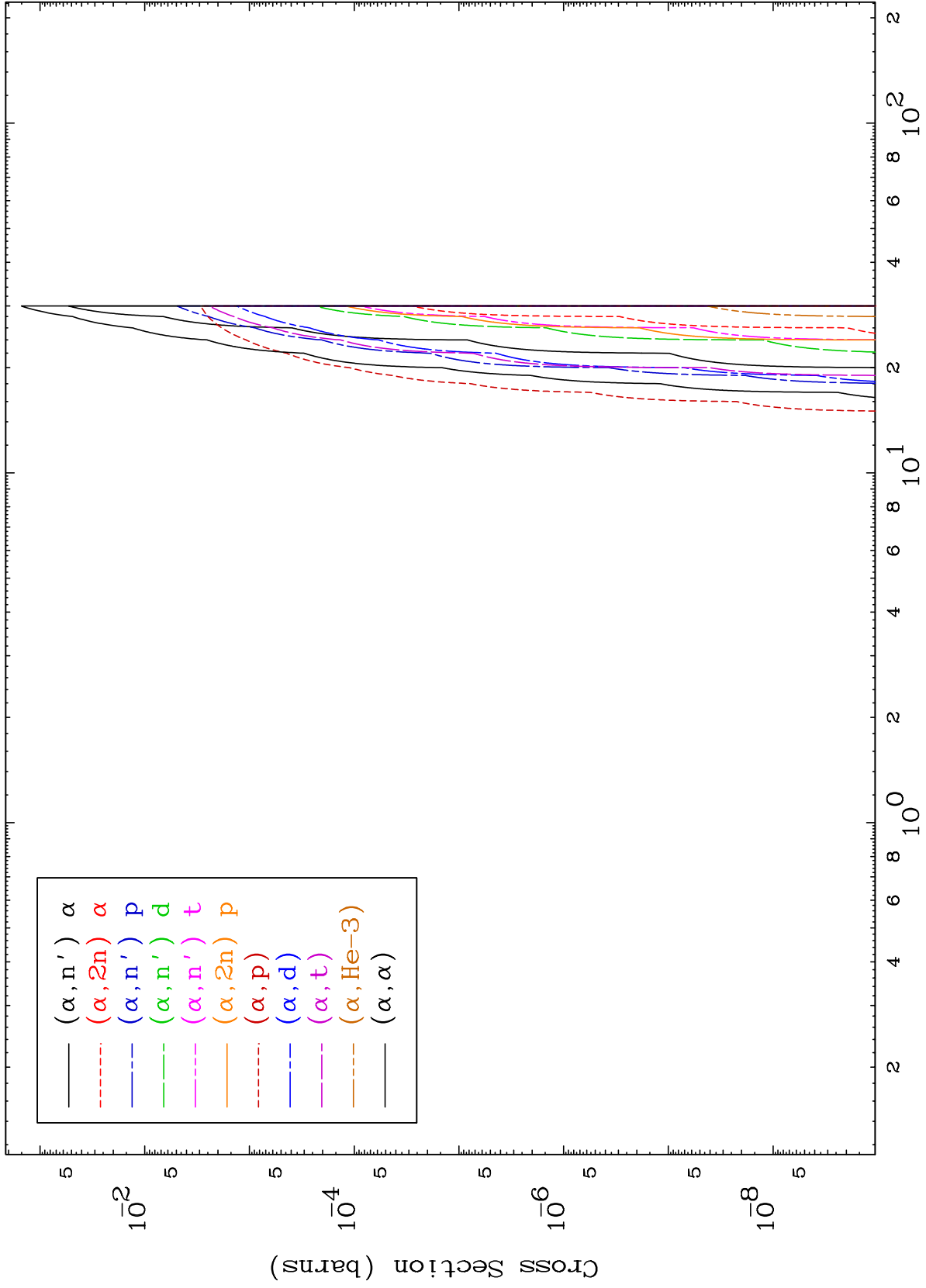
77-Ir-197



2

Incident Energy (MeV)

