

Program EVALPLOT  
(Version 2018-1)

by

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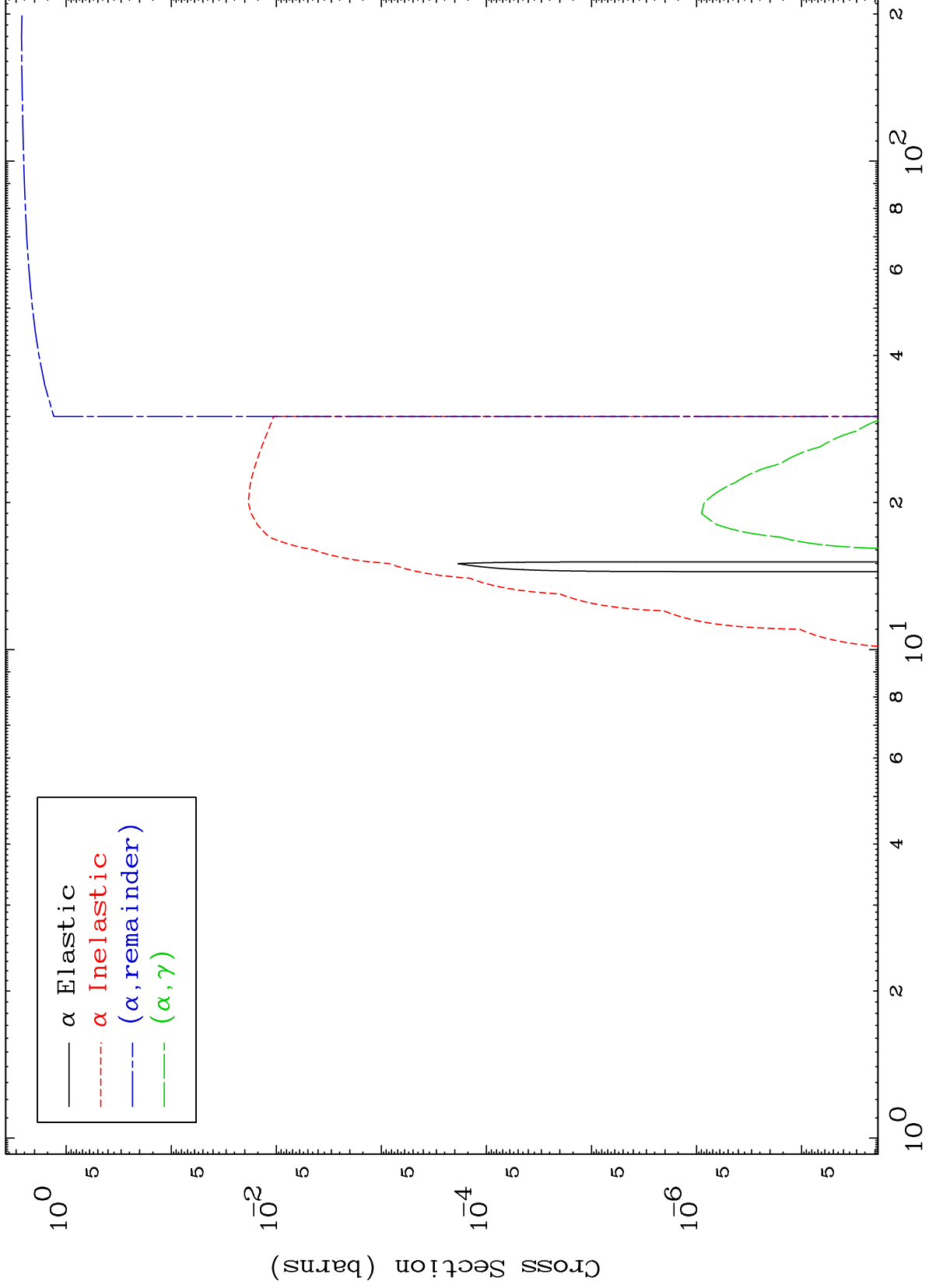
Press Mouse Button to Start

MAT 7143

$\alpha$  Major

71-Lu-181

0 Kelvin Cross Sections



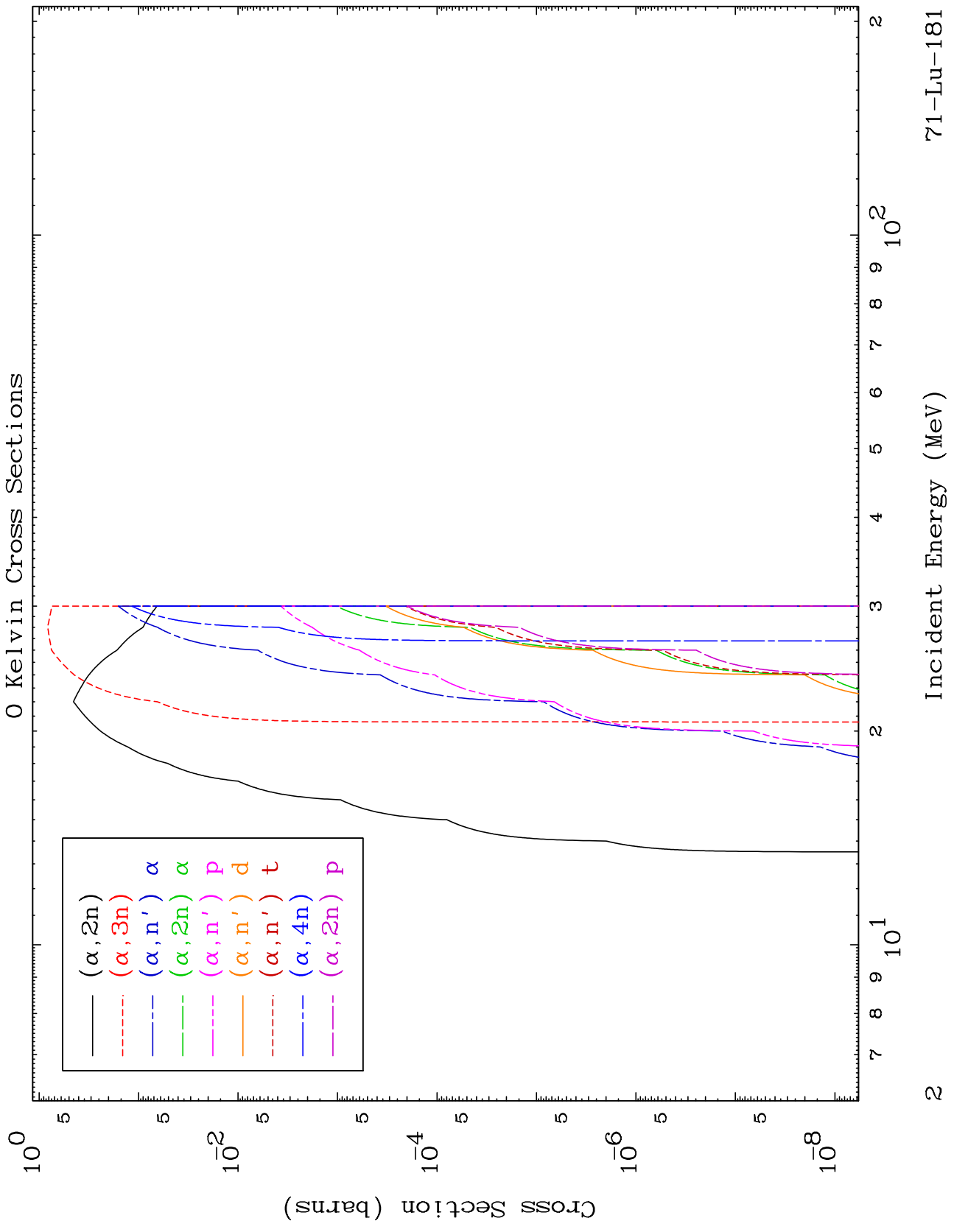
Incident Energy (MeV)

