

Program EVALPLOT  
(Version 2018-1)

by

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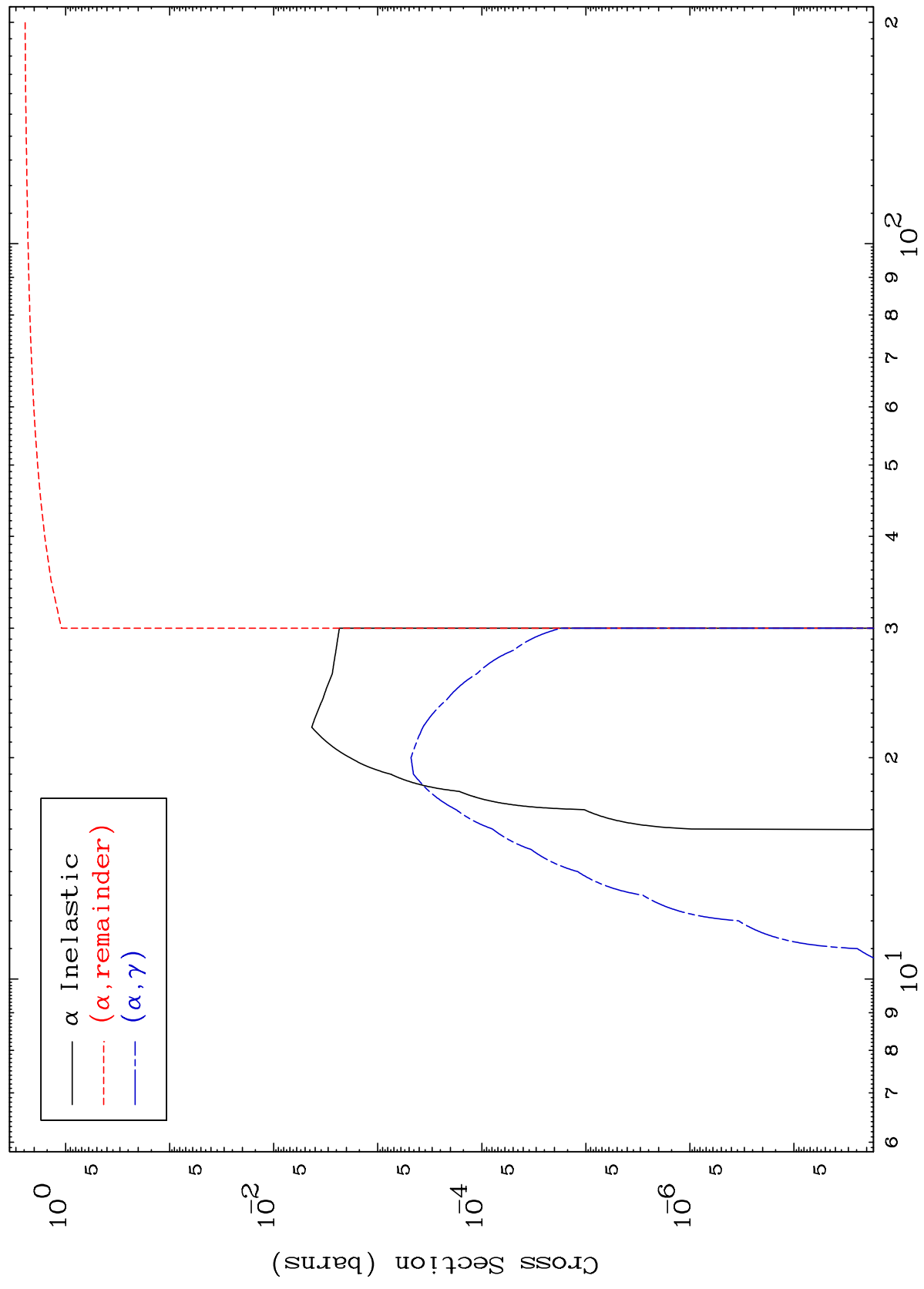
Press Mouse Button to Start