77-Ir-197

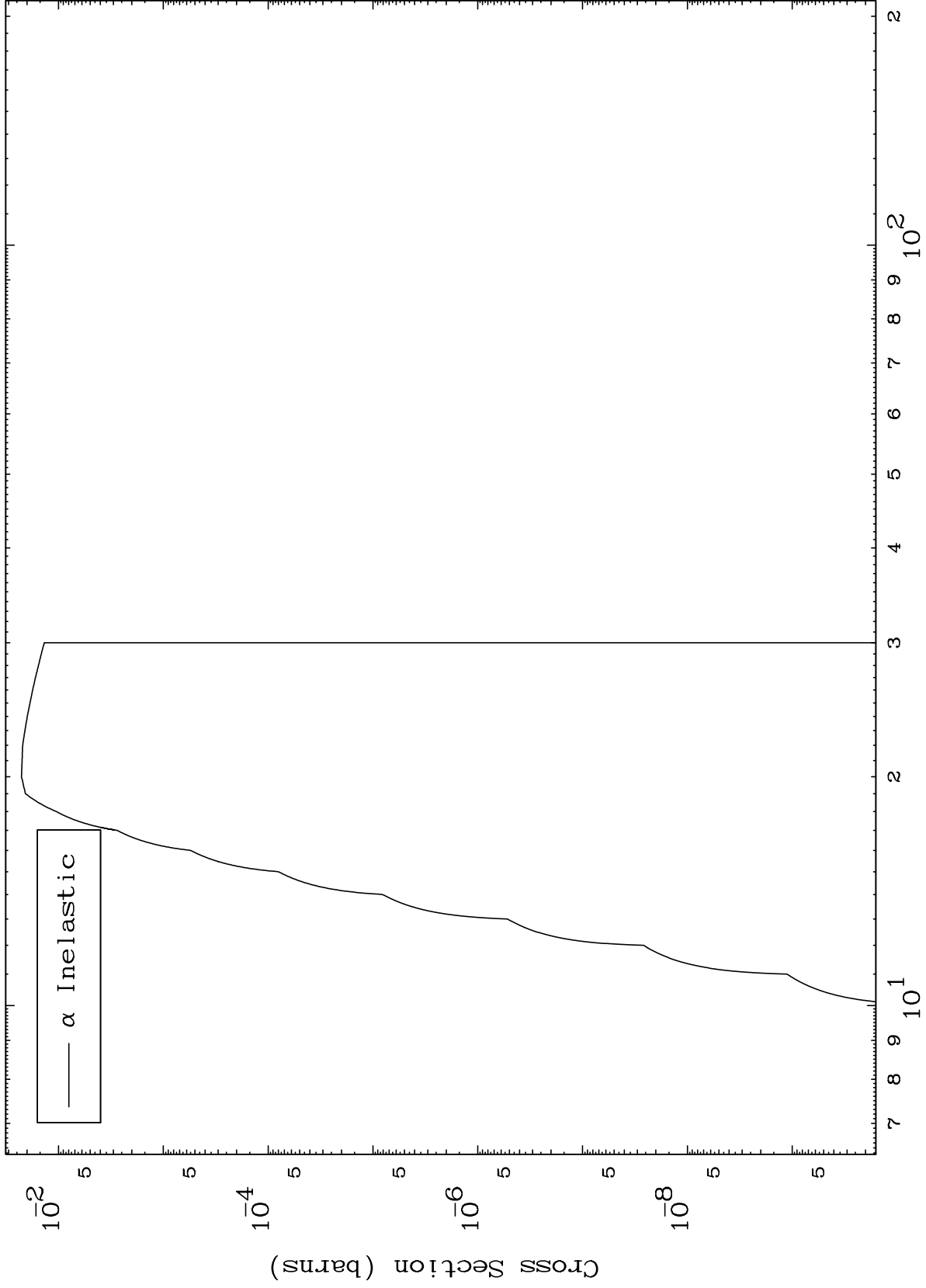


MAT 7744

( $\alpha, n'$ ) Level

77-Ir-197

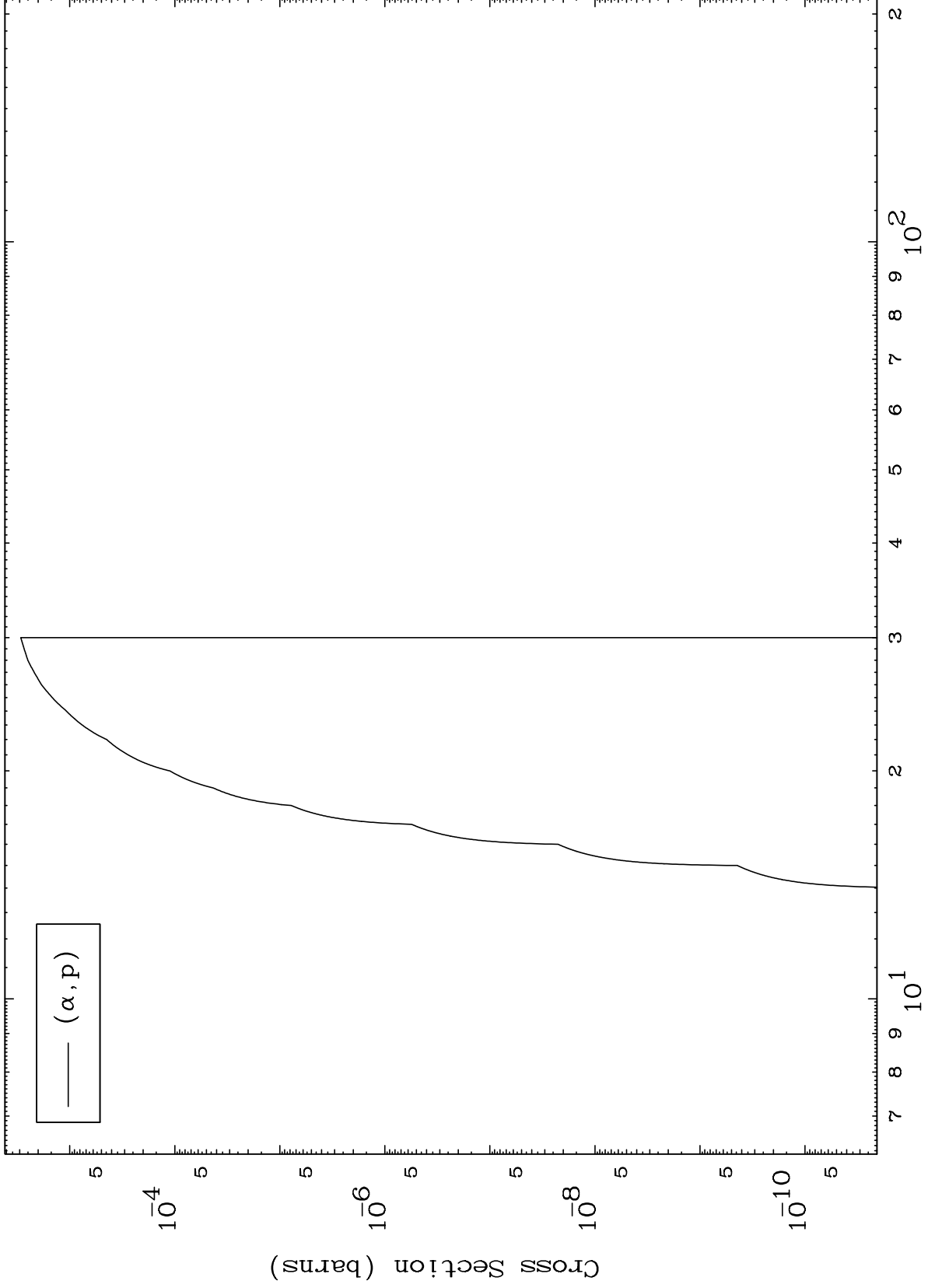
0 Kelvin Cross Sections



4

Incident Energy (MeV)