71-Lu-181

MAT 7143

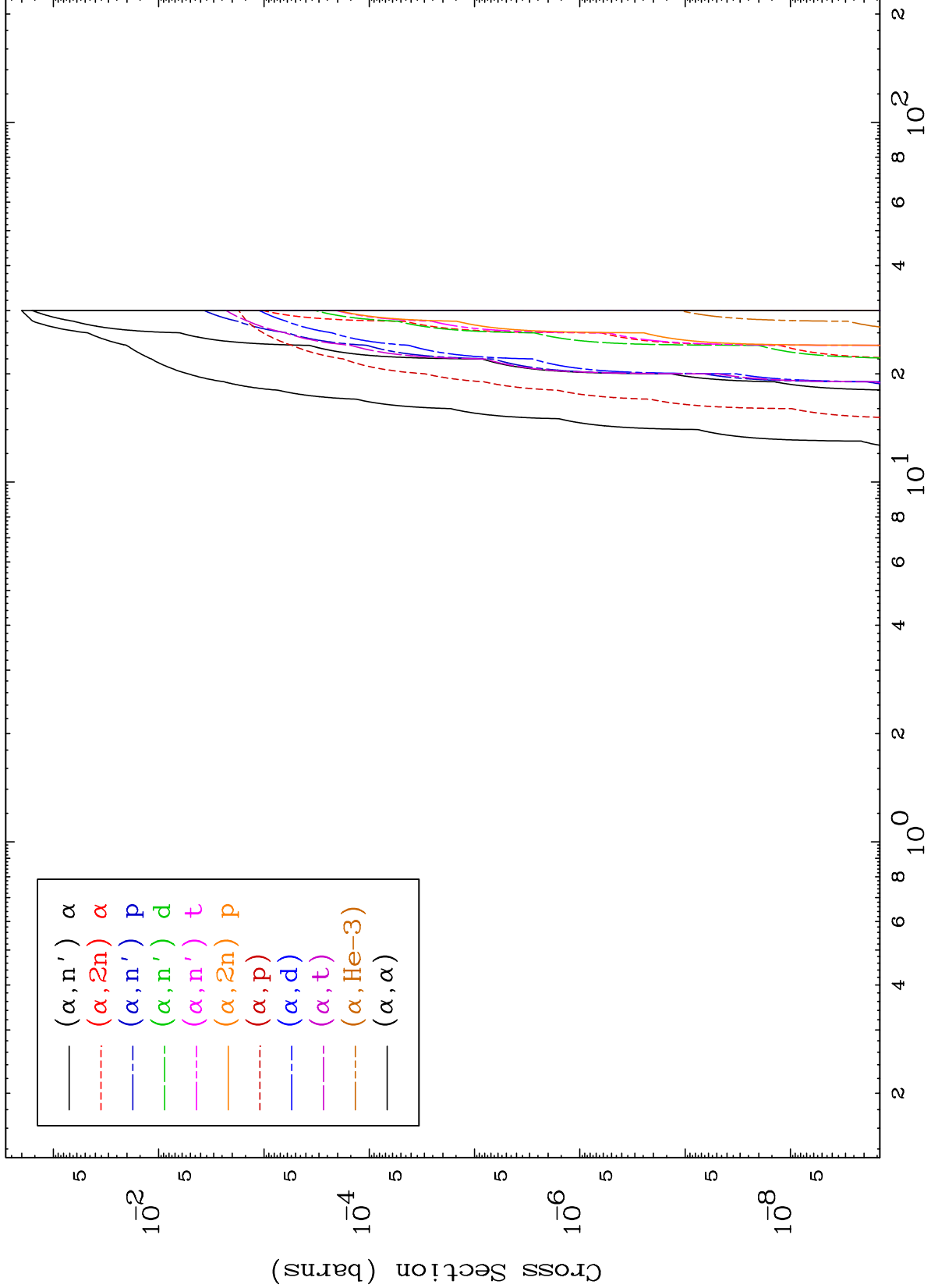
$\alpha$  Neutron Production  
0 Kelvin Cross Sections

71-Lu-181



Incident Energy (MeV)

