

Program EVALPLOT  
(Version 2018-1)

by

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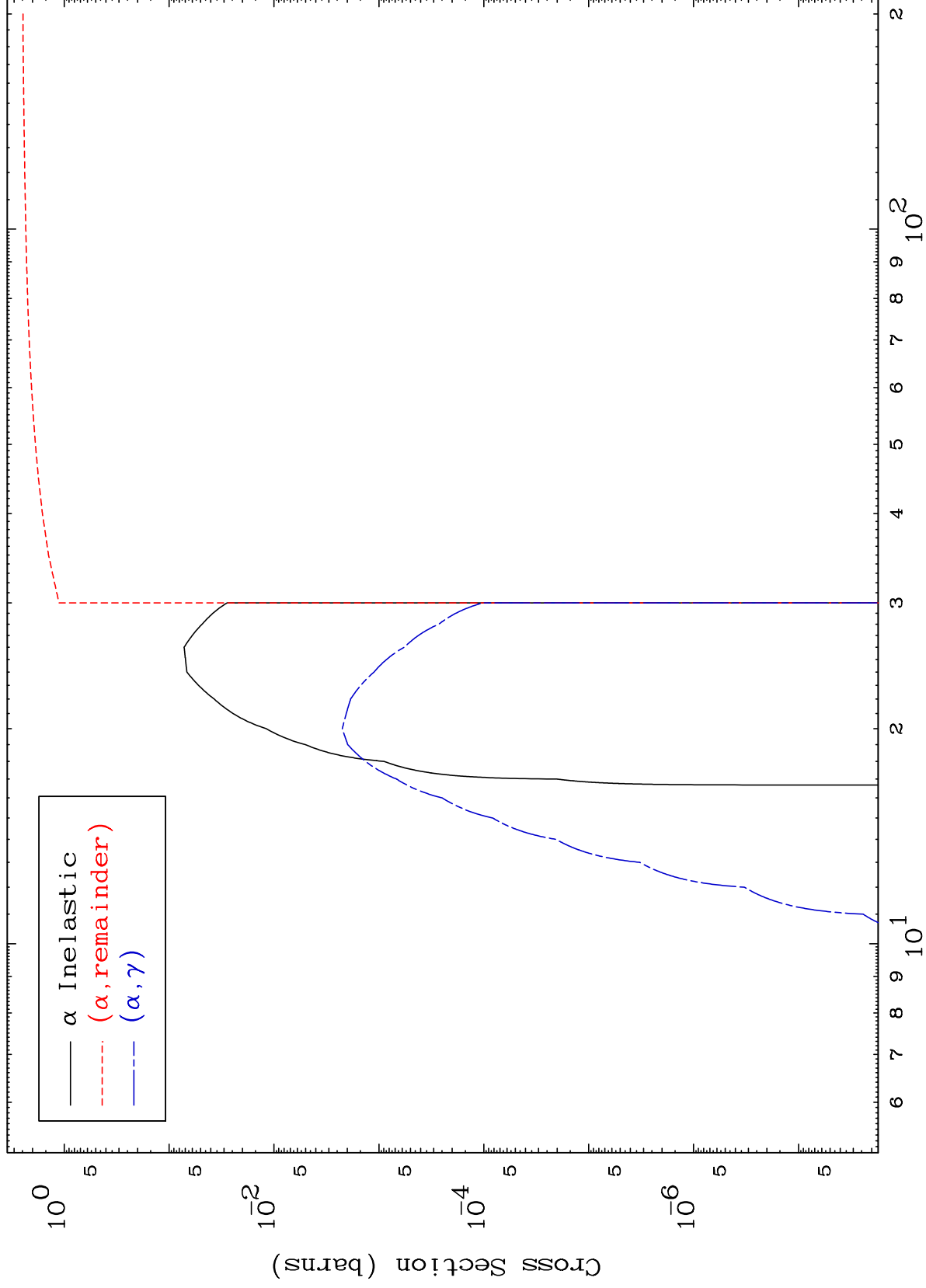
Press Mouse Button to Start