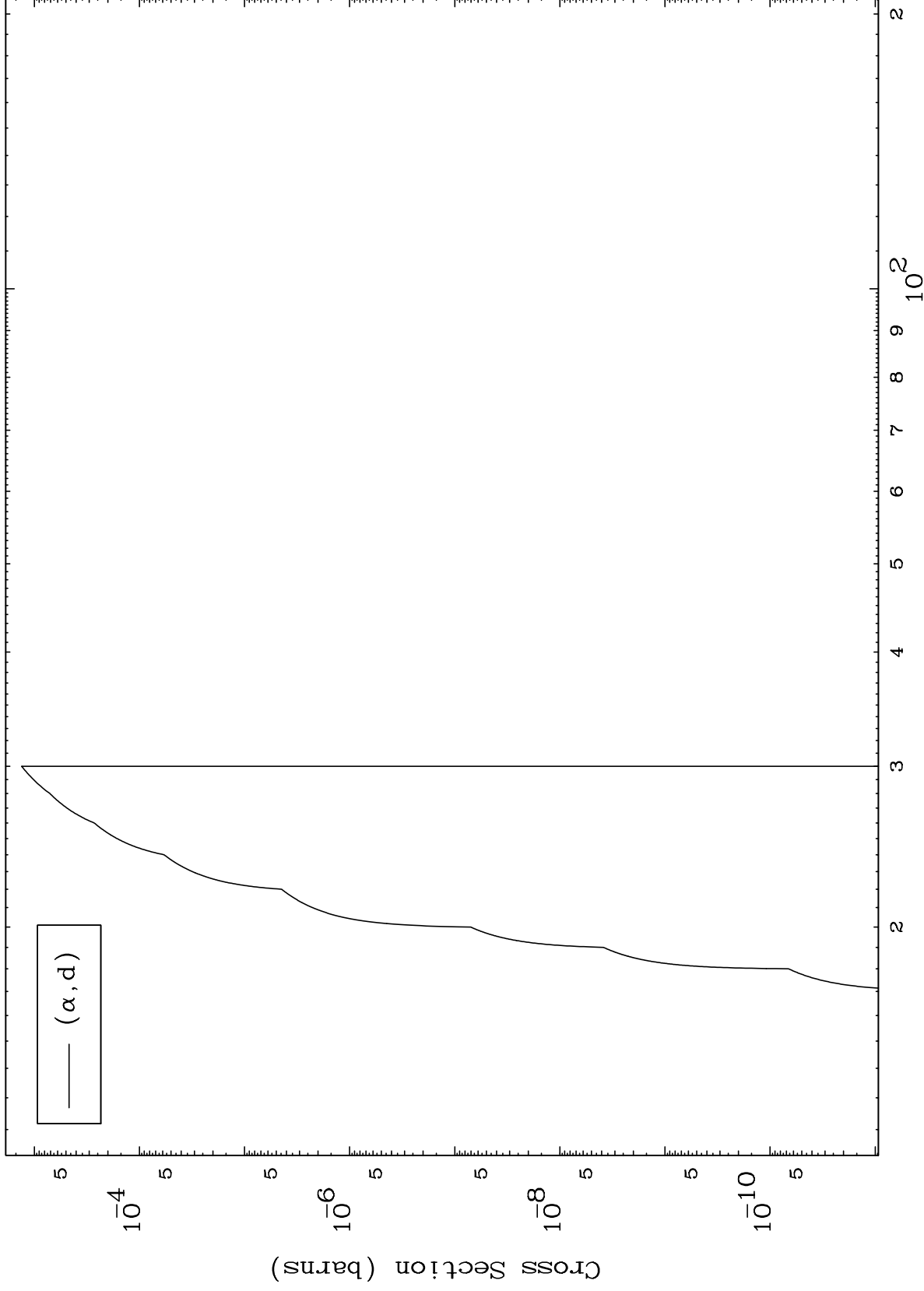
77-Ir-197

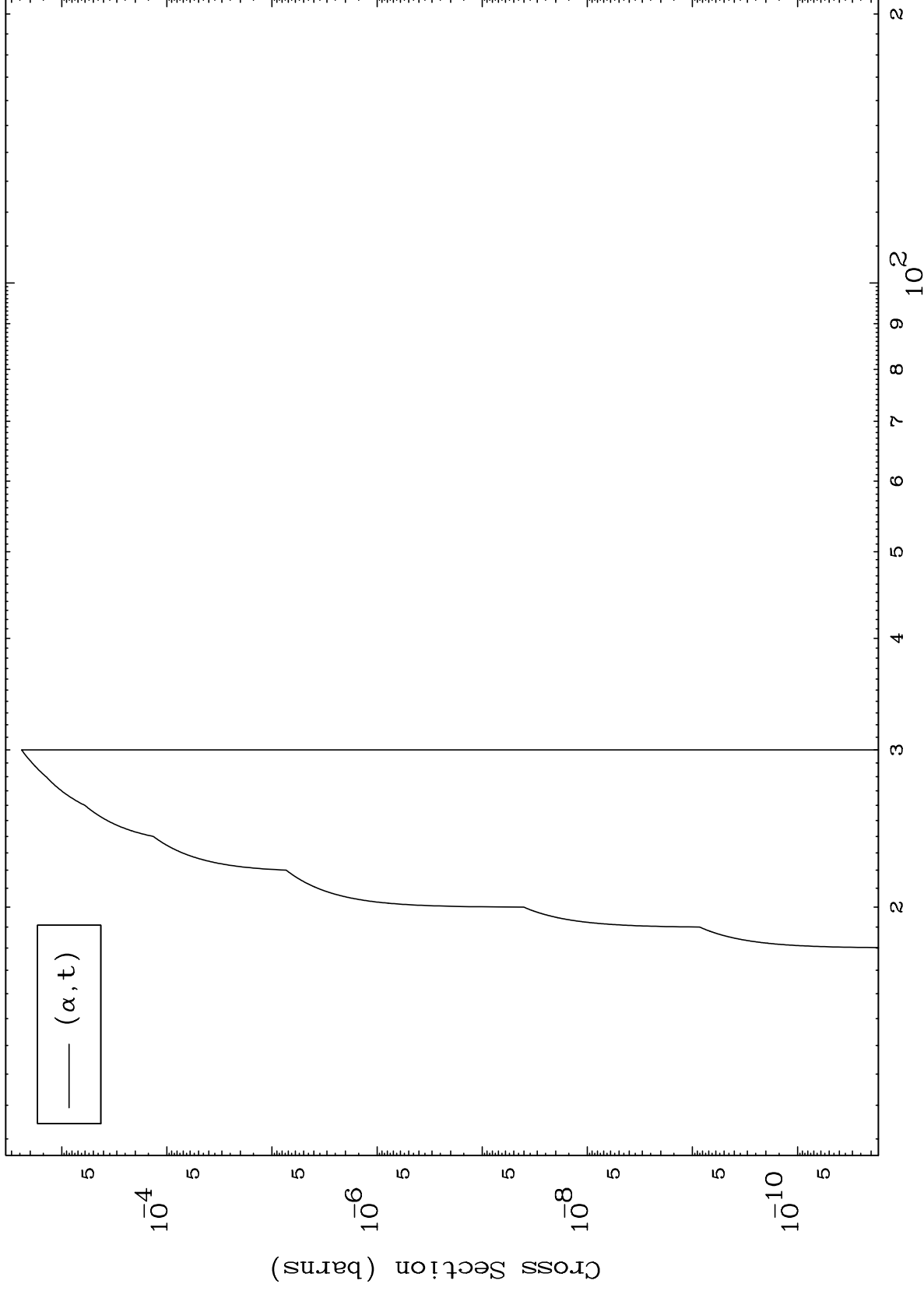


MAT 7744

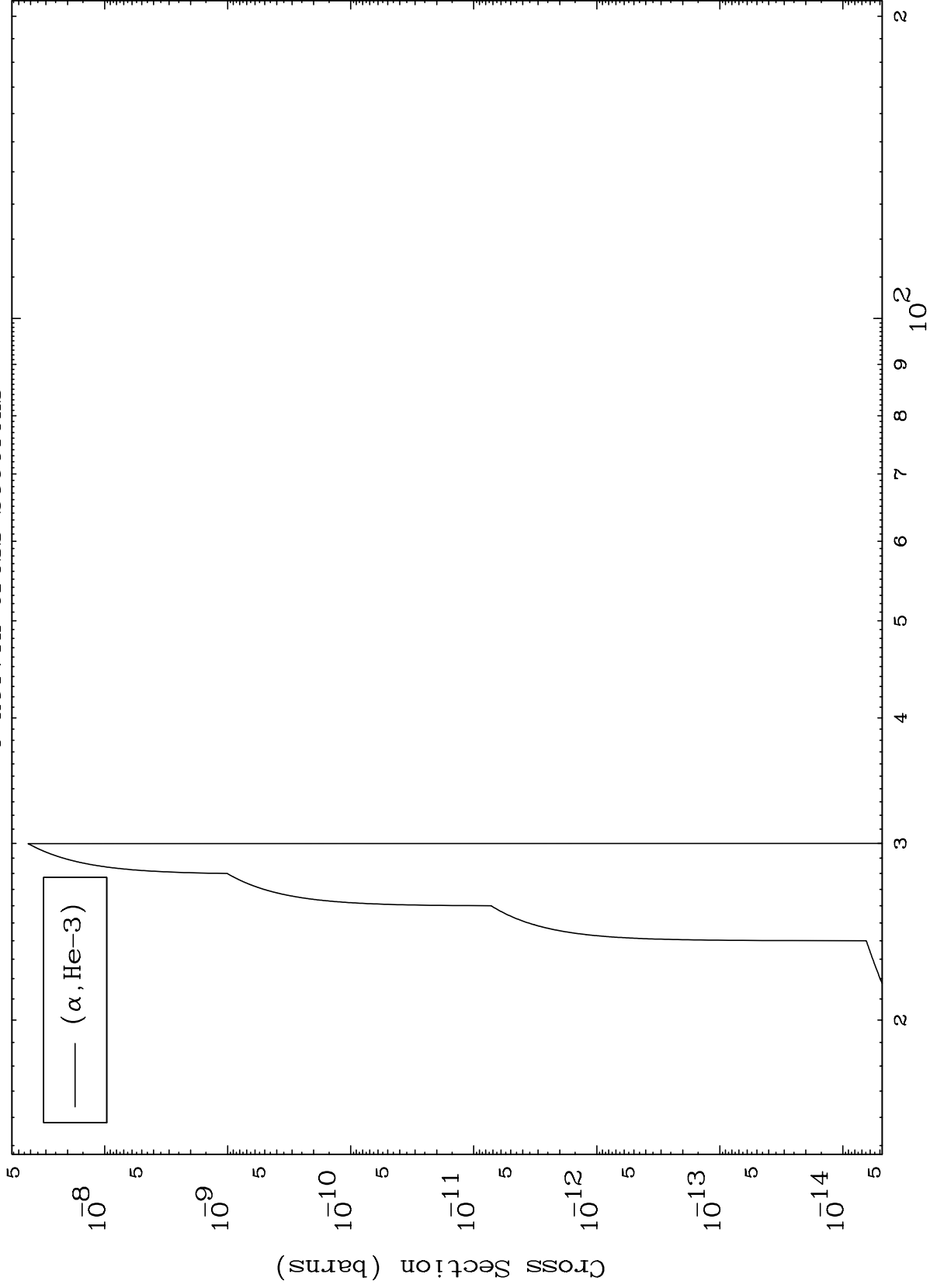
( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