71-Lu-181

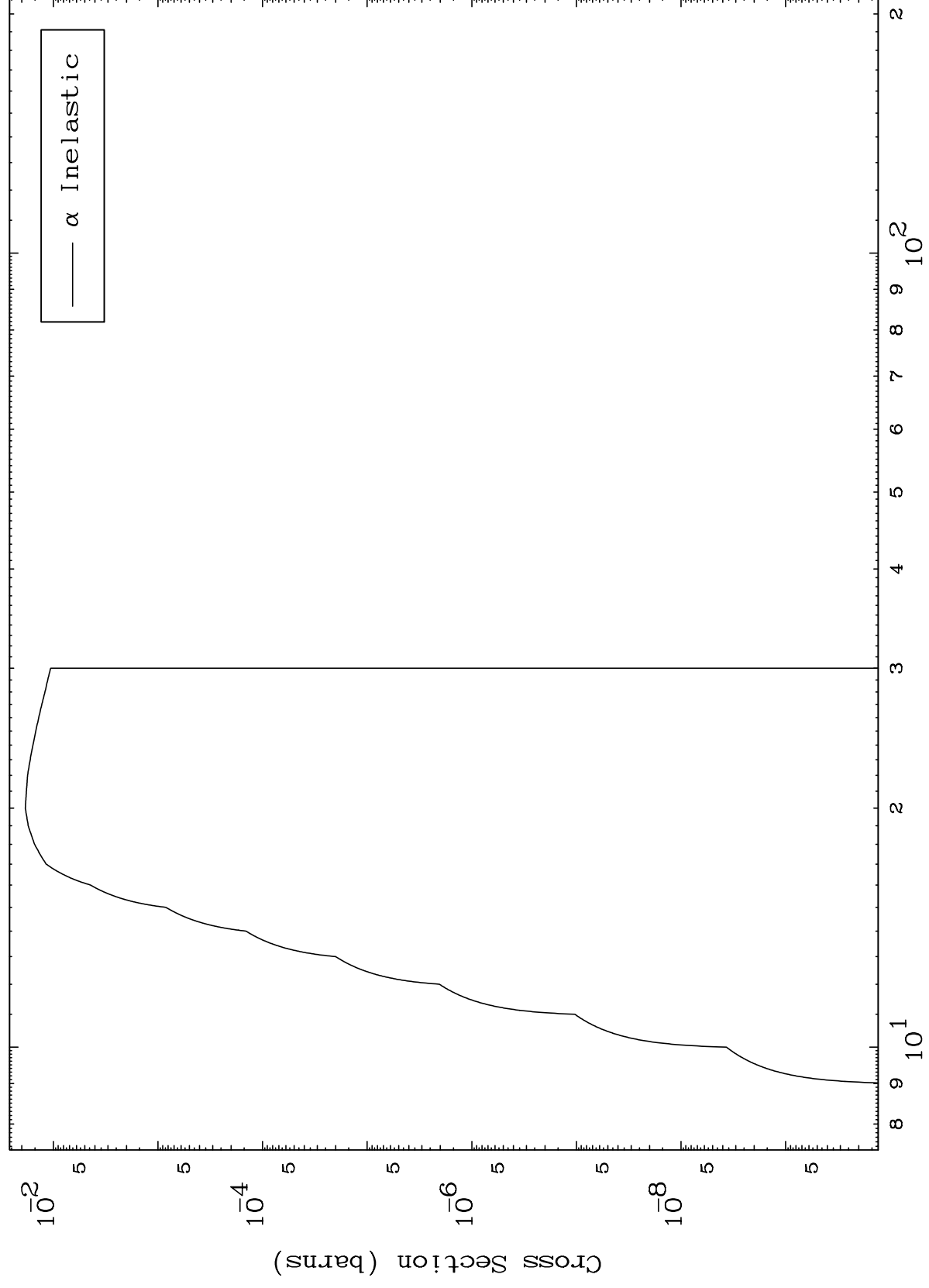


MAT 7143

( $\alpha, n'$ ) Level

71-Lu-181

0 Kelvin Cross Sections



4

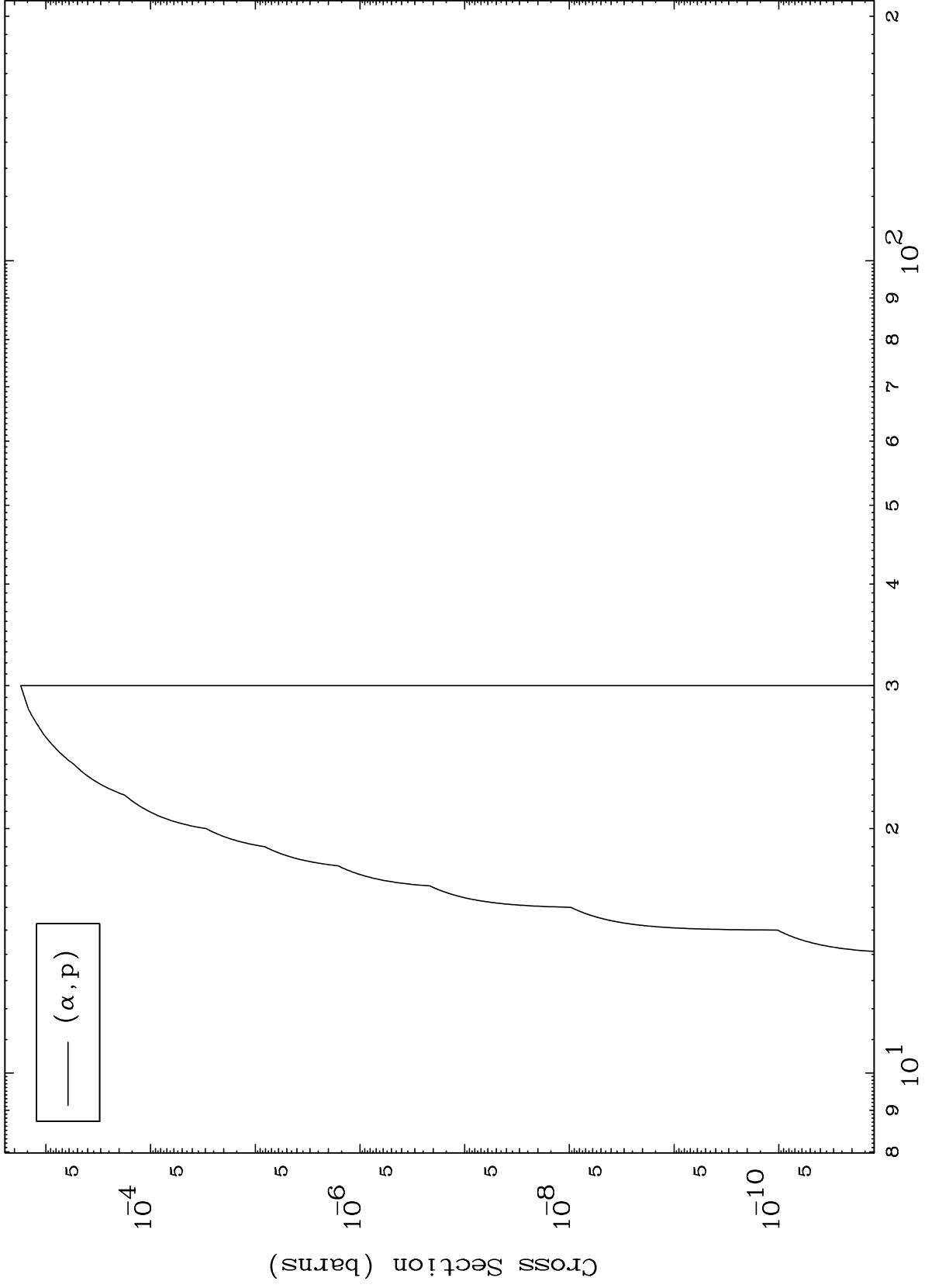
Incident Energy (MeV)