MAT 7063

0 Kelvin

$\alpha$  Major Cross Sections

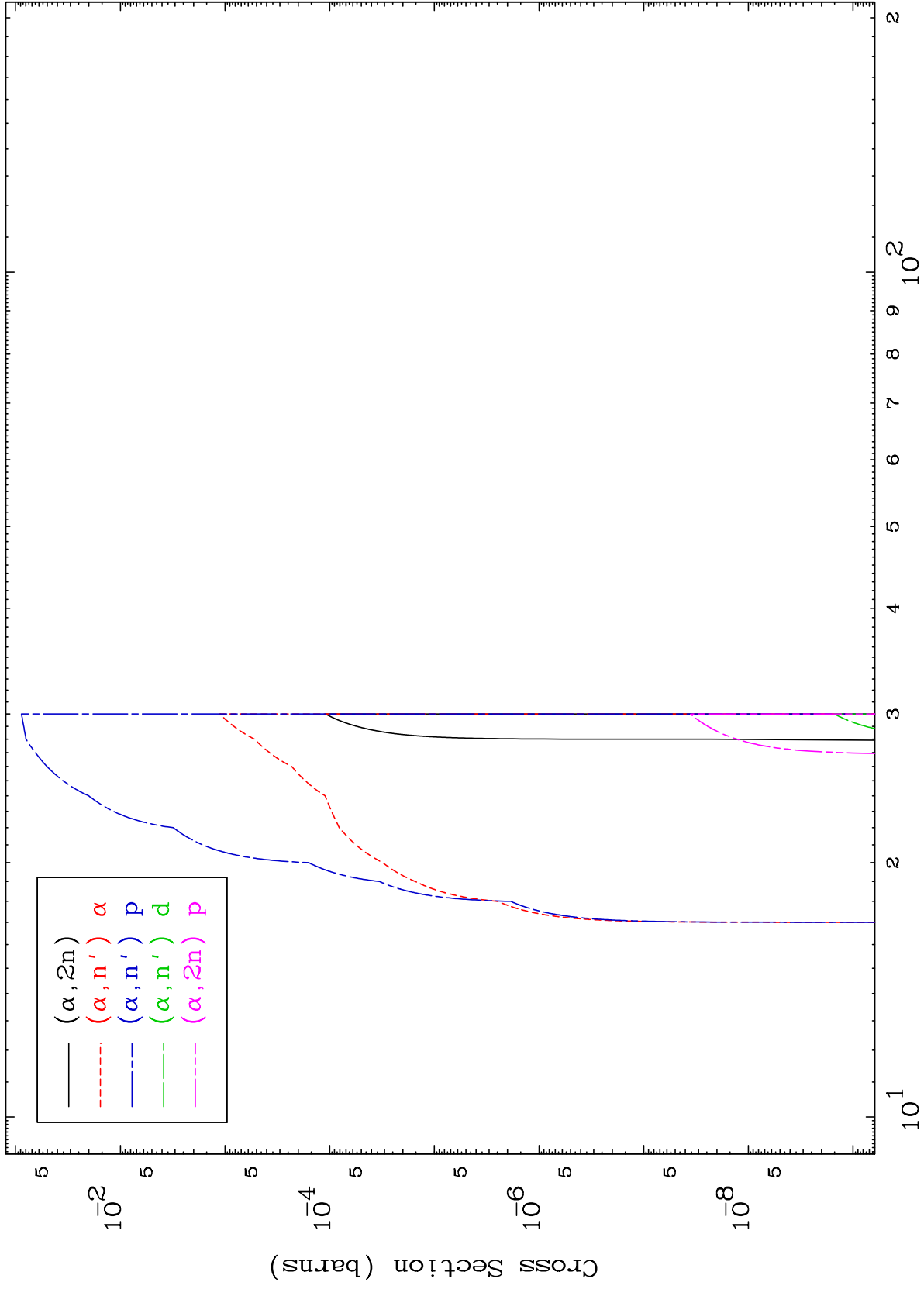
71-Lu-154

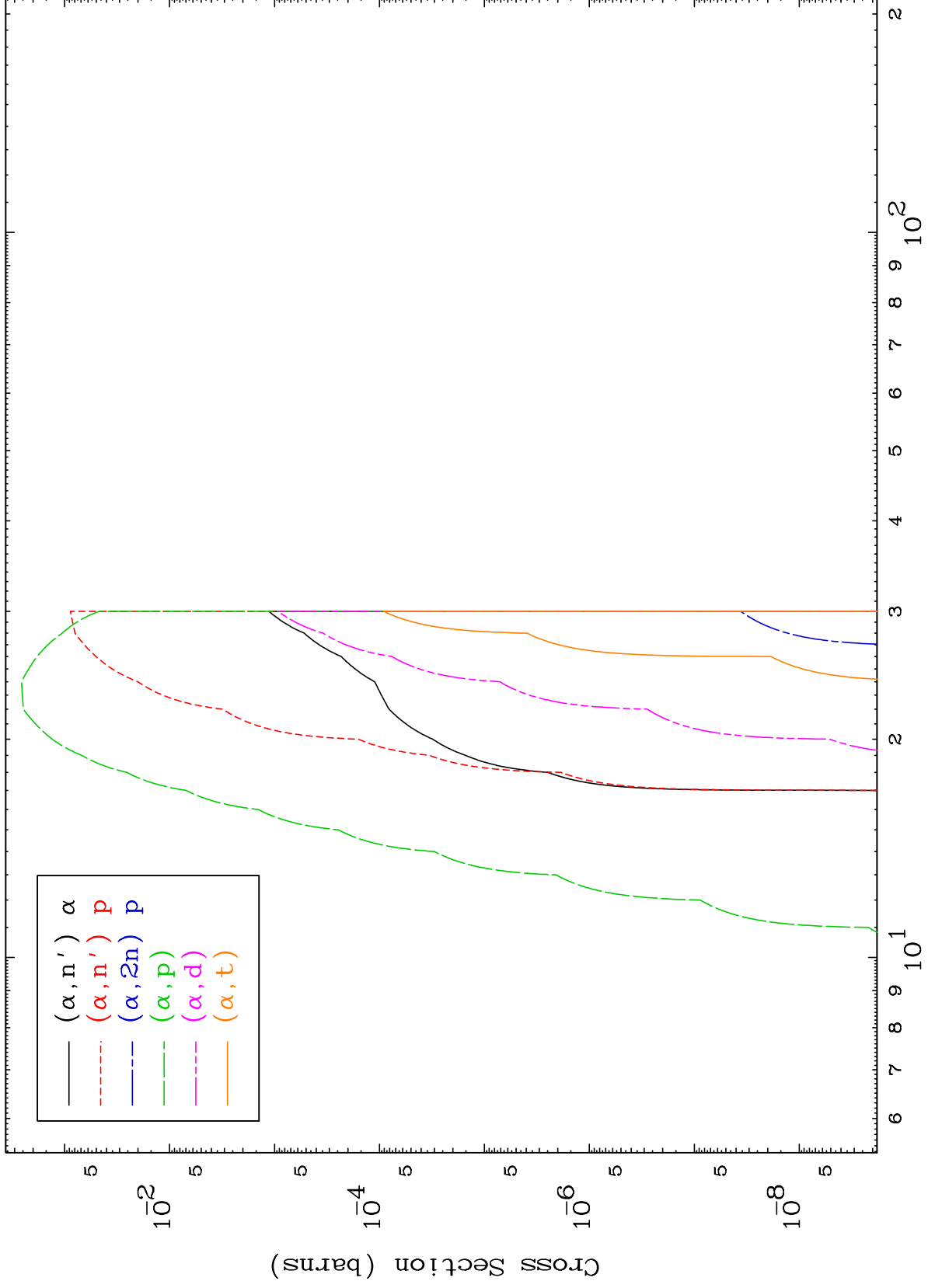


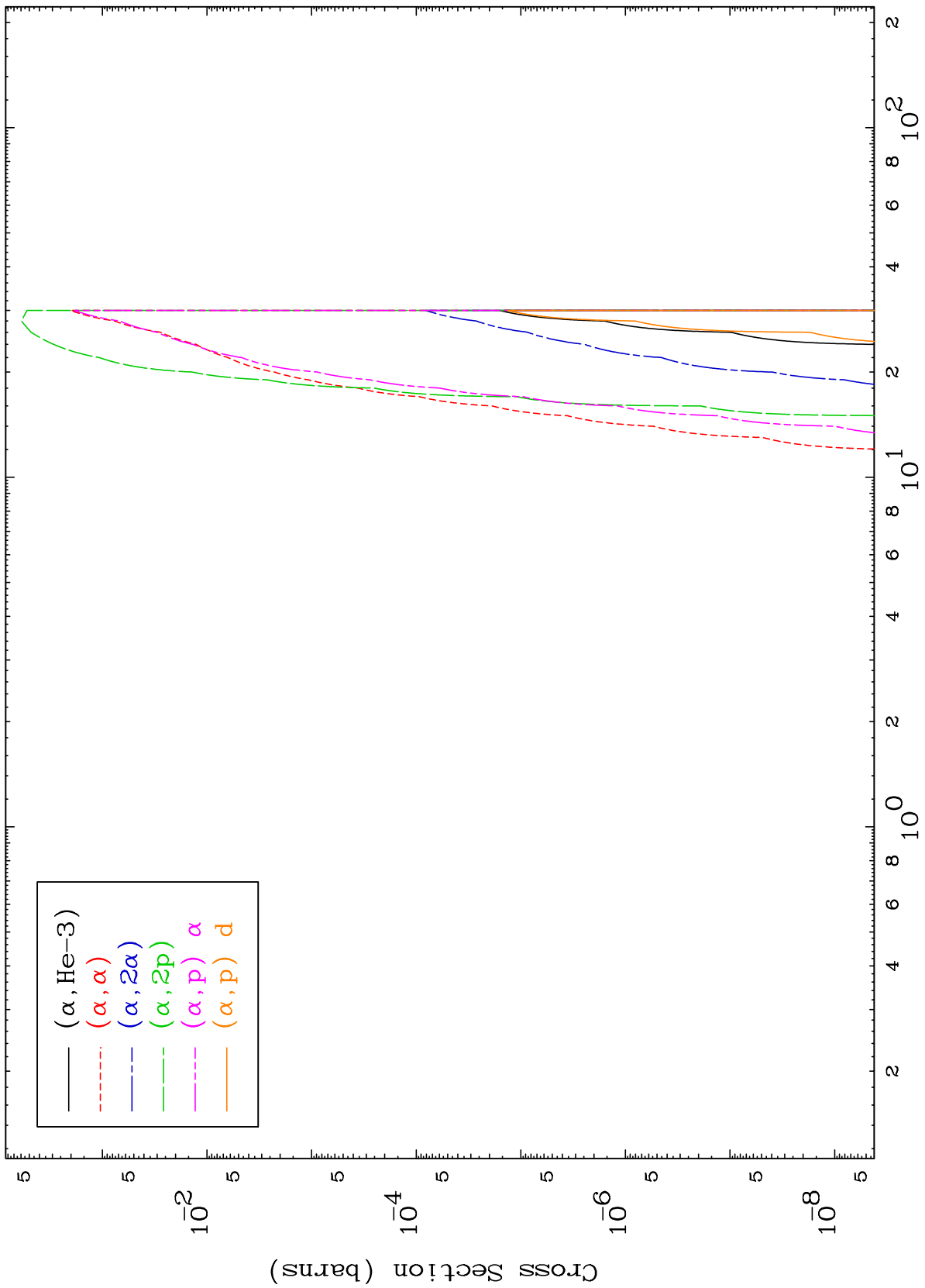
1

Incident Energy (MeV)

