

Program EVALPLOT  
(Version 2018-1)

by

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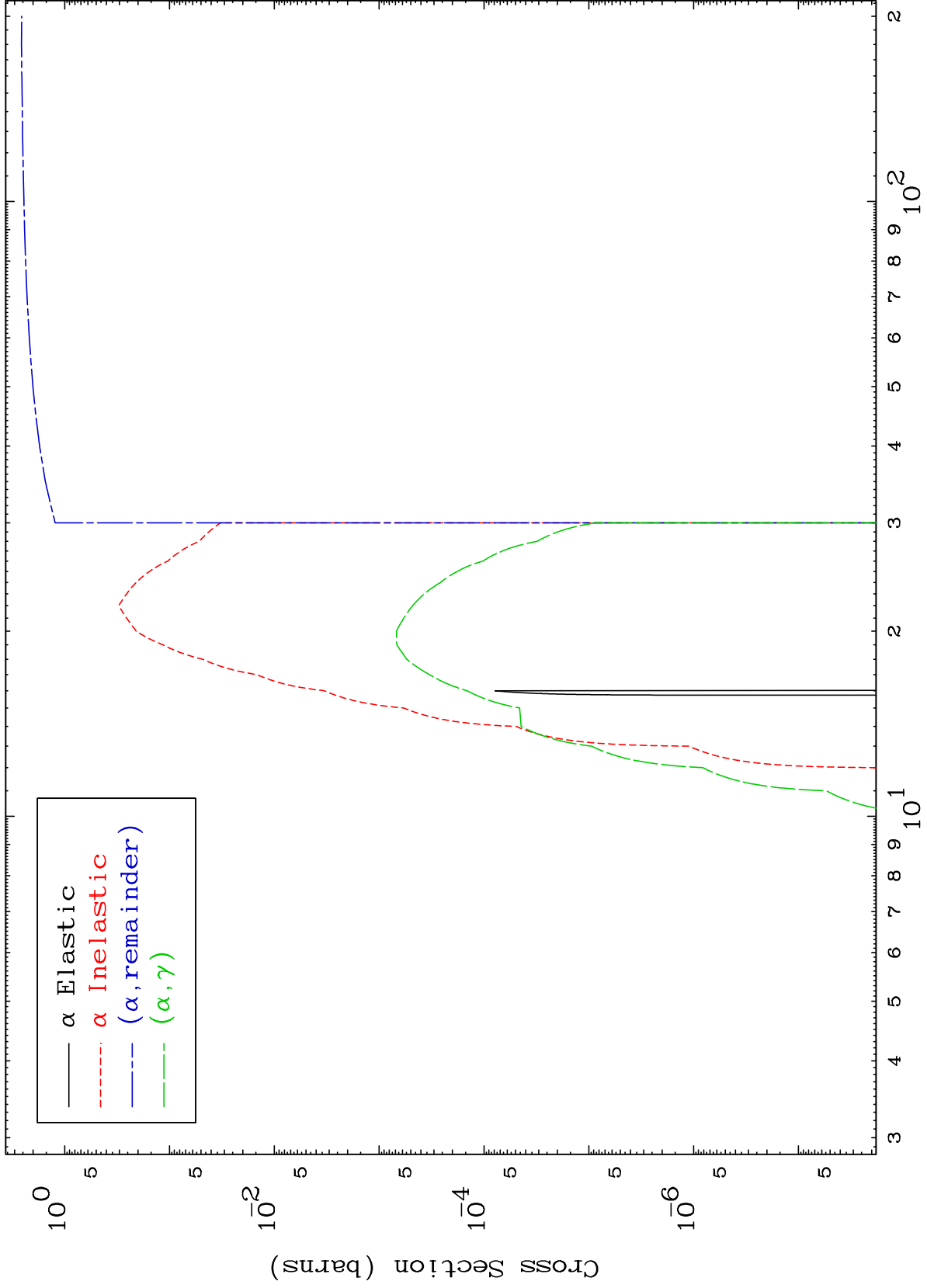
Press Mouse Button to Start

MAT 7114

0 Kelvin

$\alpha$  Major Cross Sections

71-Lu-171



1

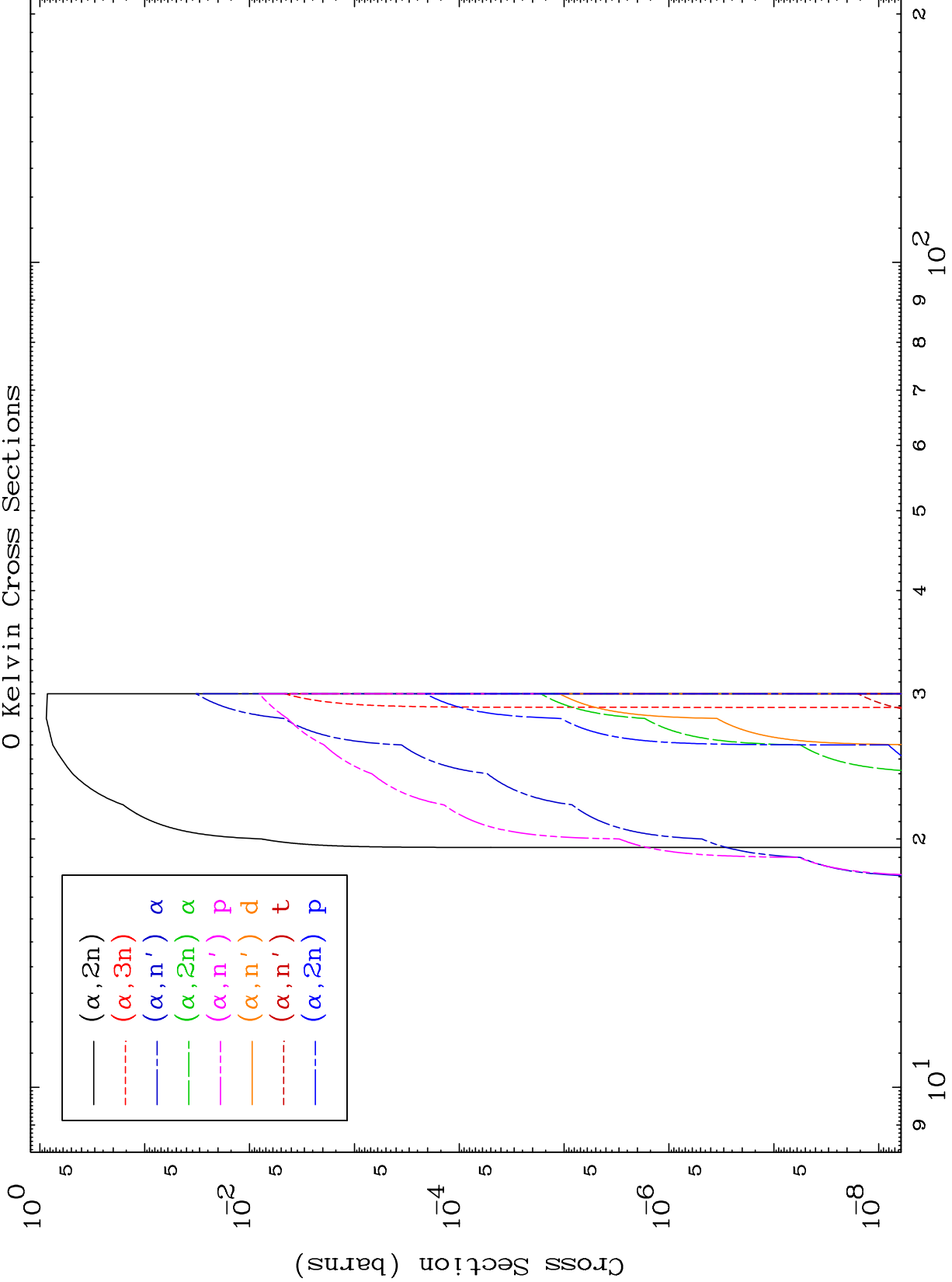
Incident Energy (MeV)

