

Program EVALPLOT  
(Version 2018-1)

by

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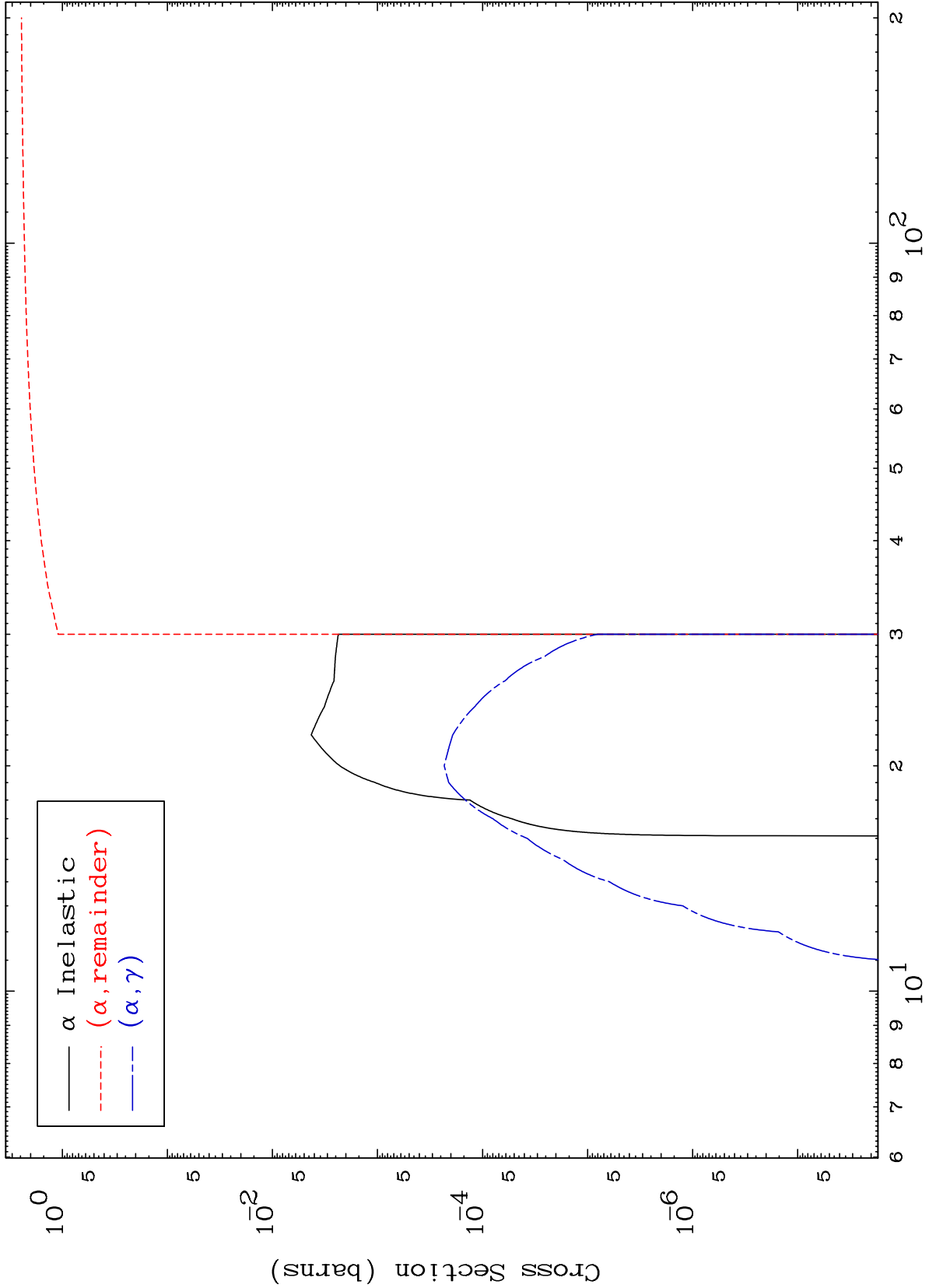
Press Mouse Button to Start