71-Lu-154





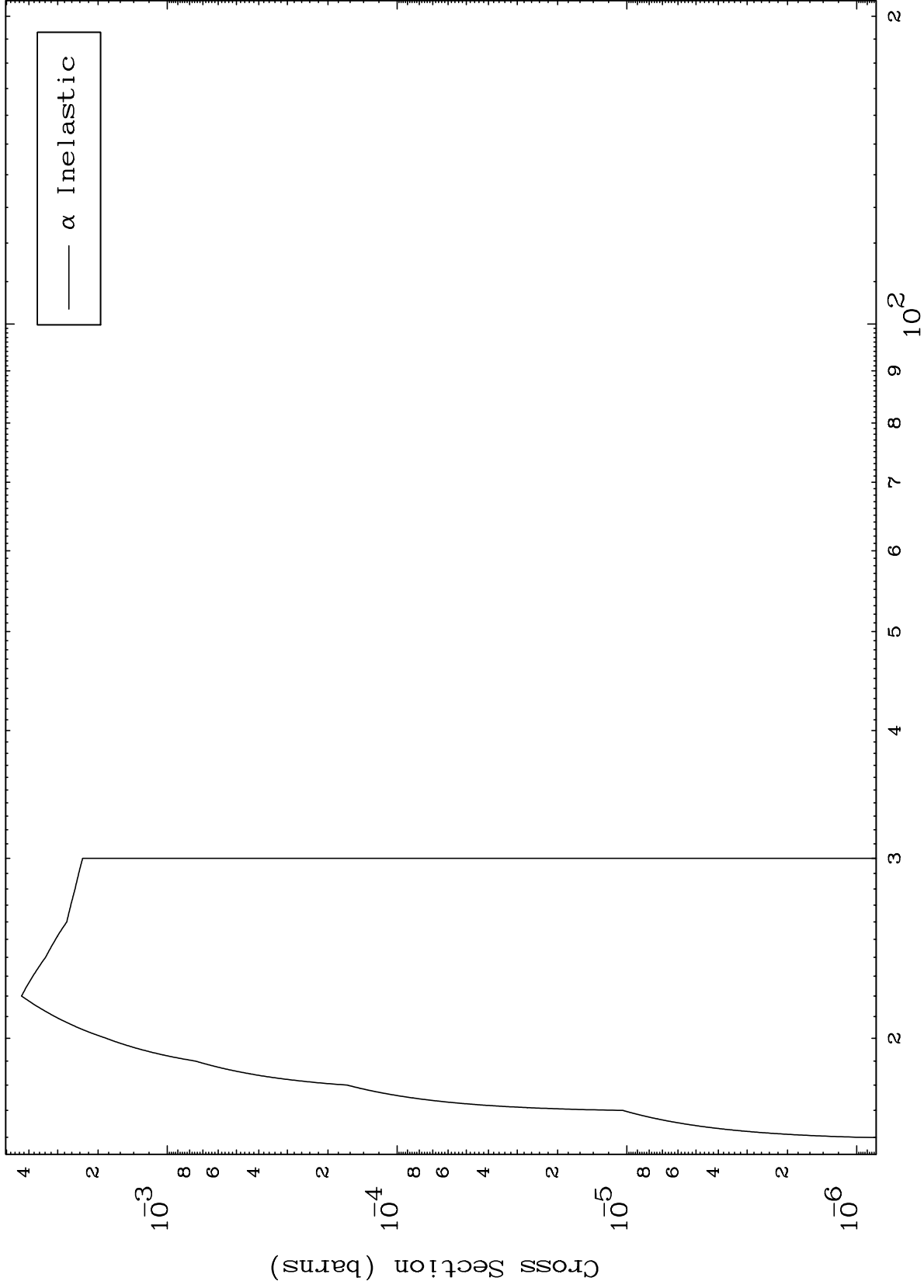


MAT 7063

( $\alpha, n'$ ) Level

71-Lu-154

0 Kelvin Cross Sections



5

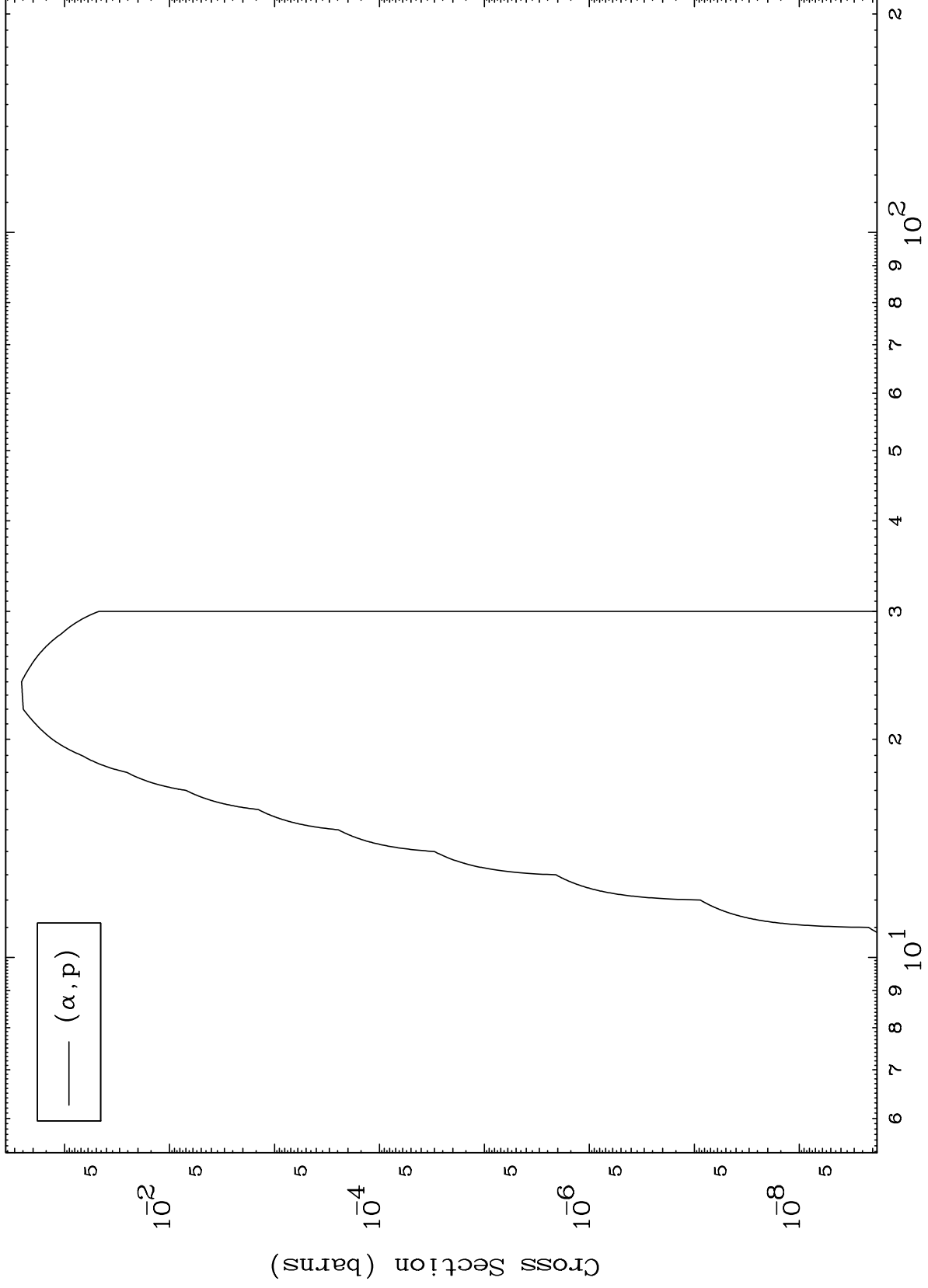
Incident Energy (MeV)