71-Lu-171

MAT 7114

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

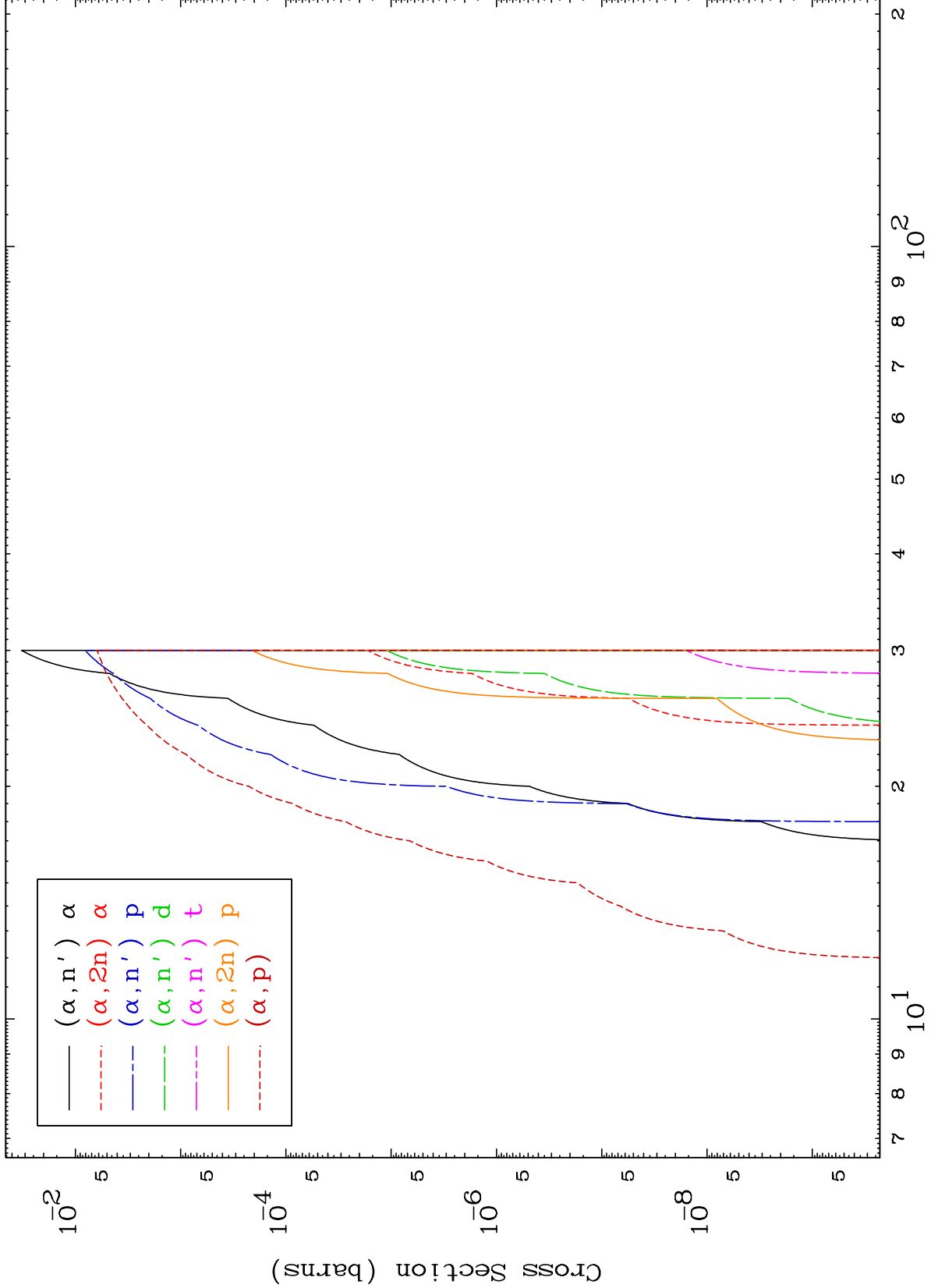
71-Lu-171

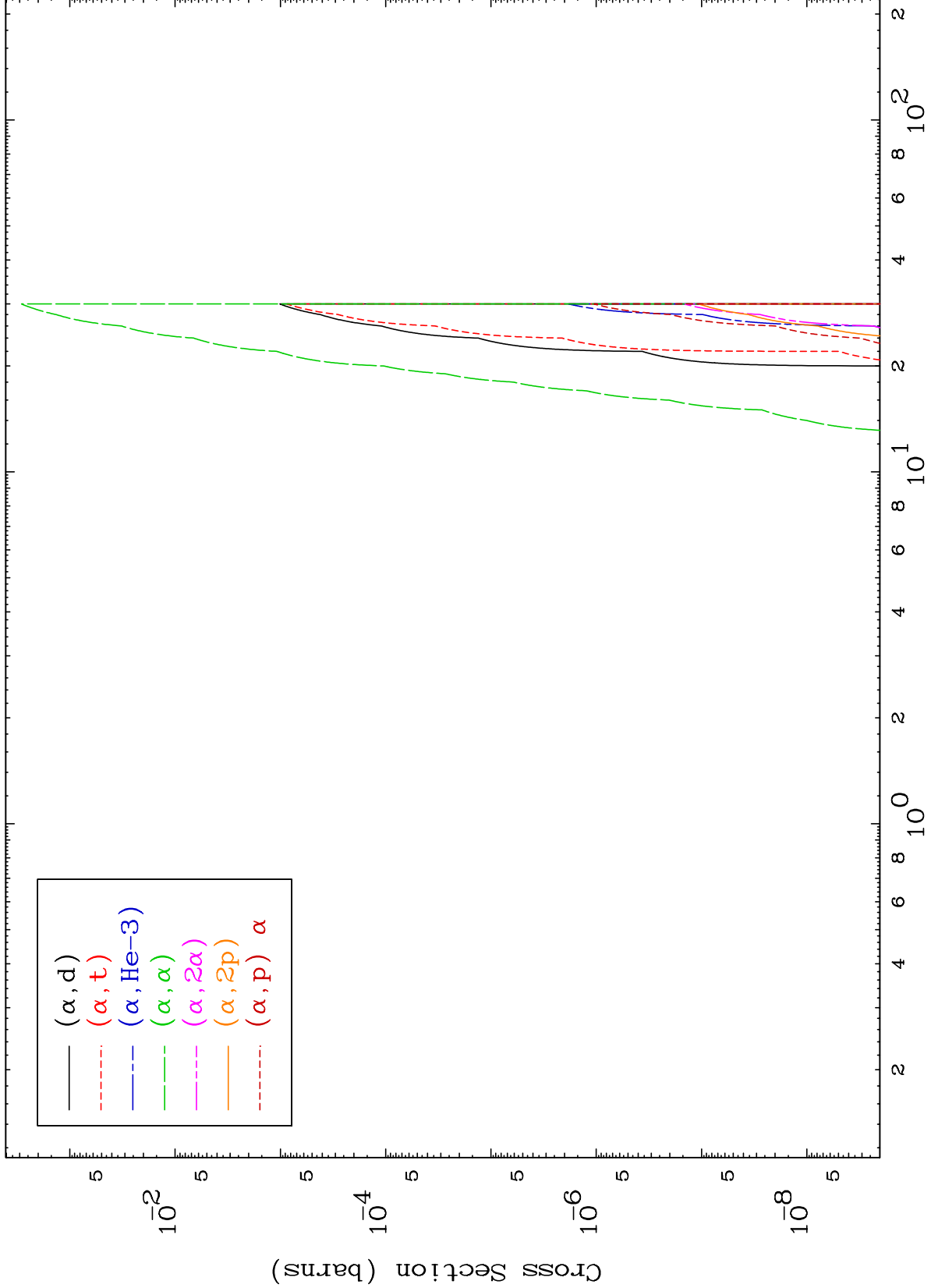


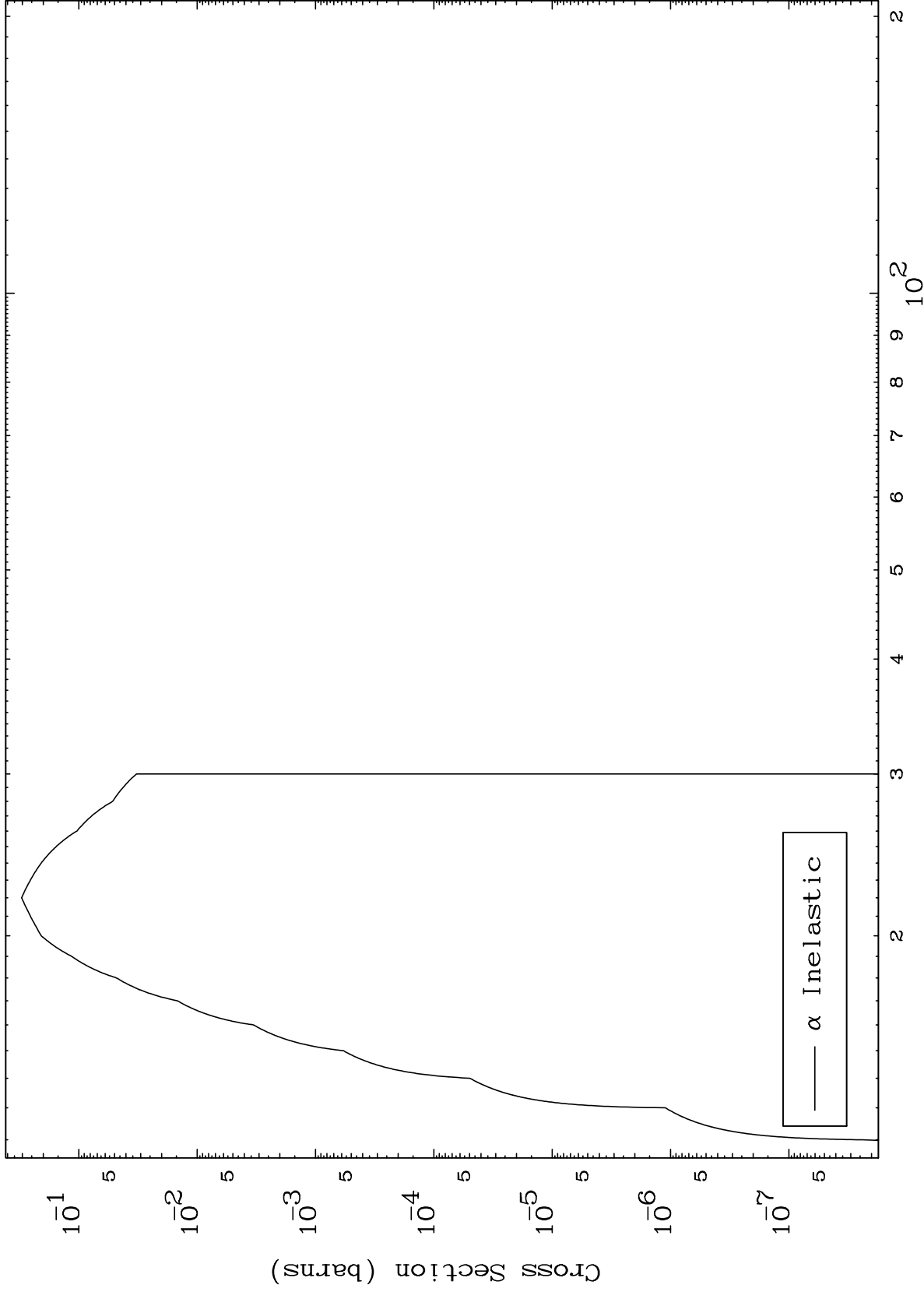
Incident Energy (MeV)