MAT 7062

0 Kelvin

$\alpha$  Major Cross Sections

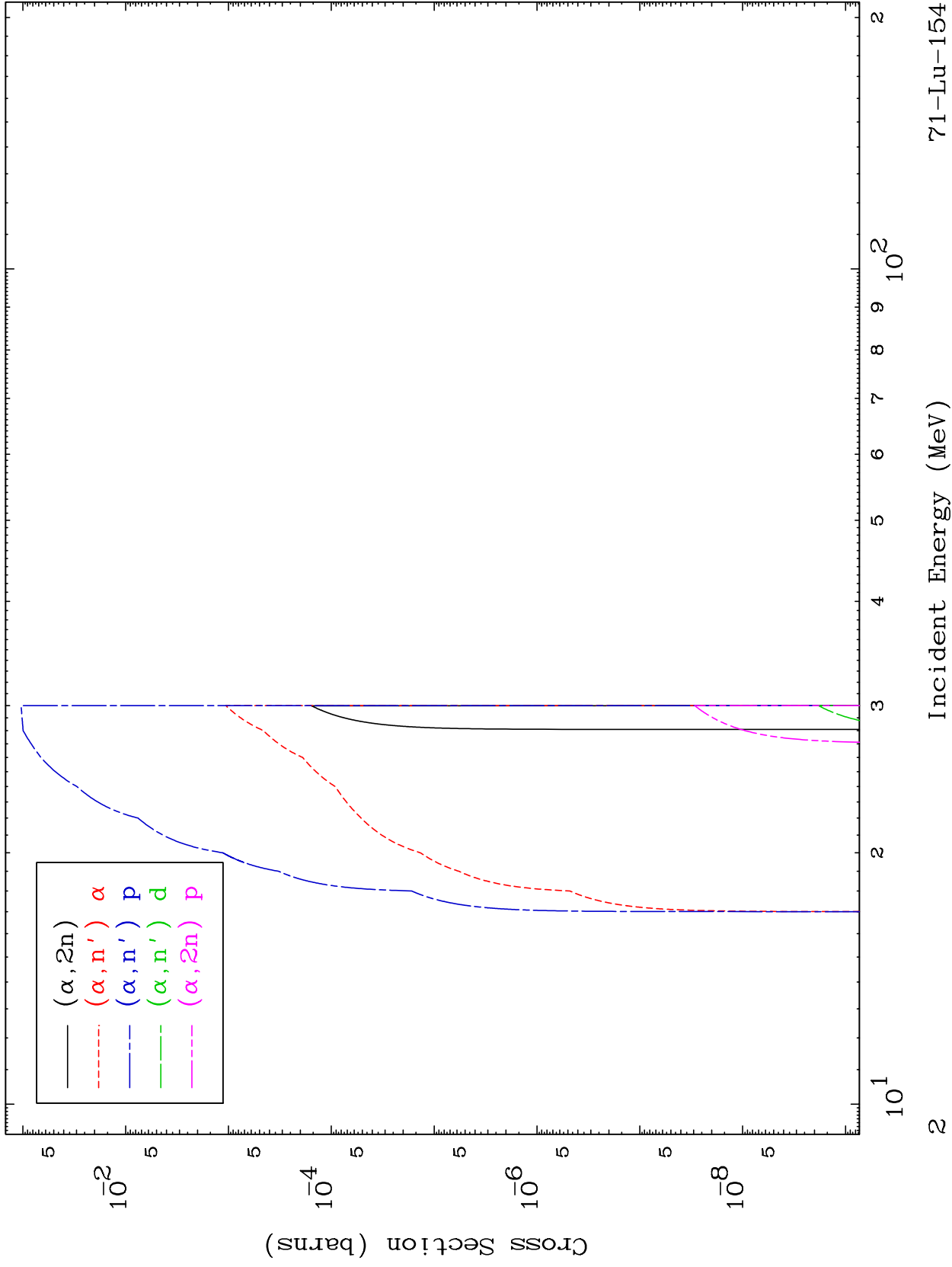
71-Lu-154

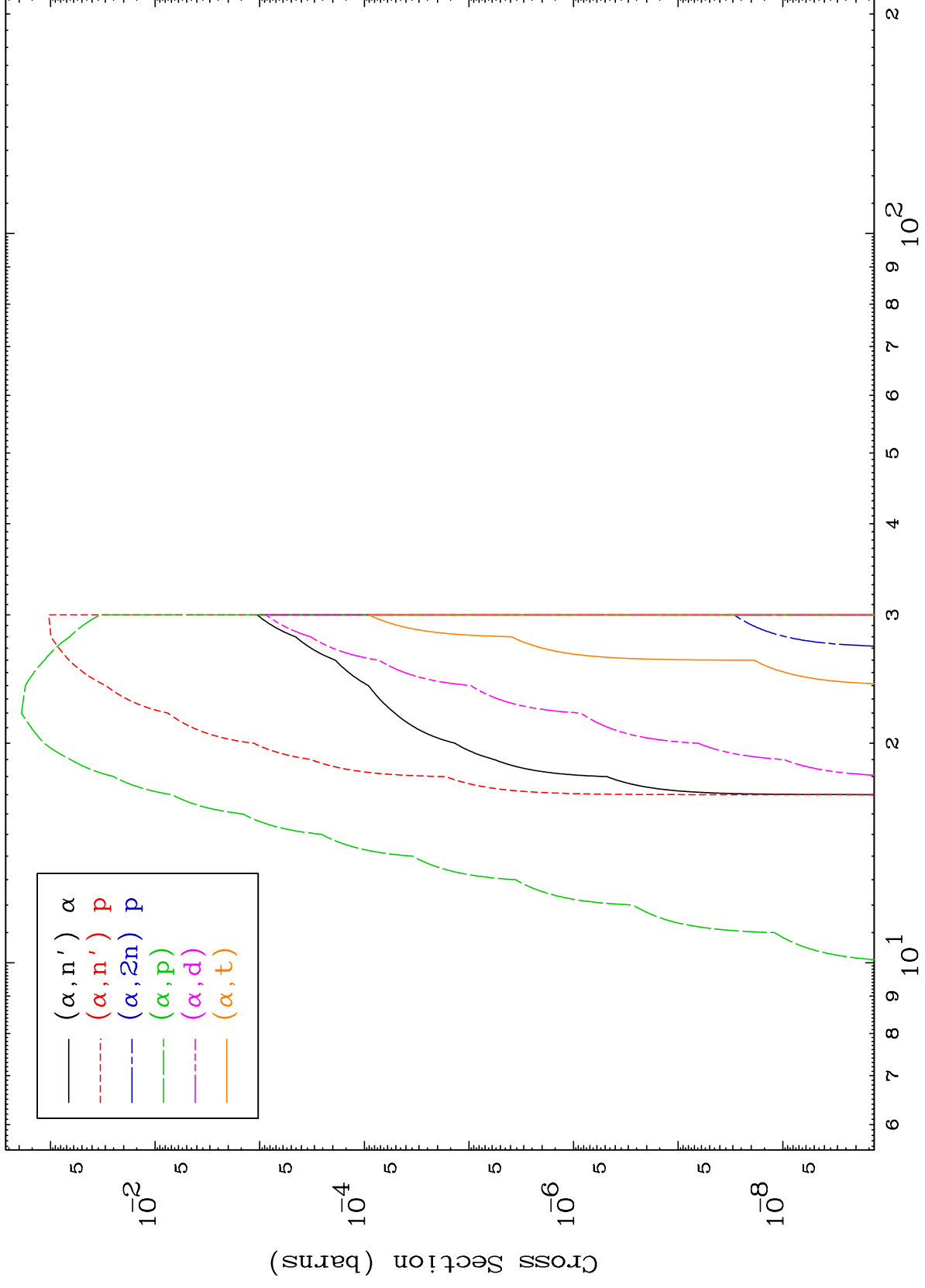


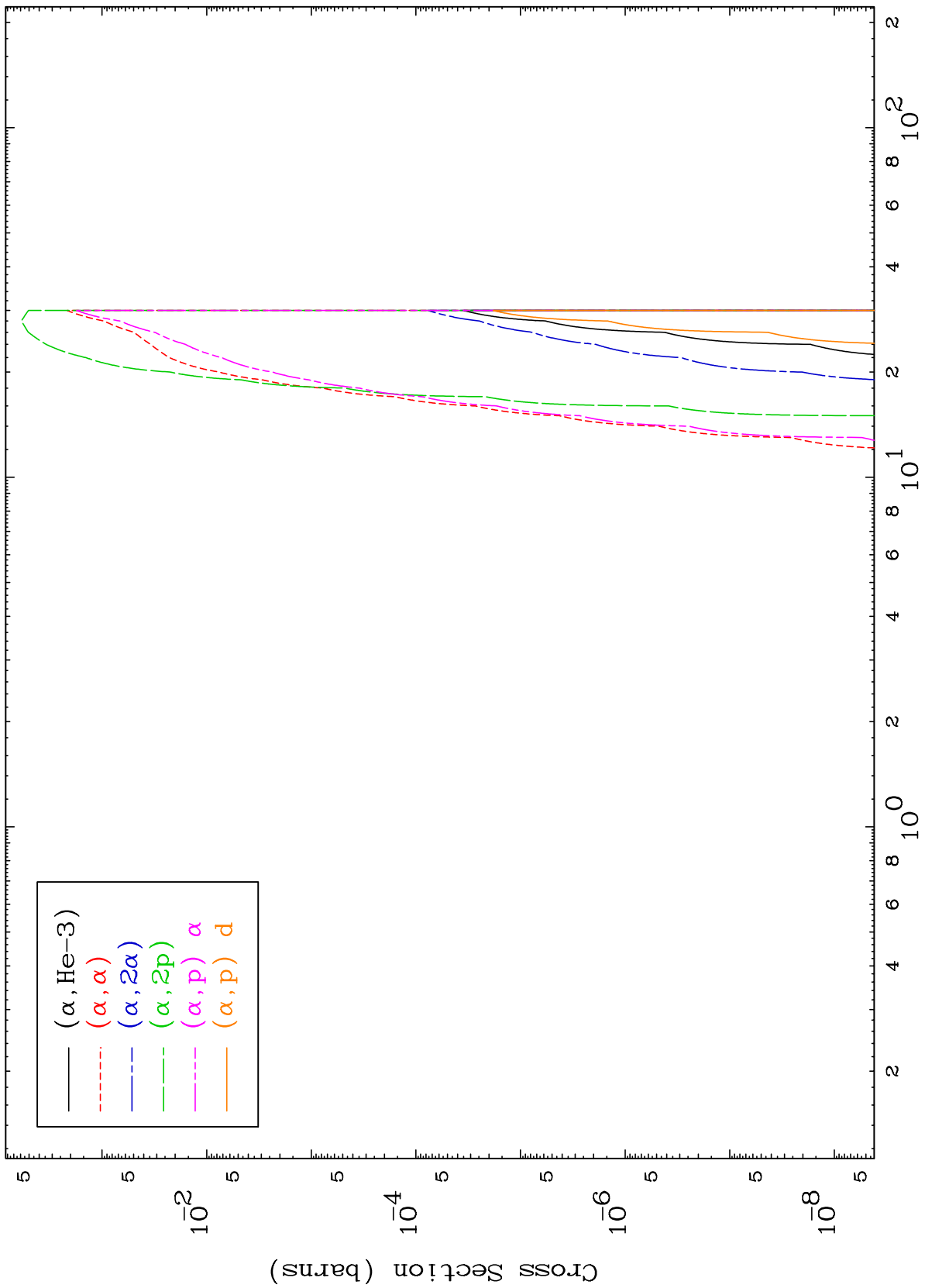
1

Incident Energy (MeV)