MAT 7071

0 Kelvin

$\alpha$  Major Cross Sections

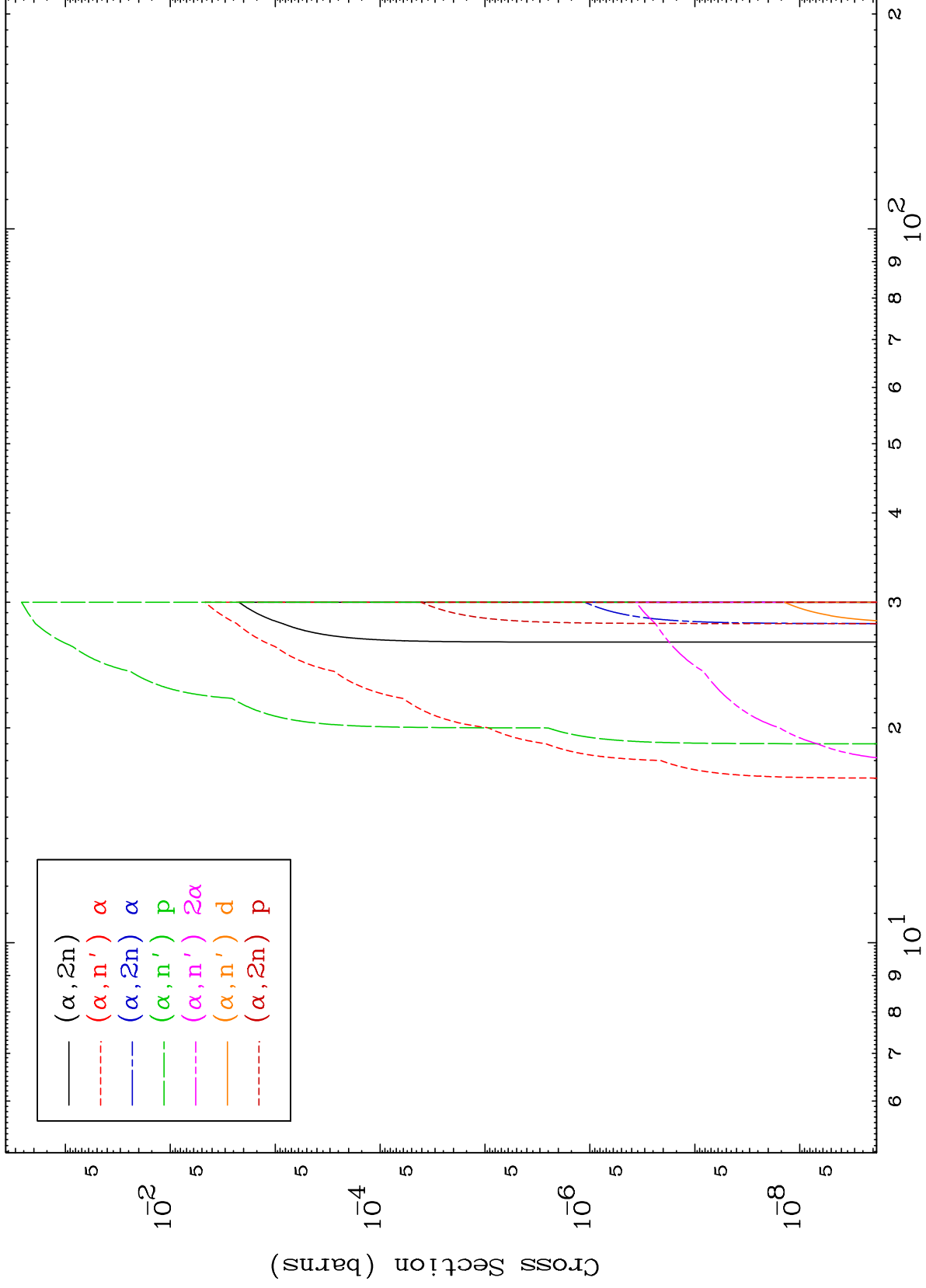
71-Lu-157

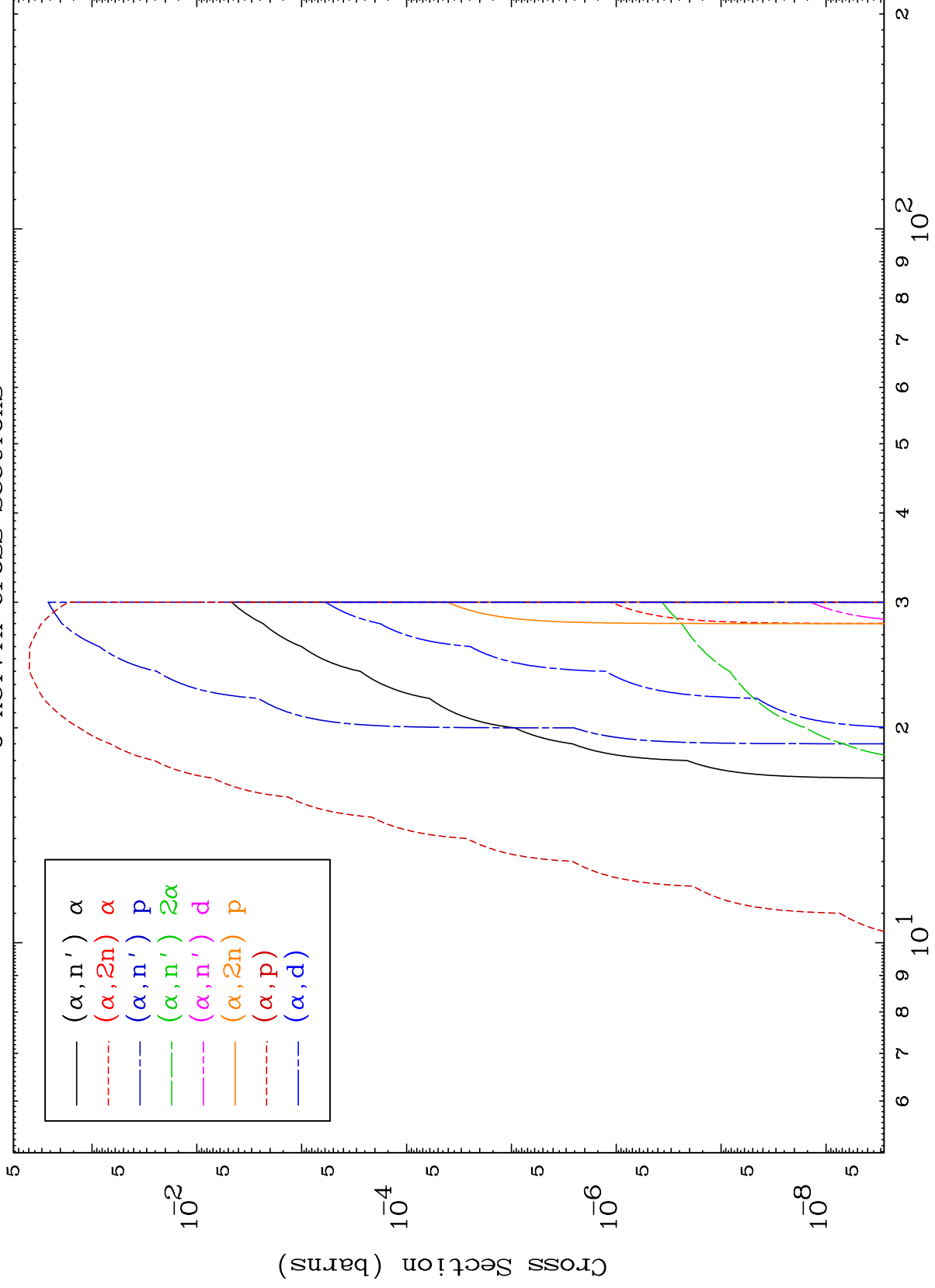


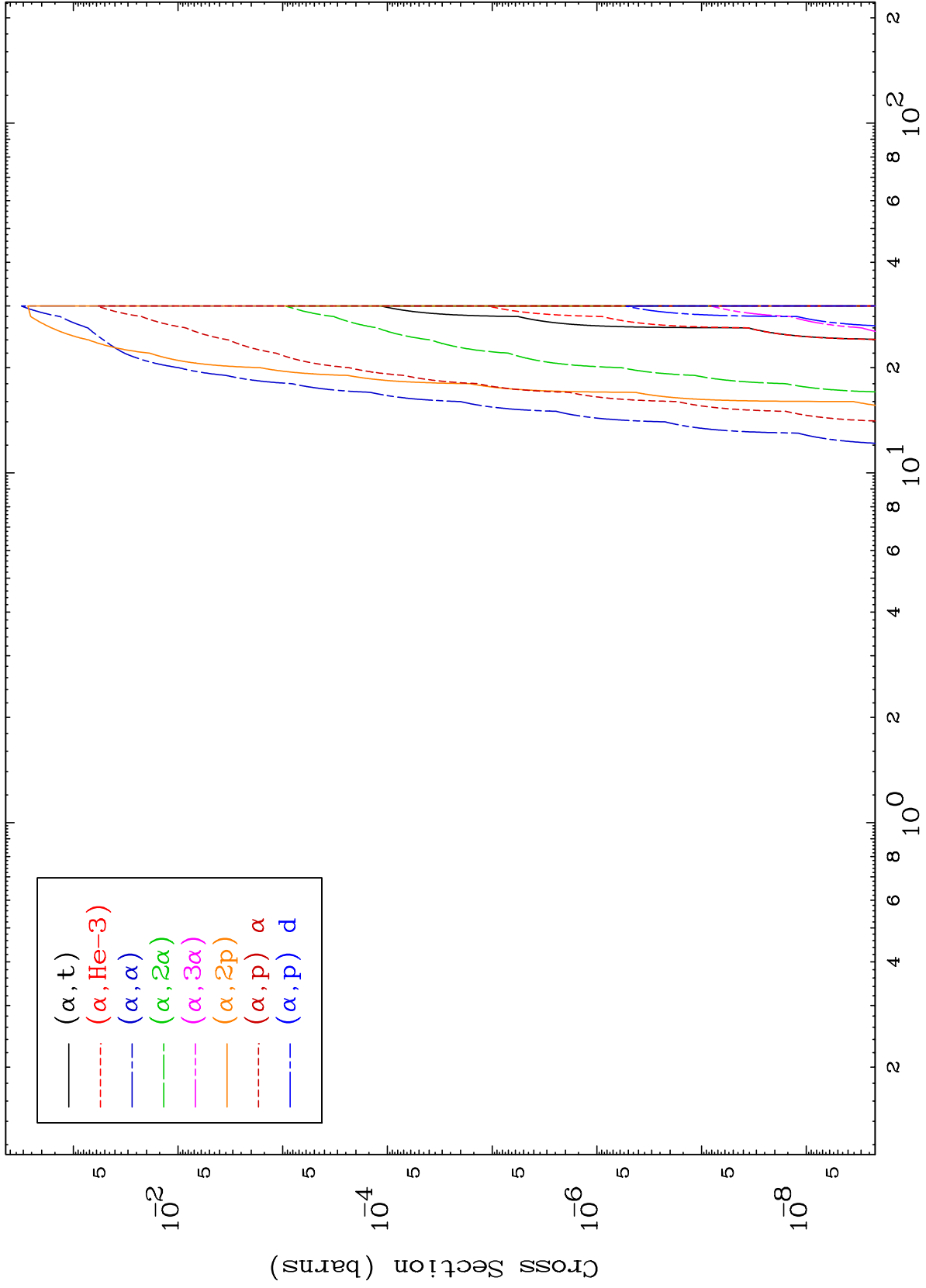
Incident Energy (MeV)