71-Lu-181

MAT 7143

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

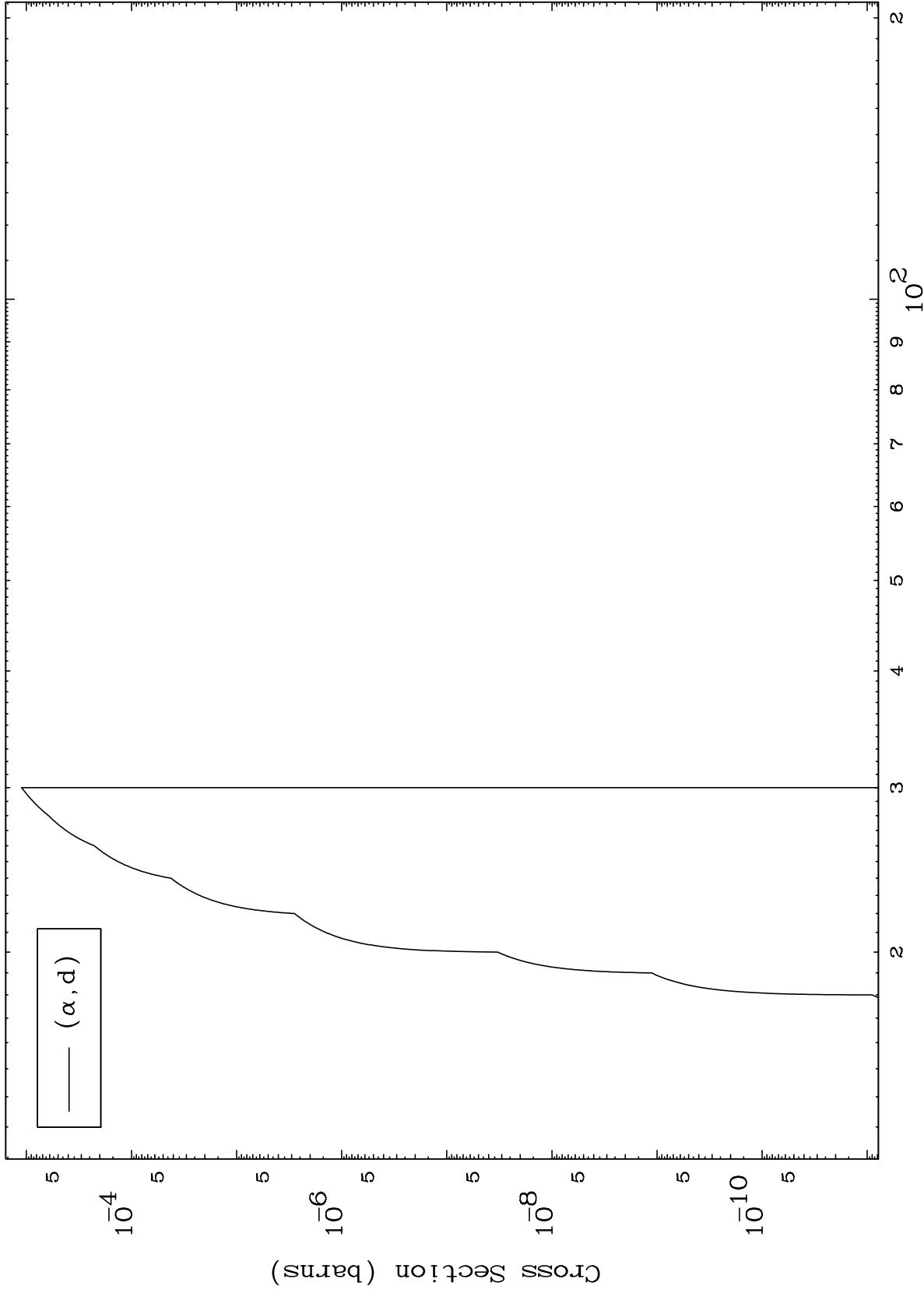
71-Lu-181

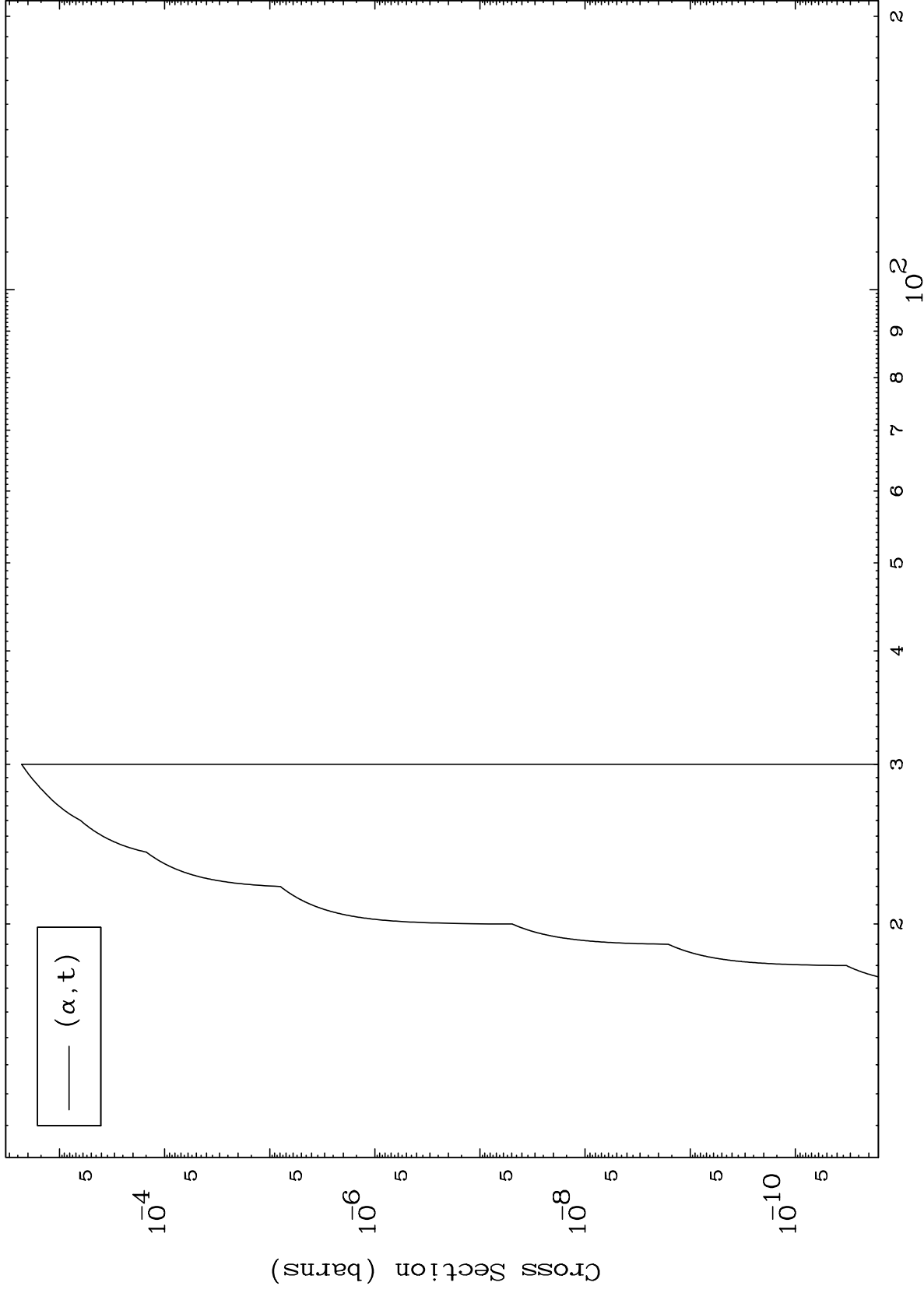


5

Incident Energy (MeV)