71-Lu-154





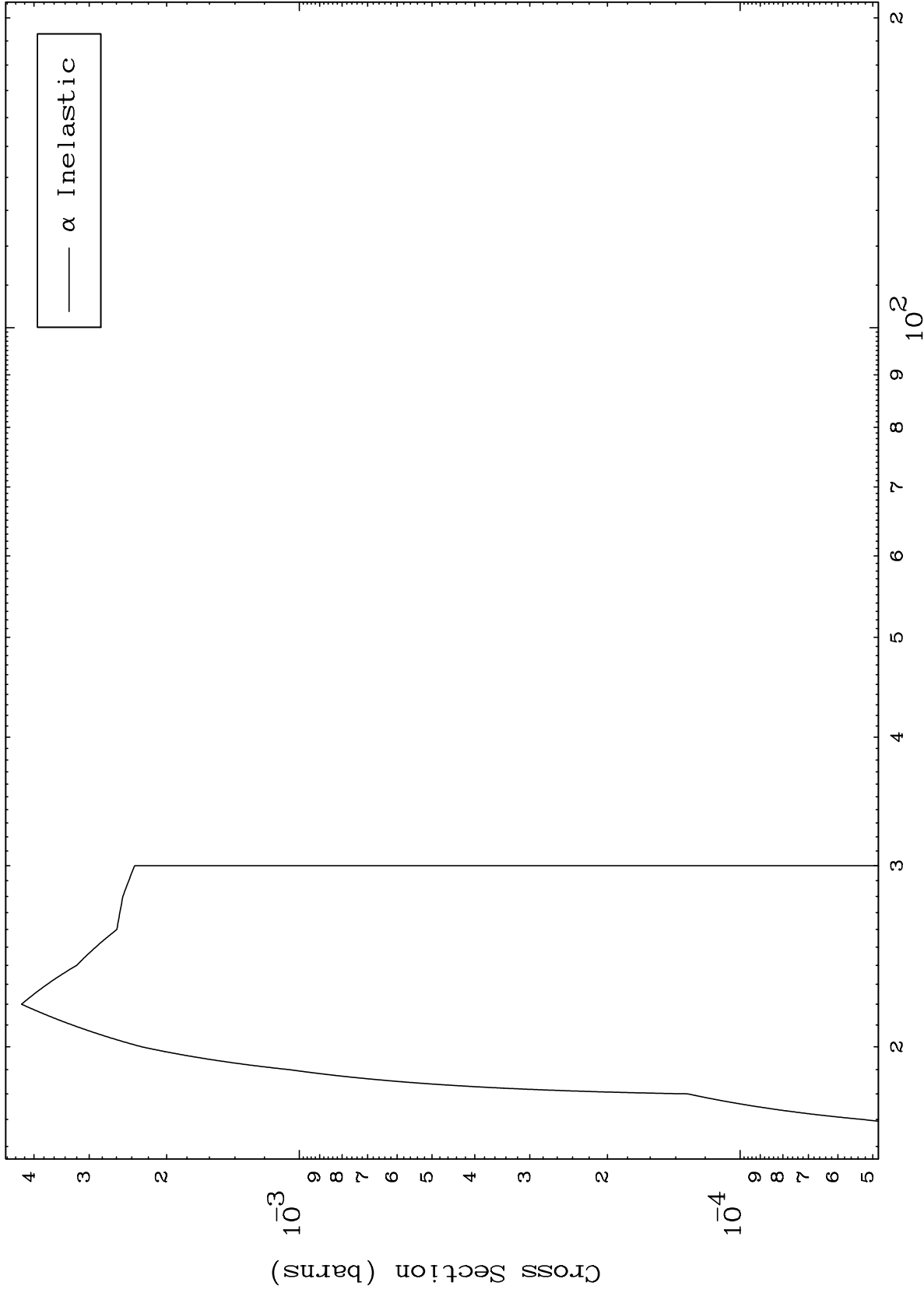


MAT 7062

( $\alpha, n'$ ) Level

71-Lu-154

0 Kelvin Cross Sections



5

Incident Energy (MeV)