71-Lu-157

1





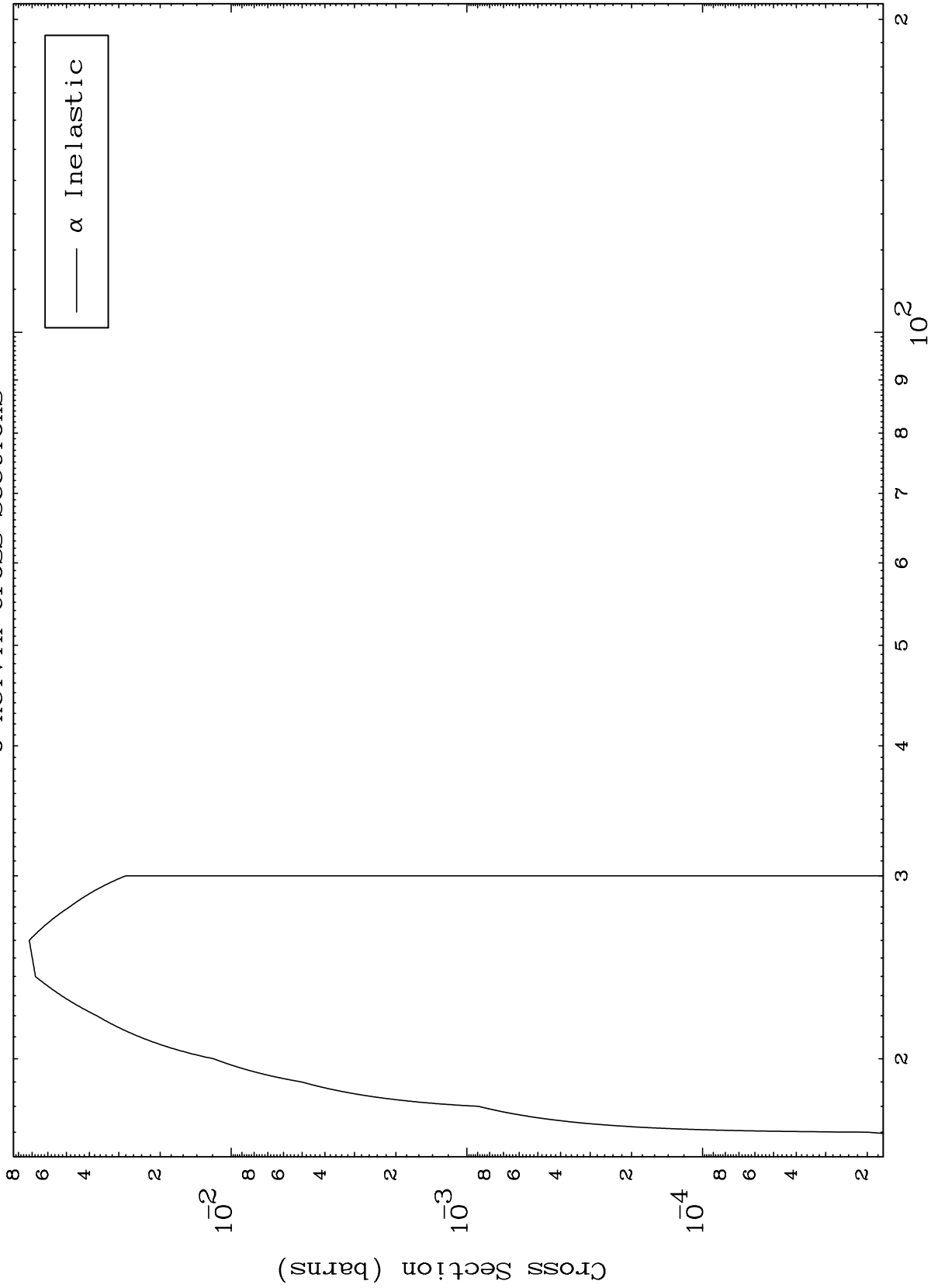


MAT 7071

( $\alpha, n'$ ) Level

71-Lu-157

0 Kelvin Cross Sections



5

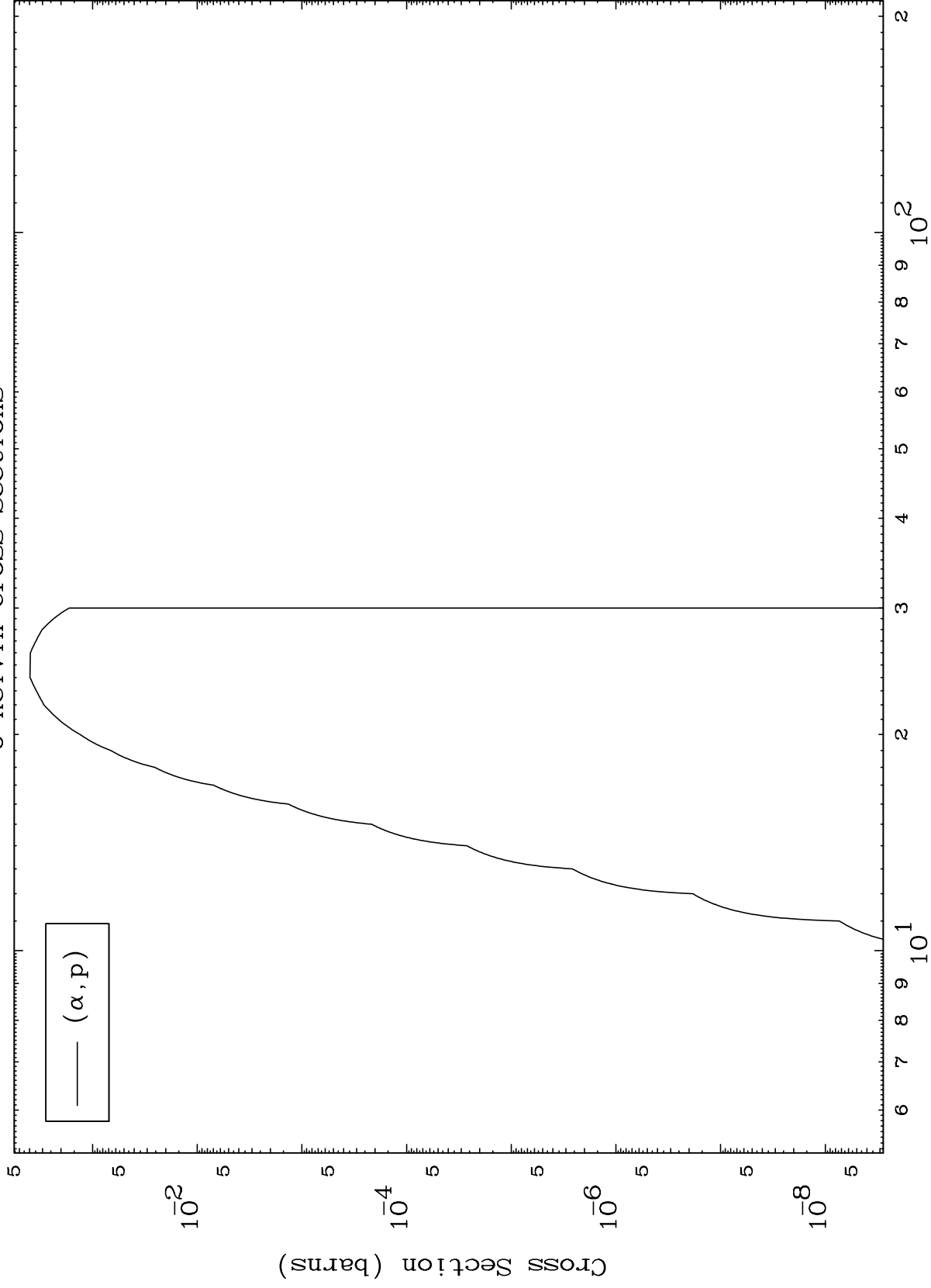
Incident Energy (MeV)

71-Lu-157

MAT 7071

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

71-Lu-157



6

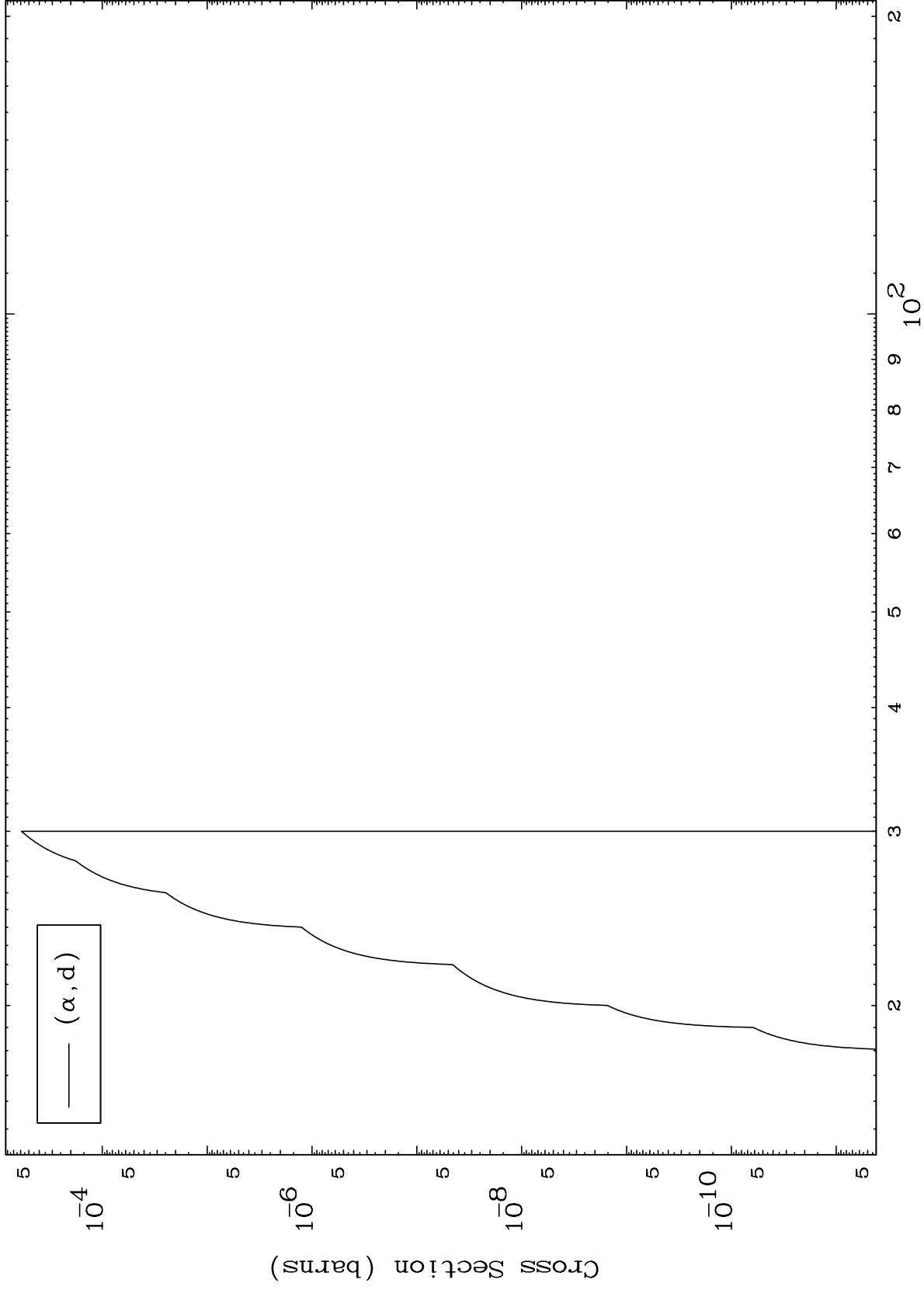
Incident Energy (MeV)