71-Lu-154

MAT 7063

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

71-Lu-154



6

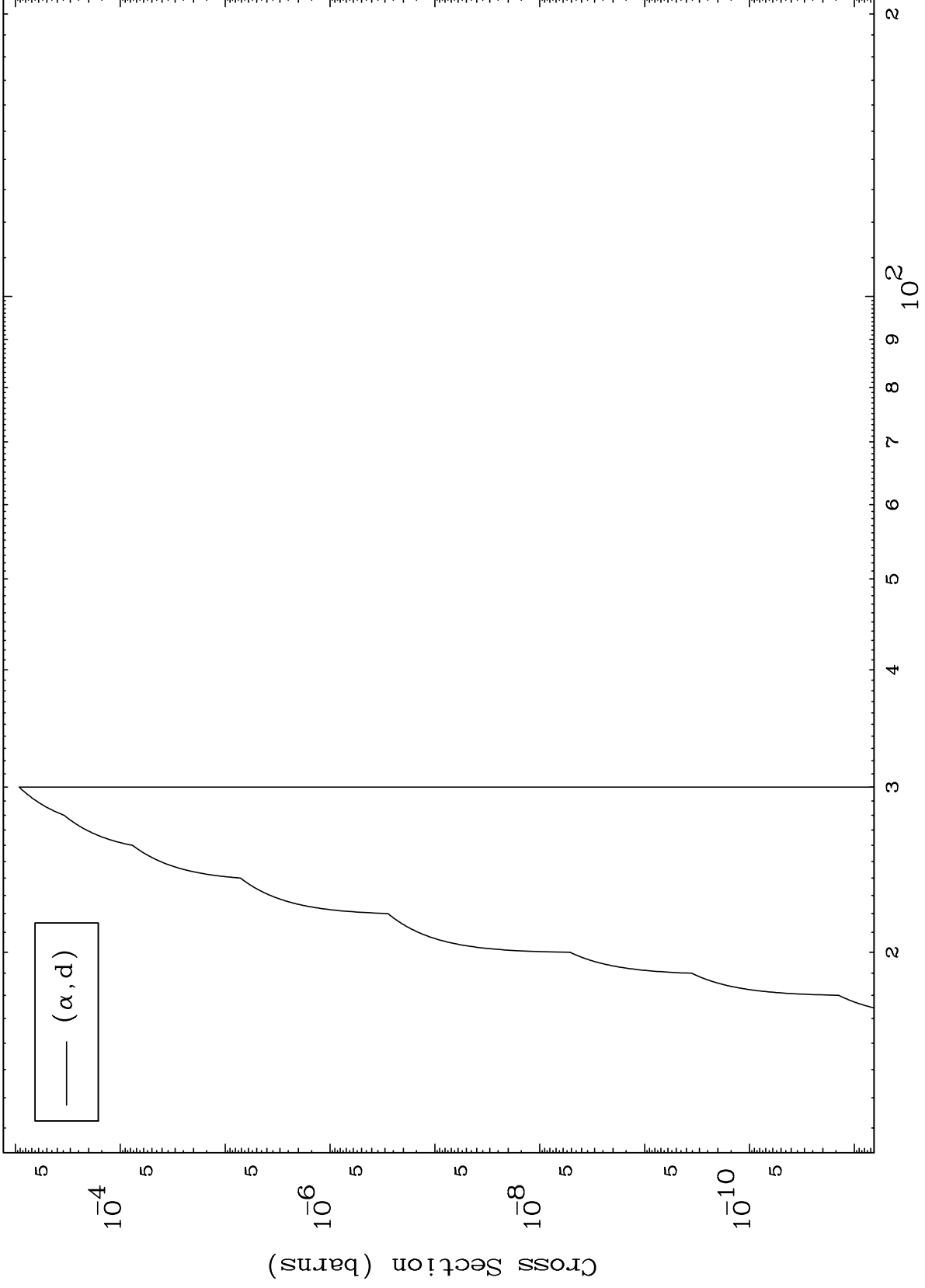
Incident Energy (MeV)

71-Lu-154

MAT 7063

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

71-Lu-154



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Incident Energy (MeV)