71-Lu-171

MAT 7114



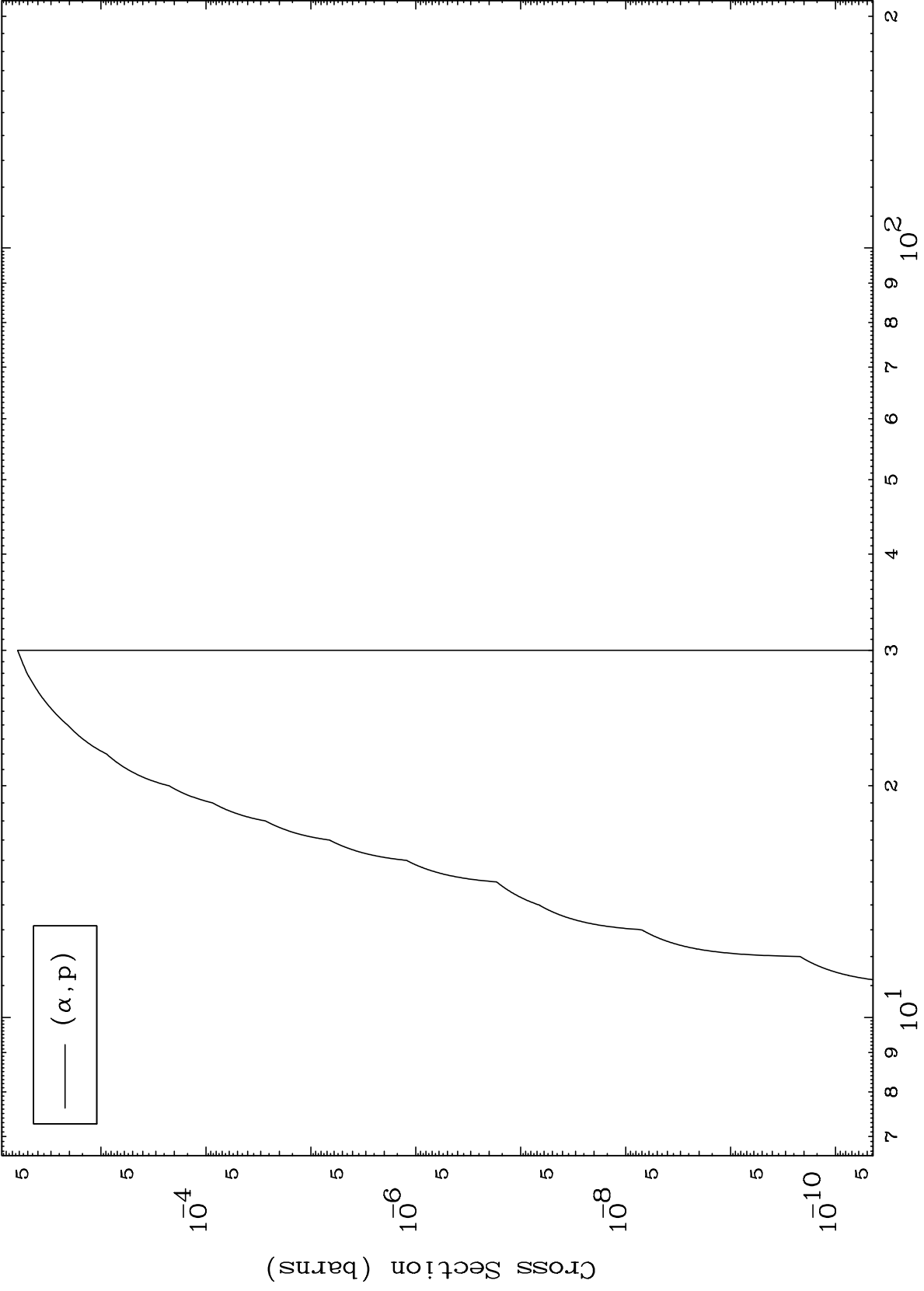




MAT 7114

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

71-Lu-171



6

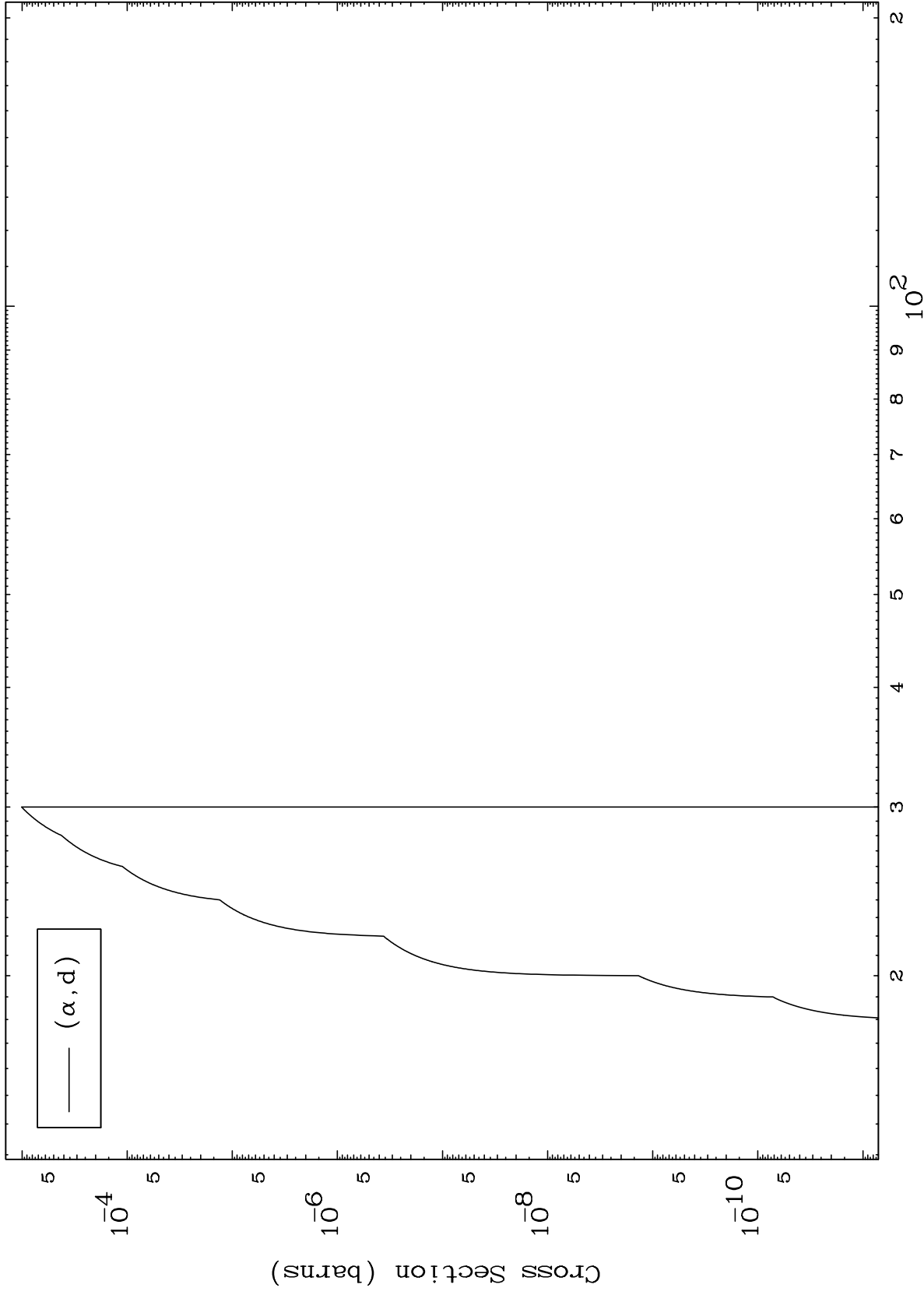
Incident Energy (MeV)