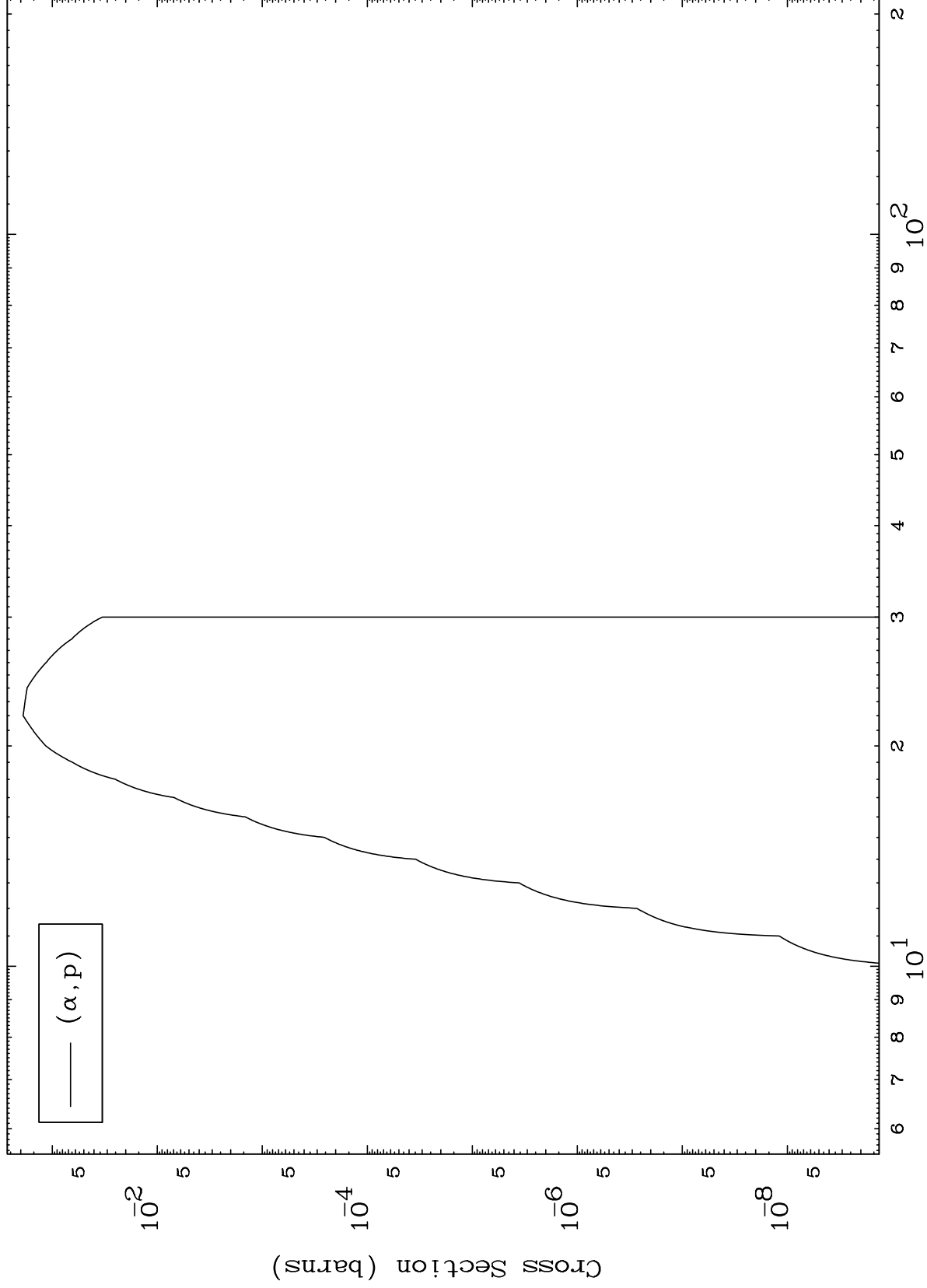
71-Lu-154

MAT 7062

( $\alpha, p$ ) Levels

71-Lu-154

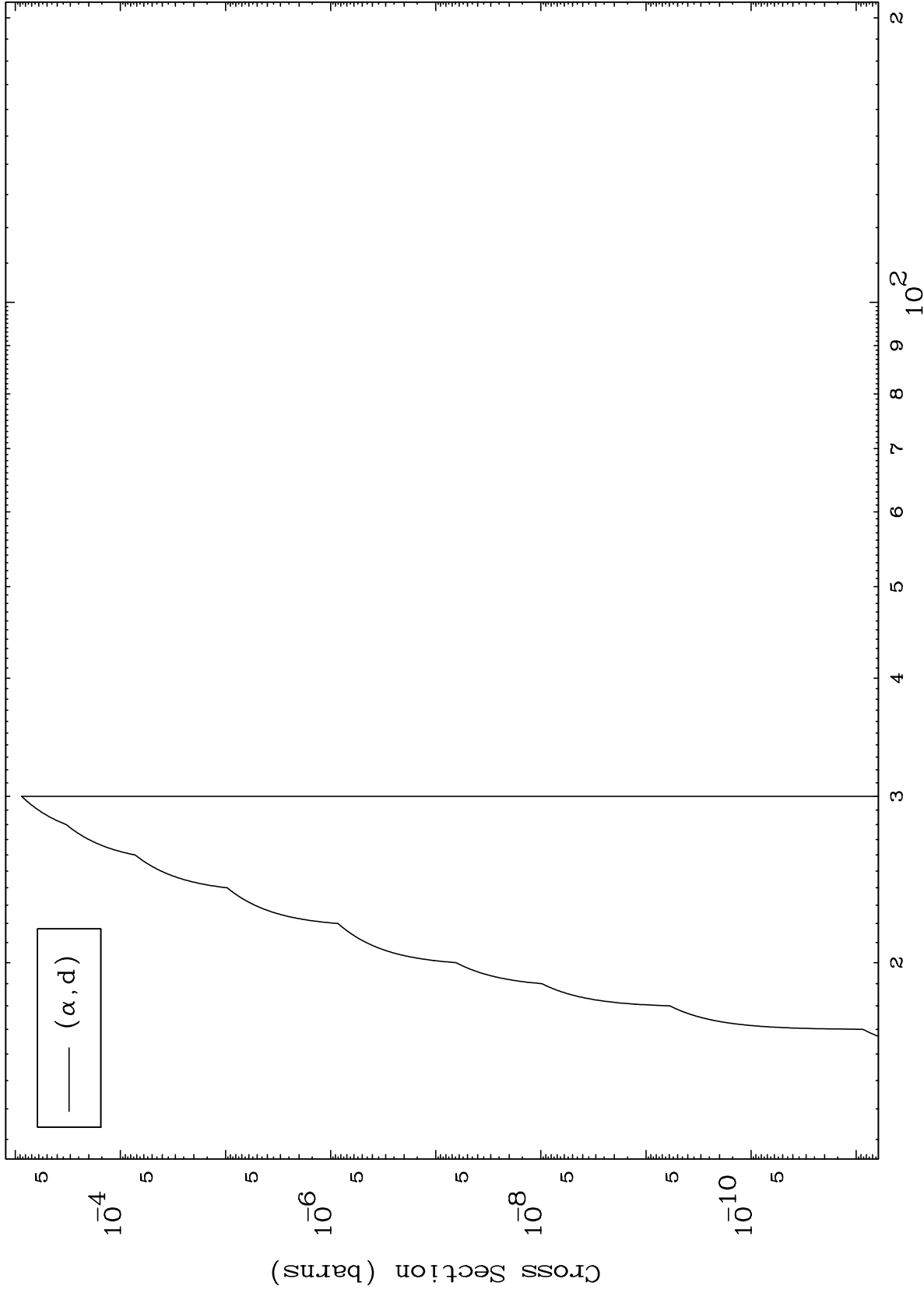
0 Kelvin Cross Sections



6

Incident Energy (MeV)