71-Lu-157

MAT 7071

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

71-Lu-157

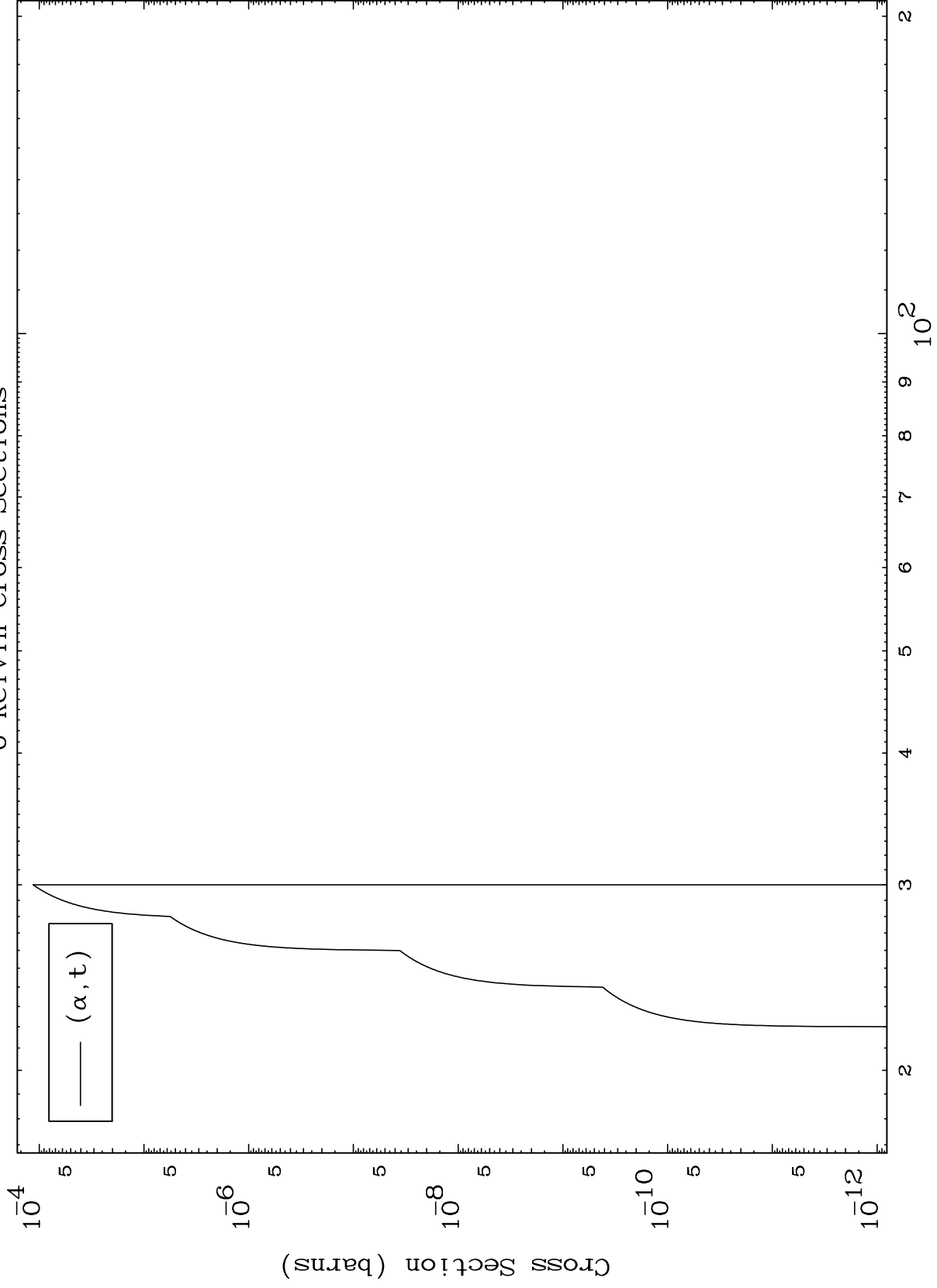


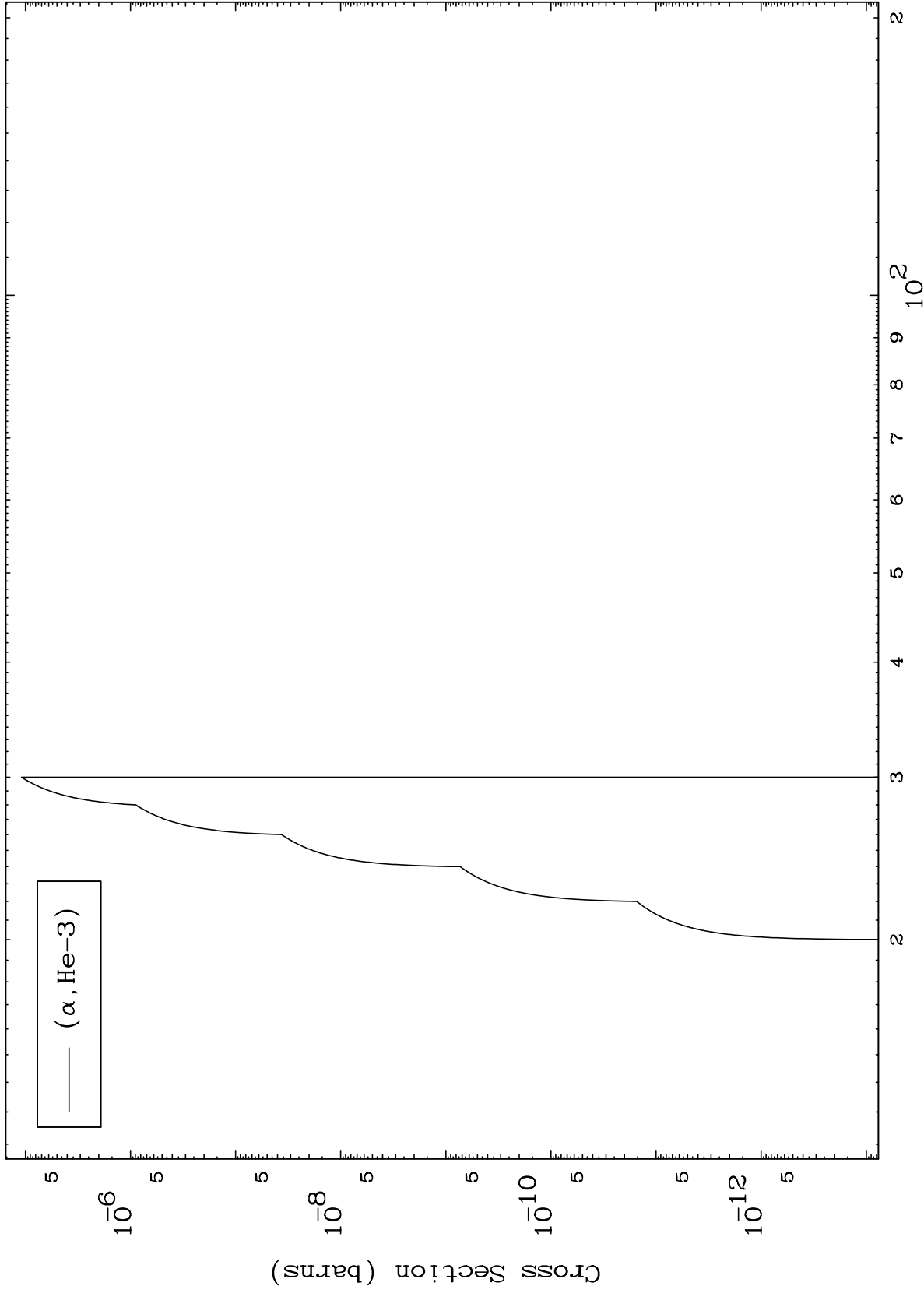
7

Incident Energy (MeV)

