

Program EVALPLOT
(Version 2018-1)

by

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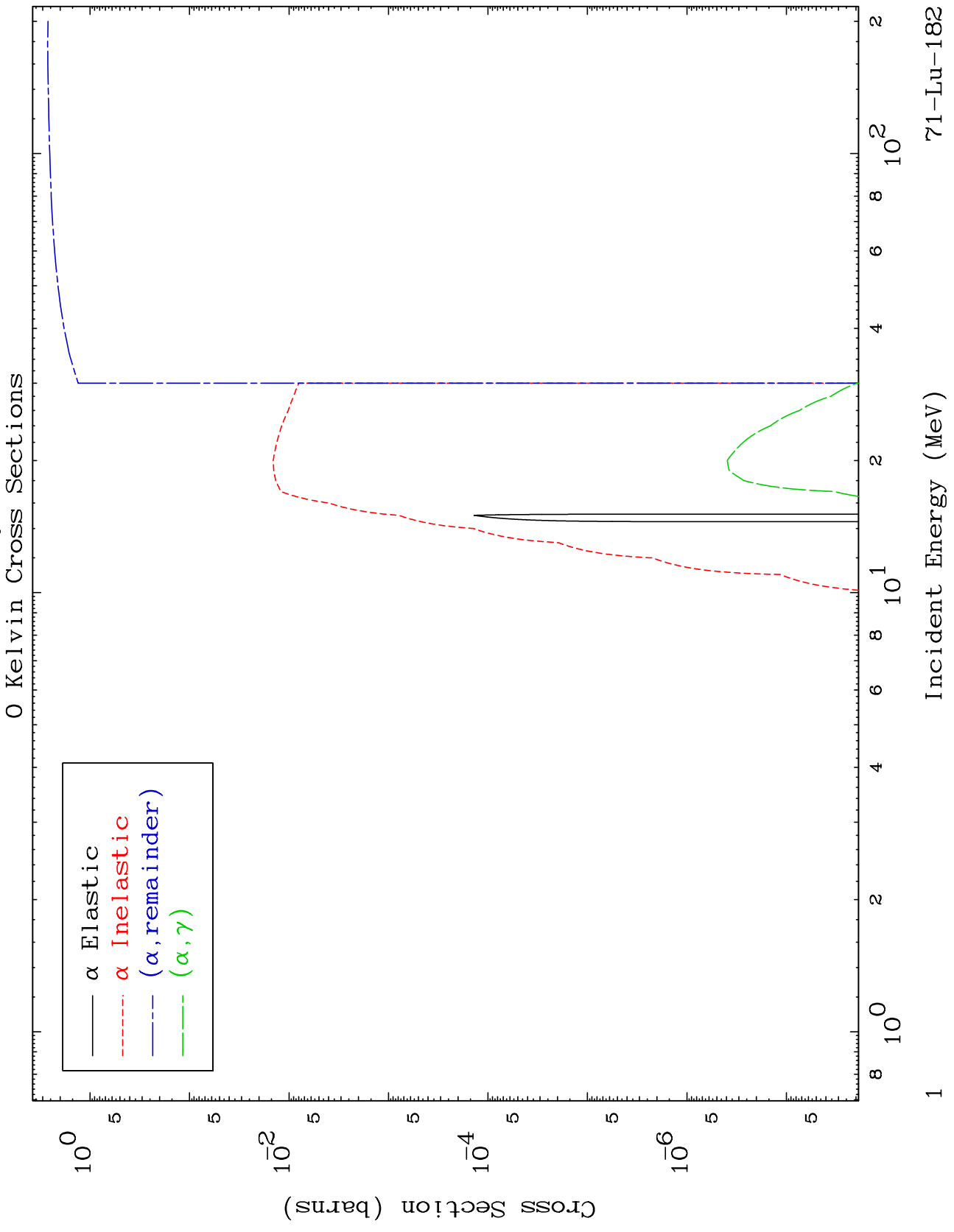
Press Mouse Button to Start