77-Ir-197





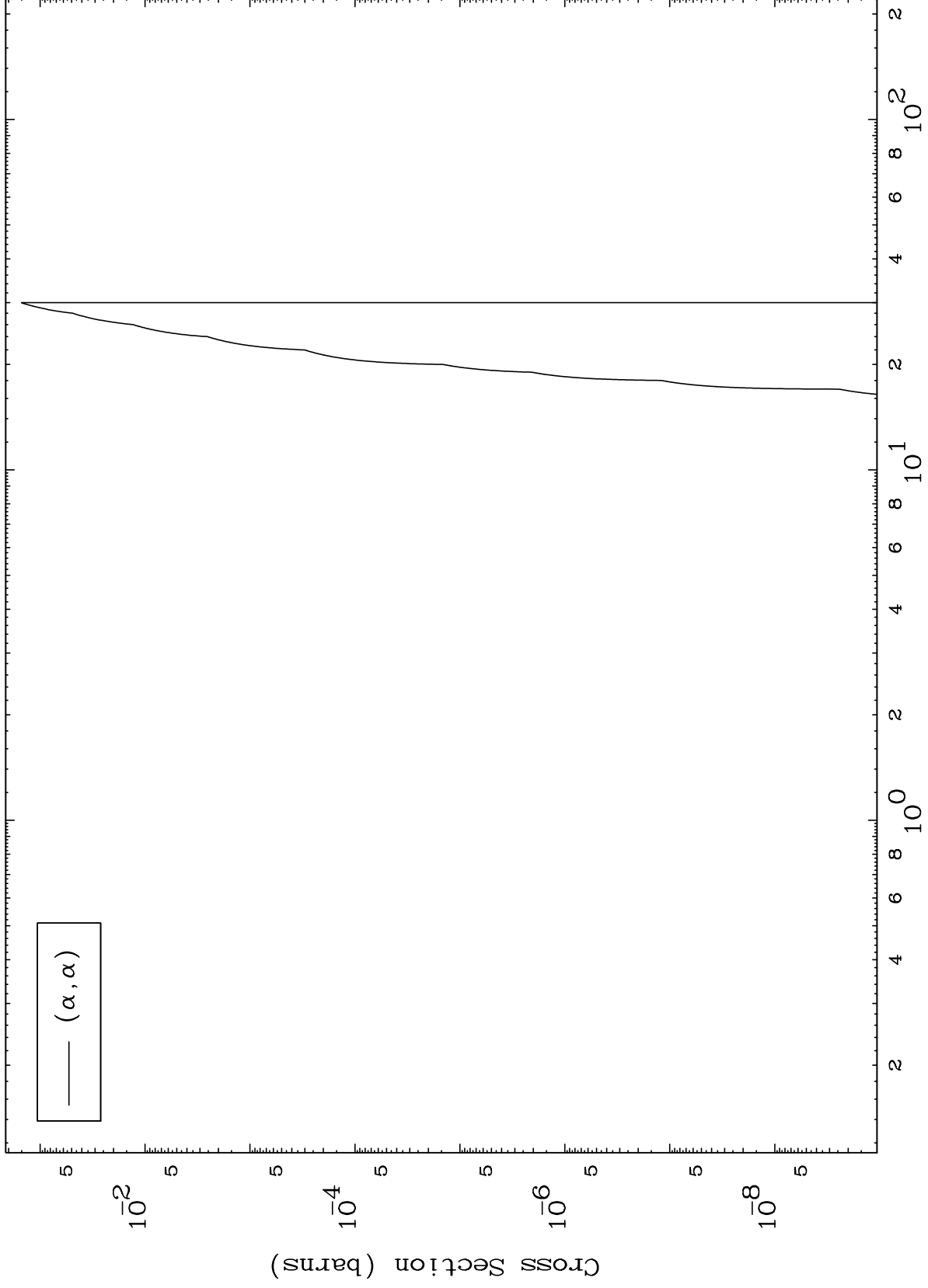




MAT 7744

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

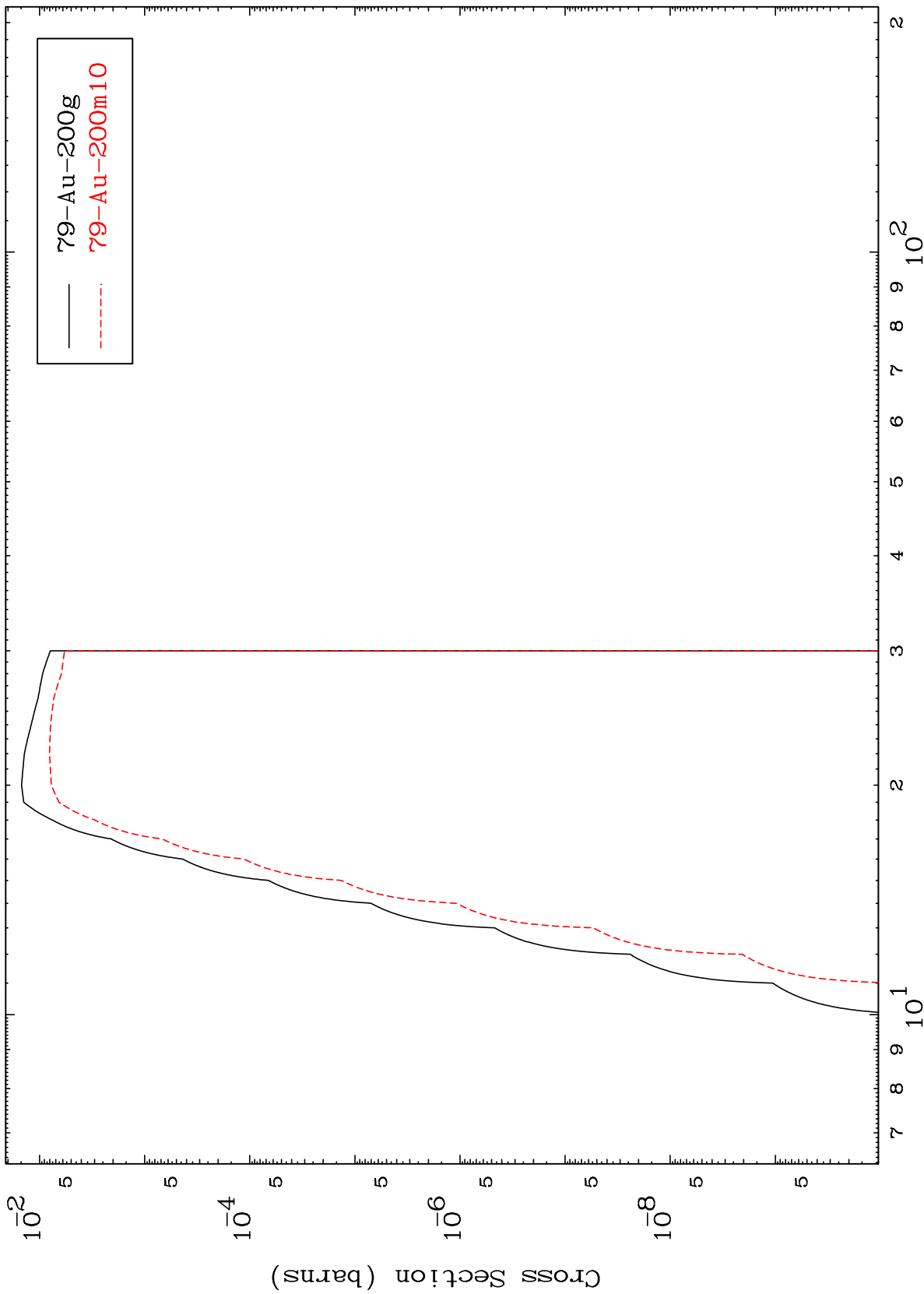
77-Ir-197



MAT 7744

77-Ir-197

$\alpha$  Inelastic  
Radionuclide Production Cross Section

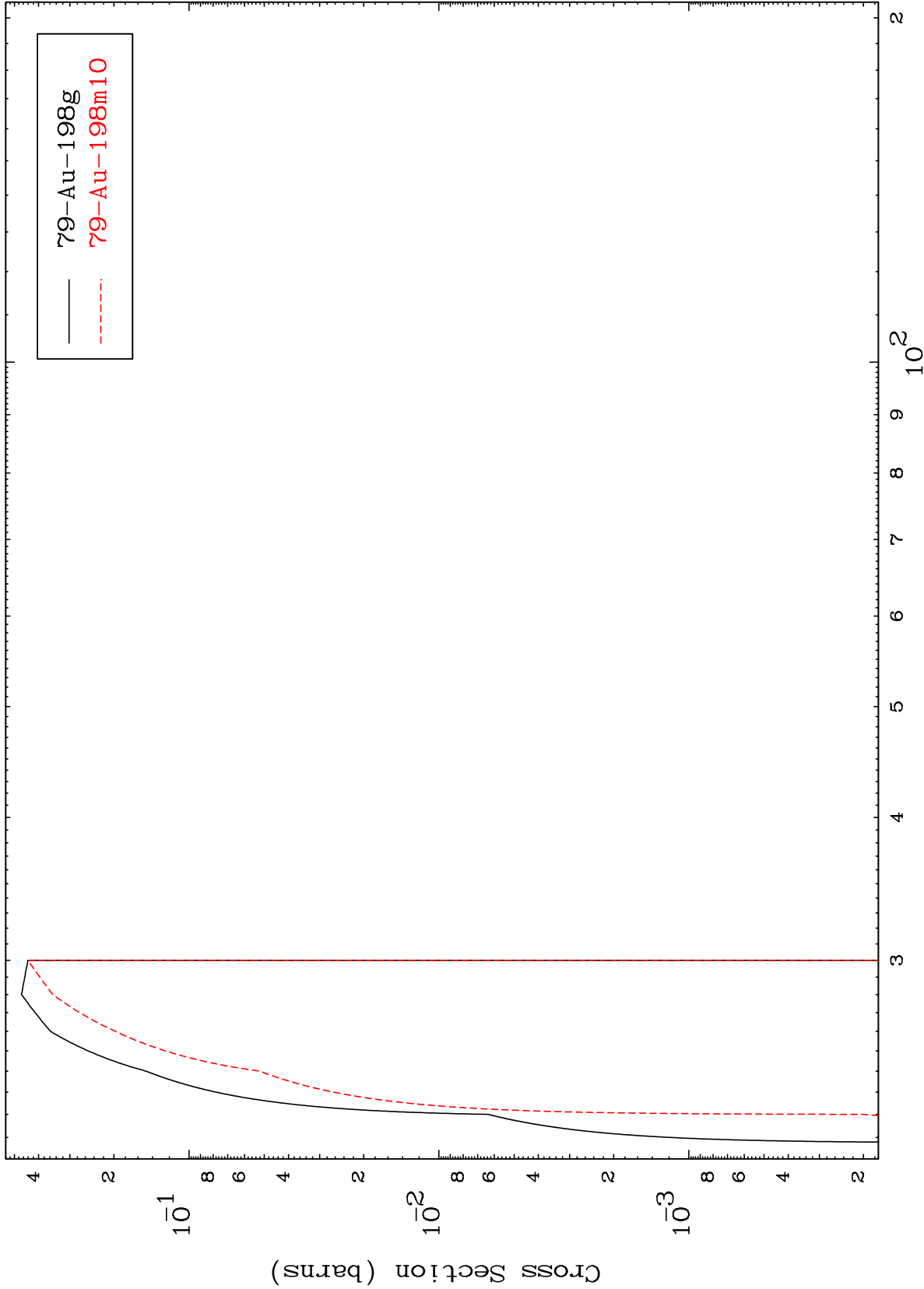


77-Ir-197

Incident Energy (MeV)

10

Radionuclide Production Cross Section