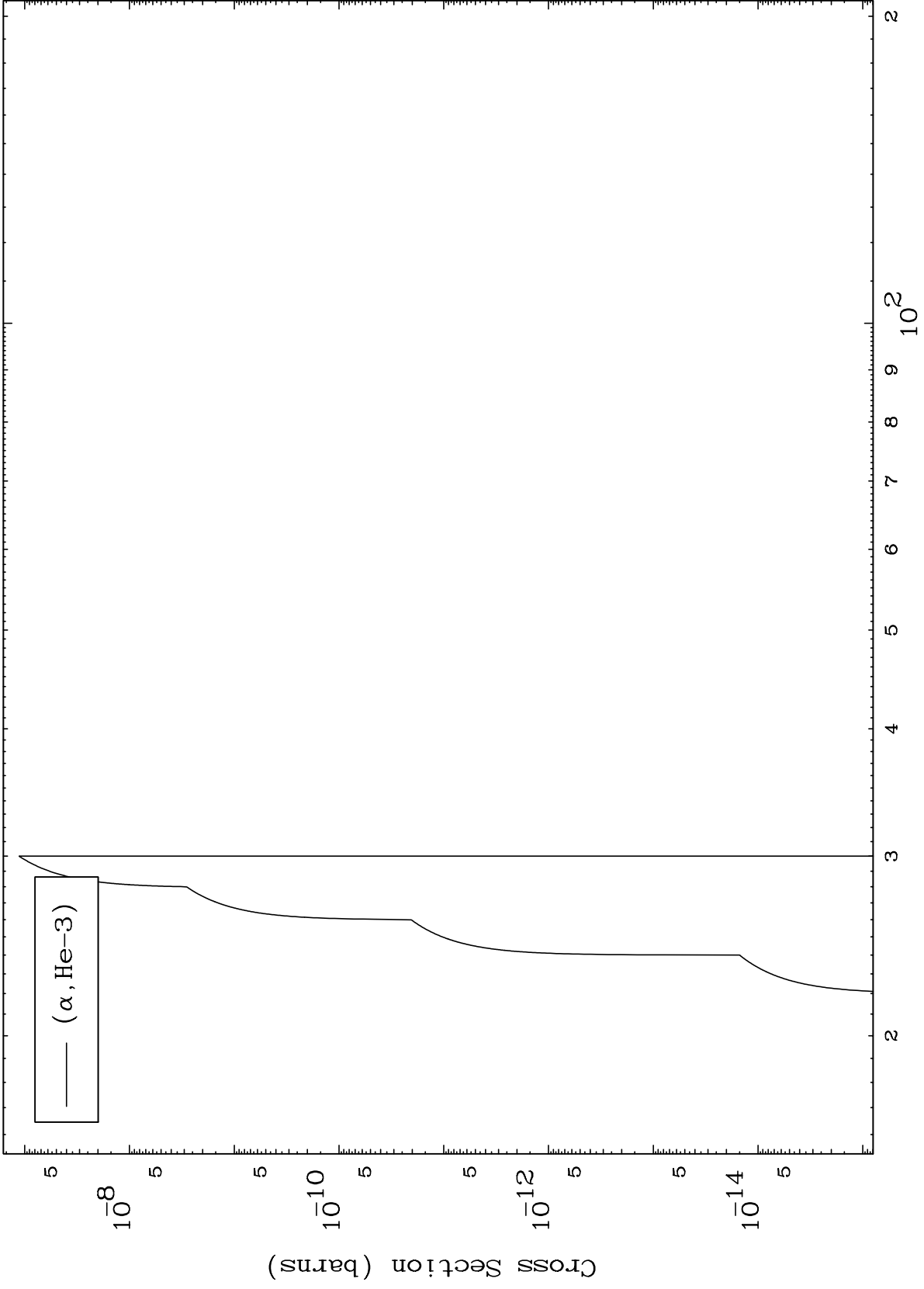
71-Lu-181







( $\alpha, \text{He}3$ ) Levels  
0 Kelvin Cross Sections

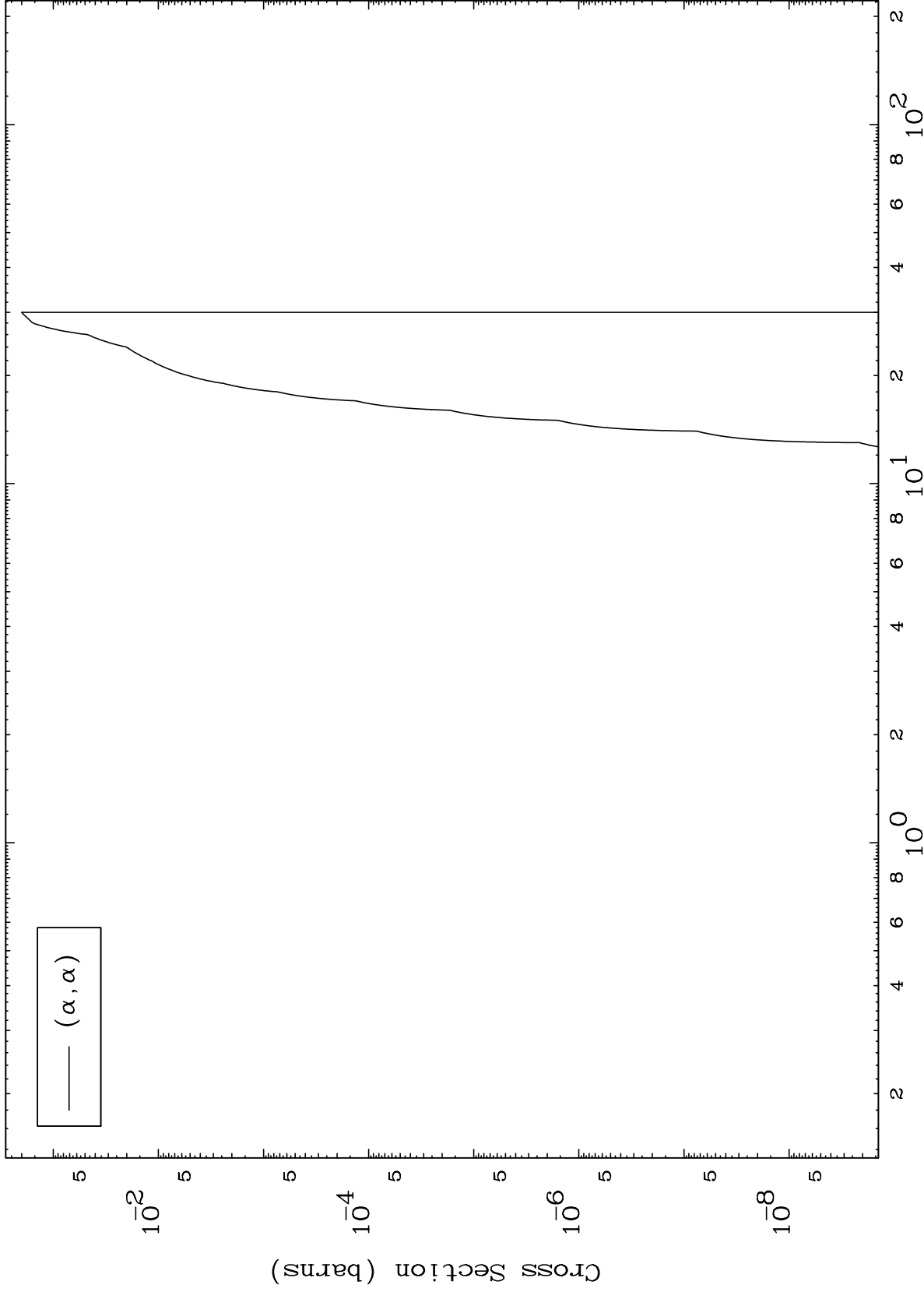


MAT 7143

( $\alpha, \alpha$ ) Levels

71-Lu-181

0 Kelvin Cross Sections



9

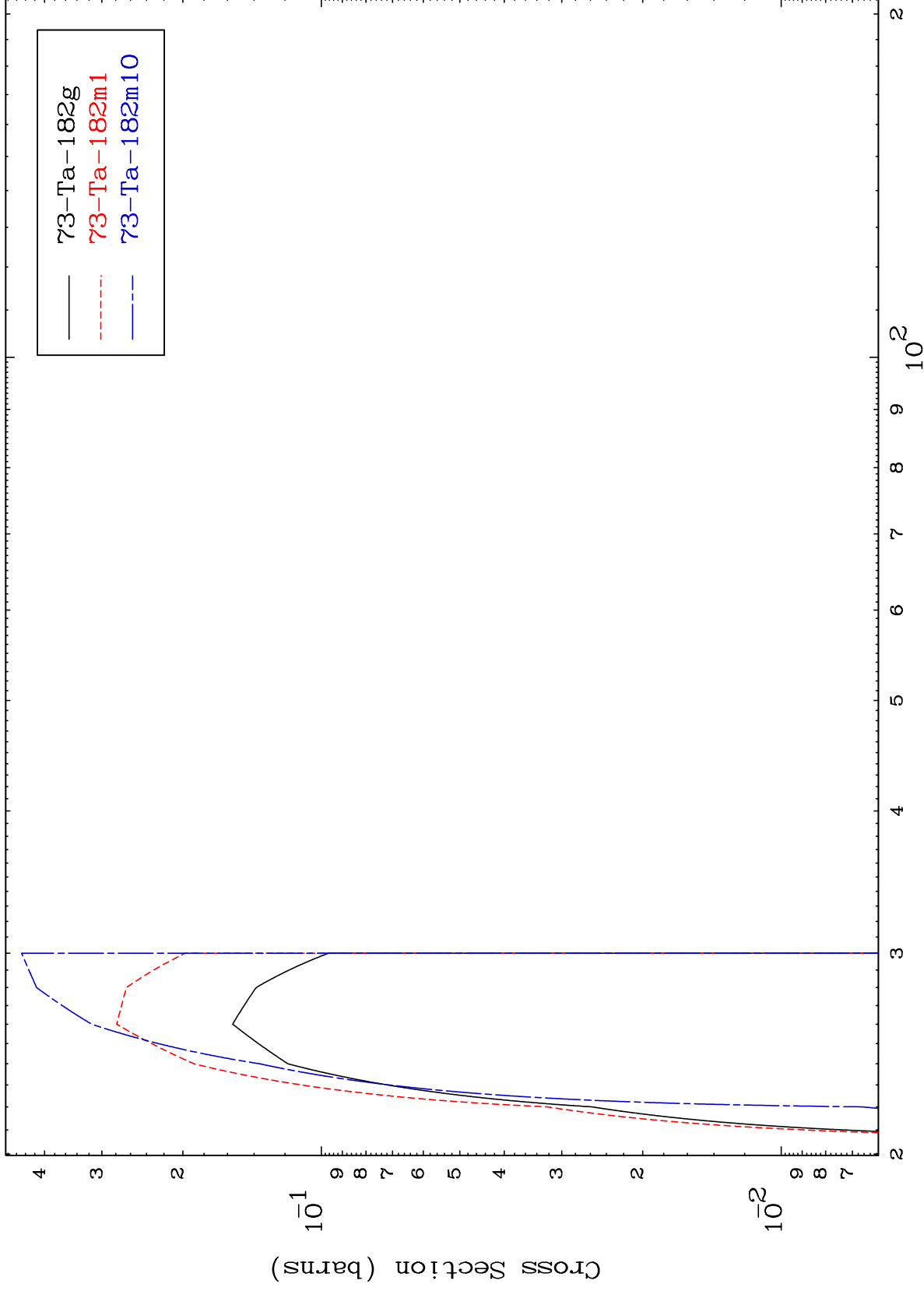
Incident Energy (MeV)

71-Lu-181

MAT 7143

71-Lu-181

( $\alpha, 3n$ )  
Radionuclide Production Cross Section



71-Lu-181

Incident Energy (MeV)