71-Lu-154

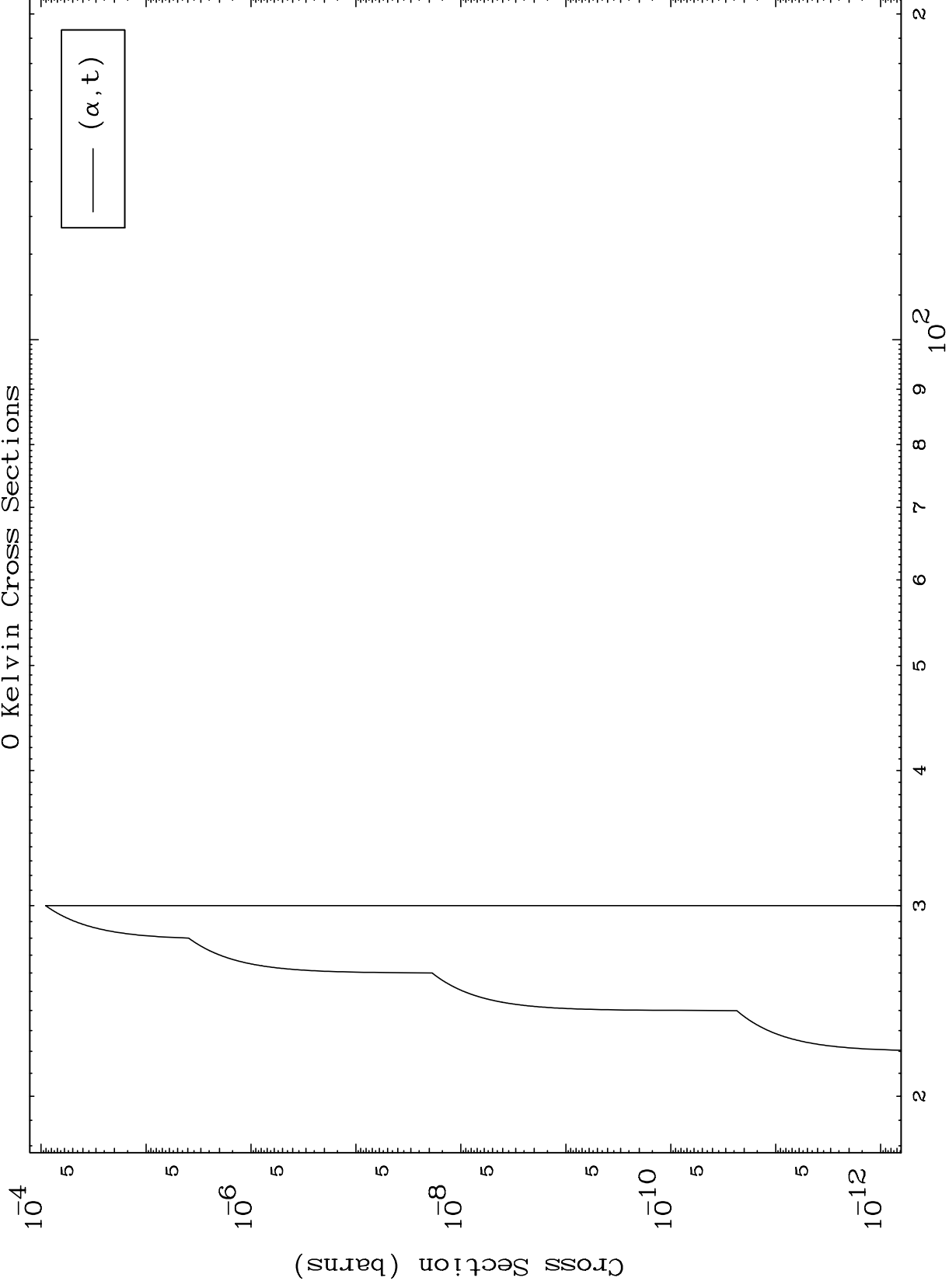




MAT 7062

71-Lu-154

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections



71-Lu-154

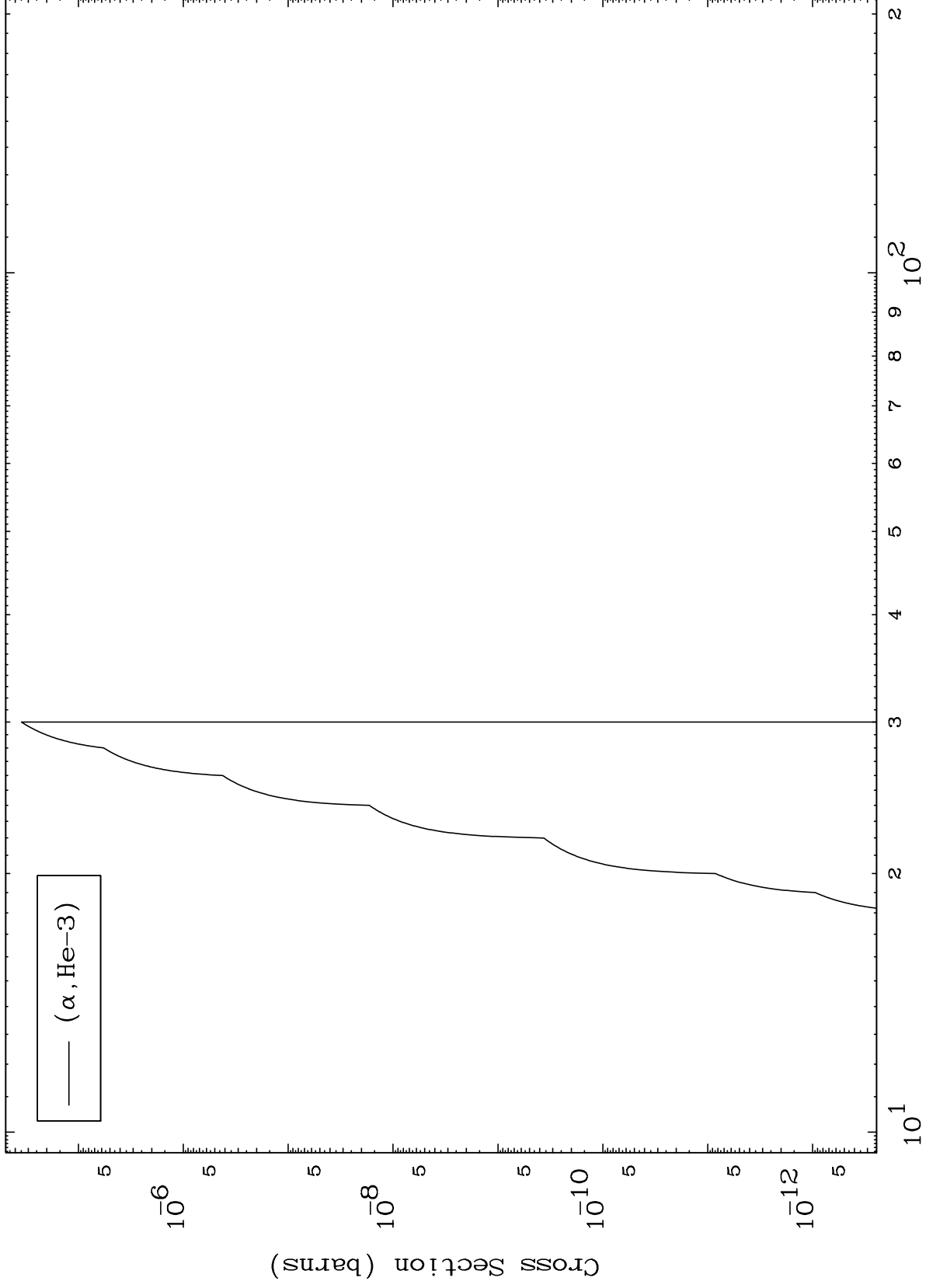
Incident Energy (MeV)

8

MAT 7062

( $\alpha$ , He3) Levels  
0 Kelvin Cross Sections

71-Lu-154



Incident Energy (MeV)