71-Lu-171

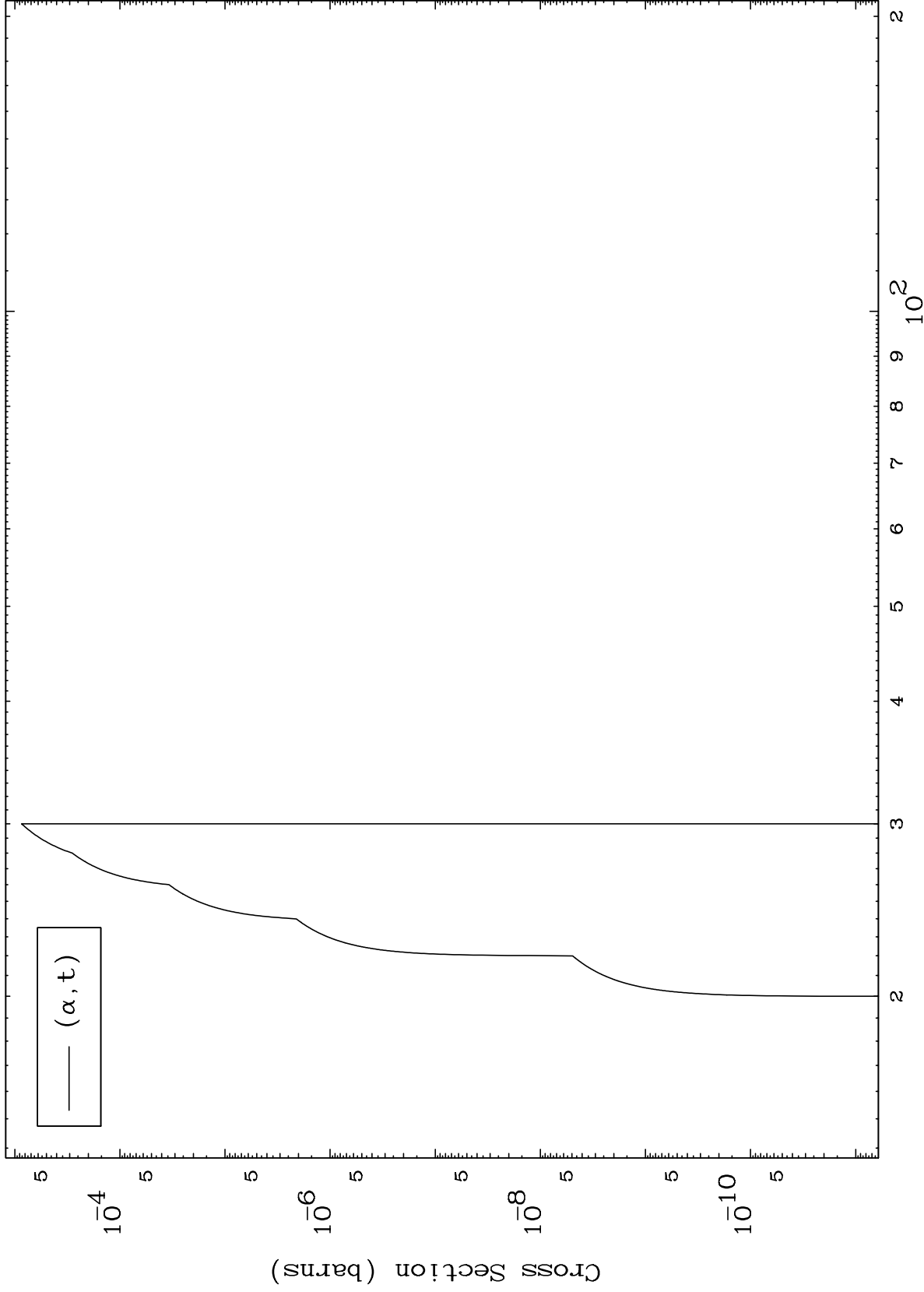
MAT 7114

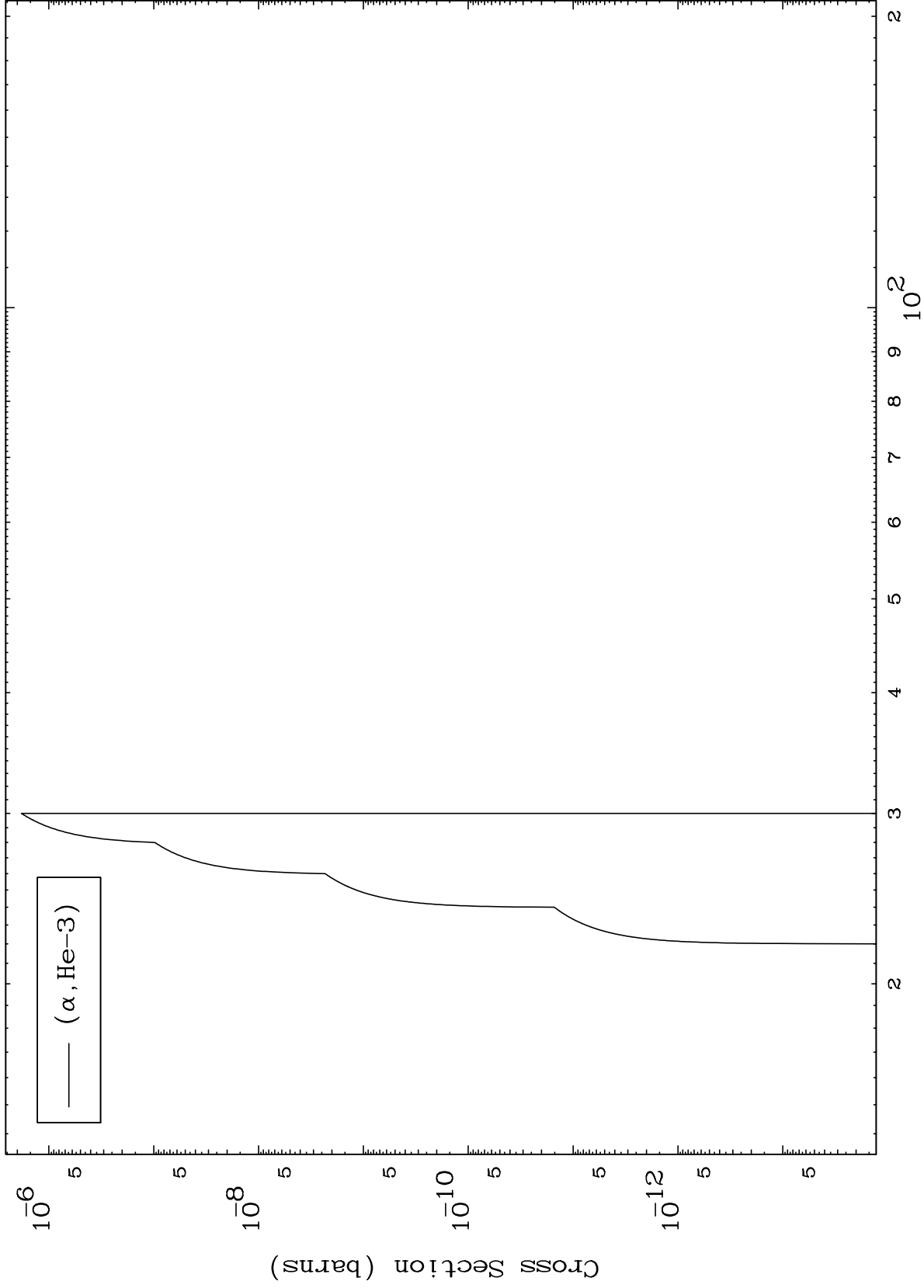
( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