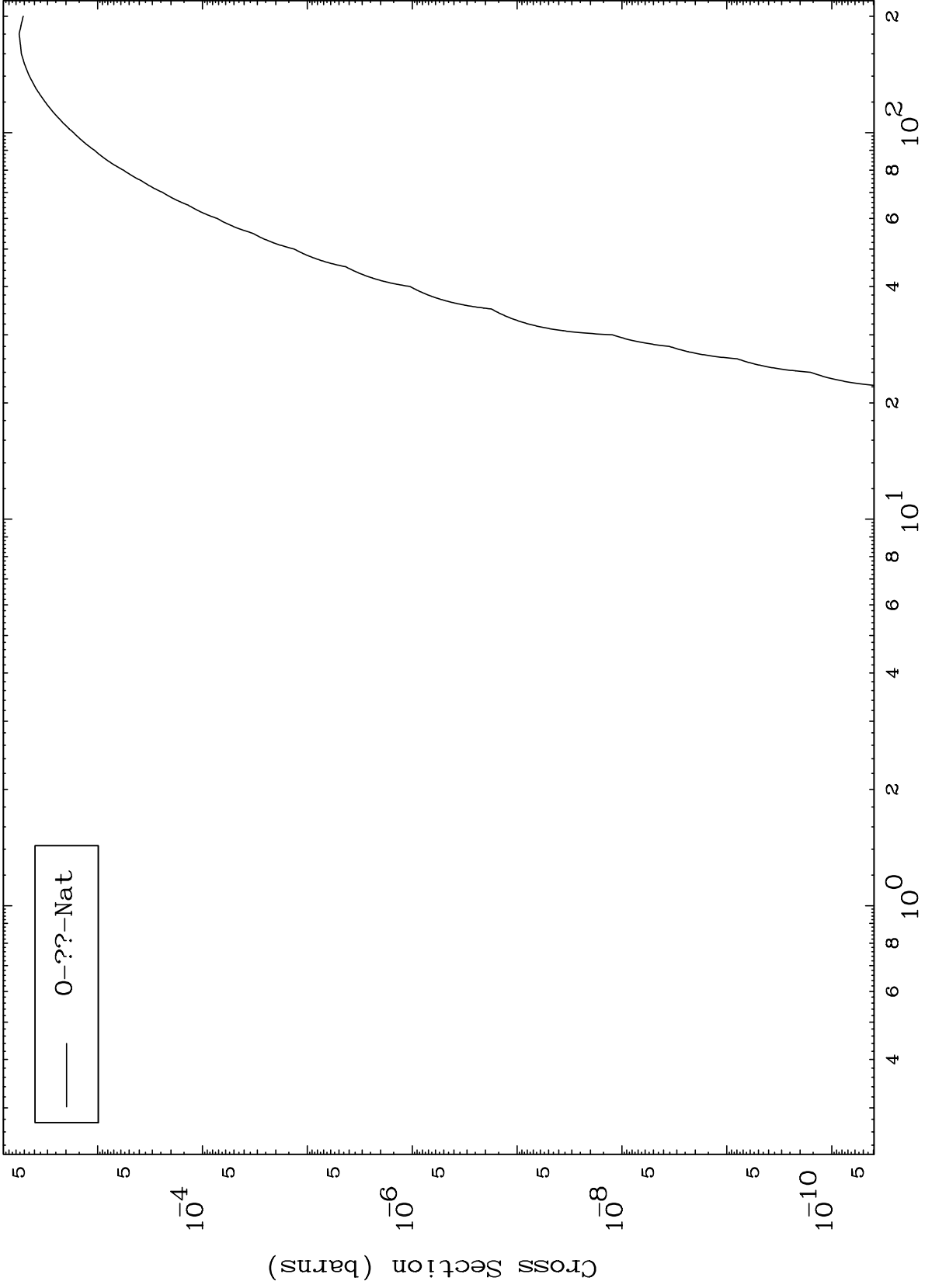
10

MAT 7143

$\alpha$  Fission

71-Lu-181

Radionuclide Production Cross Section

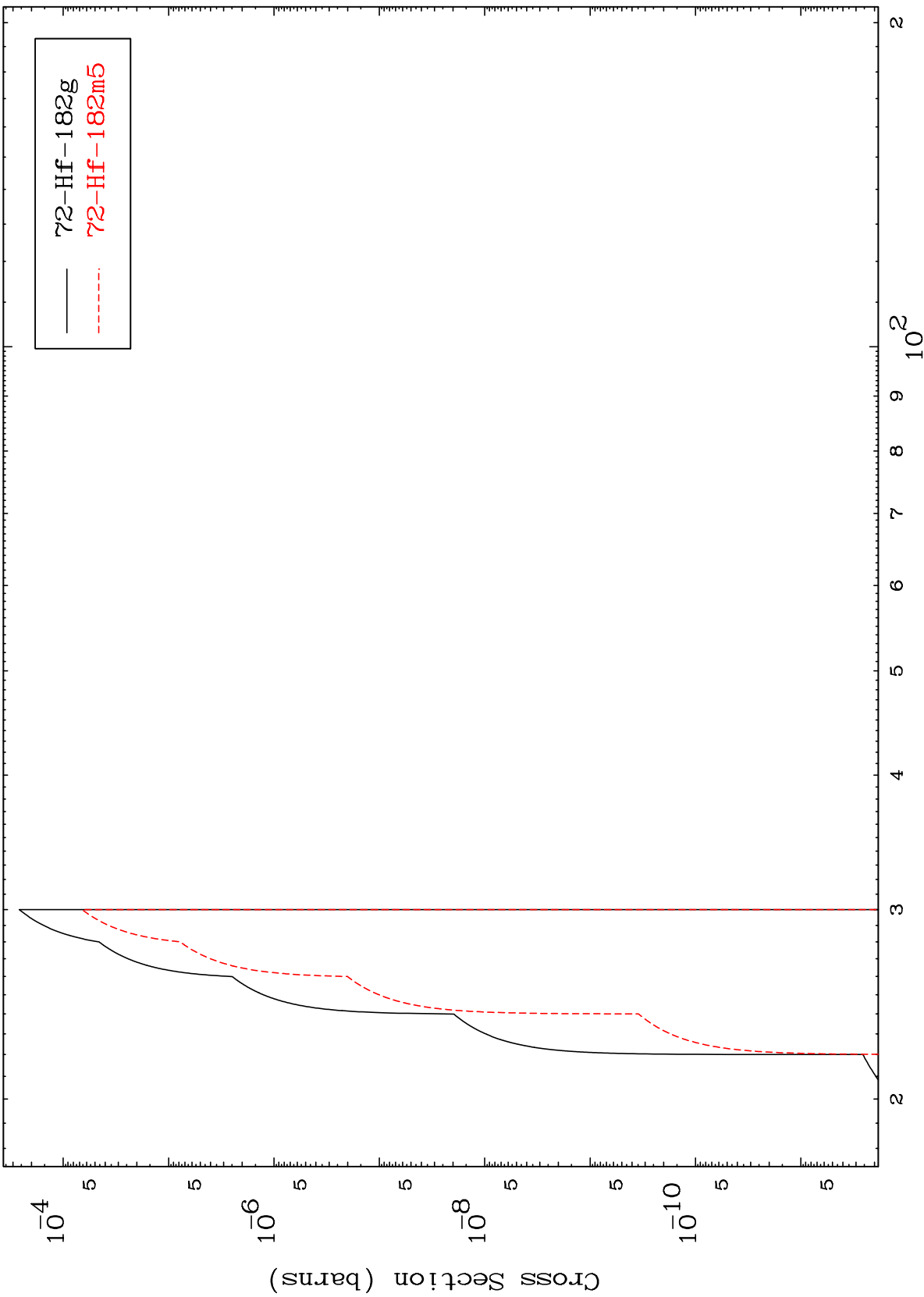


MAT 7143

( $\alpha, n'$ ) d

71-Lu-181

Radionuclide Production Cross Section



12

Incident Energy (MeV)