MAT 7146

0 Kelvin

Major

71-Lu-182



71-Lu-182

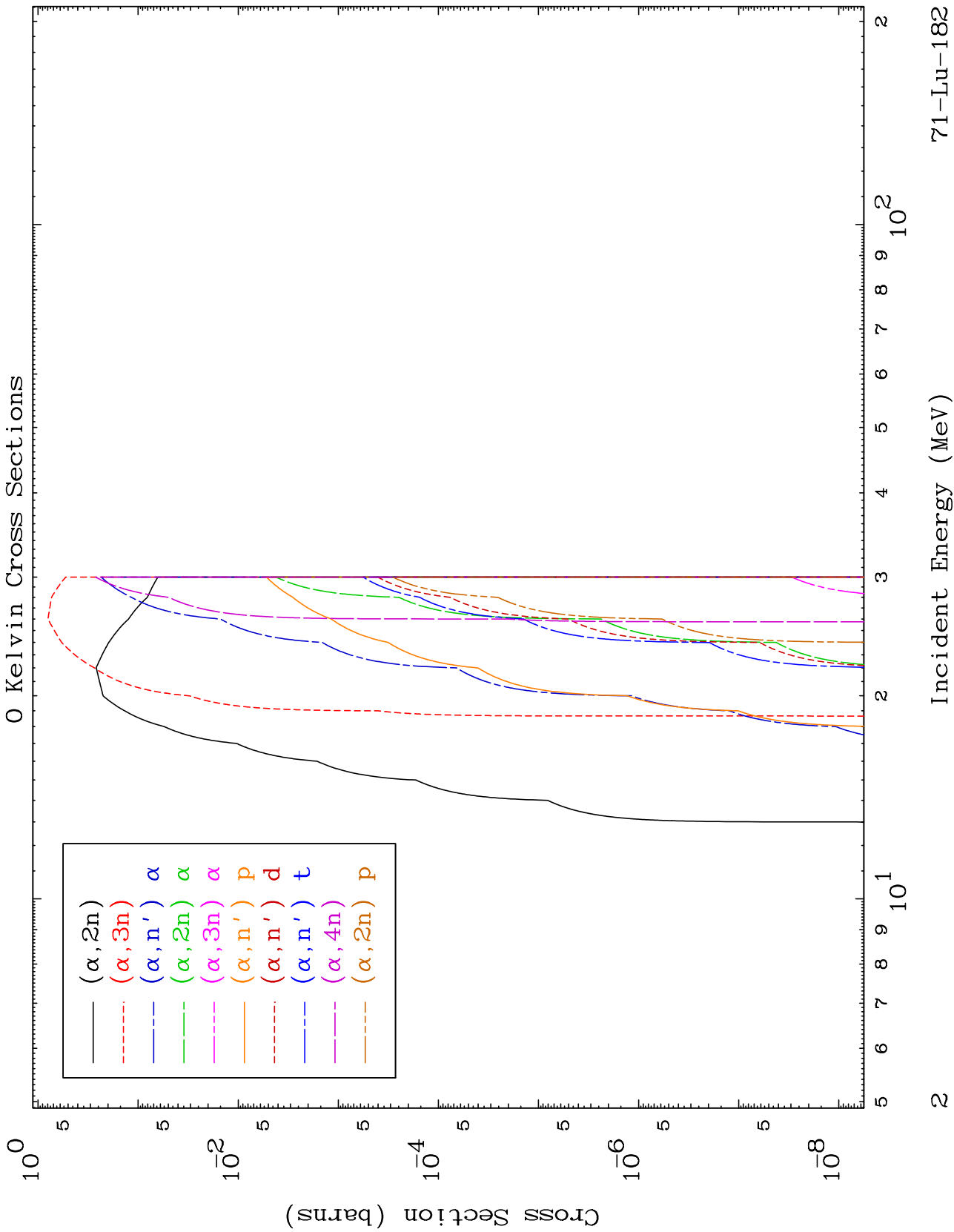
Incident Energy (MeV)