71-Lu-171





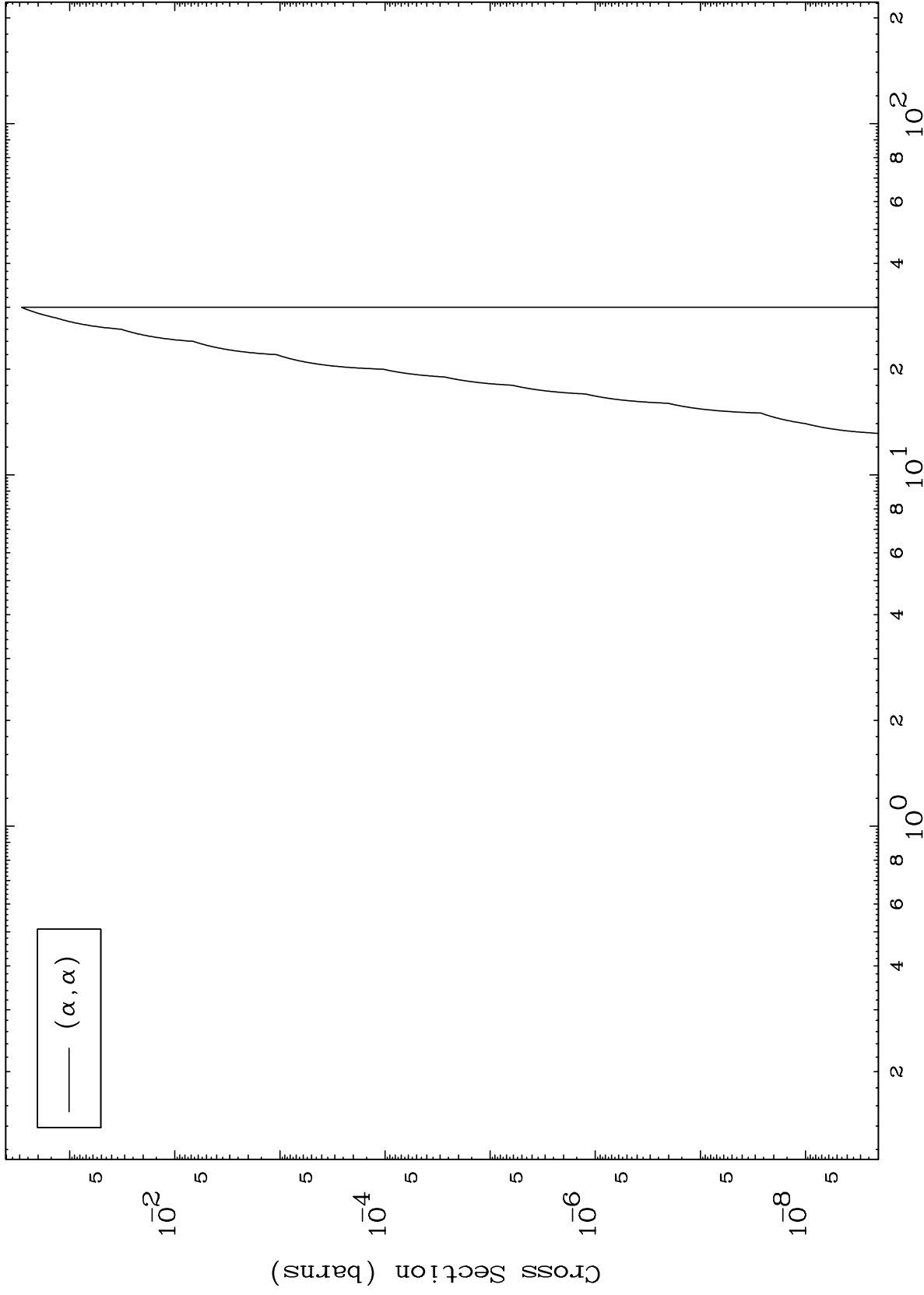




MAT 7114

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

71-Lu-171



10

Incident Energy (MeV)

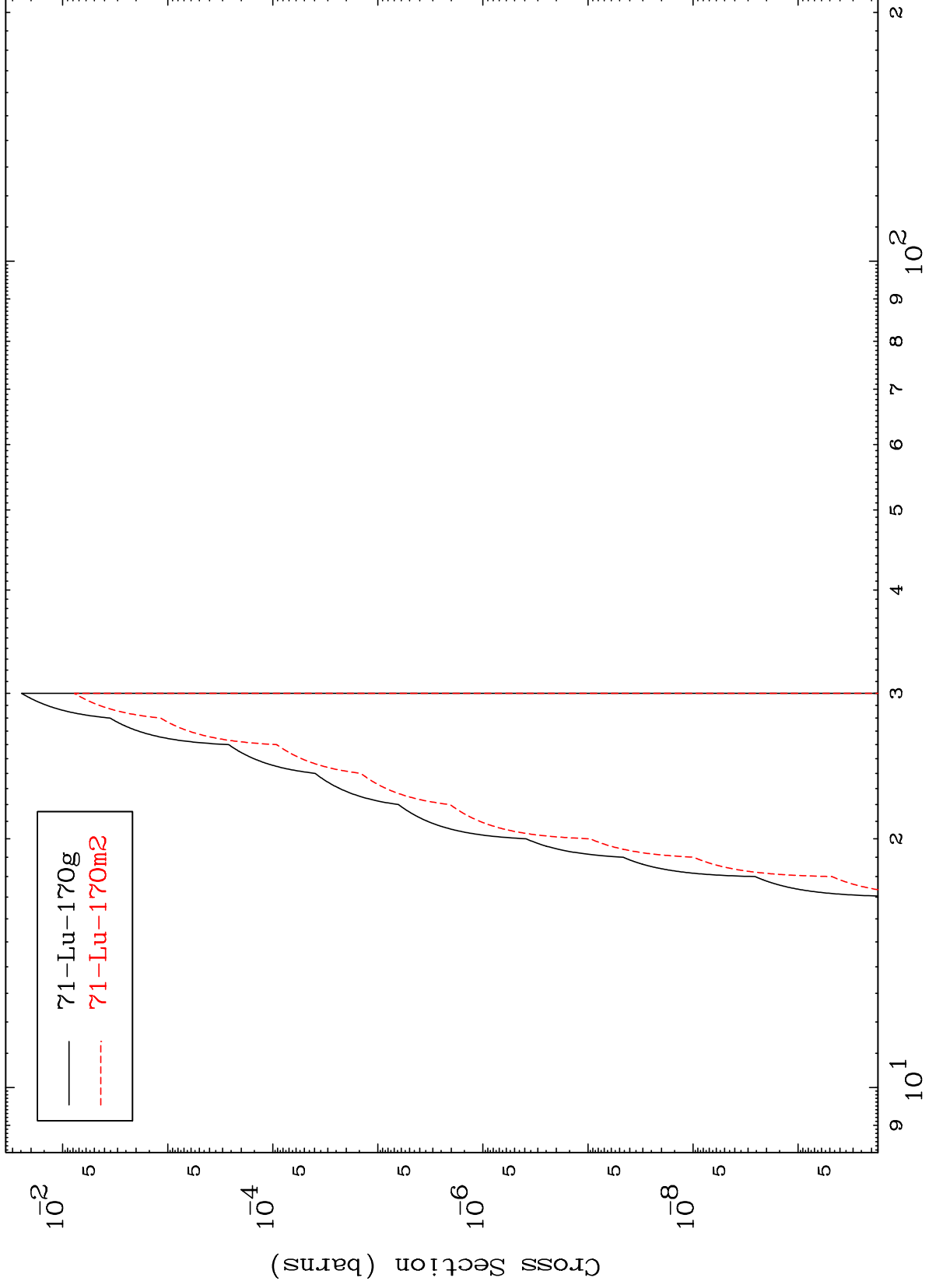
71-Lu-171

MAT 7114

( $\alpha, n'$ )  $\alpha$

71-Lu-171

Radionuclide Production Cross Section



11

Incident Energy (MeV)

