

Program EVALPLOT  
(Version 2018-1)

by

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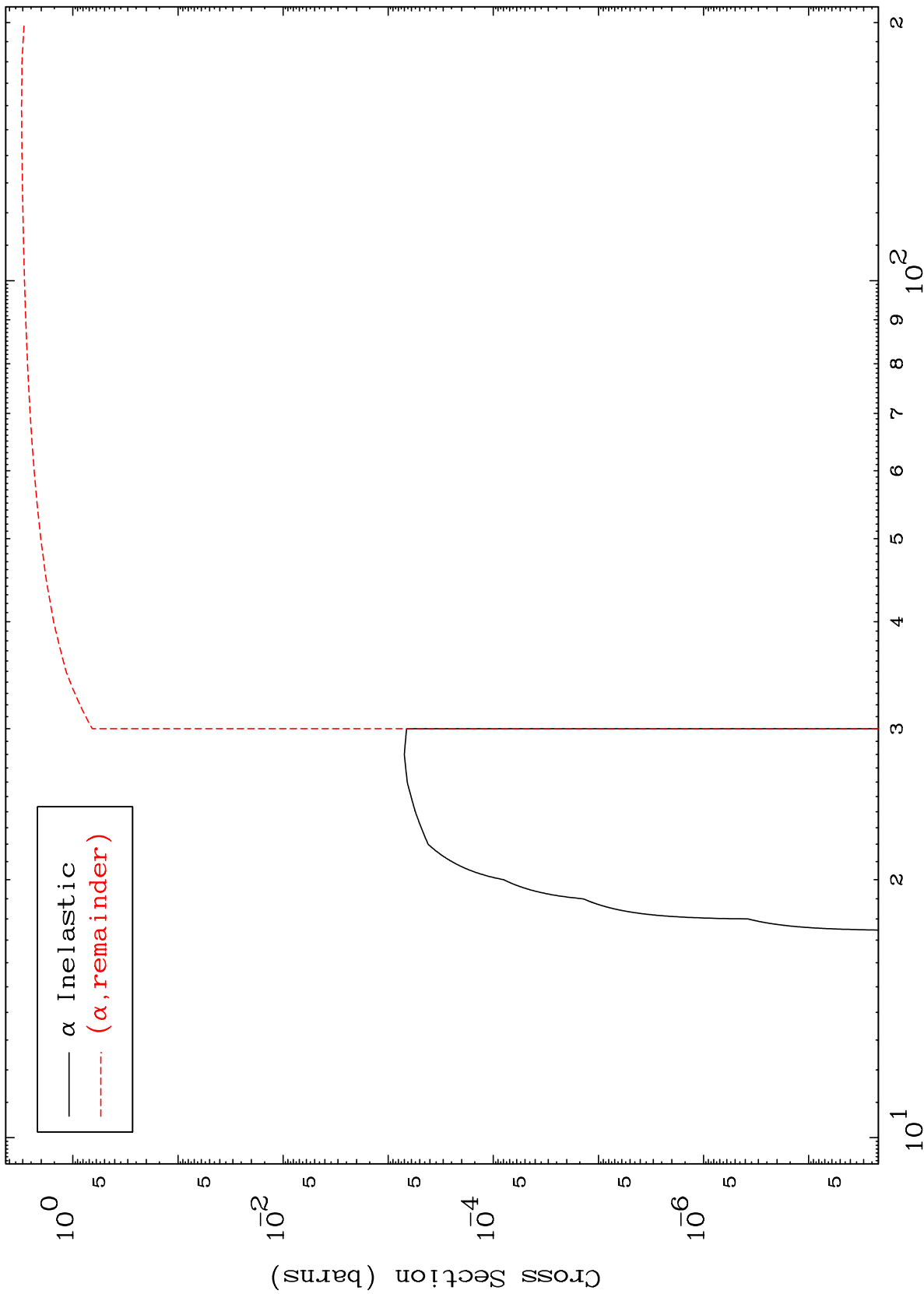
Press Mouse Button to Start

MAT 9989

$\alpha$  Major

103-Lr-255

0 Kelvin Cross Sections



—  $\alpha$  Inelastic  
- - - ( $\alpha$ , remainder)

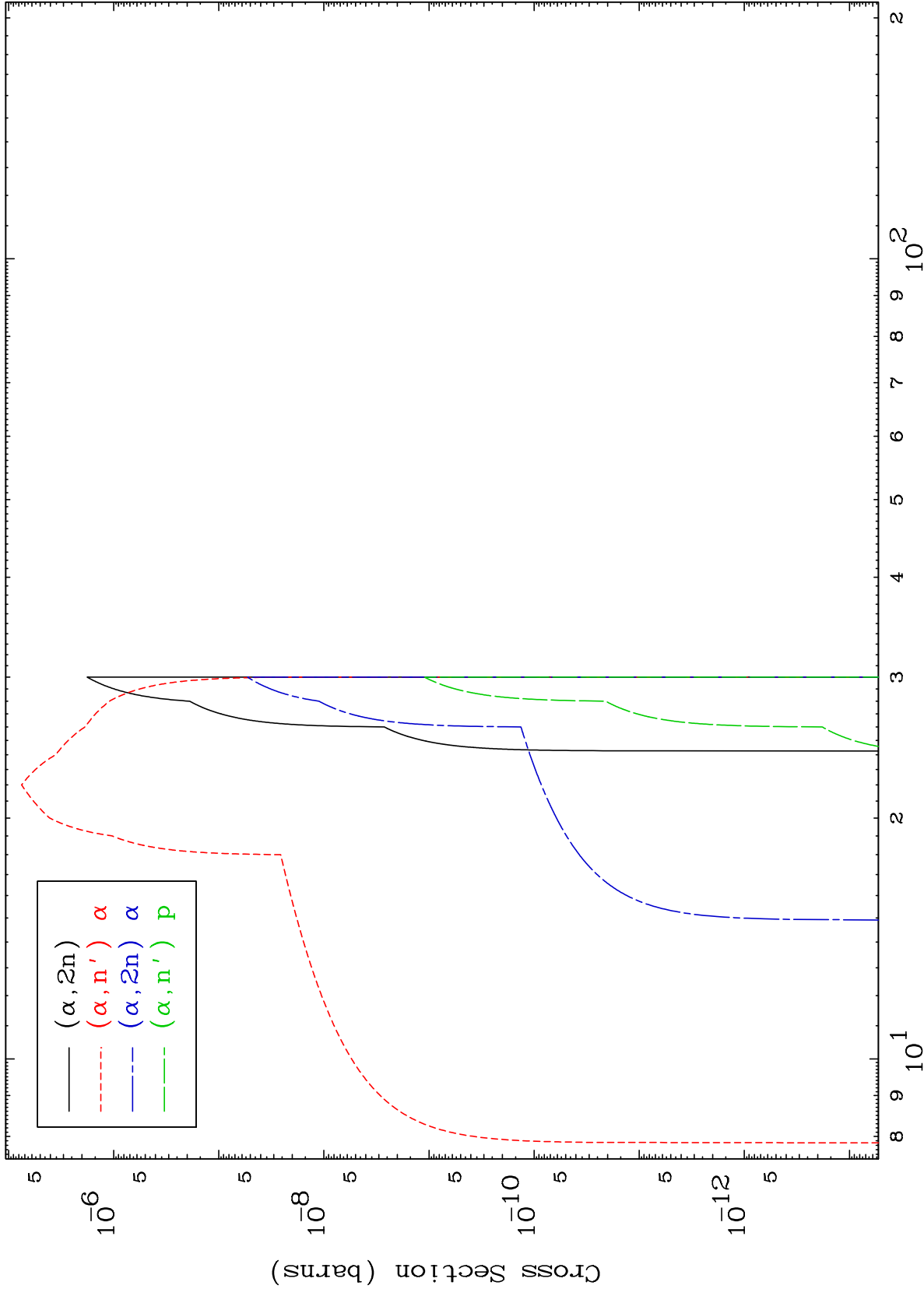
Incident Energy (MeV)