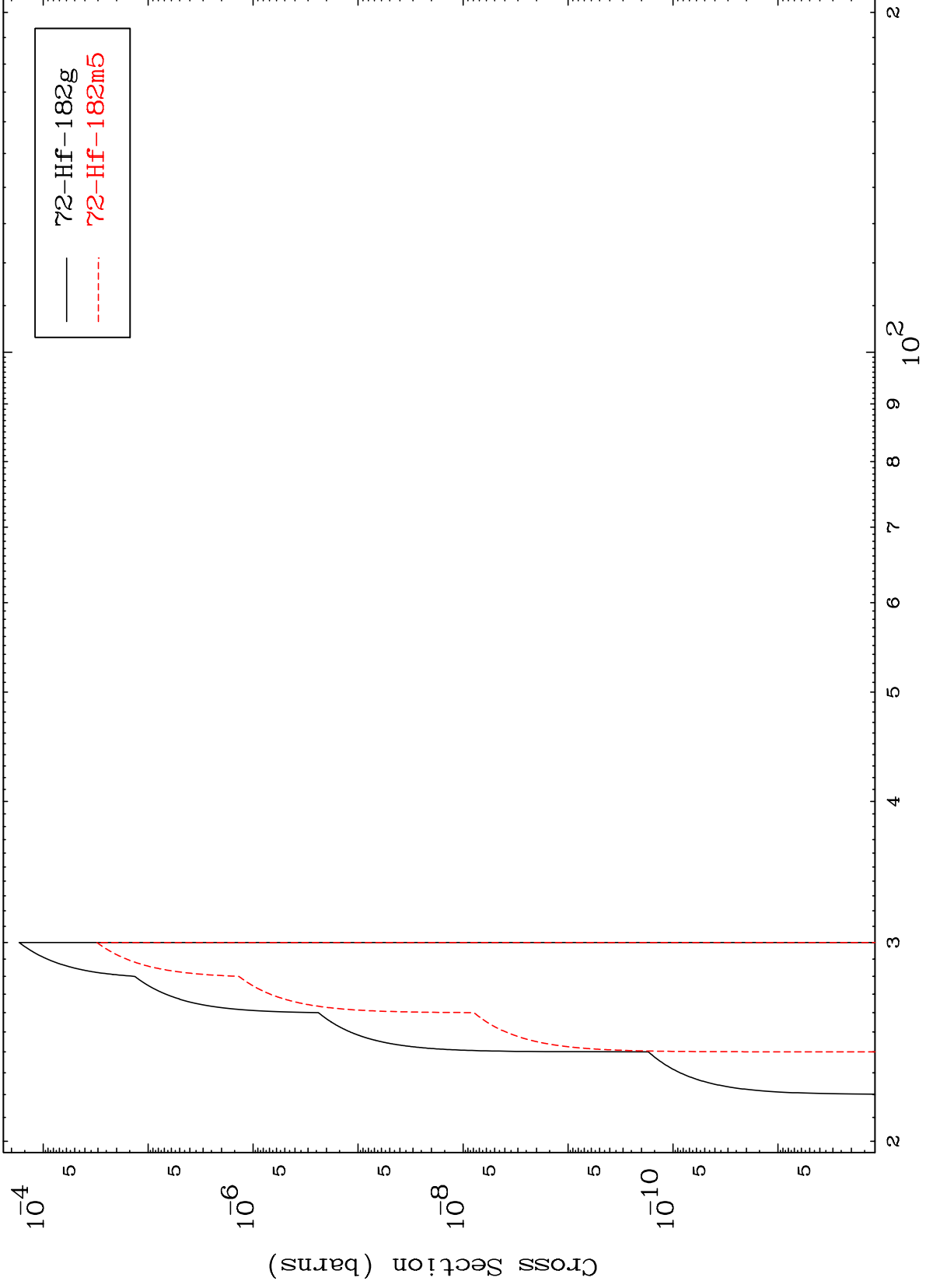
71-Lu-181

MAT 7143

$(\alpha, 2n)$  p

71-Lu-181

Radionuclide Production Cross Section



13

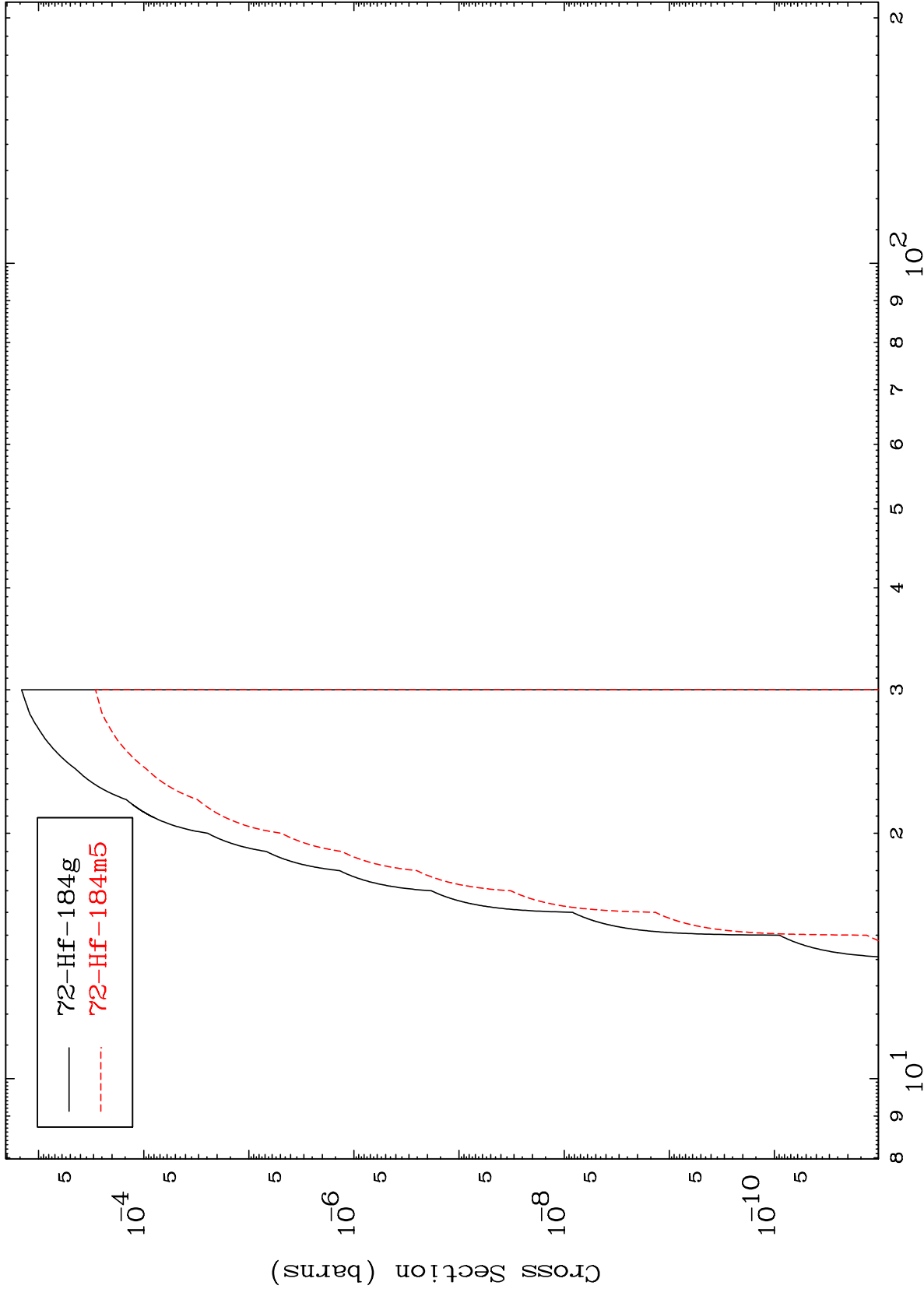
Incident Energy (MeV)

71-Lu-181

MAT 7143

71-Lu-181

( $\alpha, p$ )  