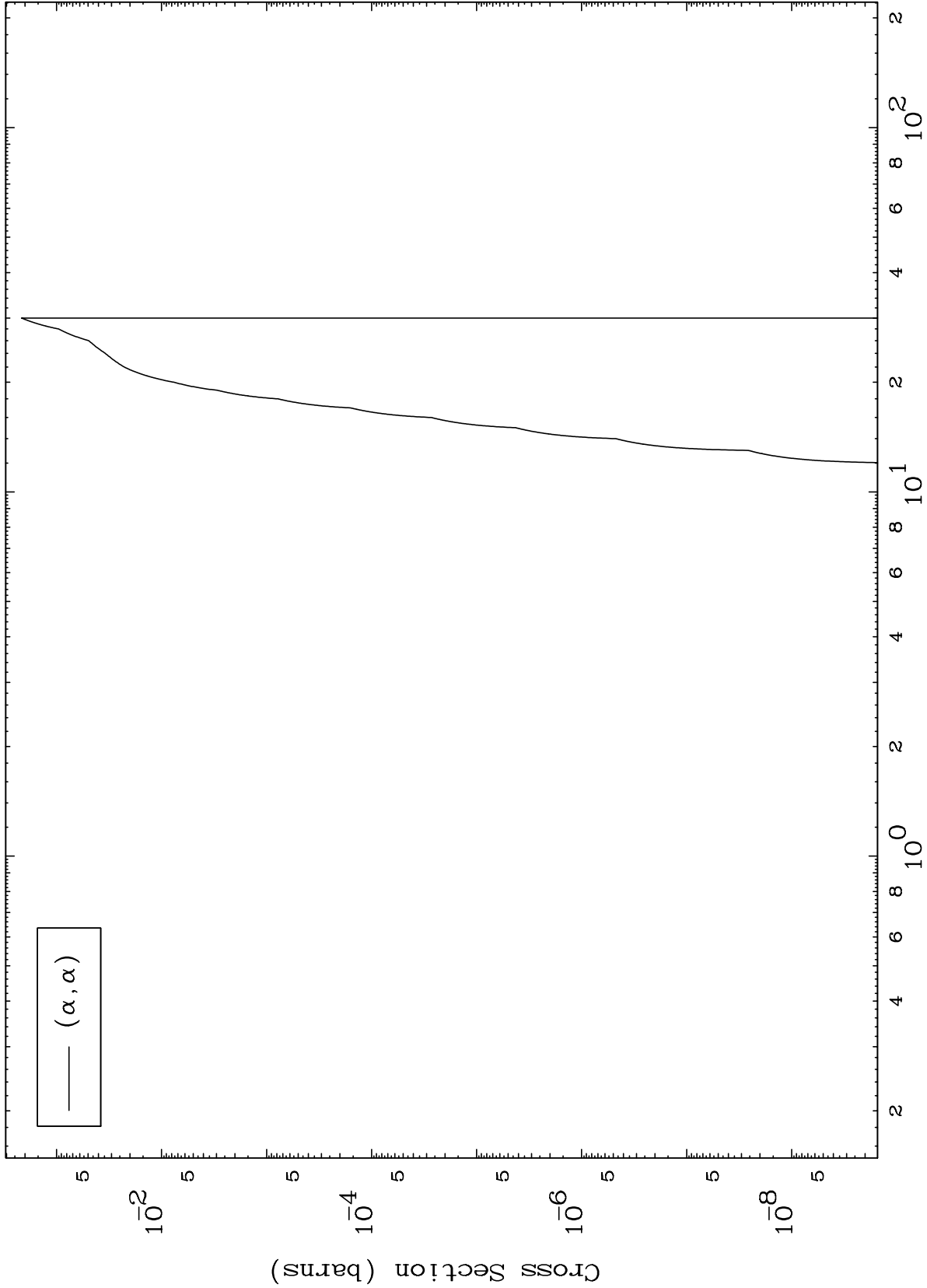
71-Lu-154

MAT 7062

( $\alpha, \alpha$ ) Levels

71-Lu-154

0 Kelvin Cross Sections



10

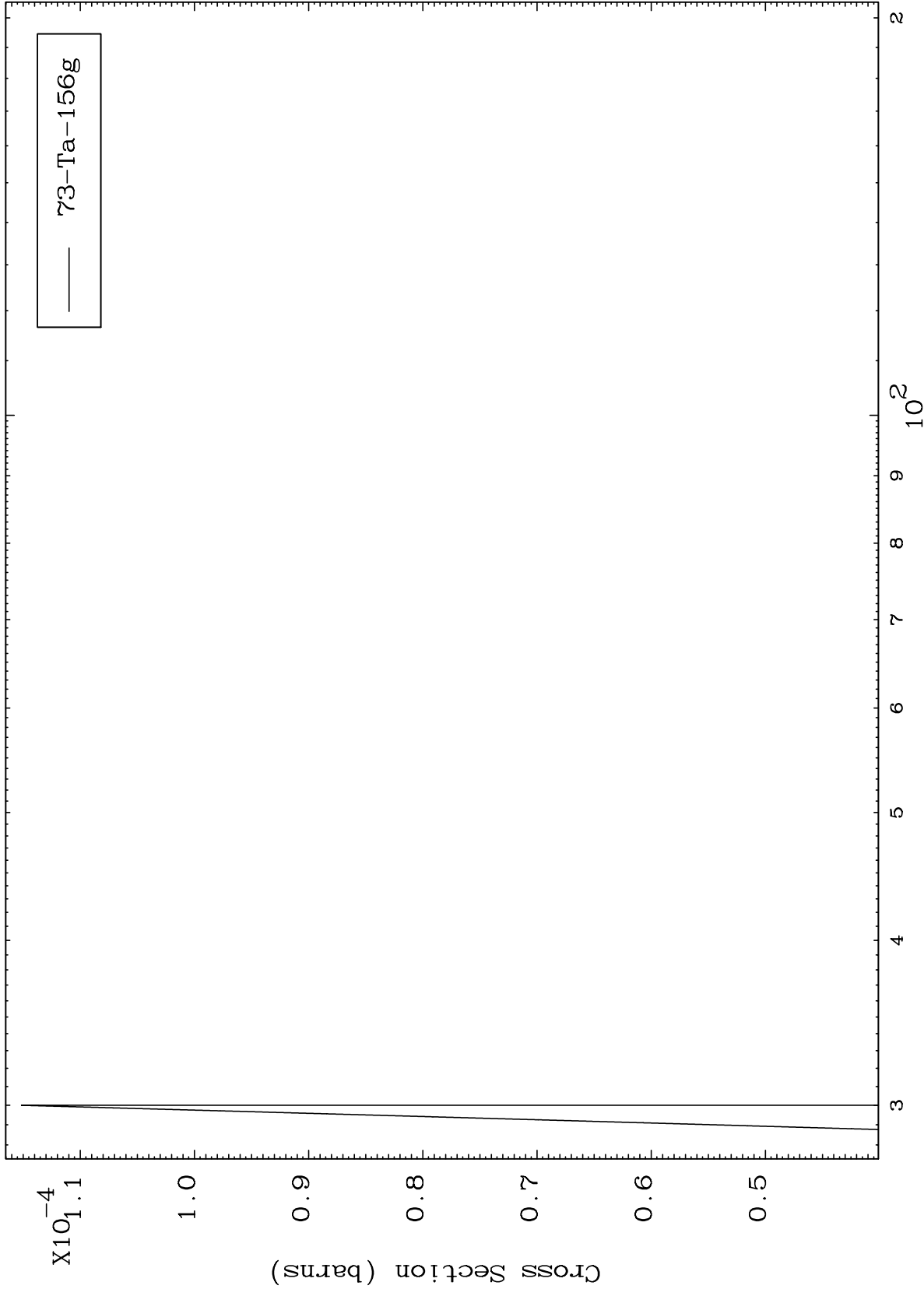
Incident Energy (MeV)