103-Lr-255

MAT 9989

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

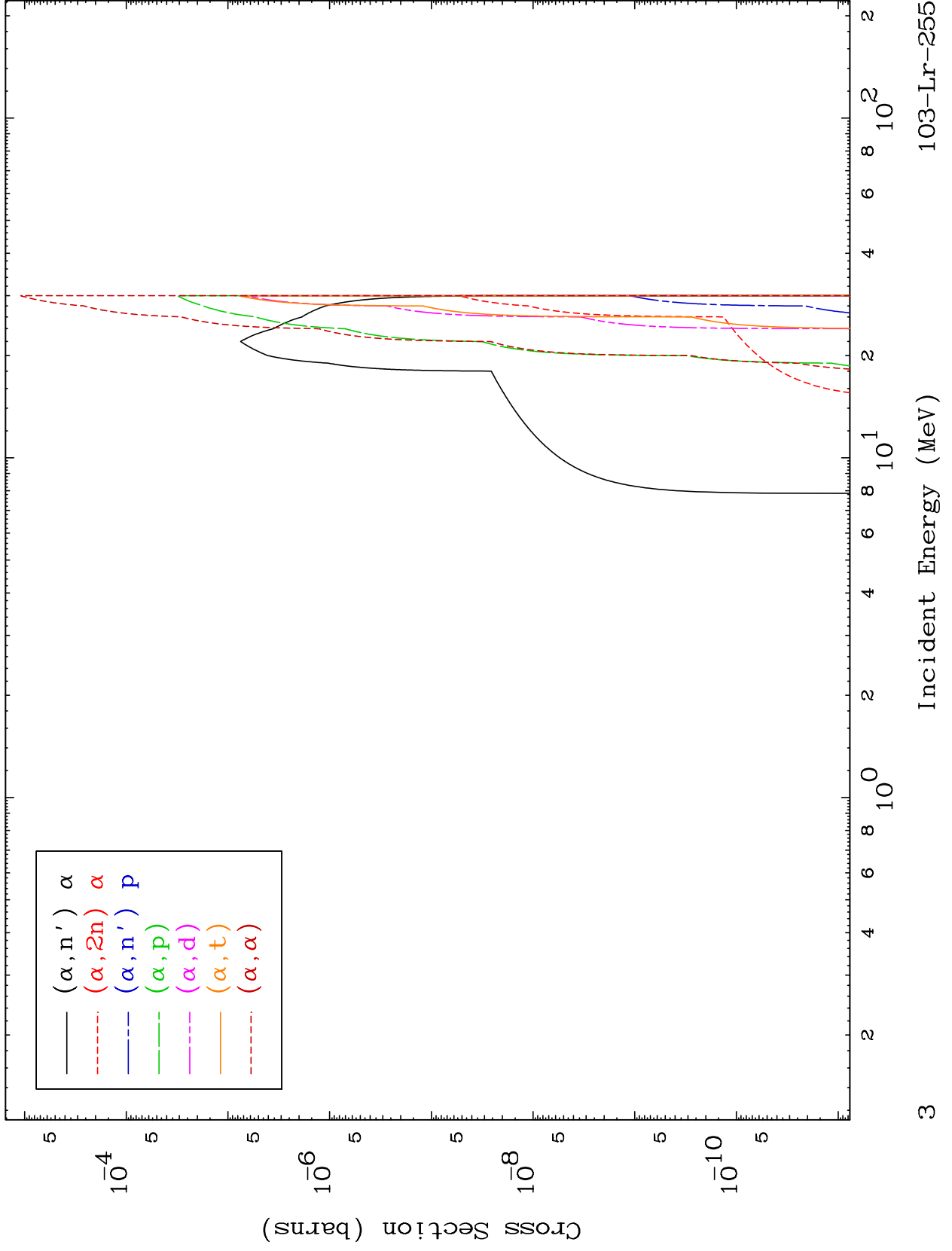
103-Lr-255



103-Lr-255

Incident Energy (MeV)