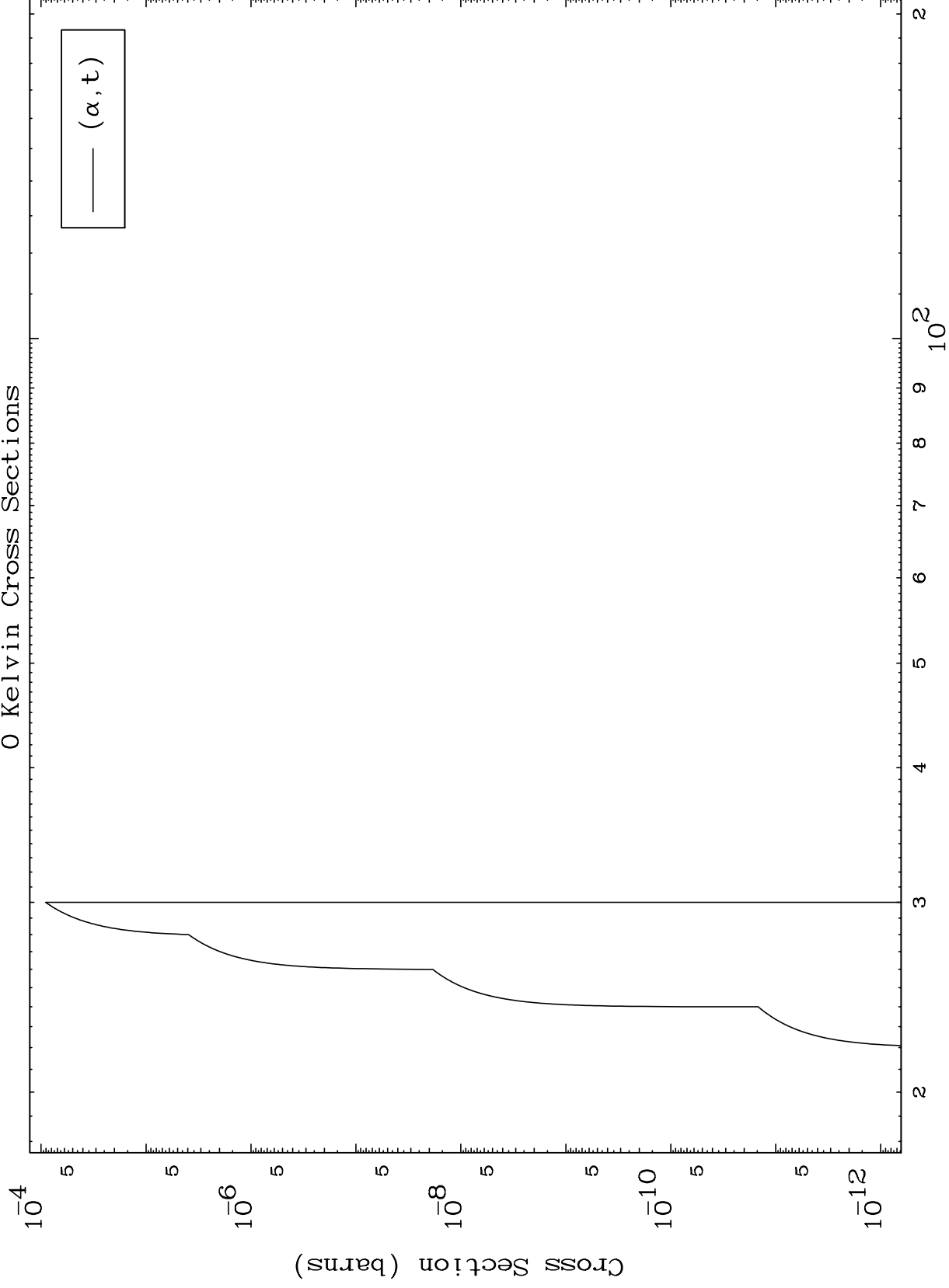
71-Lu-154



MAT 7063

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

71-Lu-154



8

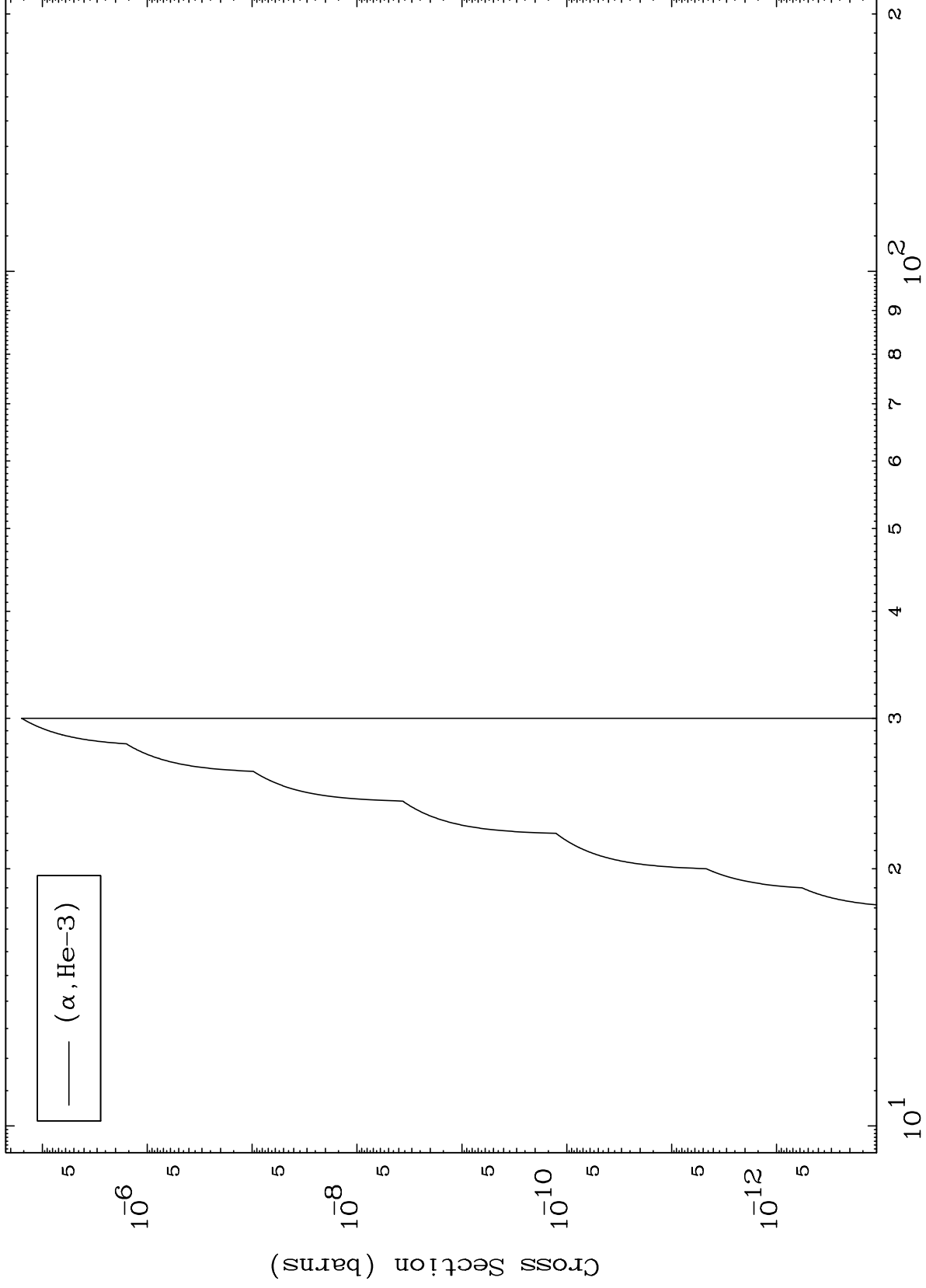
Incident Energy (MeV)

71-Lu-154

MAT 7063

( $\alpha, \text{He}3$ ) Levels  
0 Kelvin Cross Sections

71-Lu-154



Incident Energy (MeV)