71-Lu-157





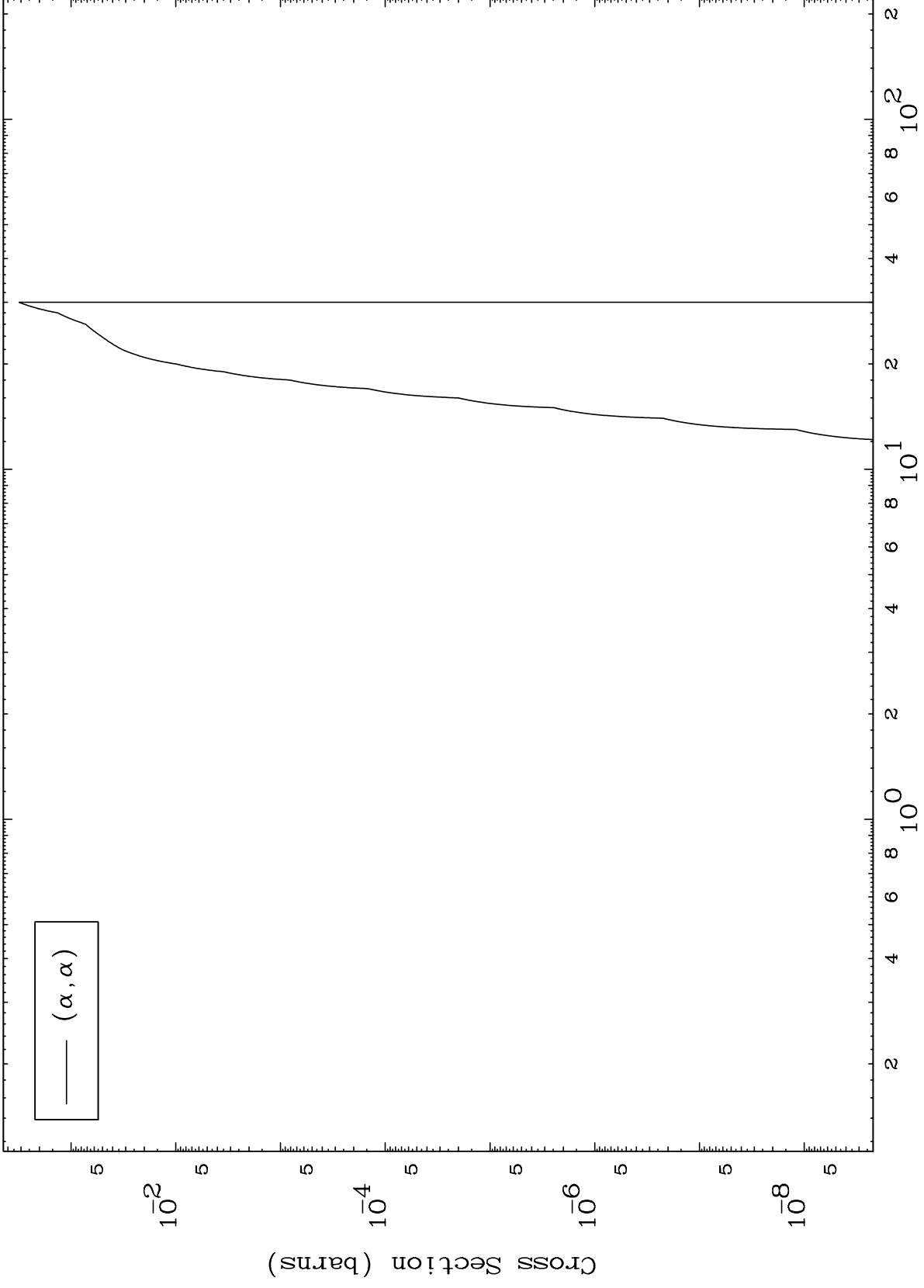


MAT 7071

( $\alpha, \alpha$ ) Levels

71-Lu-157

0 Kelvin Cross Sections

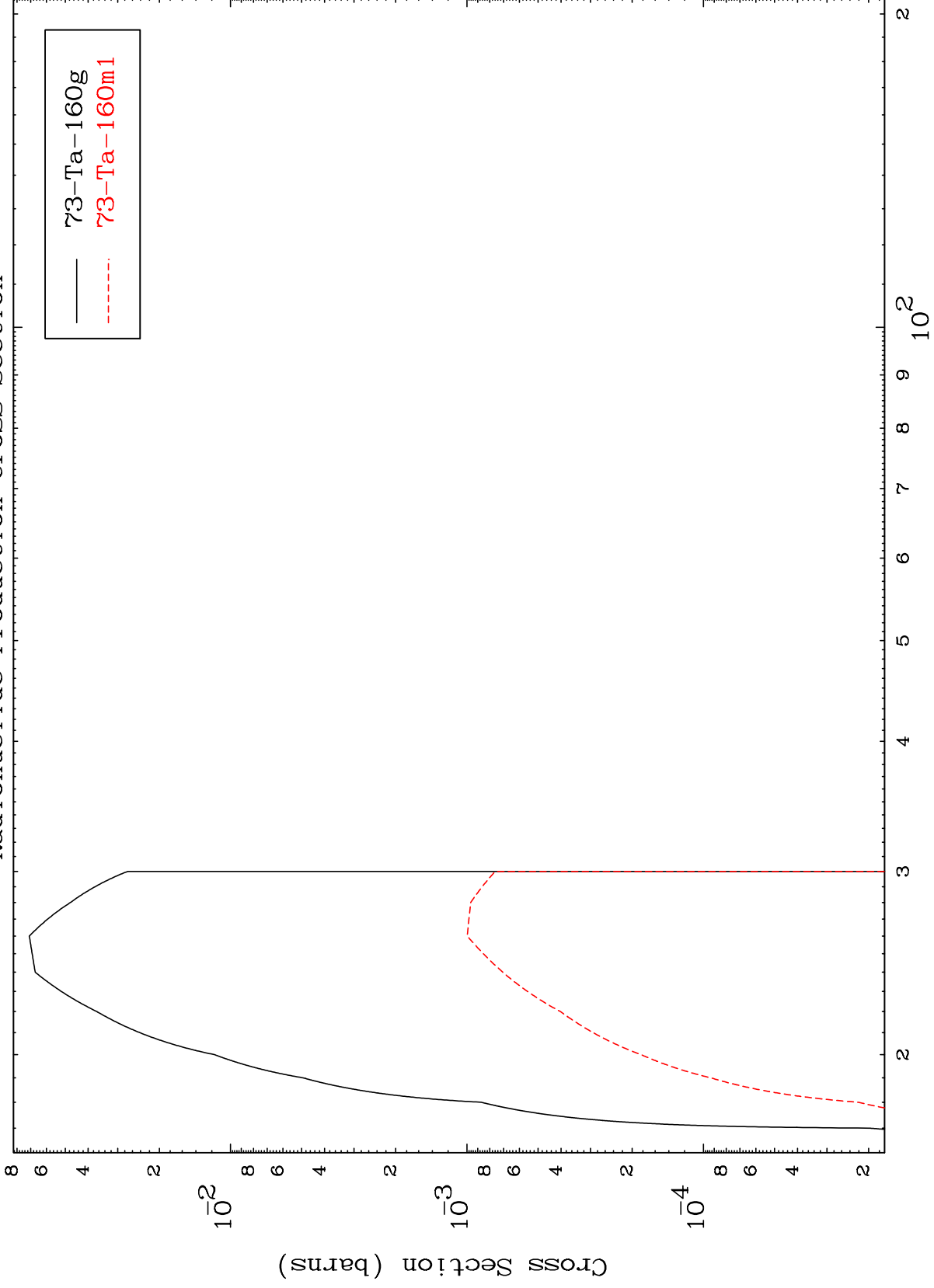


10

Incident Energy (MeV)