2

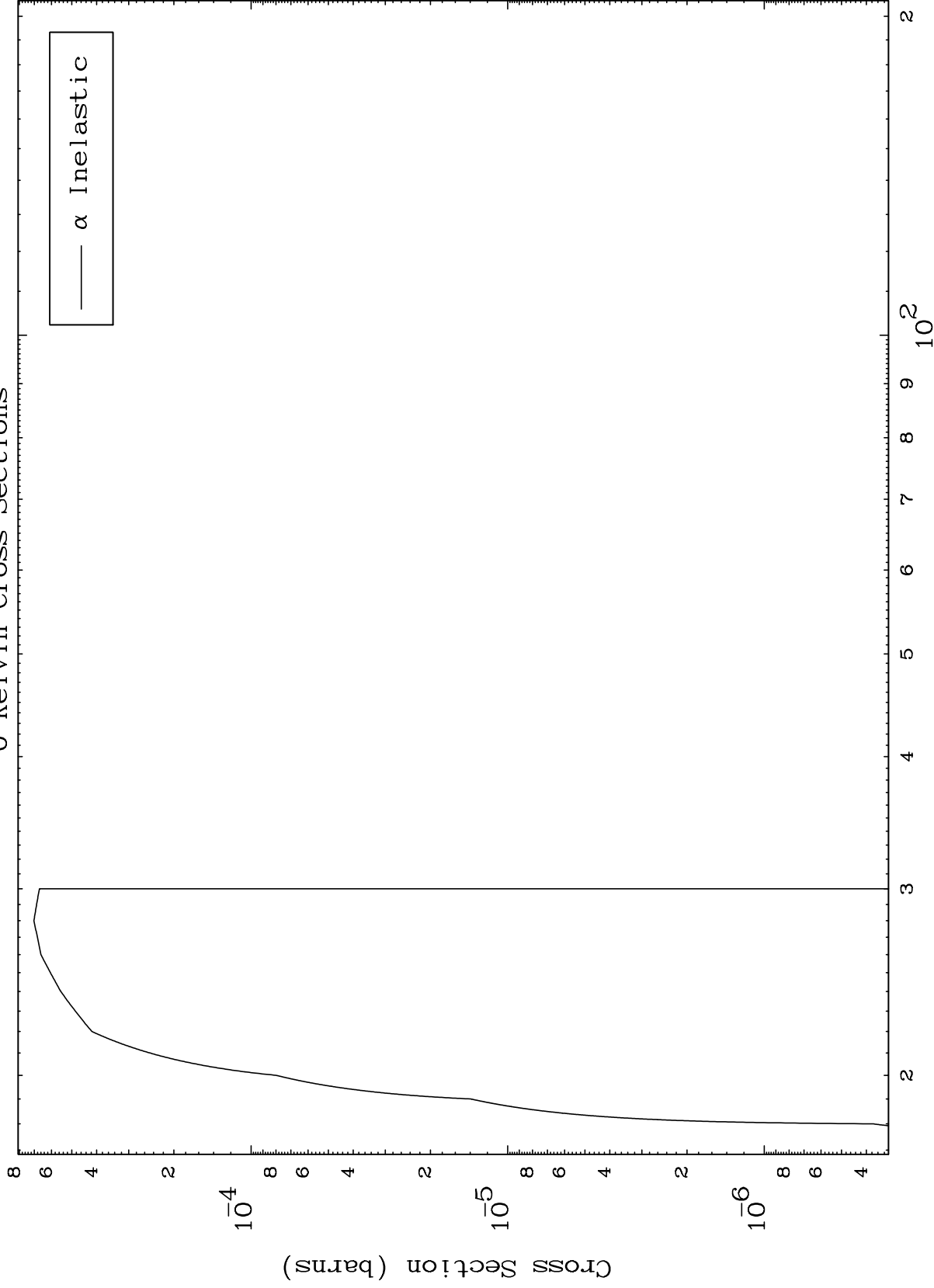


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( $\alpha, n'$ ) Level

103-Lr-255

0 Kelvin Cross Sections

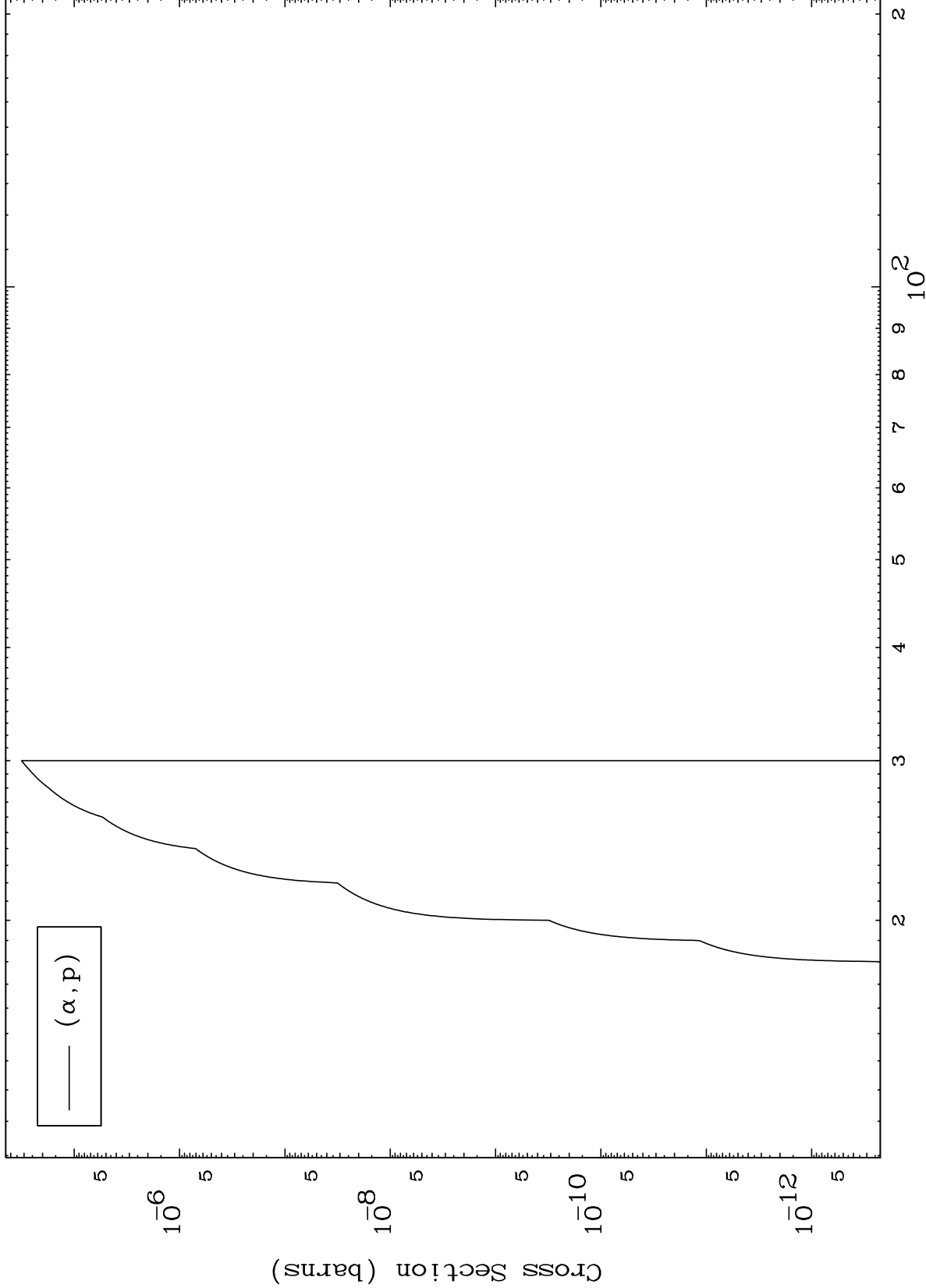


$\alpha$  Inelastic

4

Incident Energy (MeV)