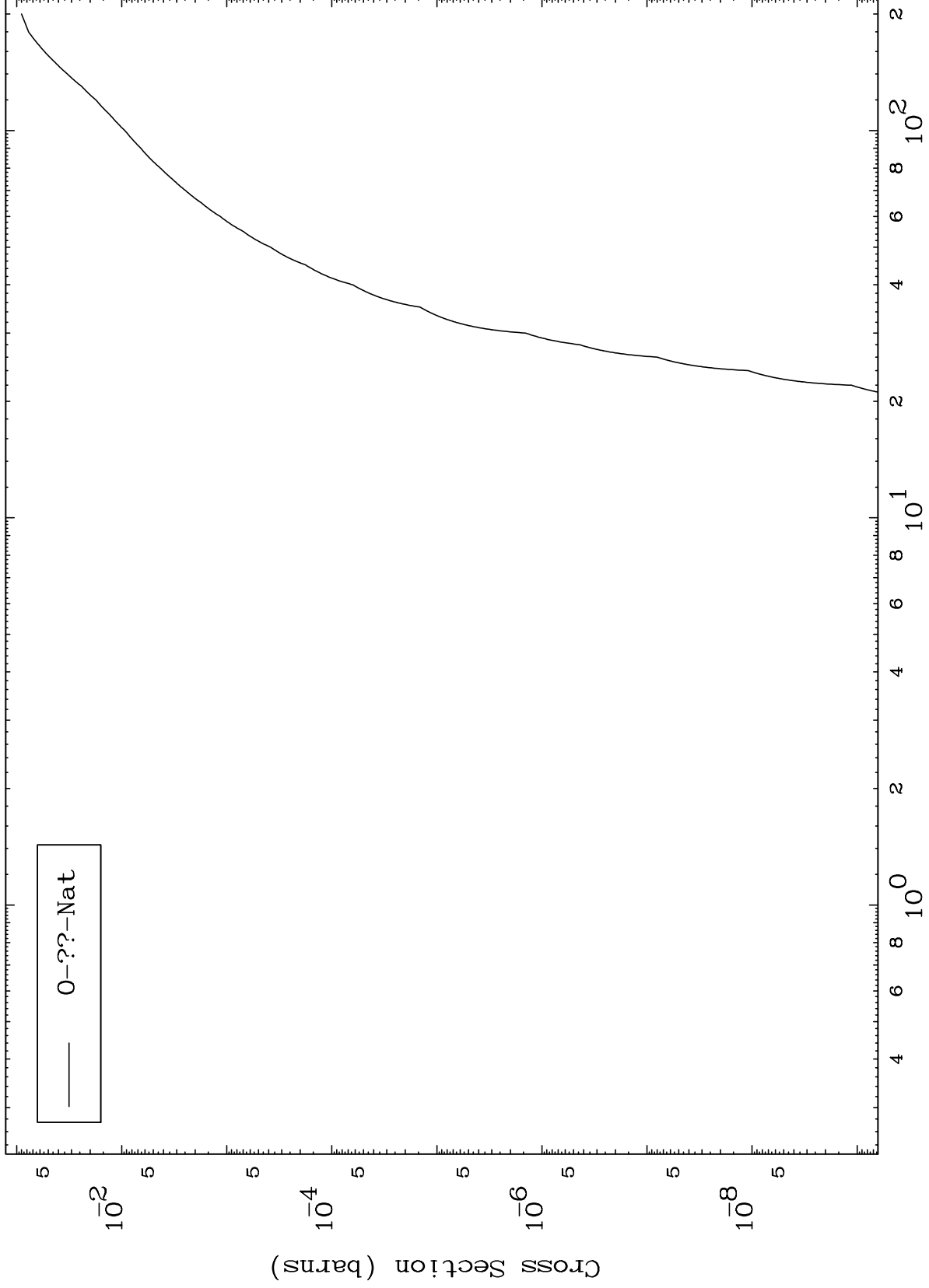


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$\alpha$  Fission

77-Ir-197

Radionuclide Production Cross Section



12

Incident Energy (MeV)

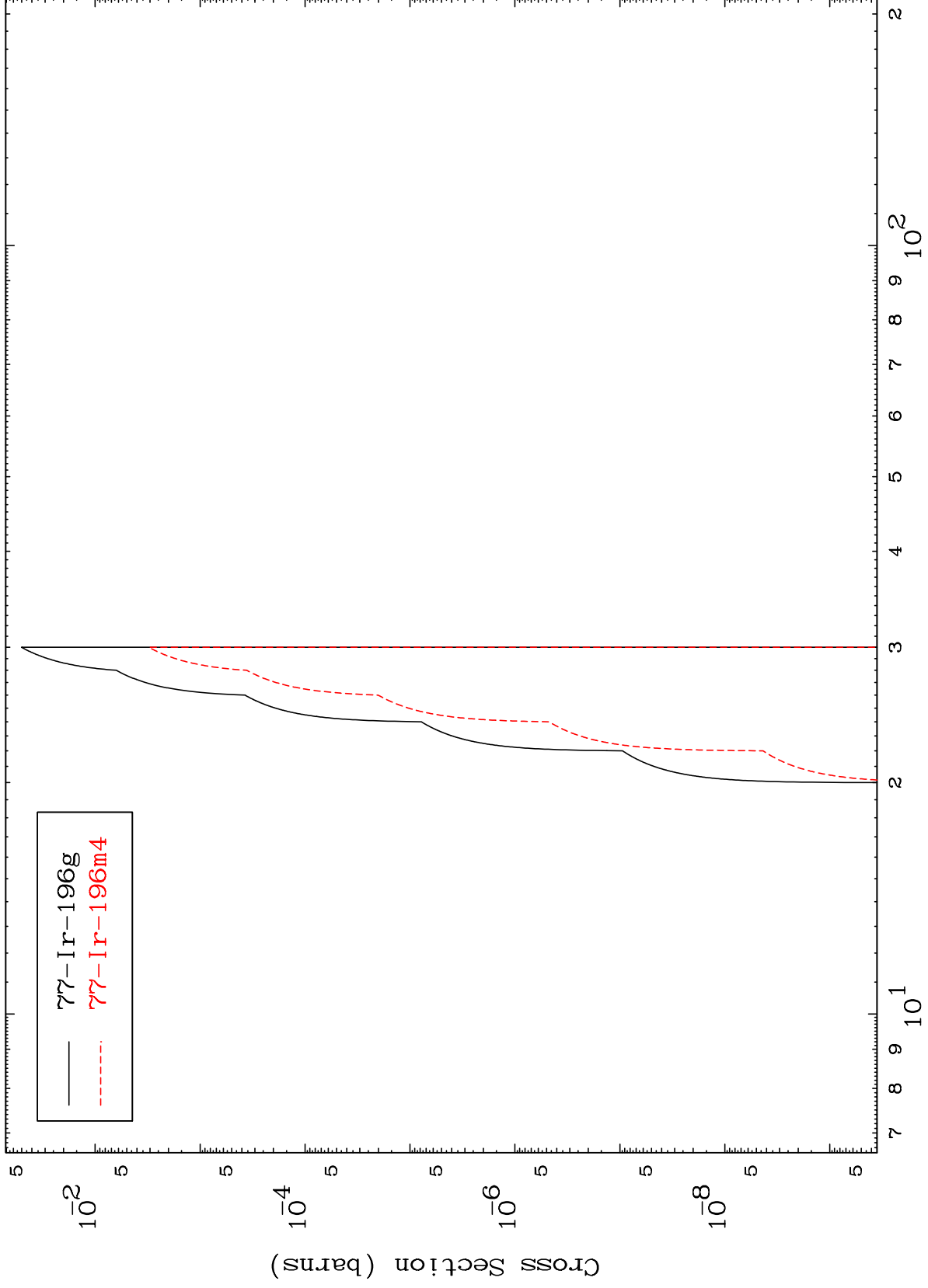
77-Ir-197

MAT 7744

77-Ir-197

( $\alpha, n'$ )  $\alpha$

Radionuclide Production Cross Section



13

Incident Energy (MeV)