71-Lu-157

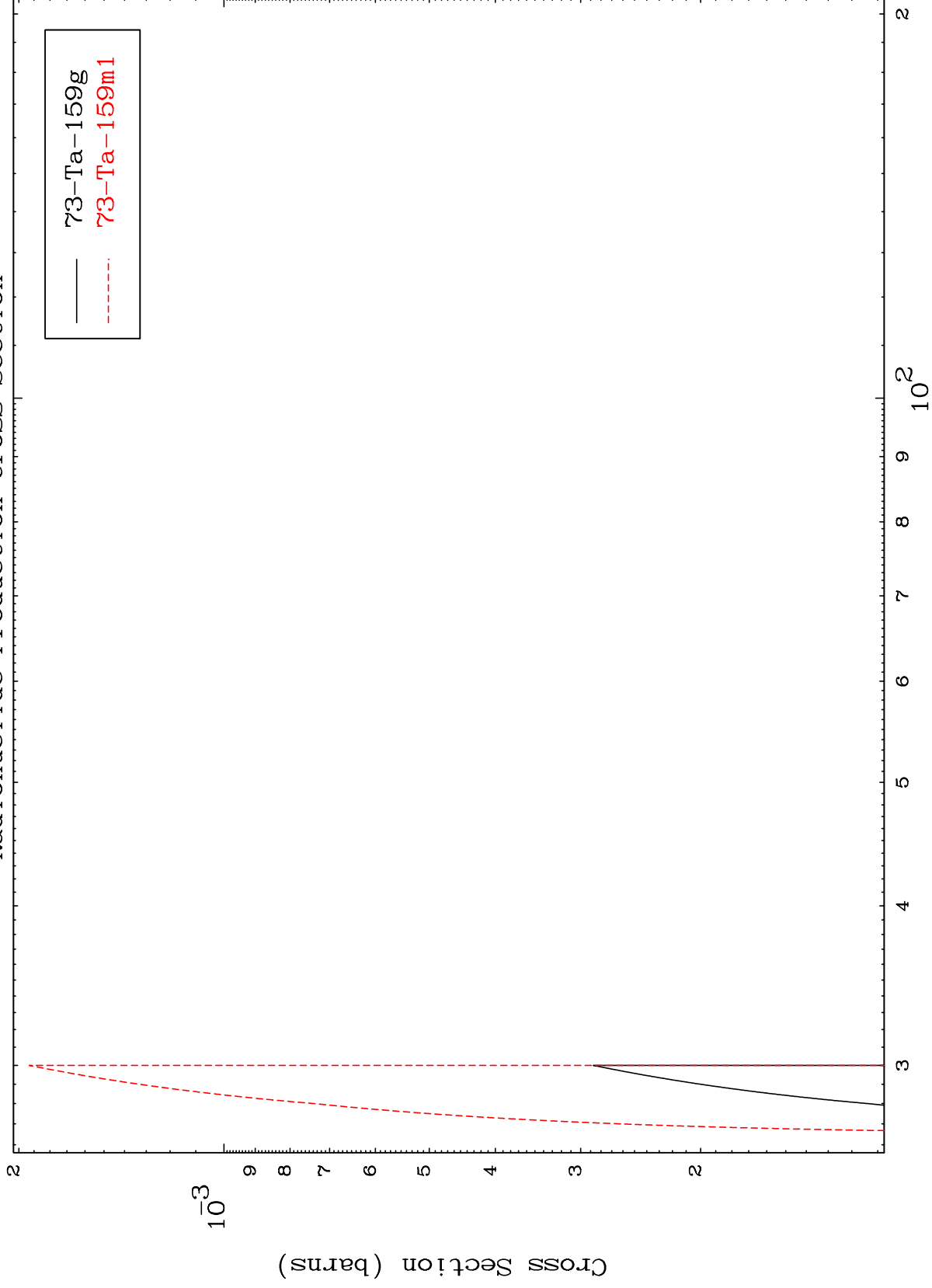
$\alpha$  Inelastic  
Radionuclide Production Cross Section



MAT 7071

71-Lu-157

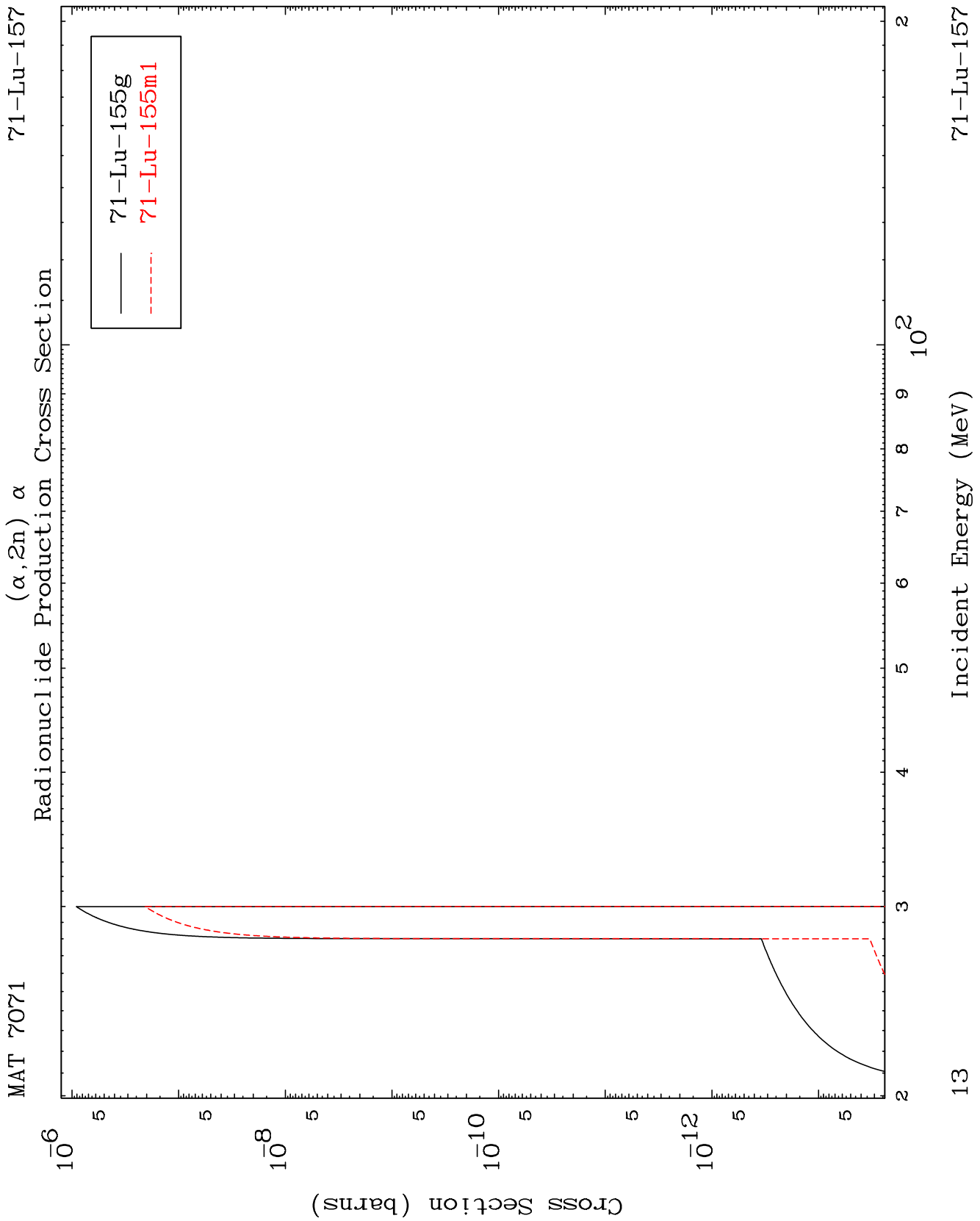
( $\alpha, 2n$ )  
Radionuclide Production Cross Section



12

Incident Energy (MeV)