1

MAT 7146

α Neutron Production
0 Kelvin Cross Sections

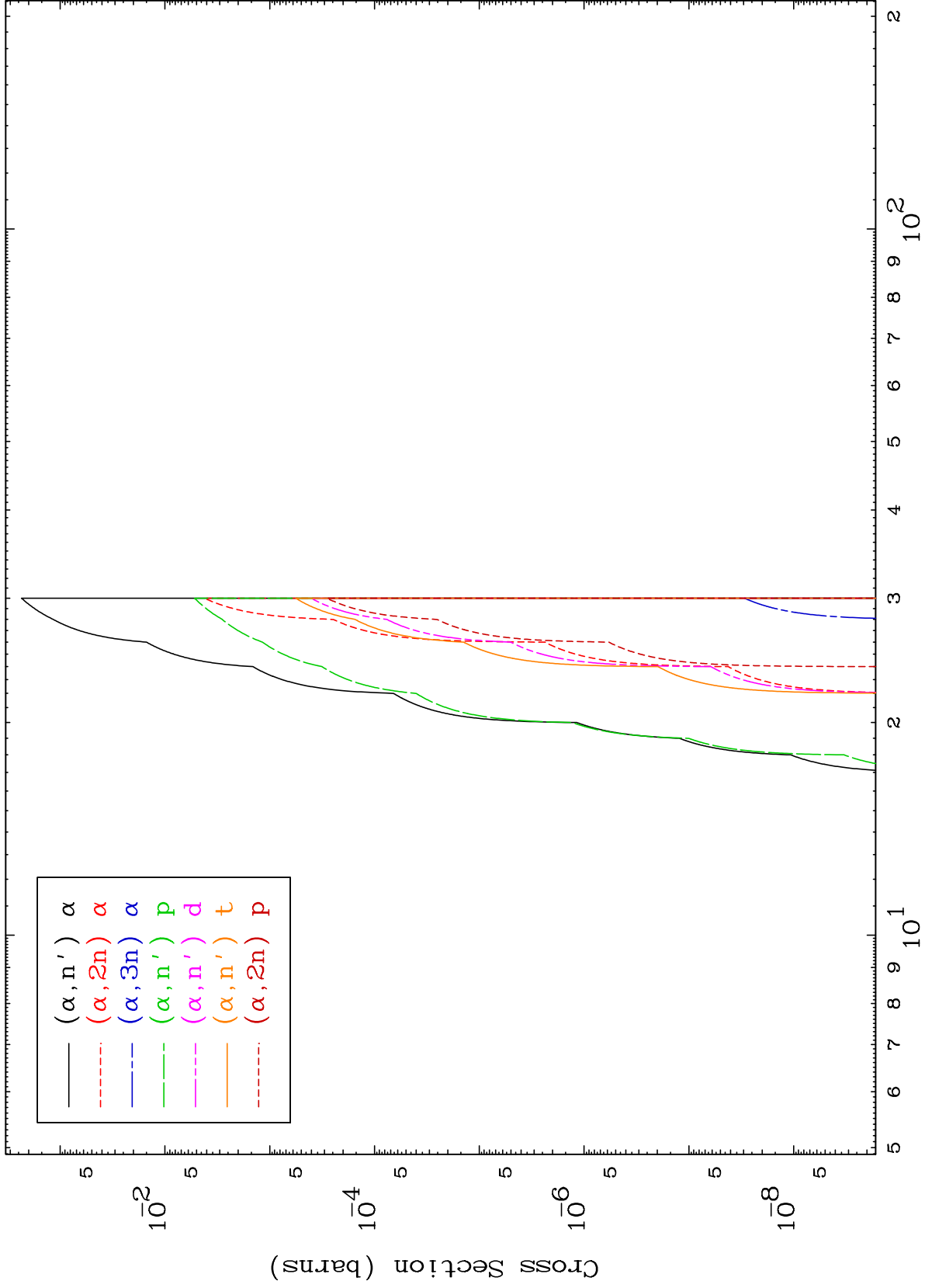