103-Lr-255

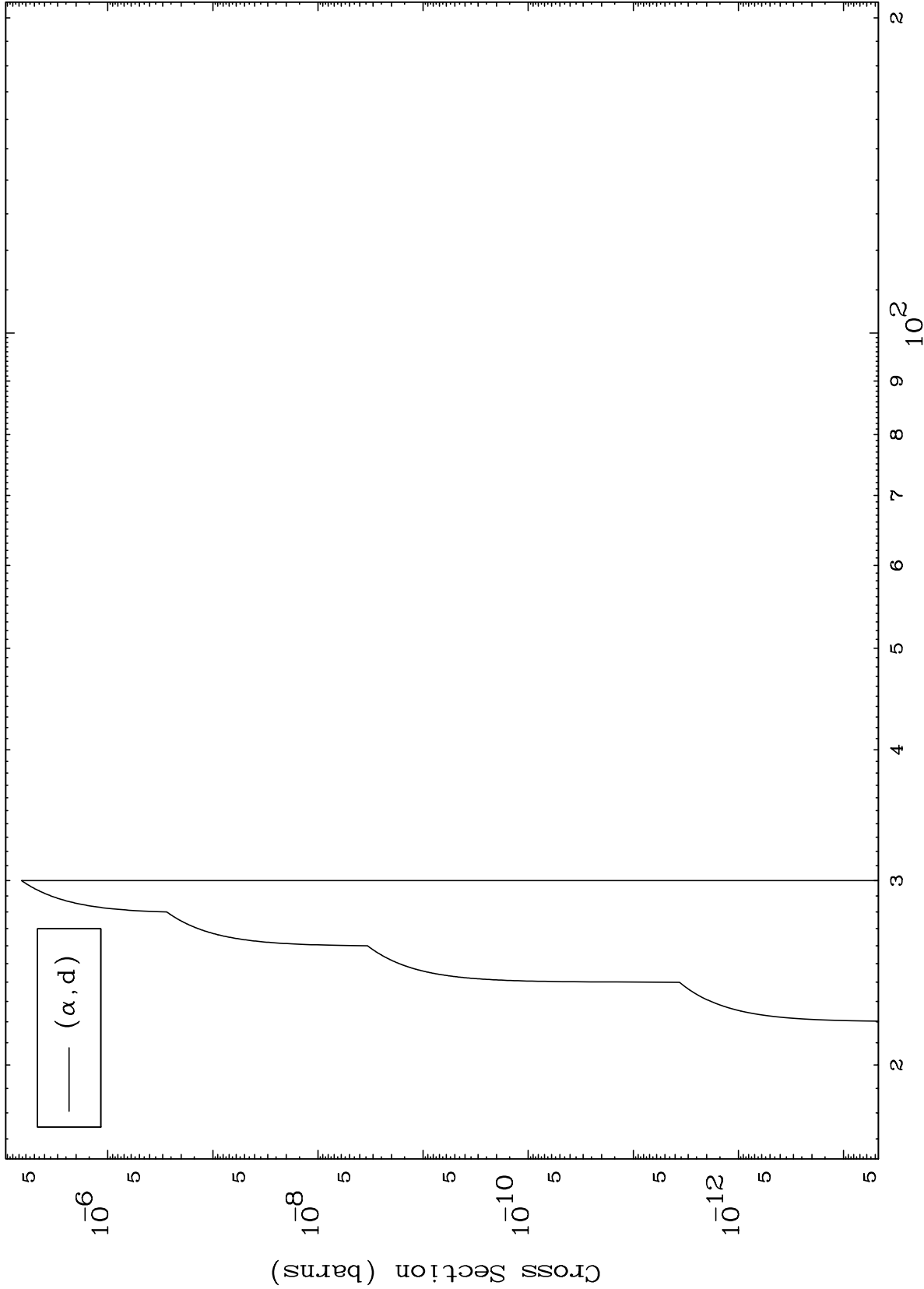


MAT 9989

( $\alpha, d$ ) Levels

103-Lr-255

0 Kelvin Cross Sections



6

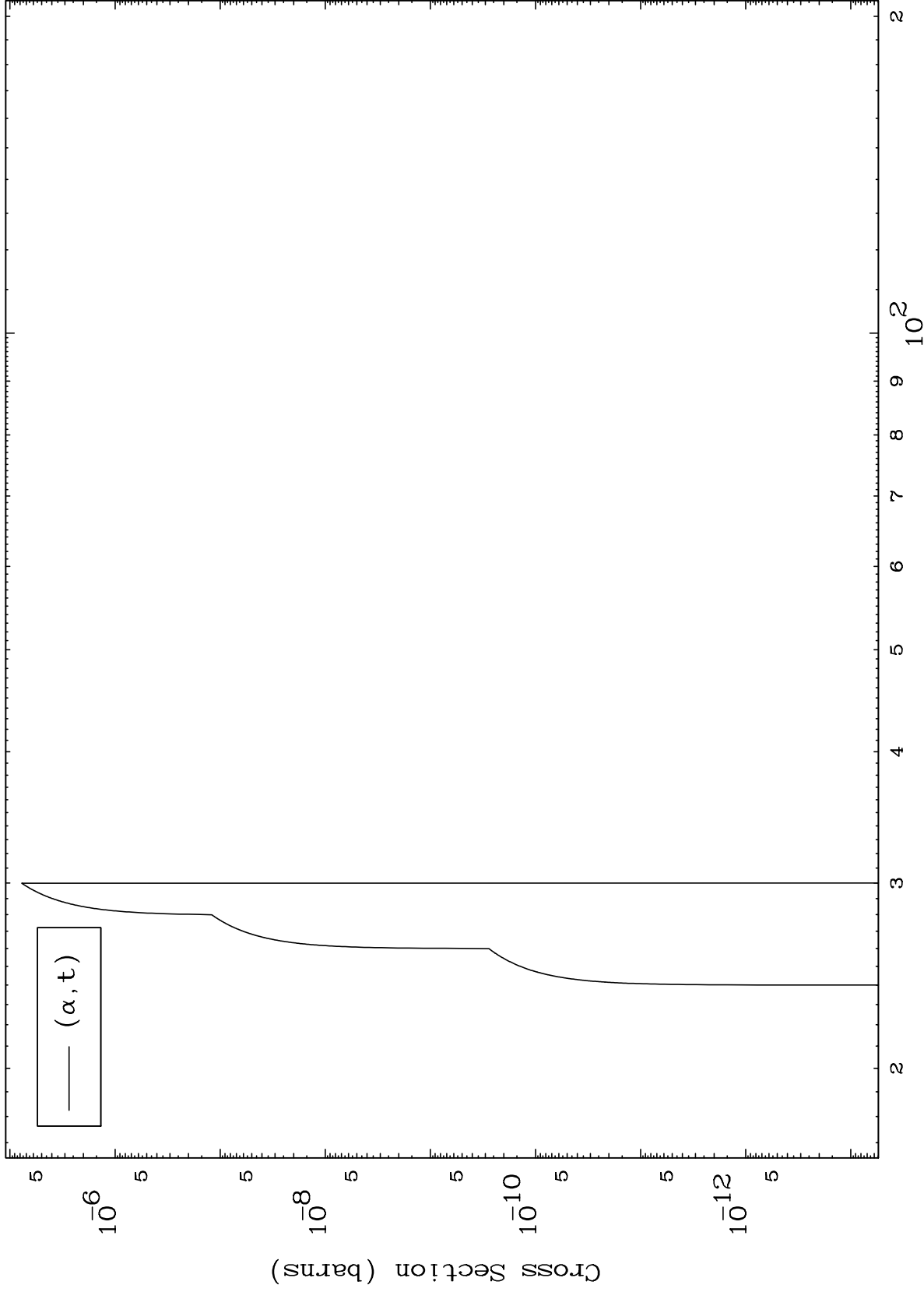
Incident Energy (MeV)

103-Lr-255

MAT 9989

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

103-Lr-255



7

Incident Energy (MeV)