71-Lu-157

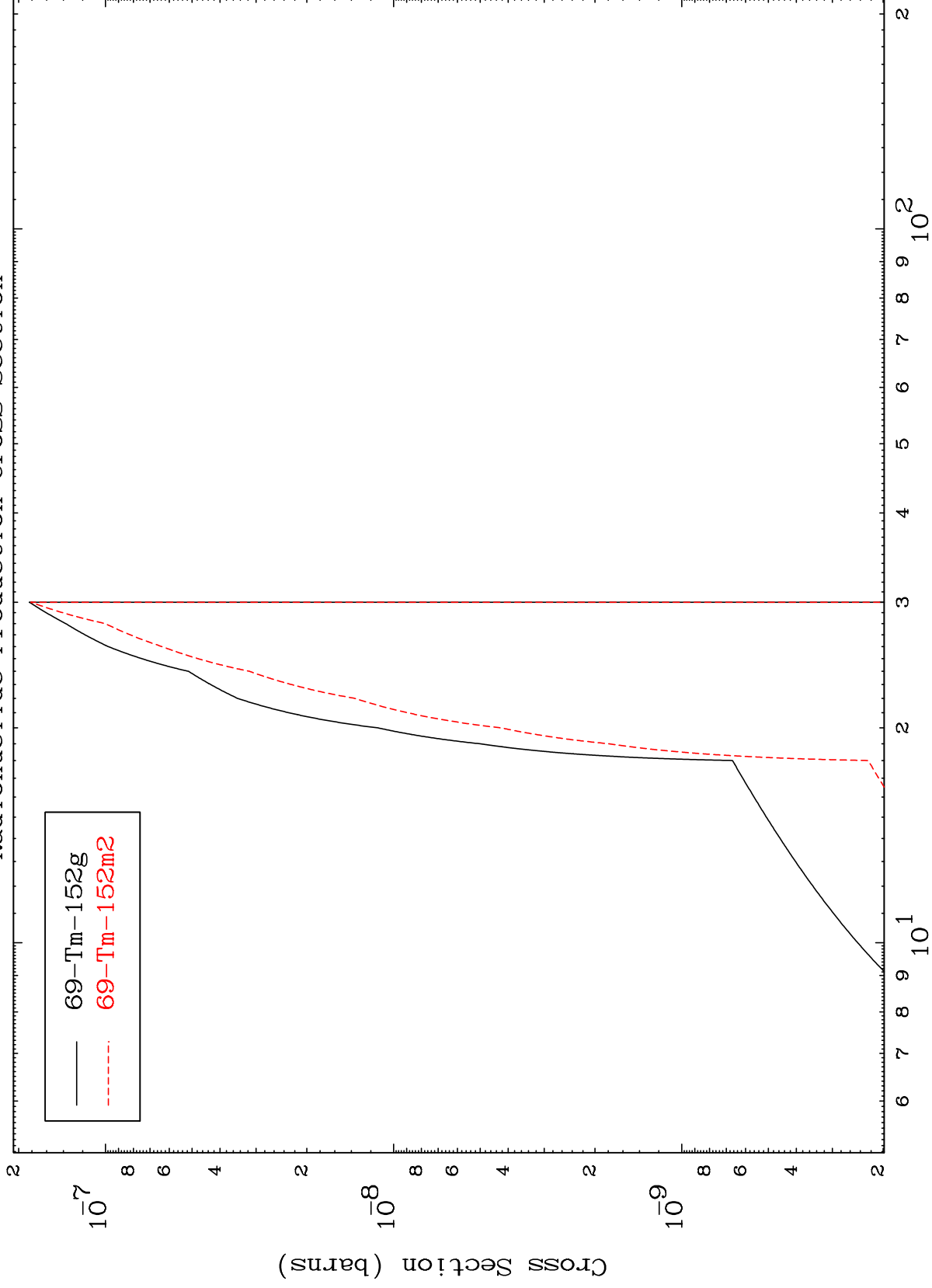


MAT 7071

( $\alpha, n'$ )  $2\alpha$

$^{71}\text{Lu-157}$

Radionuclide Production Cross Section



14

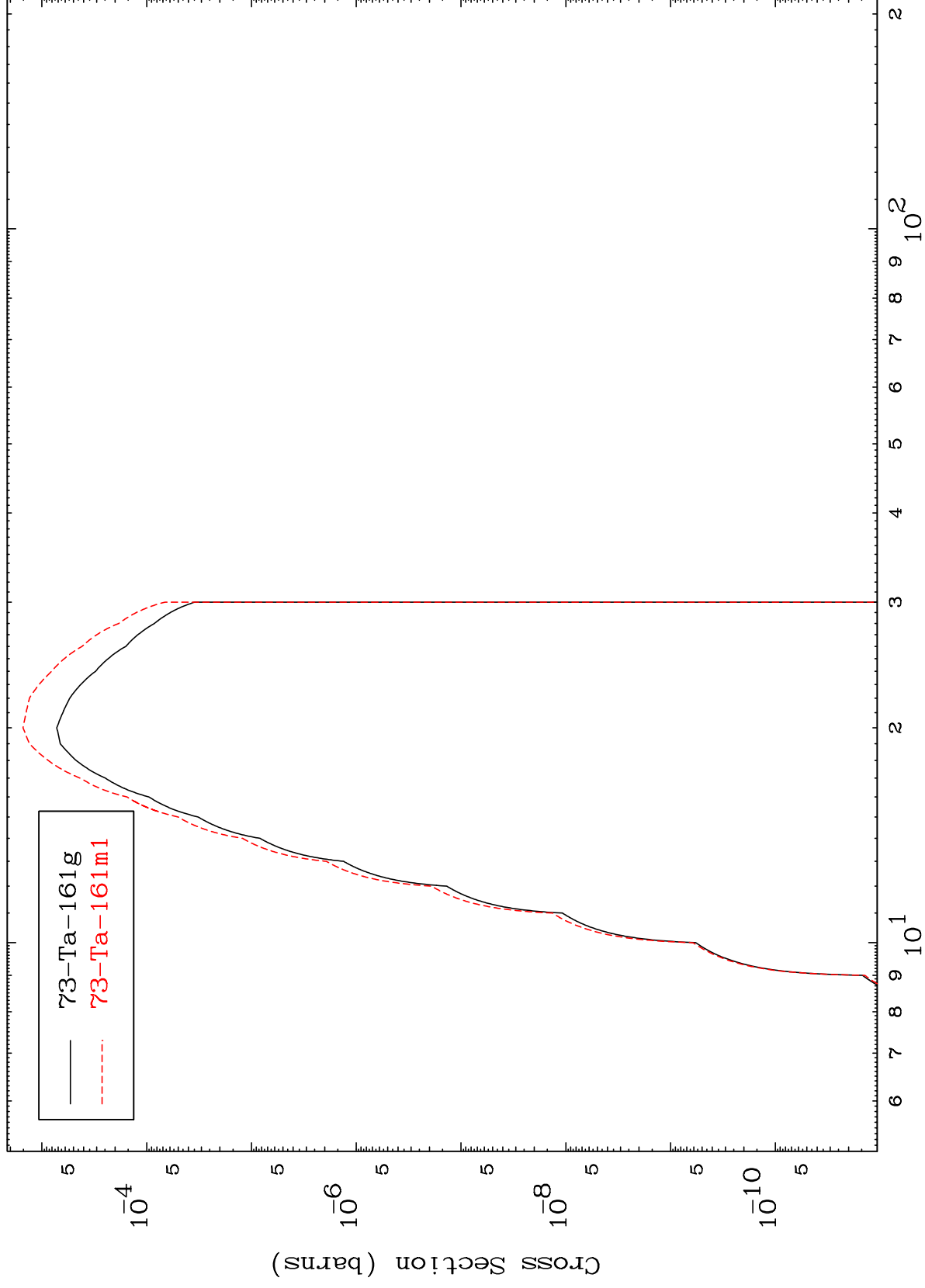
Incident Energy (MeV)

$^{71}\text{Lu-157}$

MAT 7071

71-Lu-157

( $\alpha, \gamma$ )  
Radionuclide Production Cross Section



15

Incident Energy (MeV)

71-Lu-157

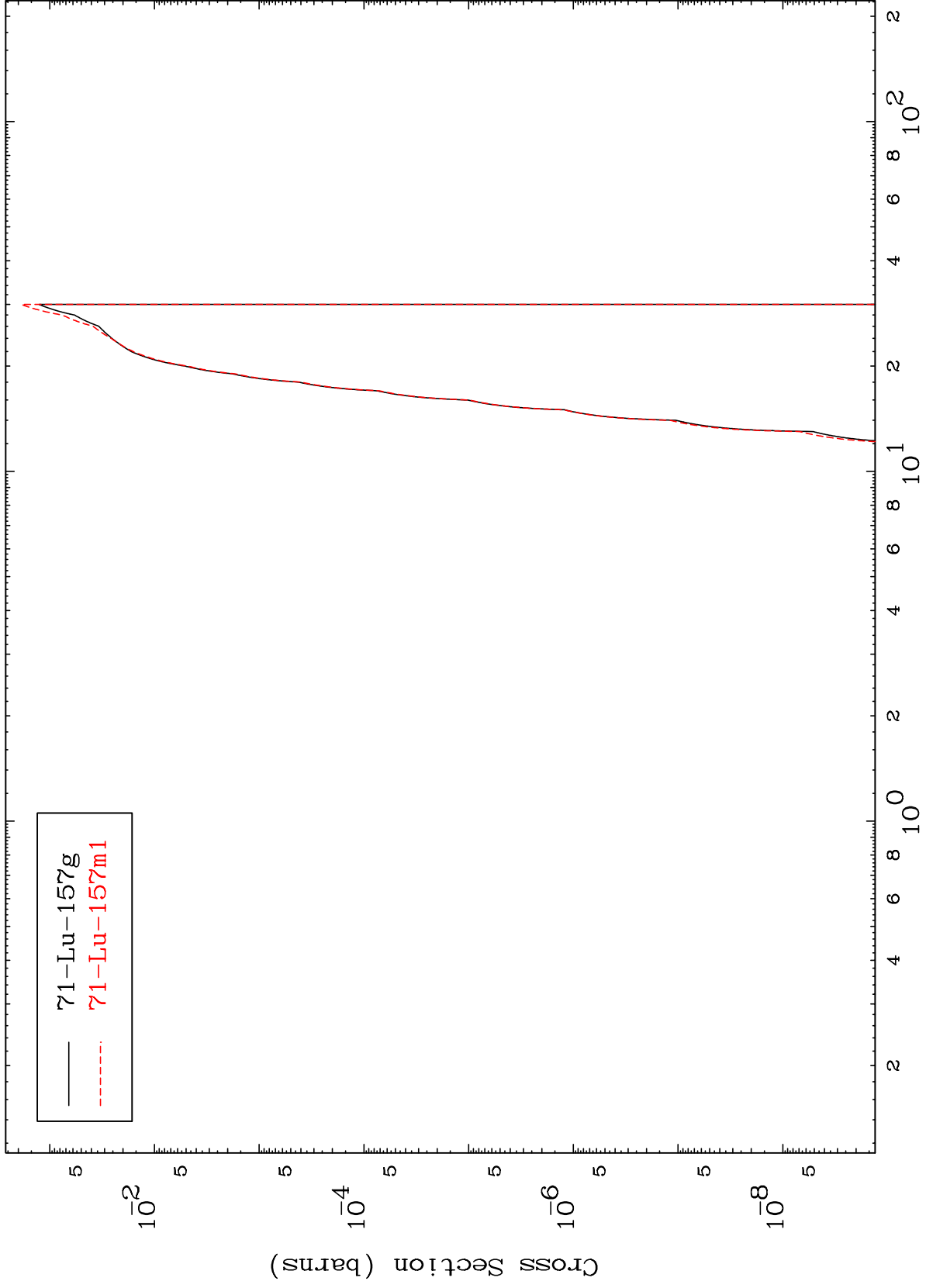


MAT 7071

<sup>71</sup>Lu-157

( $\alpha, \alpha$ )