71-Lu-182

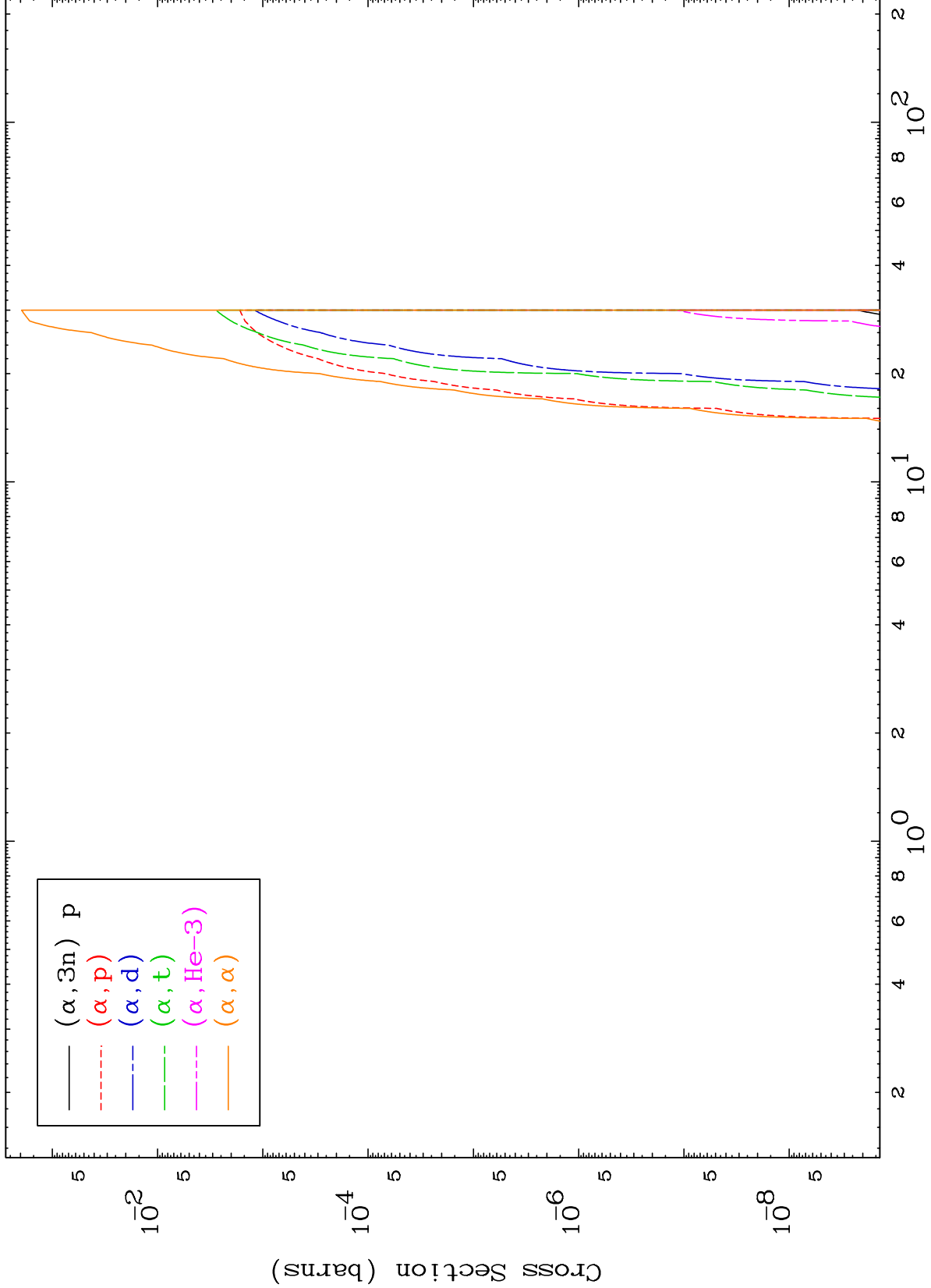


2

Incident Energy (MeV)