103-Lr-255

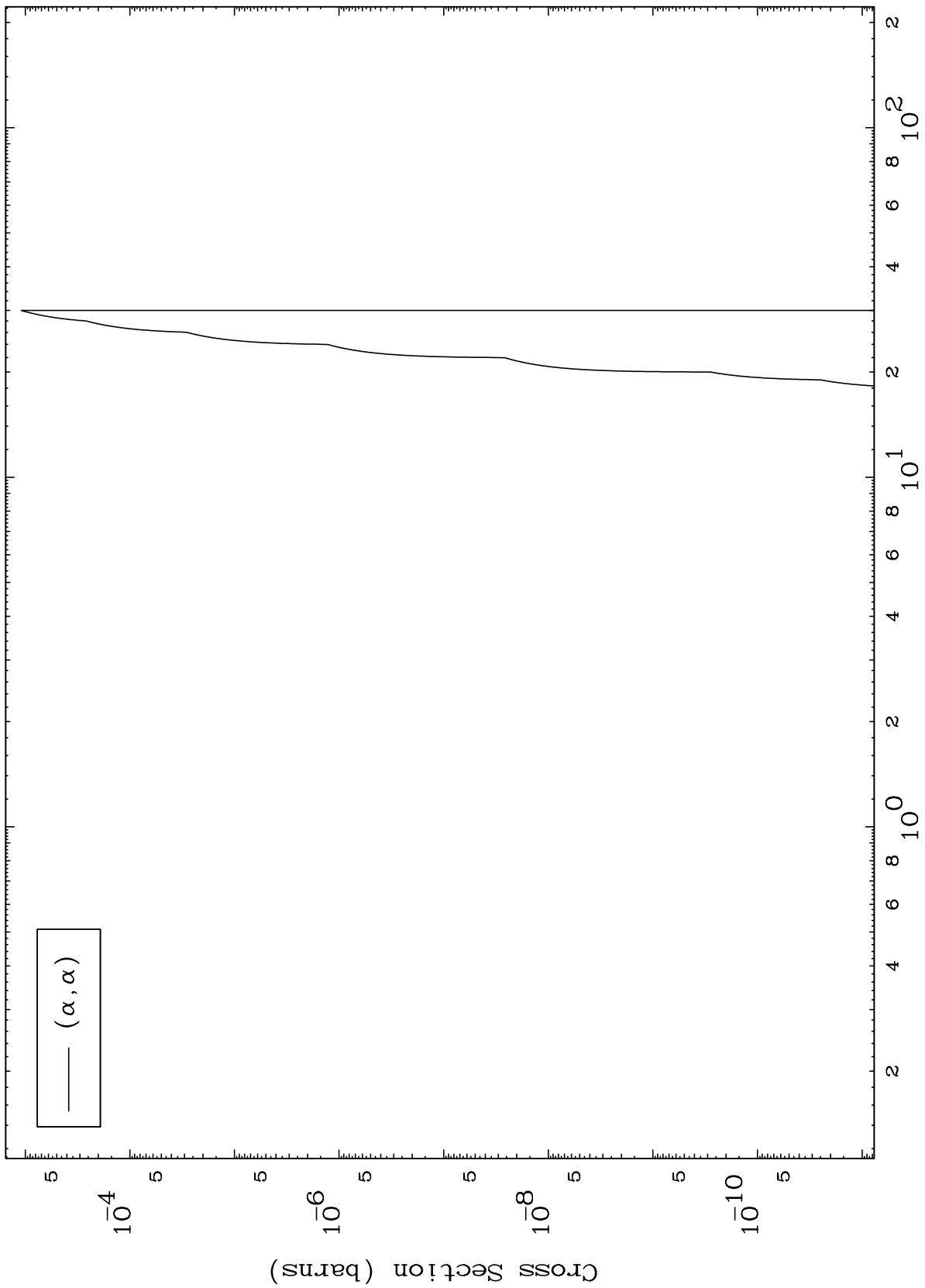


MAT 9989

( $\alpha, \alpha$ ) Levels

103-Lr-255

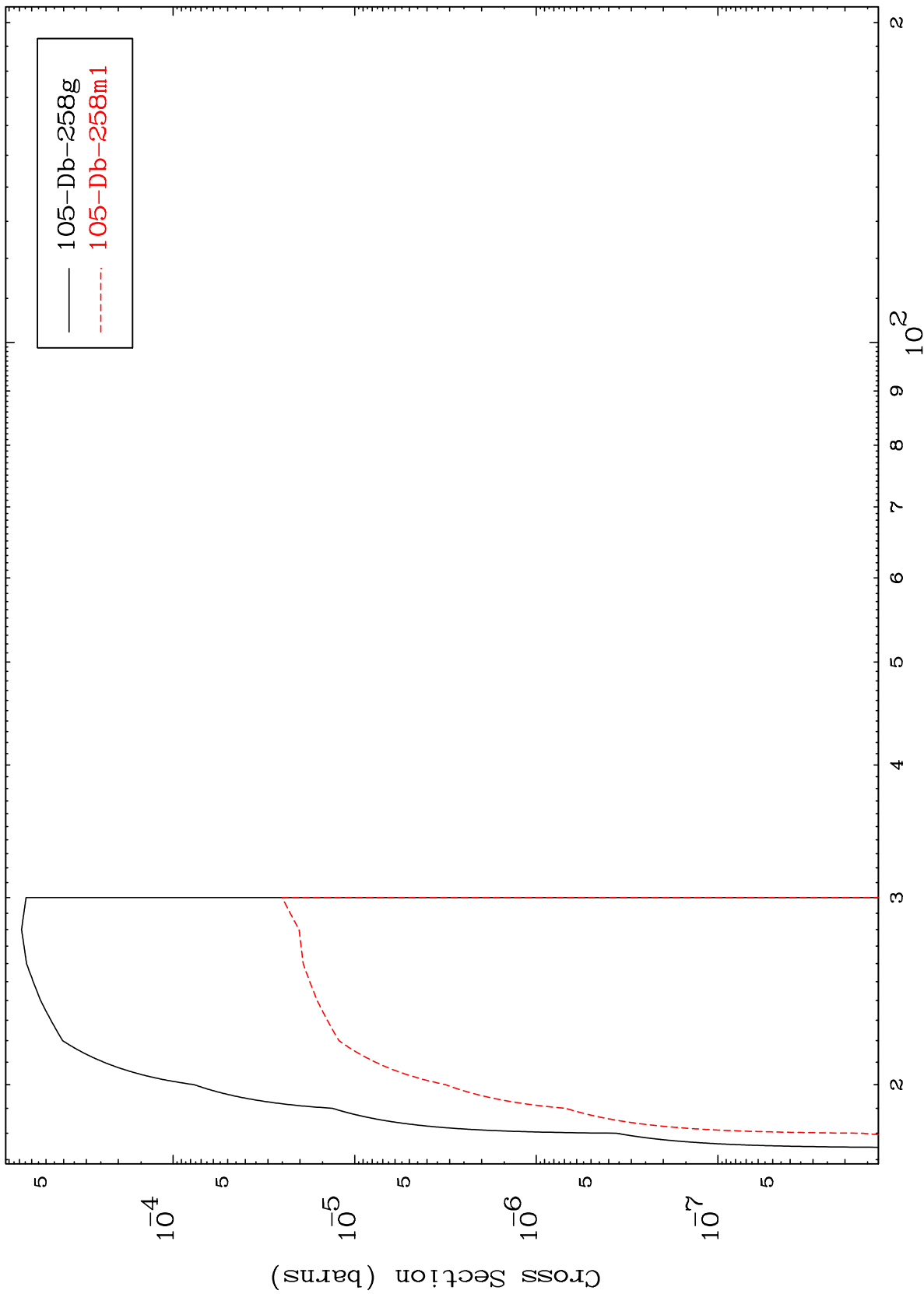
0 Kelvin Cross Sections



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103-Lr-255

Radionuclide  $\alpha$  Inelastic Production Cross Section