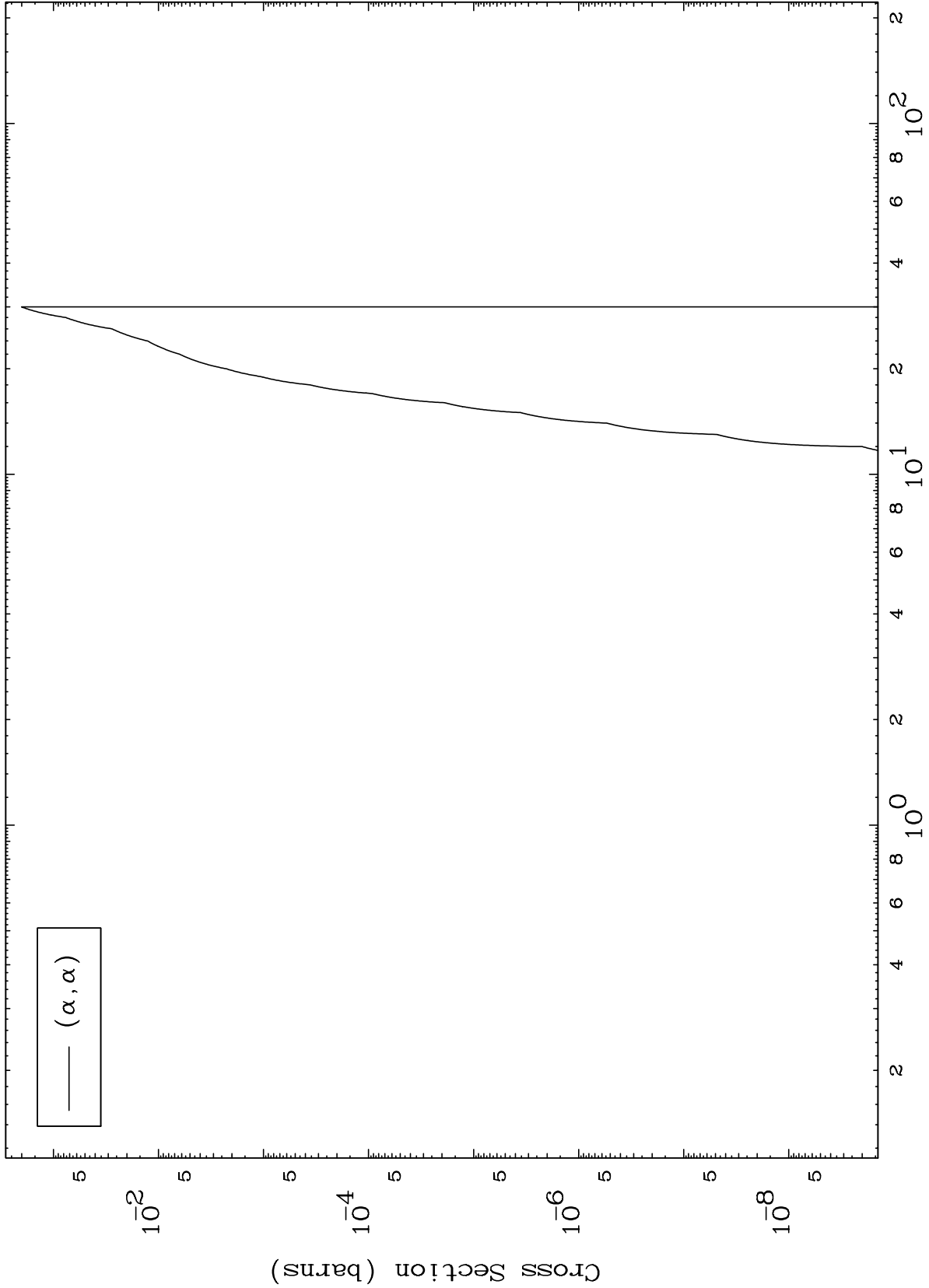
71-Lu-154

MAT 7063

( $\alpha, \alpha$ ) Levels

71-Lu-154

0 Kelvin Cross Sections



10

Incident Energy (MeV)

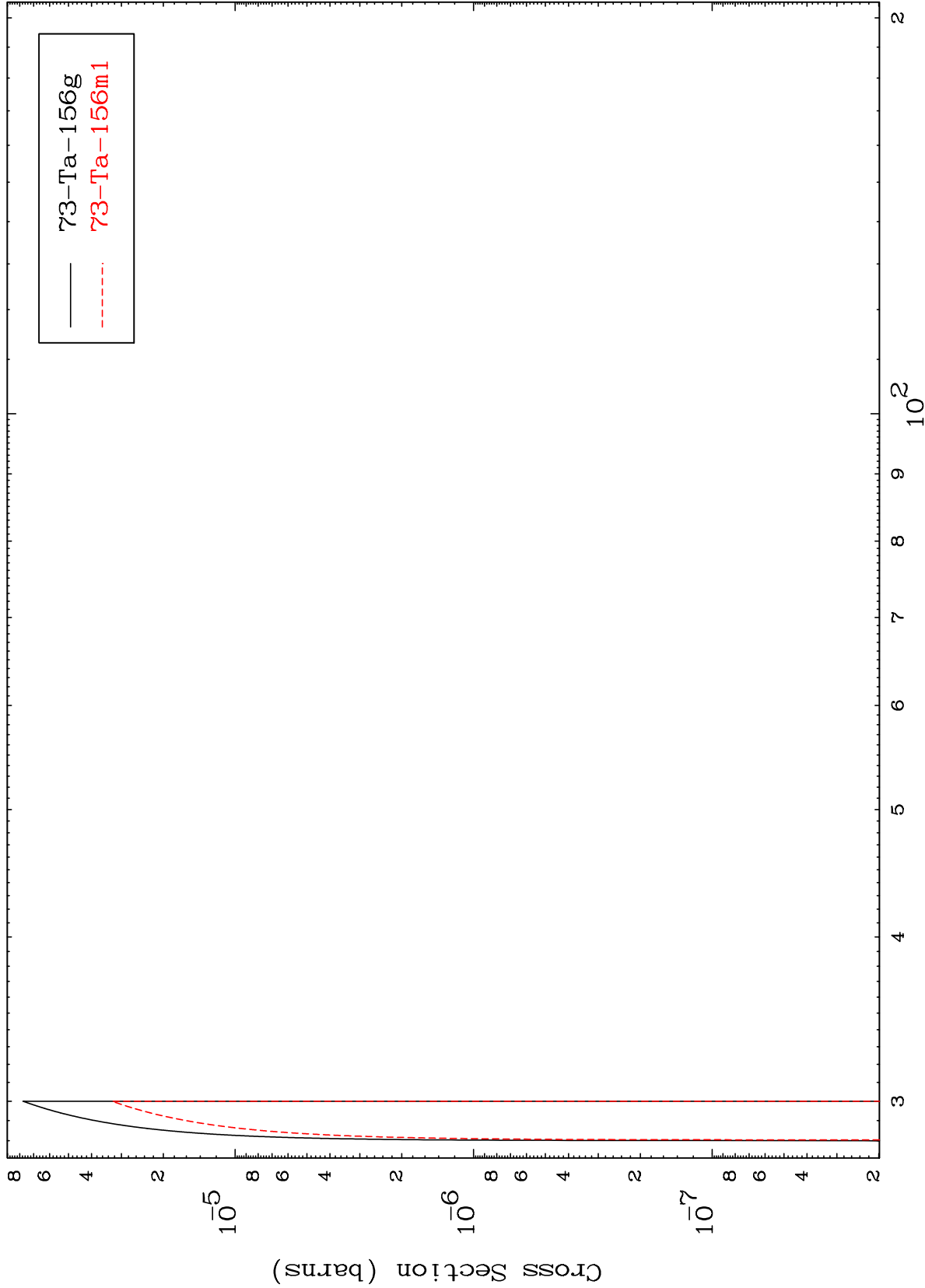
71-Lu-154

MAT 7063

( $\alpha, 2n$ )