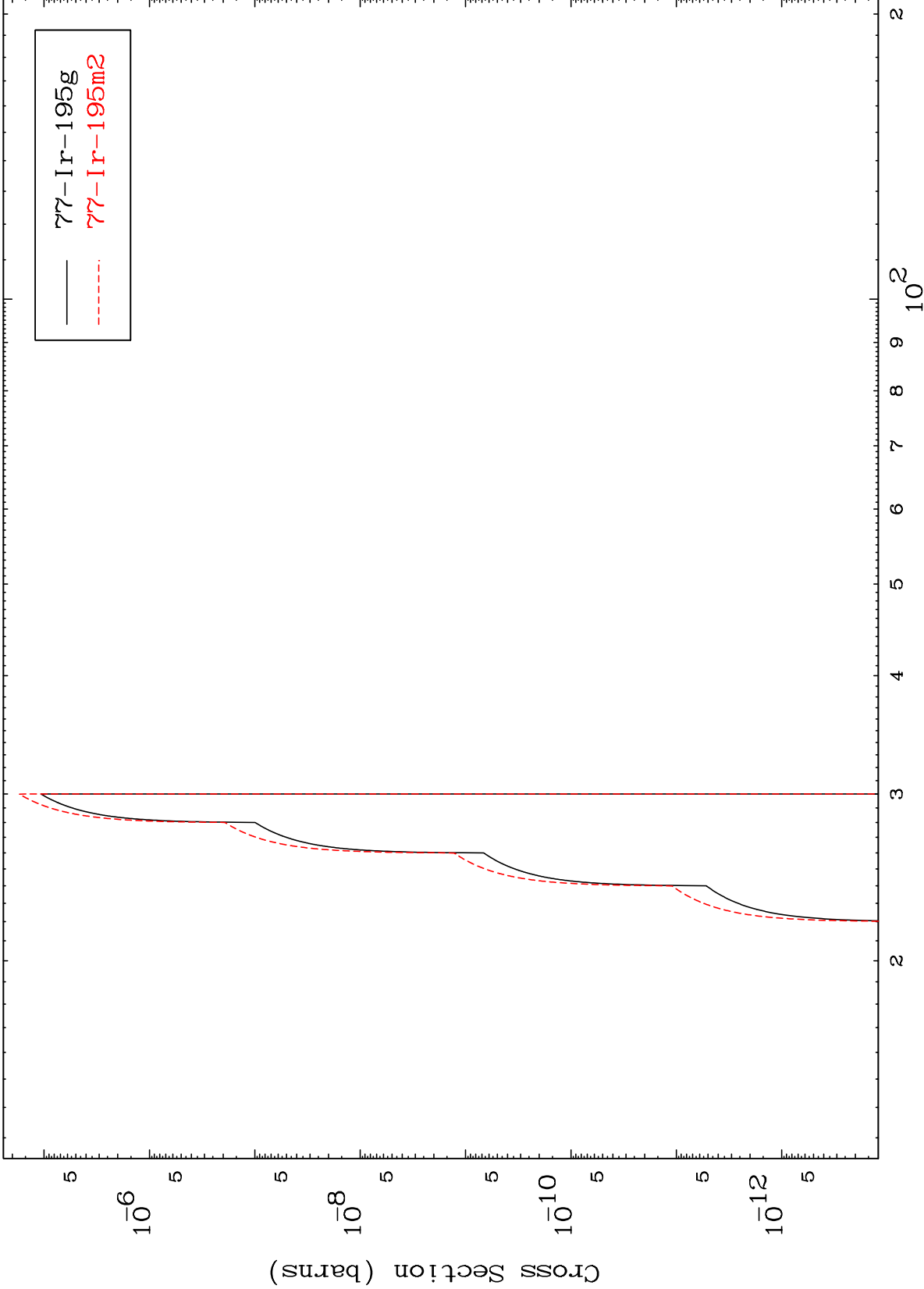
77-Ir-197

MAT 7744

( $\alpha, 2n$ )  $\alpha$

77-Ir-197

Radionuclide Production Cross Section

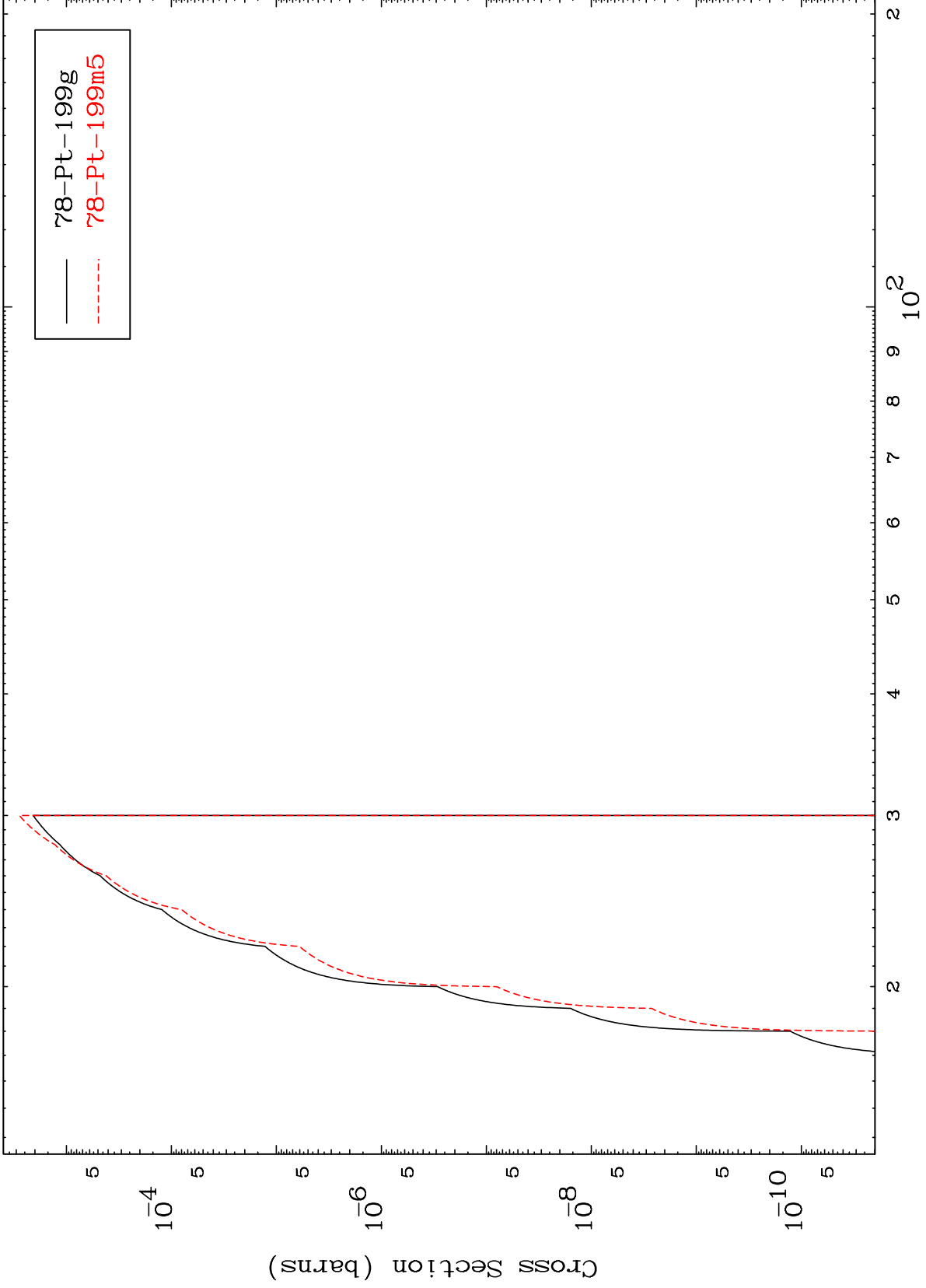


14

Incident Energy (MeV)