103-Lr-255

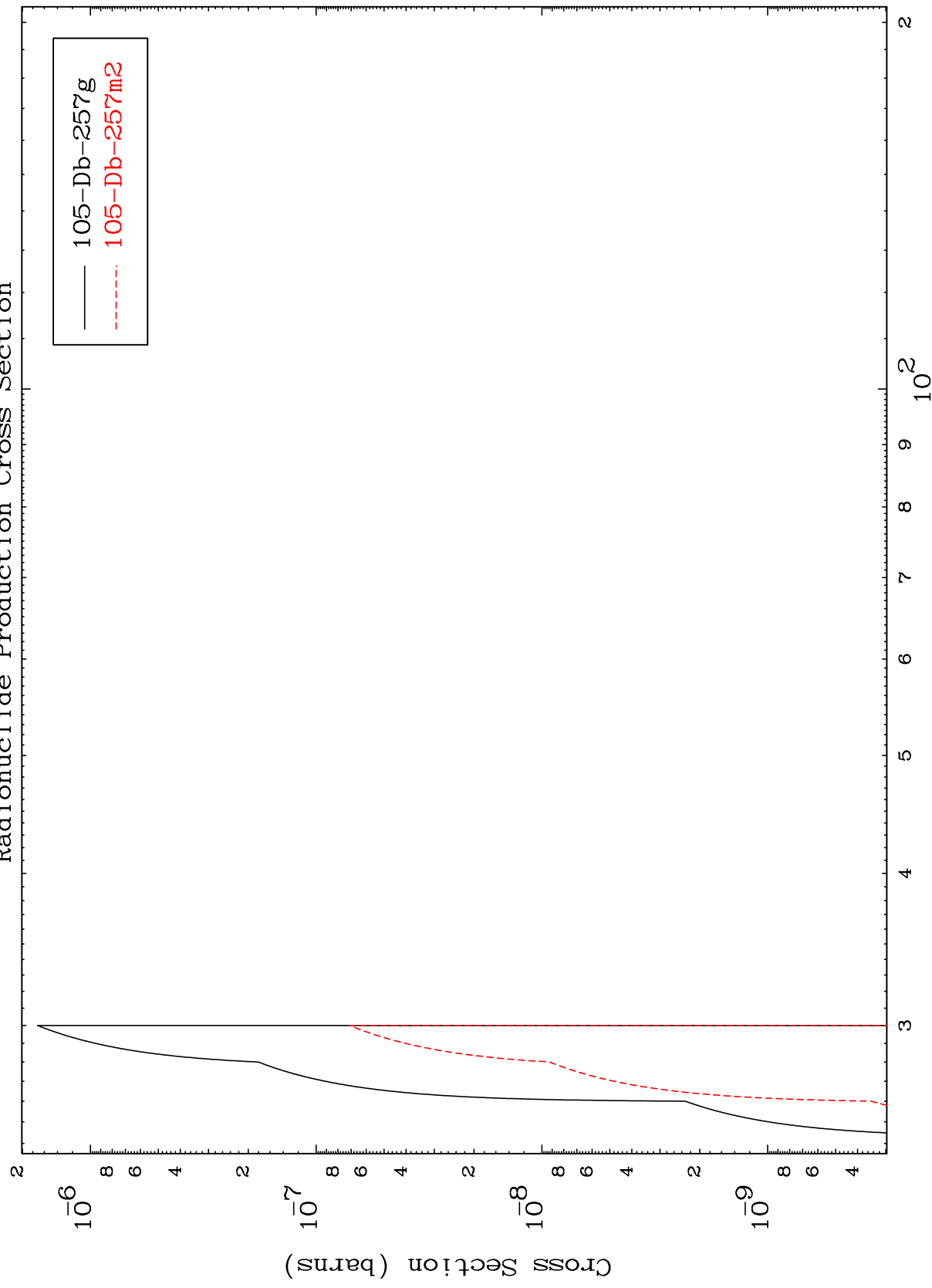
Incident Energy (MeV)

9

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103-Lr-255

( $\alpha, 2n$ )  
Radionuclide Production Cross Section



103-Lr-255

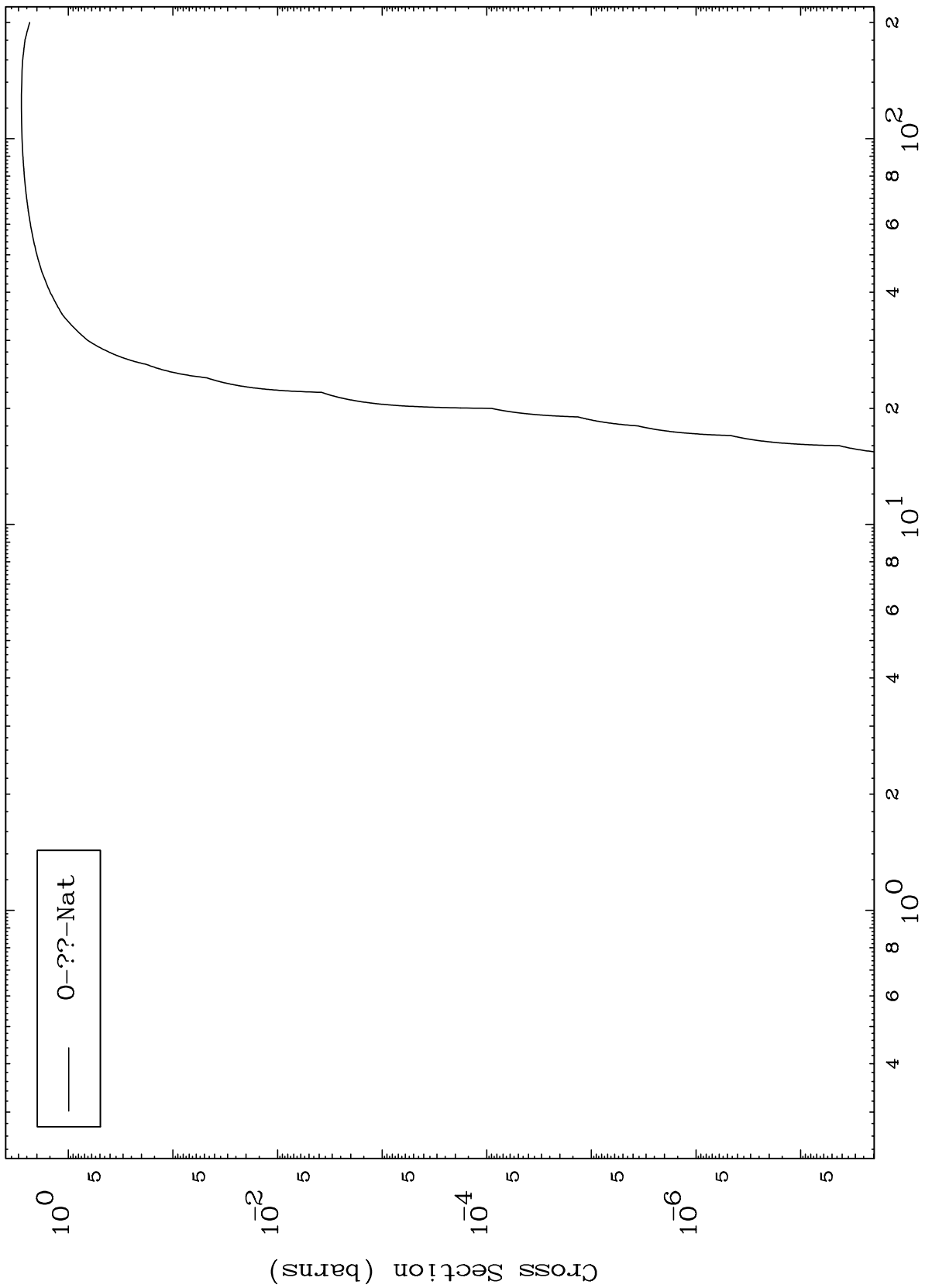
Incident Energy (MeV)