71-Lu-182



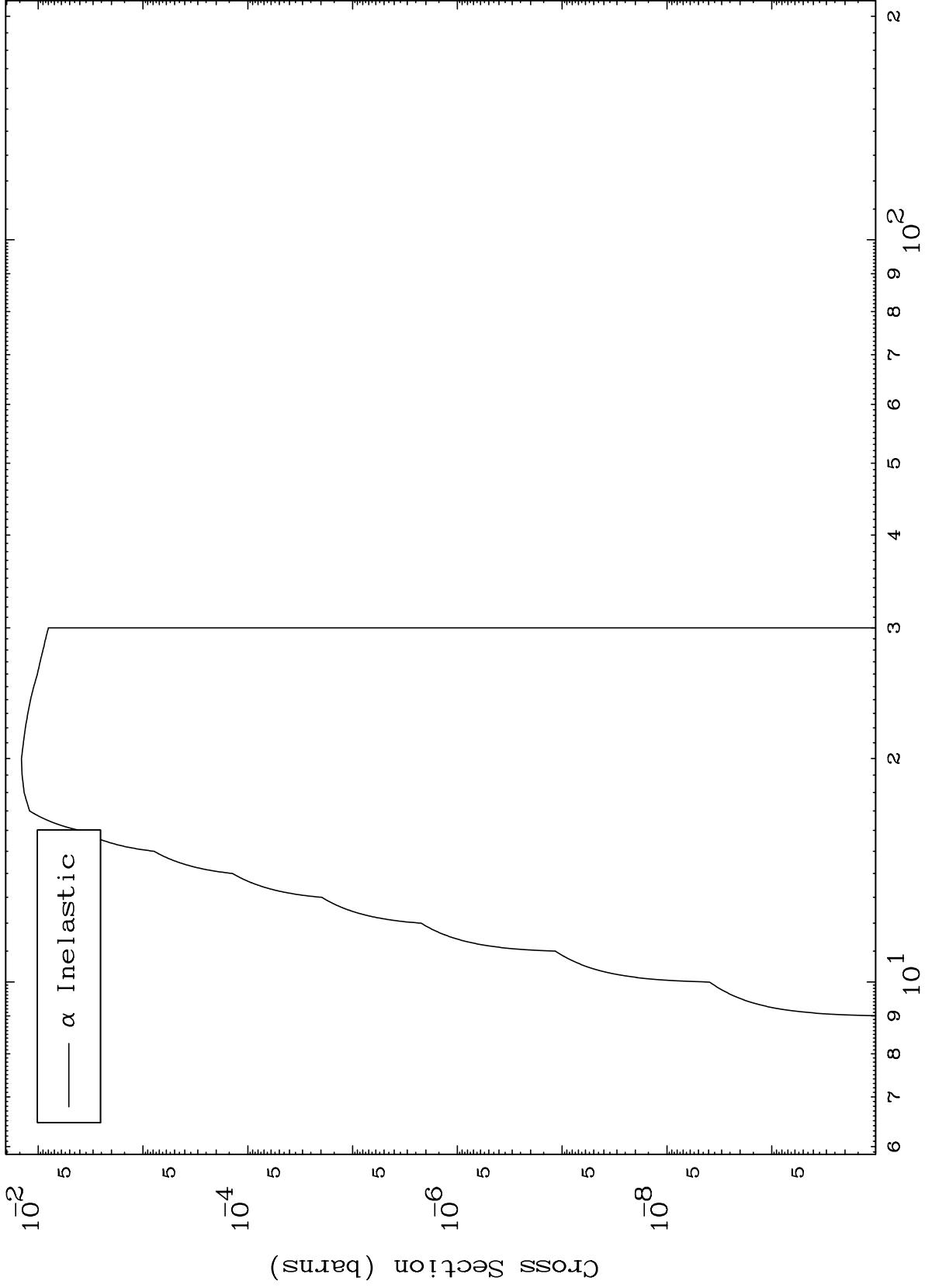


MAT 7146

(α, n') Level

71-Lu-182

0 Kelvin Cross Sections



5