Radionuclide Production Cross Section



14

Incident Energy (MeV)

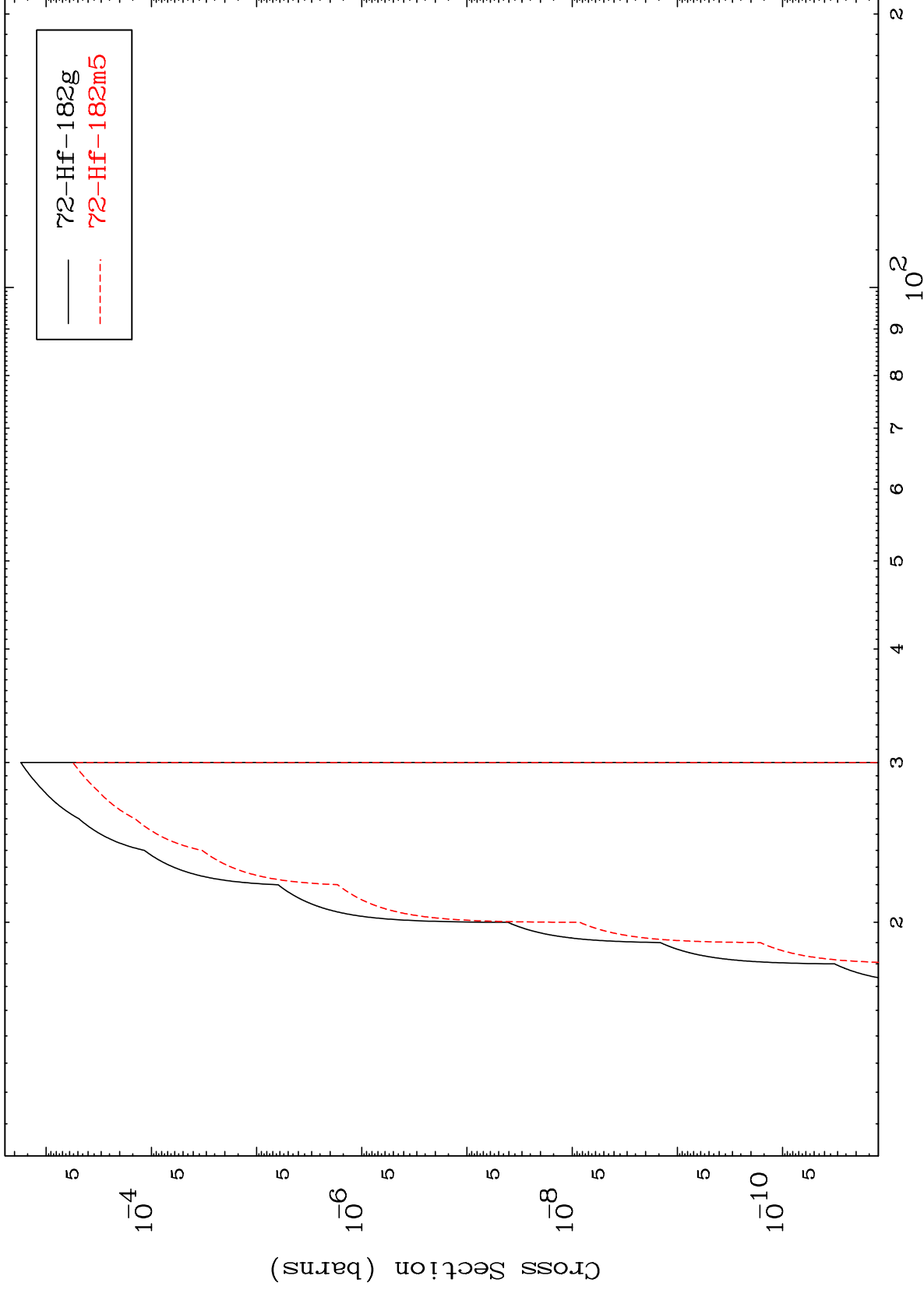
71-Lu-181

MAT 7143

( $\alpha, t$ )

71-Lu-181

Radionuclide Production Cross Section



15

Incident Energy (MeV)

71-Lu-181