10

MAT 9989

103-Lr-255

$\alpha$  Fission  
Radionuclide Production Cross Section



103-Lr-255

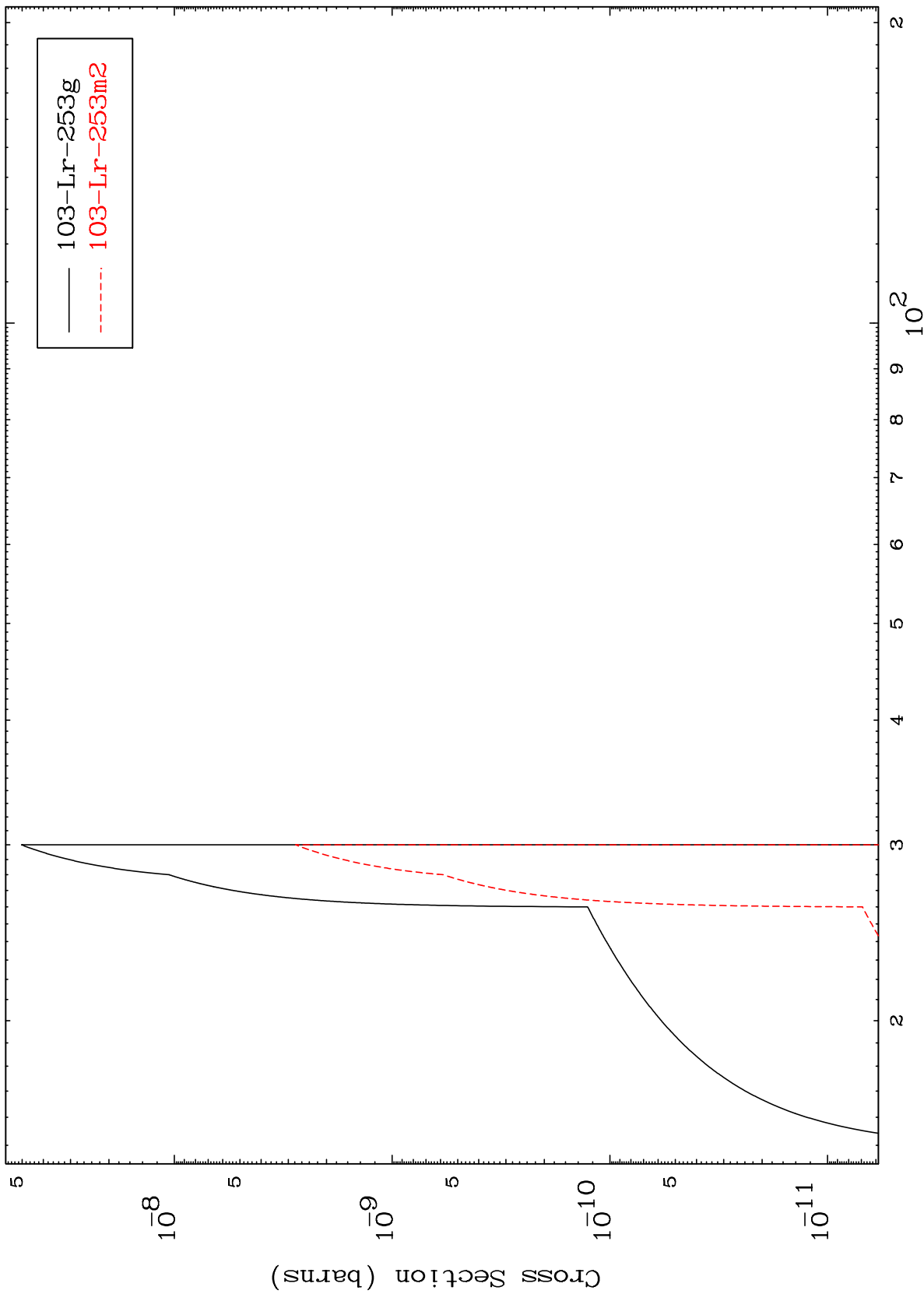
Incident Energy (MeV)

MAT 9989

103-Lr-255

$(\alpha, 2n) \alpha$

Radionuclide Production Cross Section



12

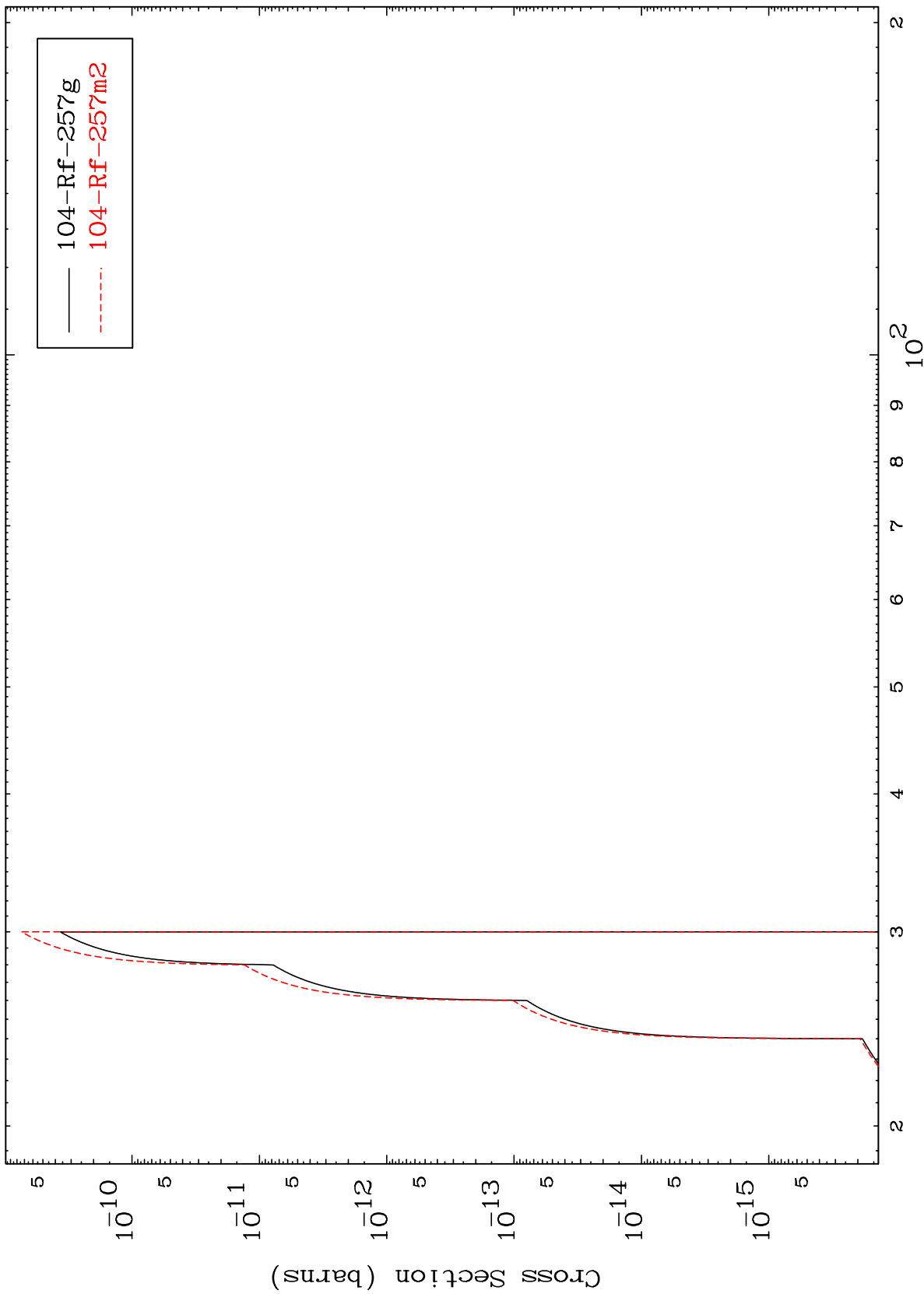
Incident Energy (MeV)