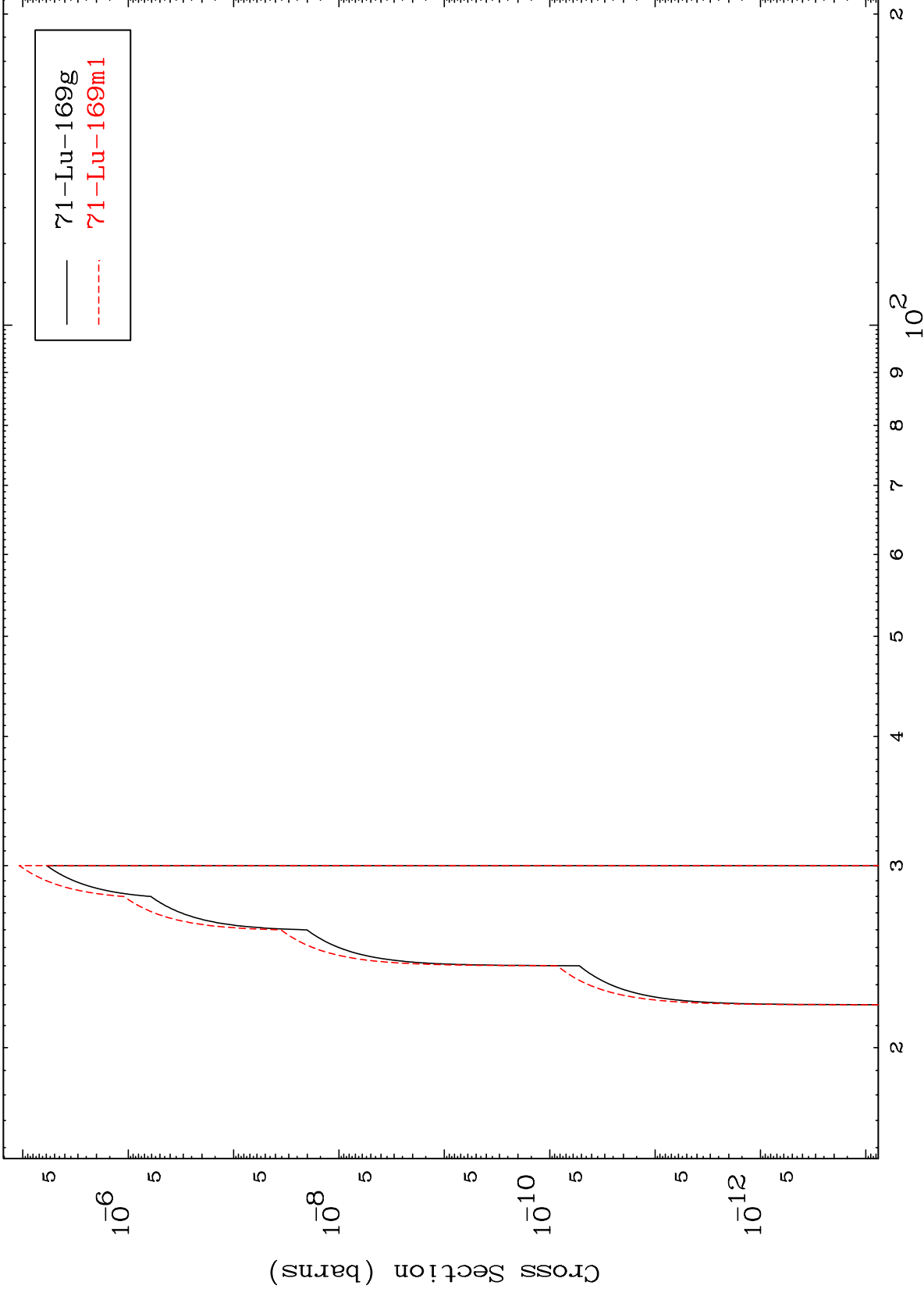
71-Lu-171

MAT 7114

$(\alpha, 2n) \alpha$

71-Lu-171

Radionuclide Production Cross Section



12

Incident Energy (MeV)