71-Lu-154

Radionuclide Production Cross Section

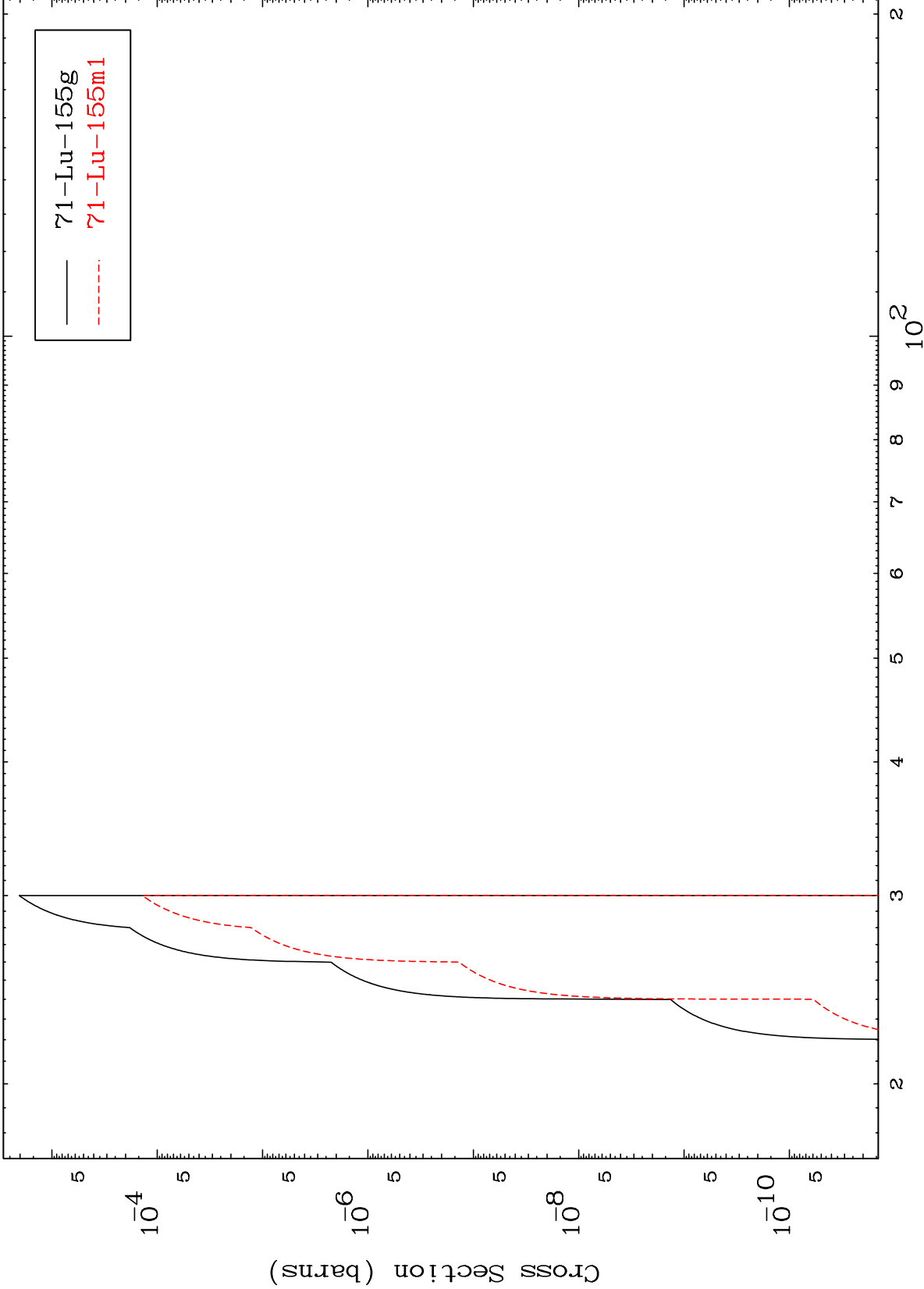


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Incident Energy (MeV)

71-Lu-154

Radionuclide Production Cross Section

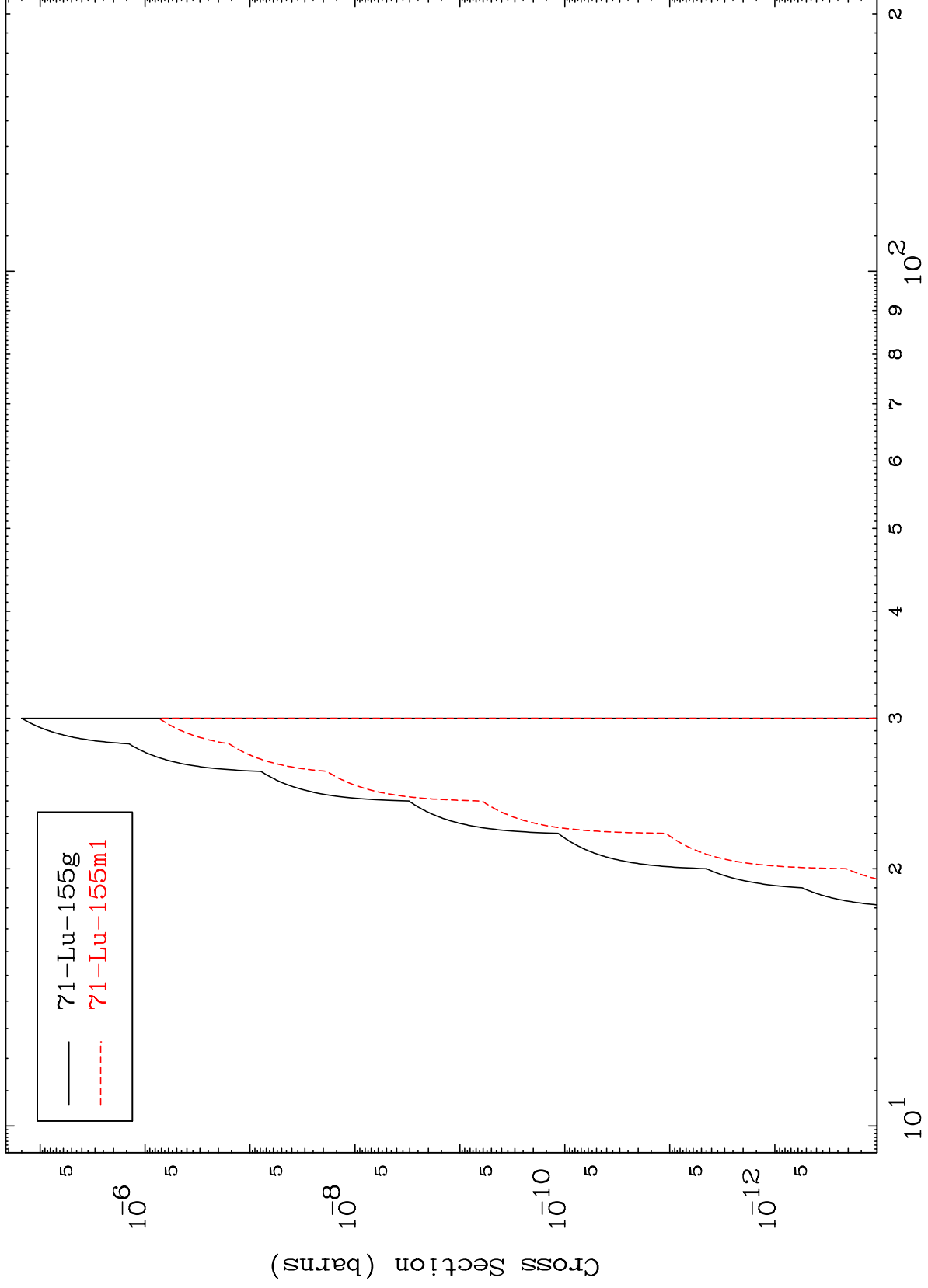


MAT 7063

( $\alpha, \text{He-3}$ )

$^{71}\text{Lu-154}$

Radionuclide Production Cross Section



—  $^{71}\text{Lu-155g}$   
- - -  $^{71}\text{Lu-155m1}$

13

Incident Energy (MeV)