71-Lu-154

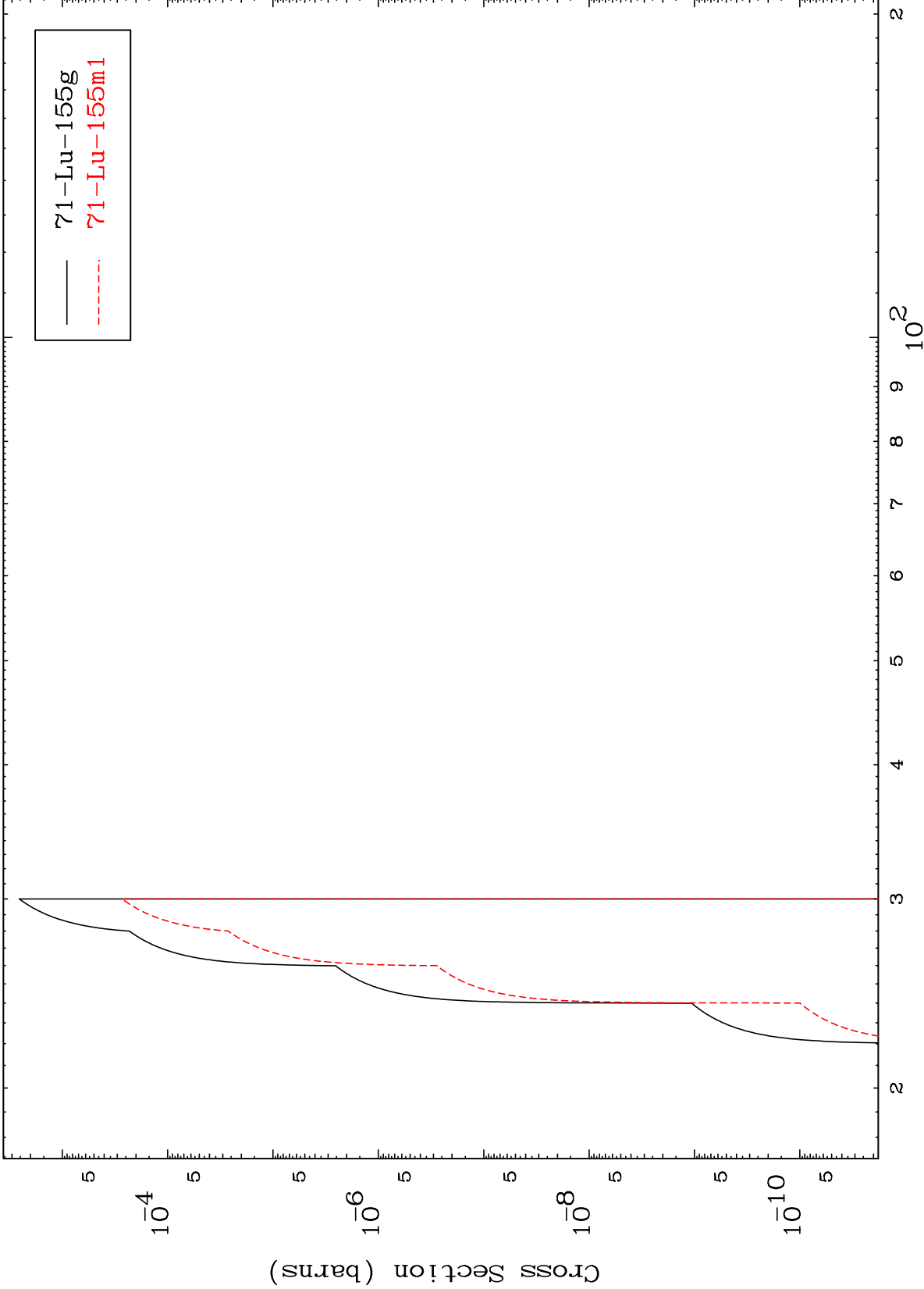
MAT 7062

71-Lu-154

( $\alpha, 2n$ )  
Radionuclide Production Cross Section



Radionuclide Production Cross Section

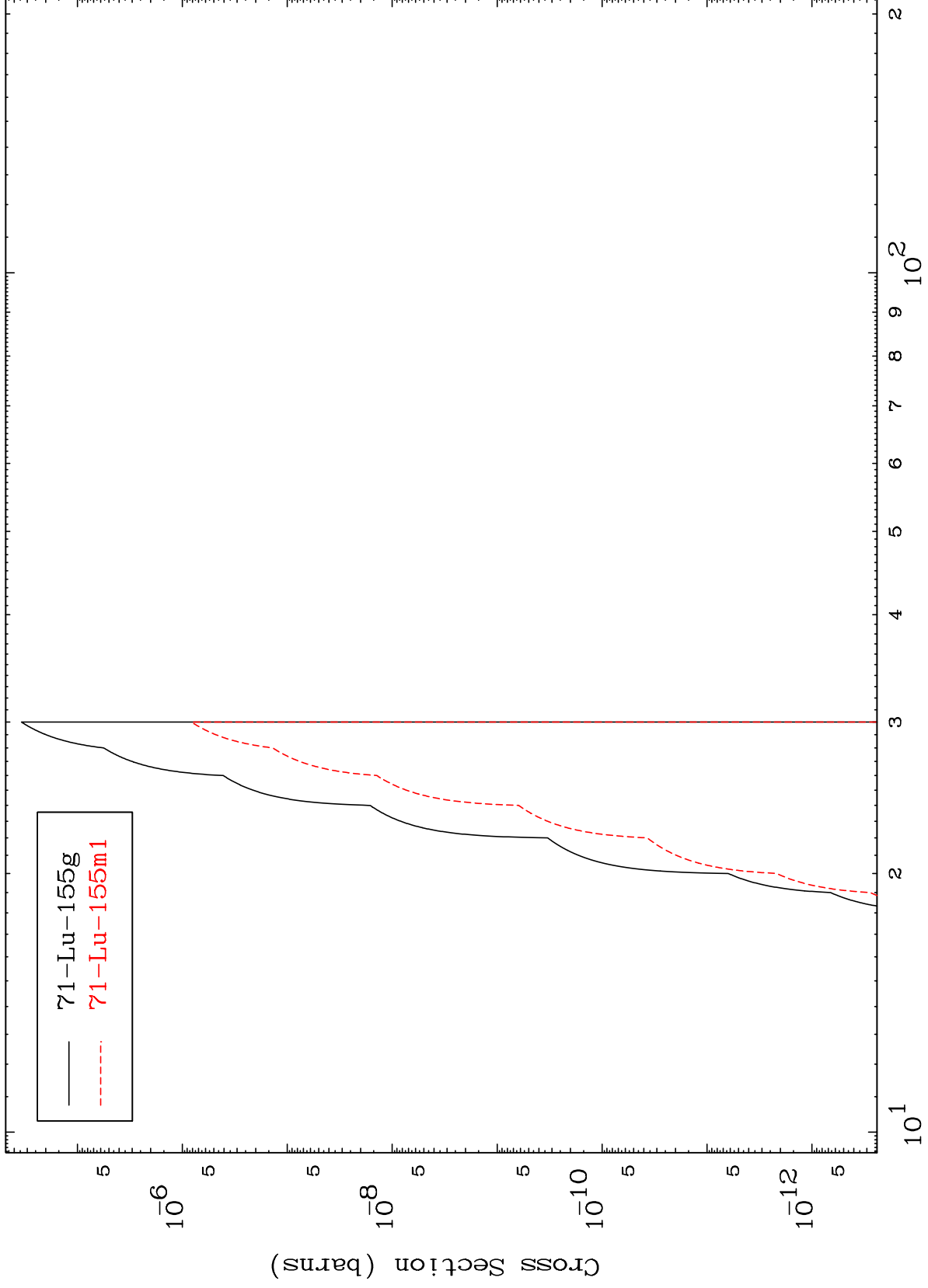


MAT 7062

( $\alpha, \text{He-3}$ )

$^{71}\text{Lu-154}$

Radionuclide Production Cross Section



13

Incident Energy (MeV)