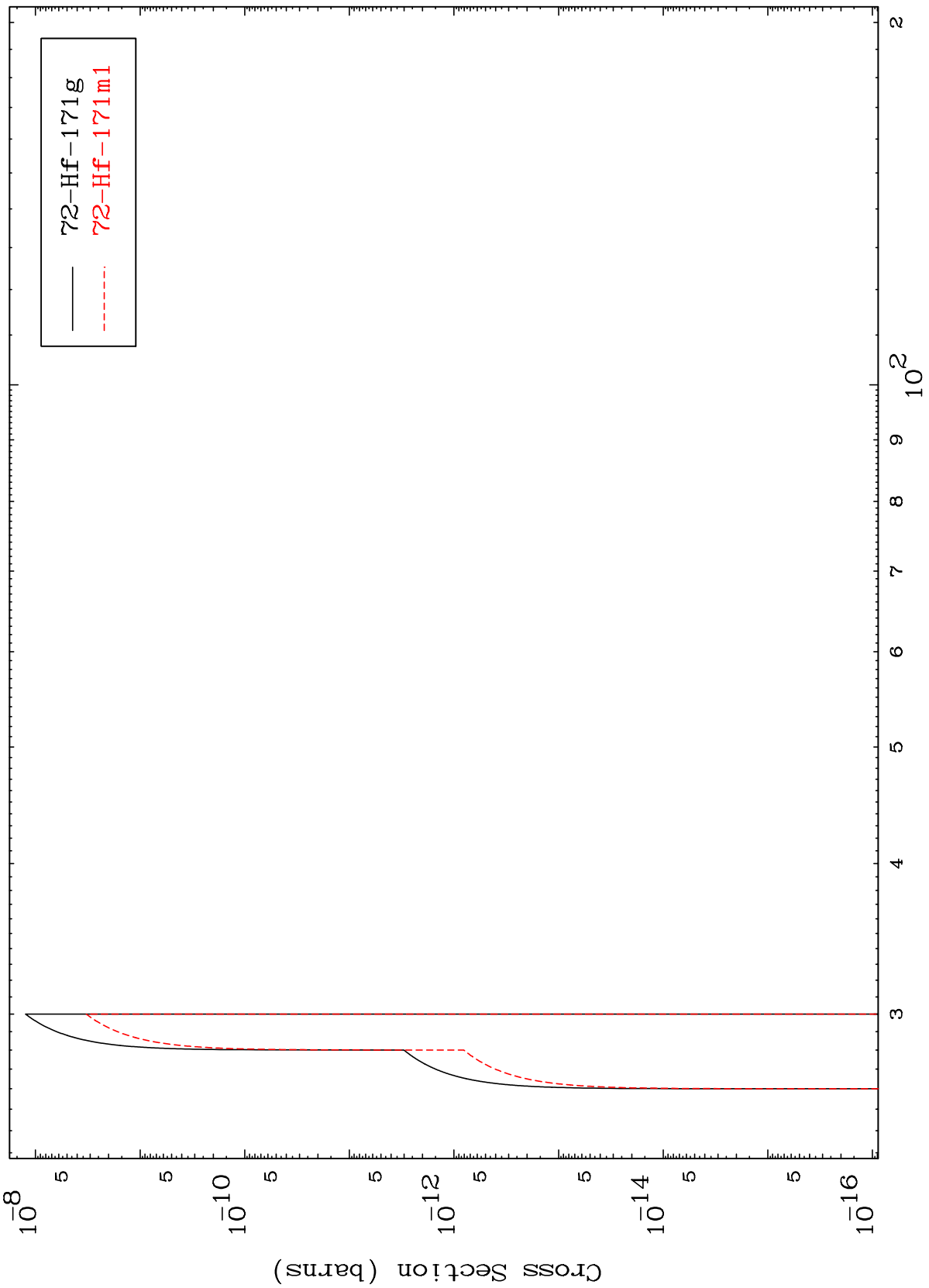
71-Lu-171

MAT 7114

$(\alpha, n')$  t

71-Lu-171

Radionuclide Production Cross Section



72-Hf-171g  
72-Hf-171m1

13

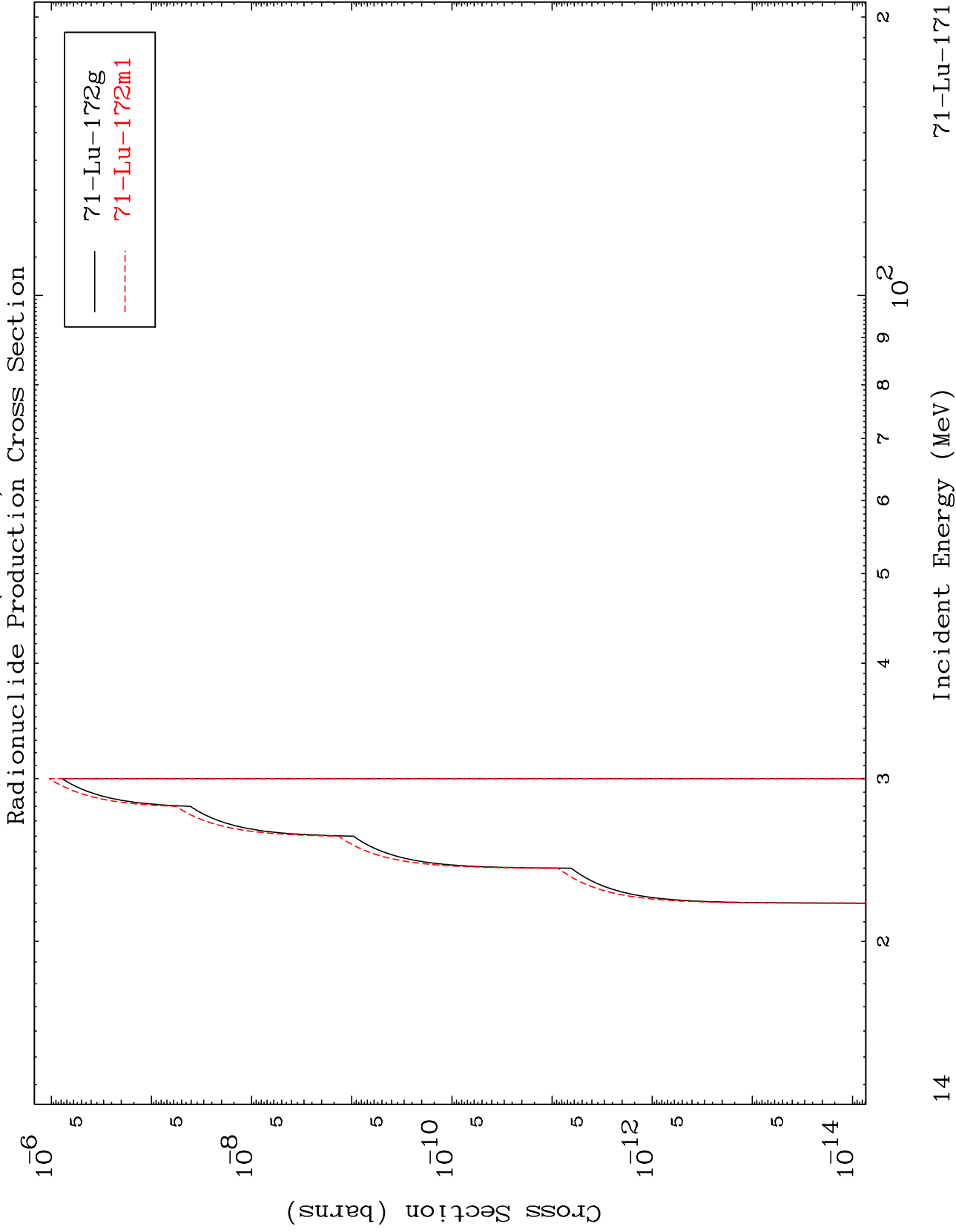
Incident Energy (MeV)

71-Lu-171