77-Ir-197

Radionuclide Production Cross Section



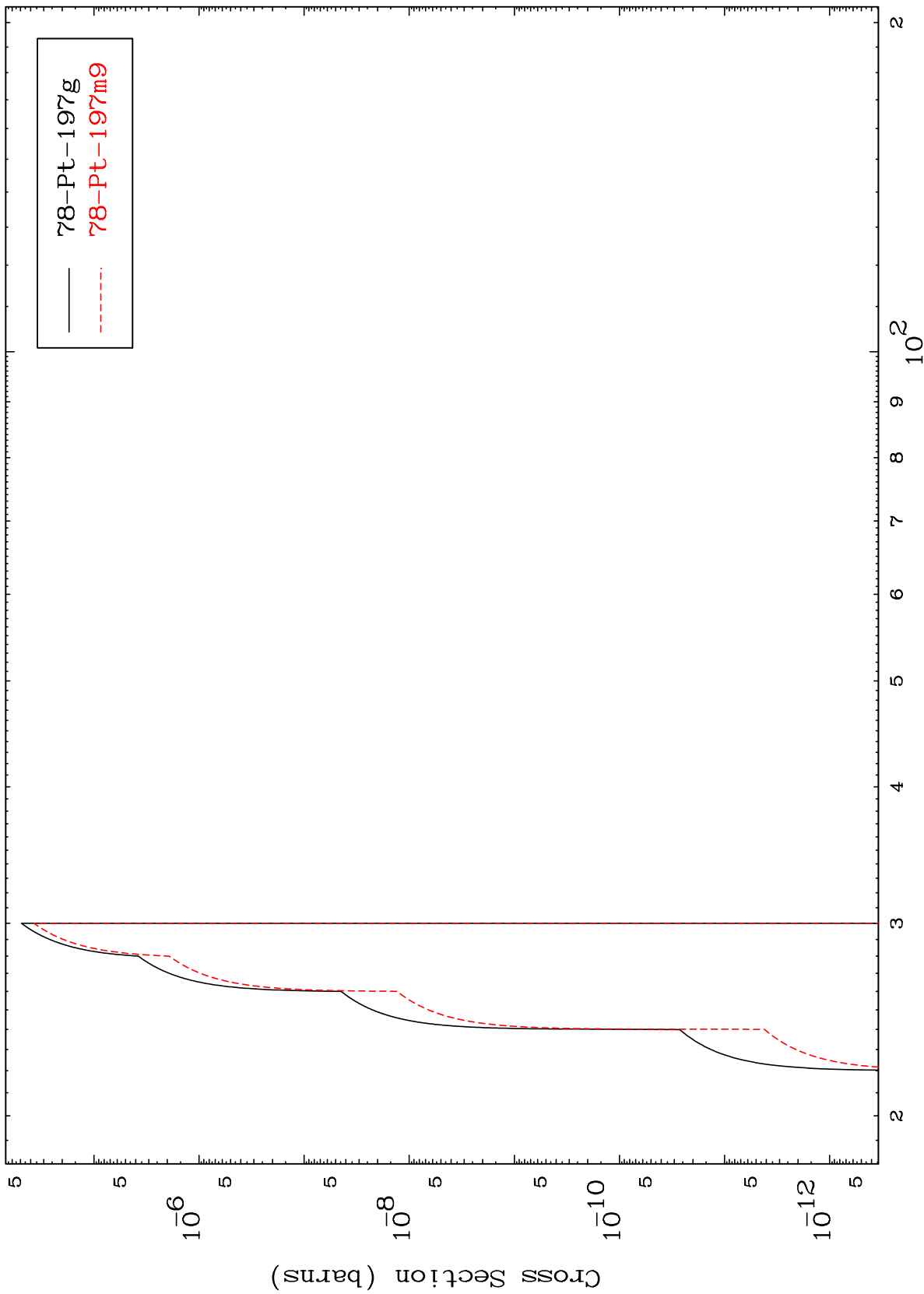


MAT 7744

$(\alpha, n')$  t

77-Ir-197

Radionuclide Production Cross Section



16

Incident Energy (MeV)

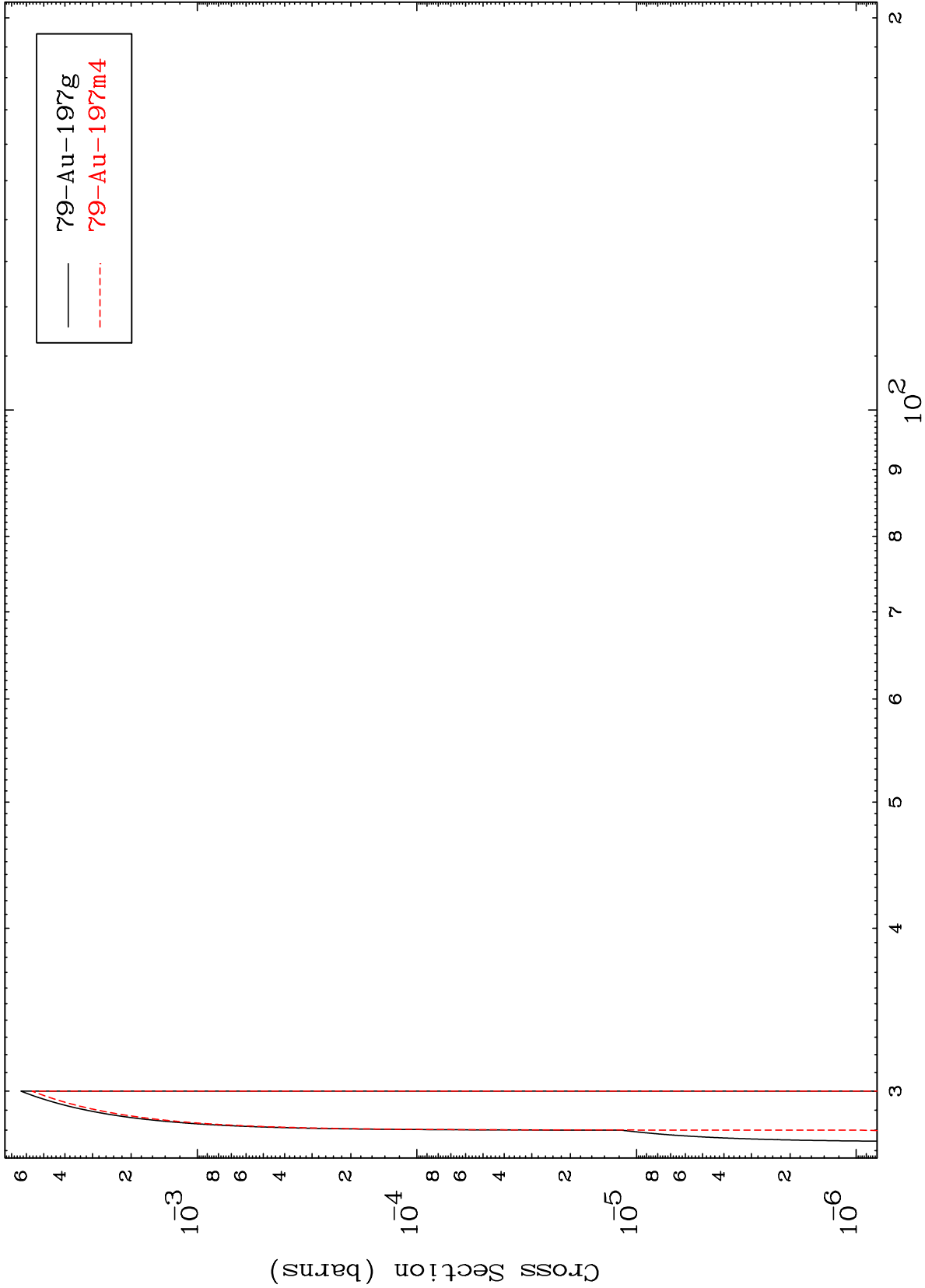
77-Ir-197

MAT 7744

( $\alpha, 4n$ )