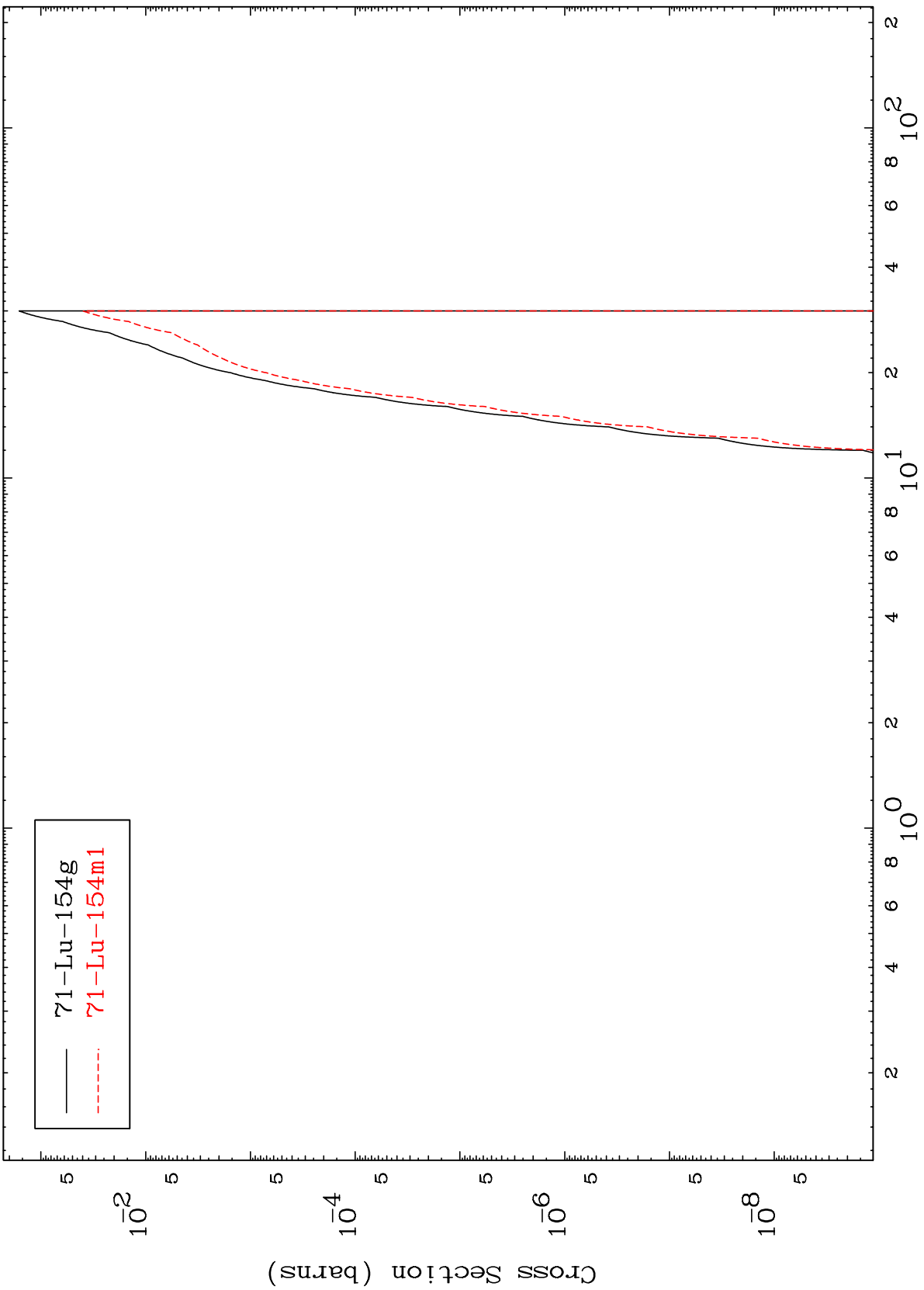
$^{71}\text{Lu-154}$

MAT 7063

( $\alpha, \alpha$ )

<sup>71</sup>Lu-154

Radionuclide Production Cross Section



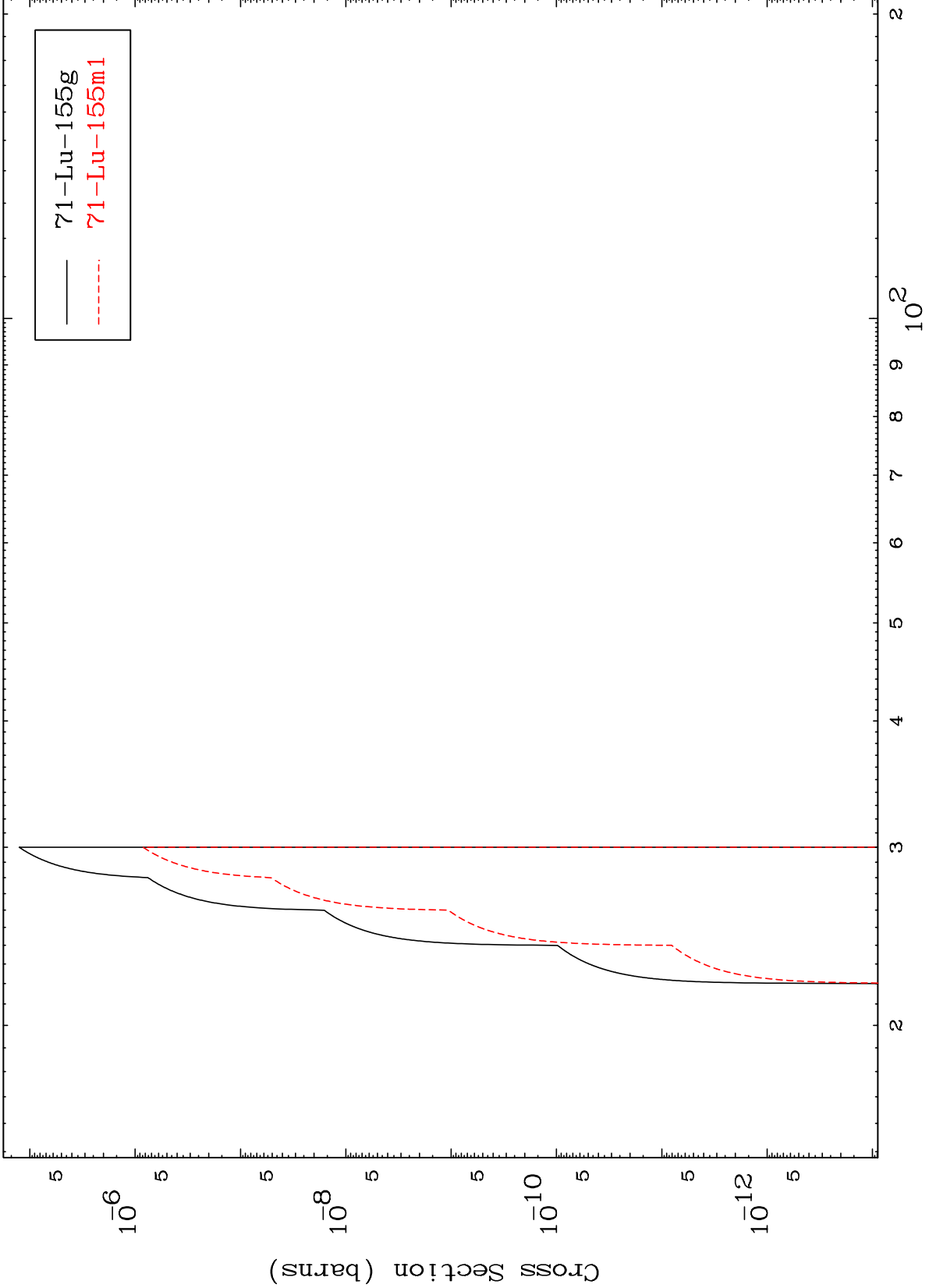
— <sup>71</sup>Lu-154g  
- - - <sup>71</sup>Lu-154m1

MAT 7063

( $\alpha, p$ ) d

<sup>71</sup>Lu-154

Radionuclide Production Cross Section



15

Incident Energy (MeV)

<sup>71</sup>Lu-154