103-Lr-255

MAT 9989

103-Lr-255

$(\alpha, n')$  p  
Radionuclide Production Cross Section



103-Lr-255

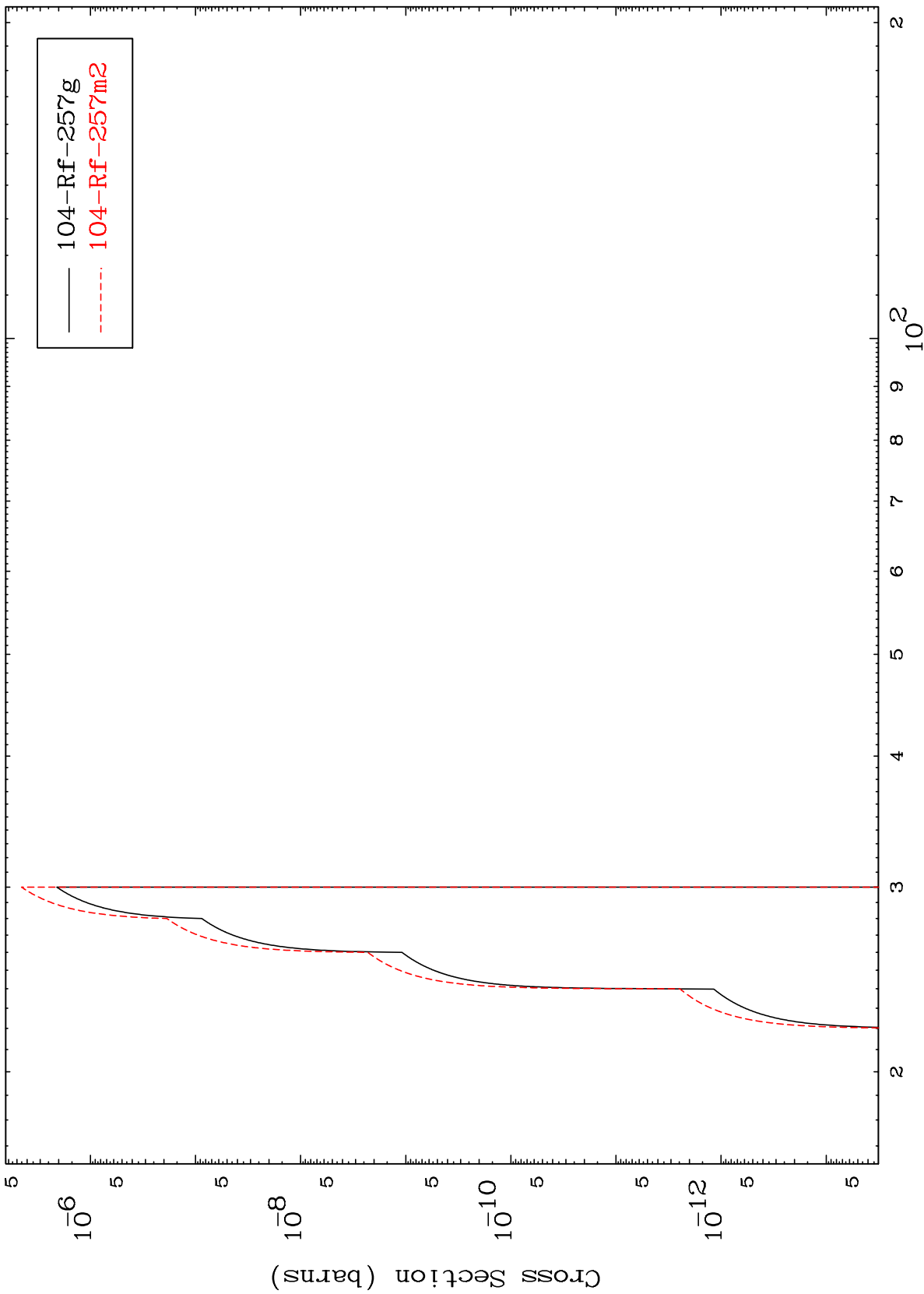
Incident Energy (MeV)

13

MAT 9989

103-Lr-255

( $\alpha, d$ )  
Radionuclide Production Cross Section



14

Incident Energy (MeV)

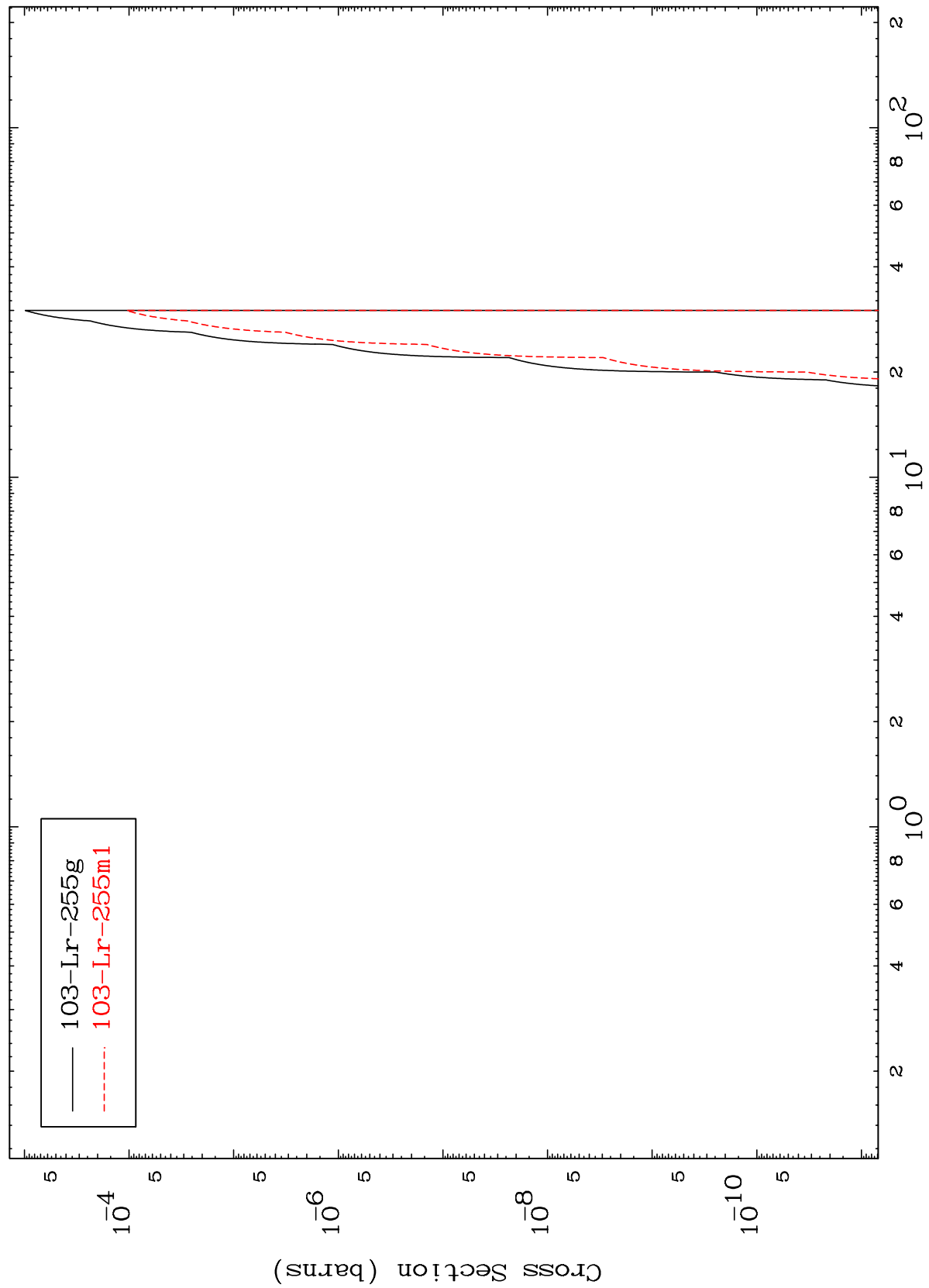
103-Lr-255

MAT 9989

103-Lr-255

( $\alpha, \alpha$ )

Radionuclide Production Cross Section



— 103-Lr-255g  
- - - 103-Lr-255m1

15

Incident Energy (MeV)

103-Lr-255