MAT 71114

( $\alpha, \text{He-3}$ )

71-Lu-171



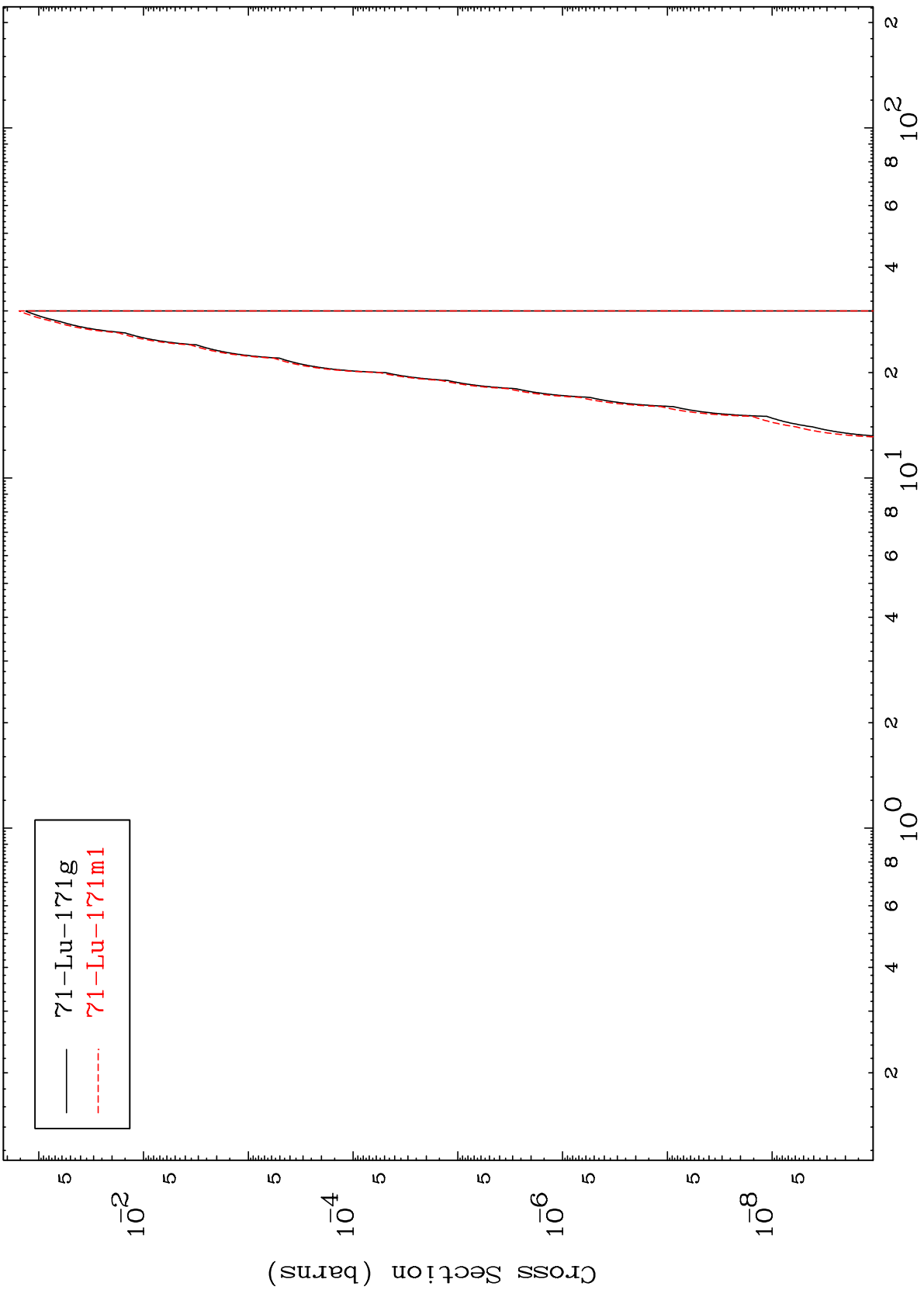
14

MAT 7114

( $\alpha, \alpha$ )

<sup>71</sup>Lu-171

Radionuclide Production Cross Section



15

Incident Energy (MeV)

<sup>71</sup>Lu-171