Radionuclide Production Cross Section



16

Incident Energy (MeV)

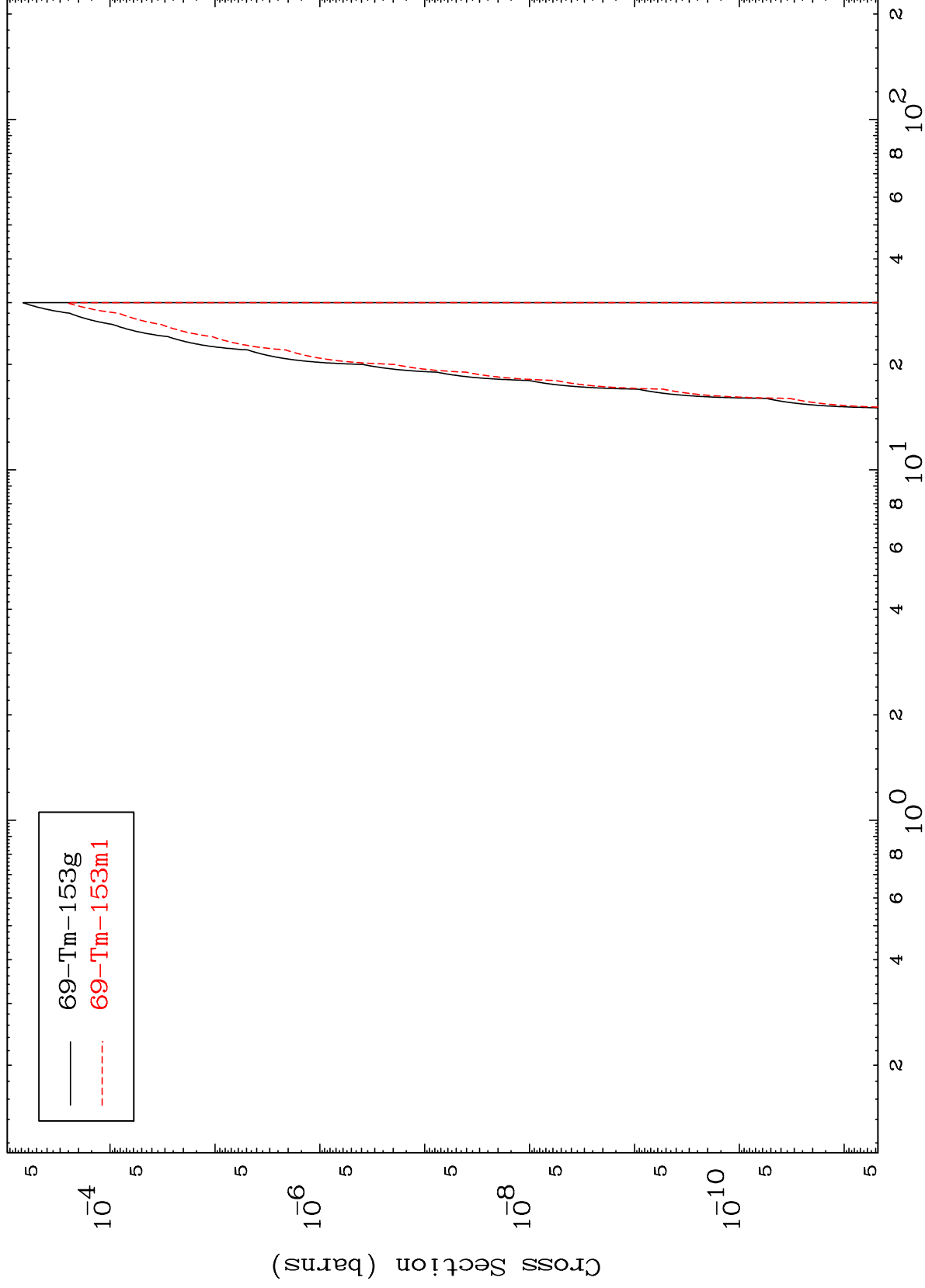
<sup>71</sup>Lu-157

MAT 7071

( $\alpha, 2\alpha$ )

71-Lu-157

Radionuclide Production Cross Section



17

Incident Energy (MeV)

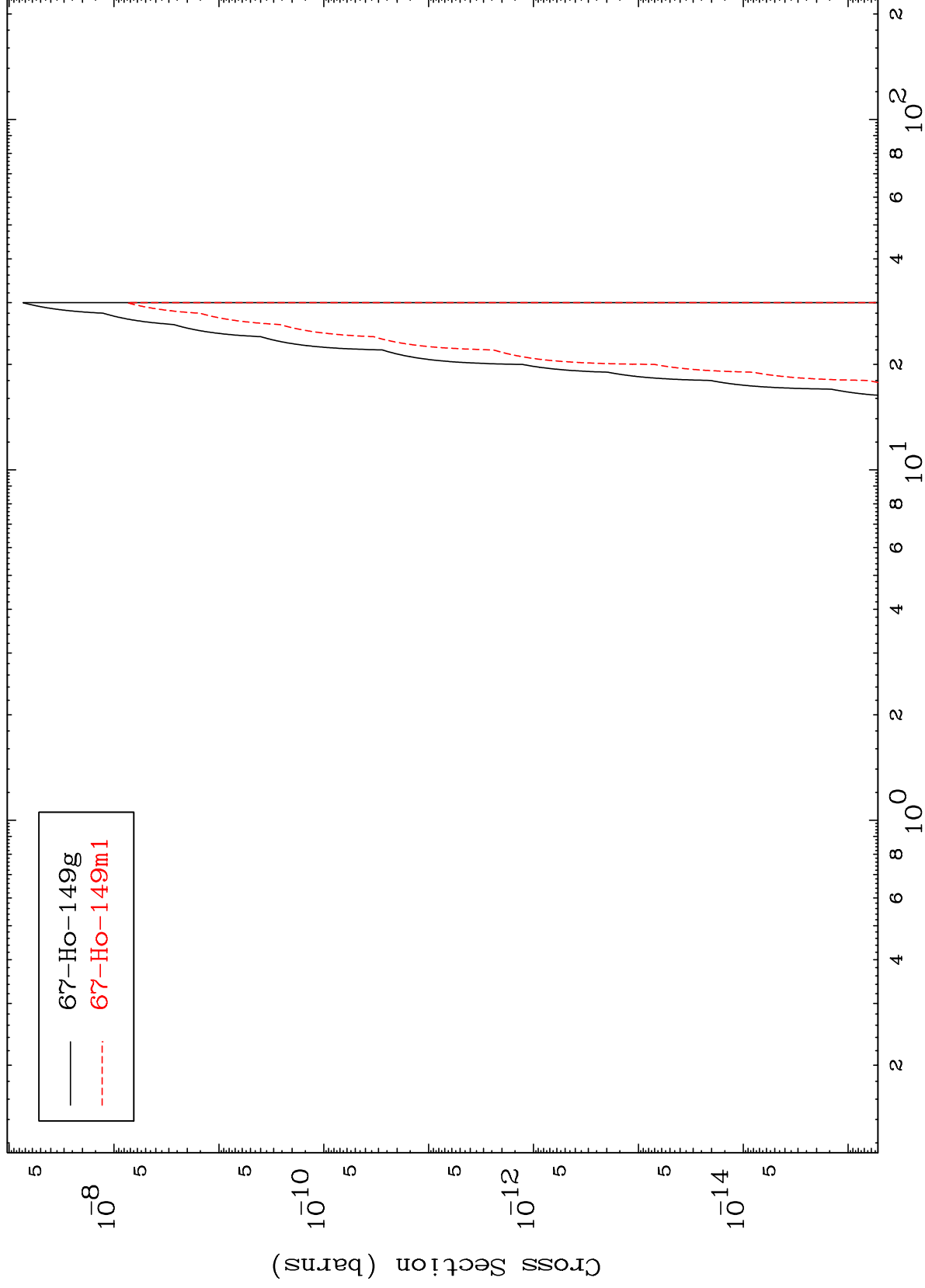
71-Lu-157

MAT 7071

( $\alpha, 3\alpha$ )

<sup>71</sup>Lu-157

Radionuclide Production Cross Section



18

Incident Energy (MeV)

<sup>71</sup>Lu-157