77-Ir-197

Radionuclide Production Cross Section



17

Incident Energy (MeV)

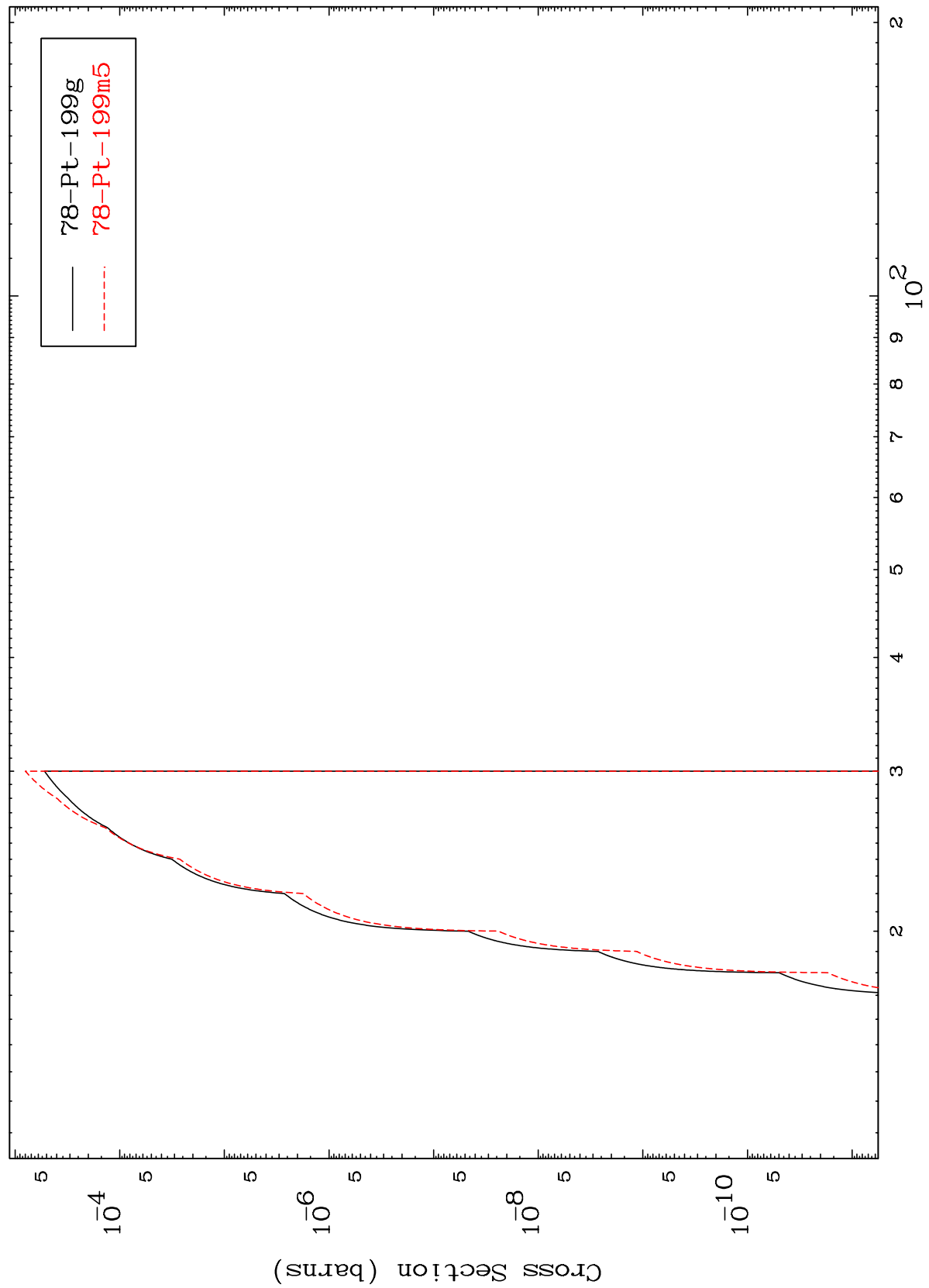
77-Ir-197

MAT 7744

( $\alpha, d$ )

77-Ir-197

Radionuclide Production Cross Section



18

Incident Energy (MeV)

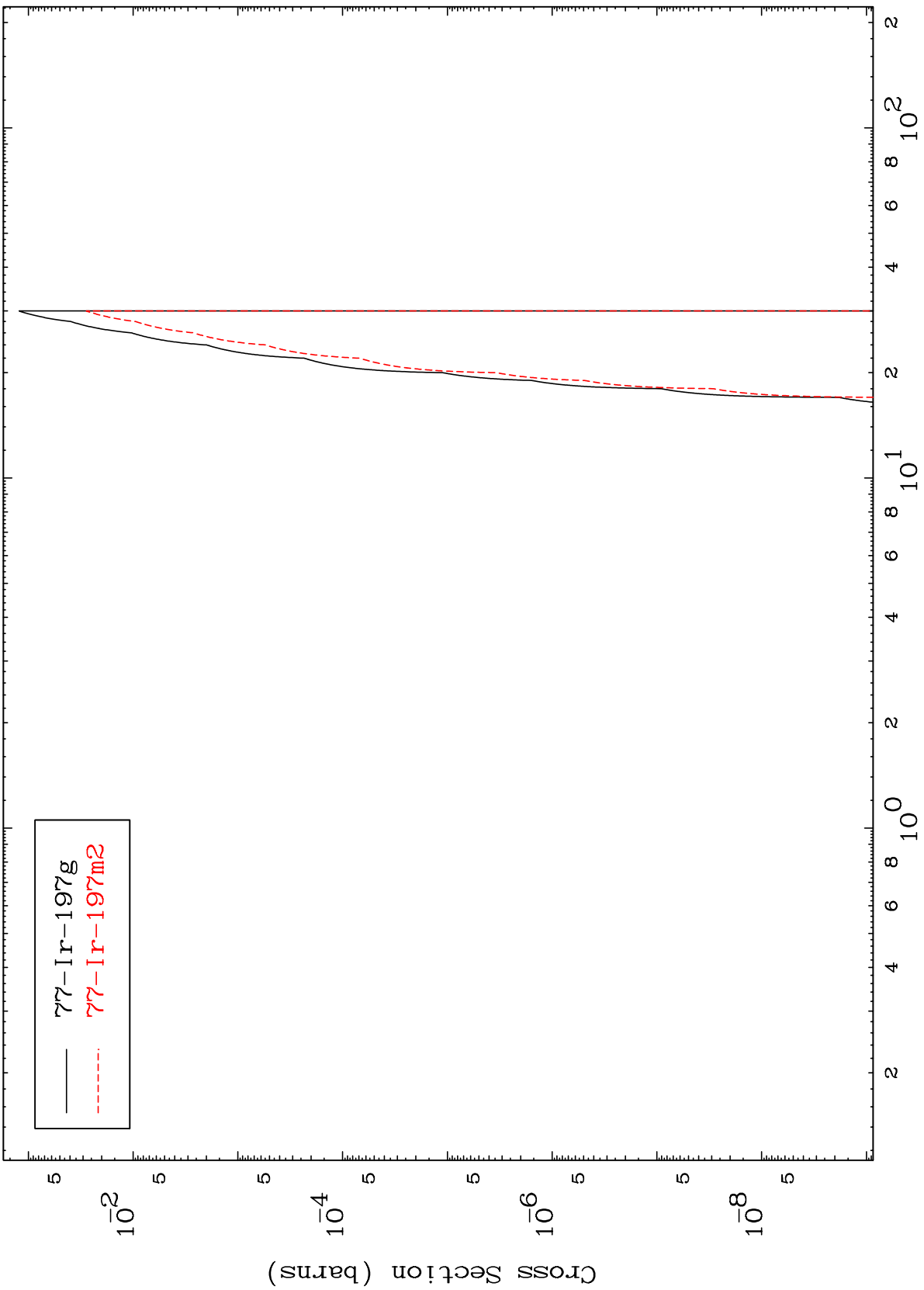
77-Ir-197

MAT 7744

( $\alpha, \alpha$ )

<sup>77</sup>Ir-197

Radionuclide Production Cross Section



— <sup>77</sup>Ir-197g  
- - - <sup>77</sup>Ir-197m2