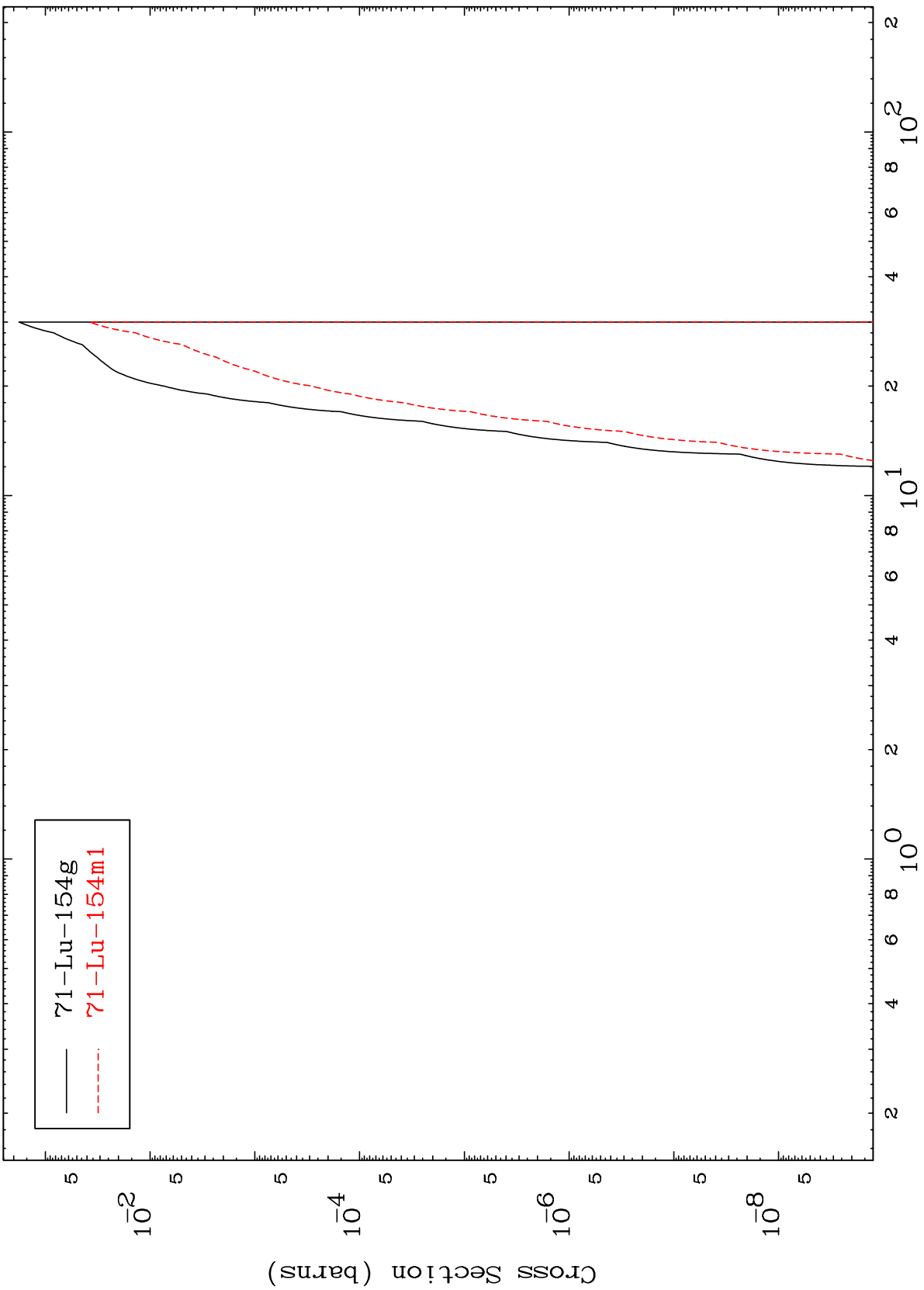
$^{71}\text{Lu-154}$

MAT 7062

( $\alpha, \alpha$ )

<sup>71</sup>Lu-154

Radionuclide Production Cross Section



14

Incident Energy (MeV)

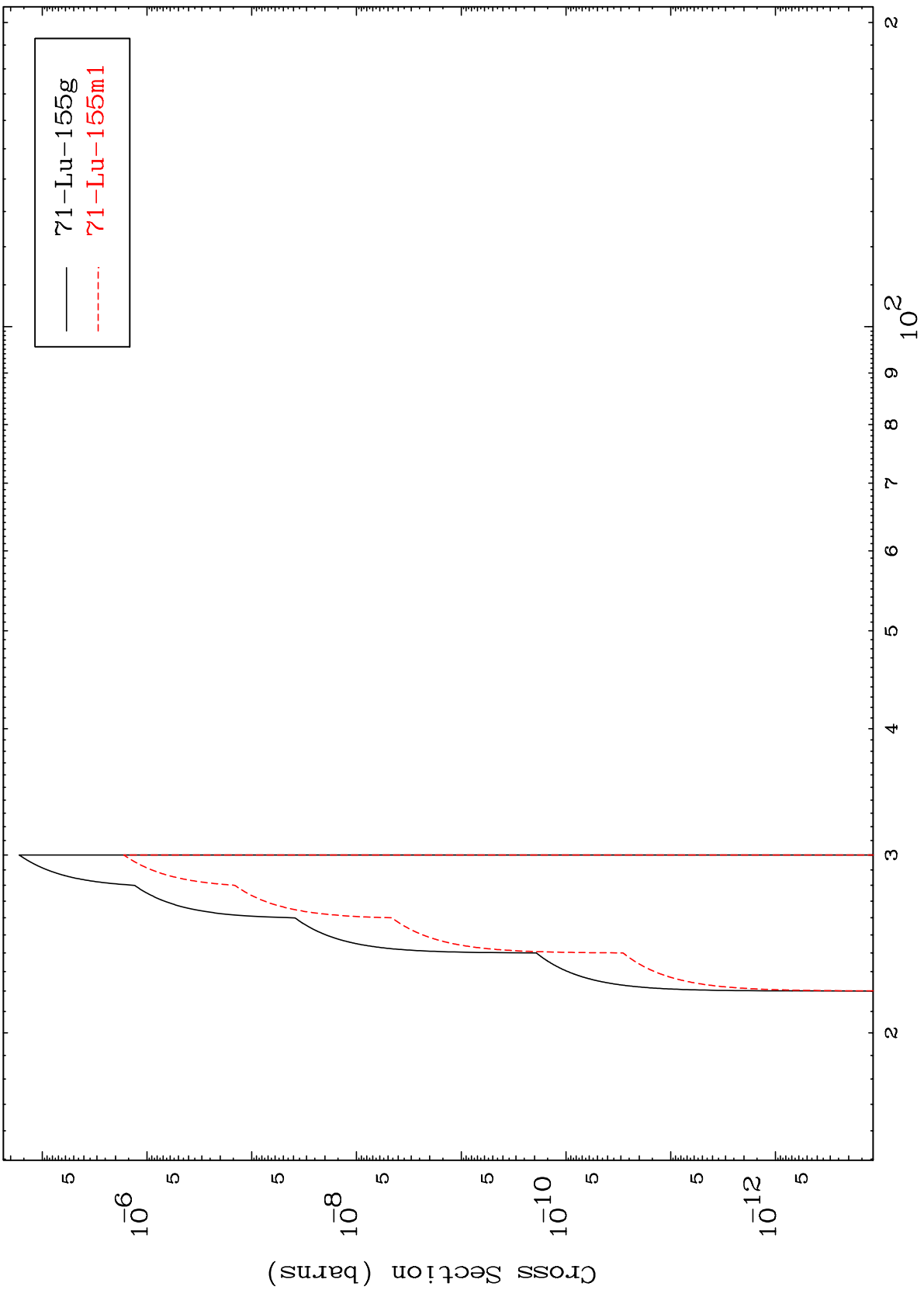
<sup>71</sup>Lu-154

MAT 7062

( $\alpha, p$ ) d

<sup>71</sup>Lu-154

Radionuclide Production Cross Section



15

Incident Energy (MeV)

<sup>71</sup>Lu-154