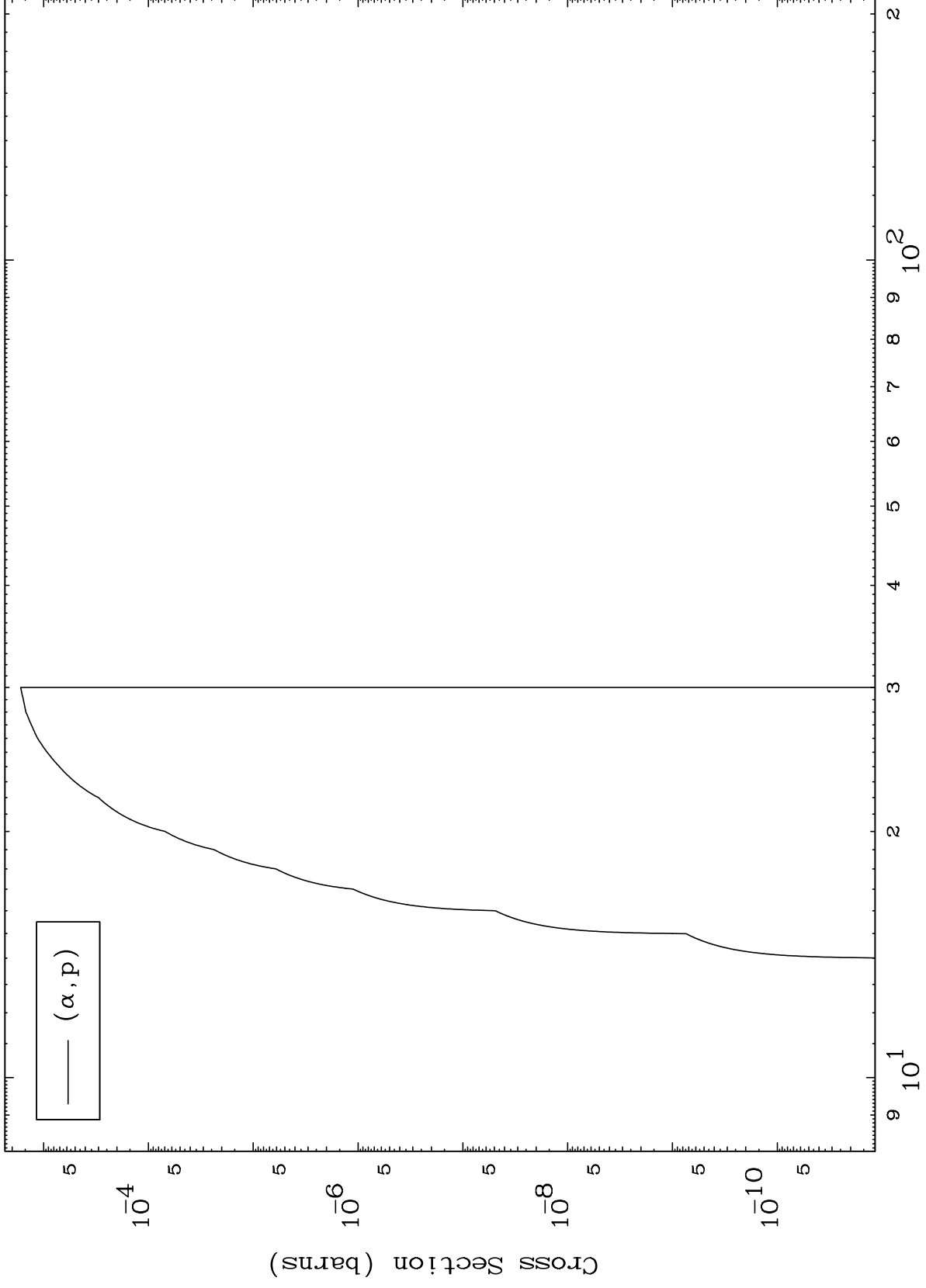
Incident Energy (MeV)

71-Lu-182

MAT 7146

(α, p) Levels
0 Kelvin Cross Sections

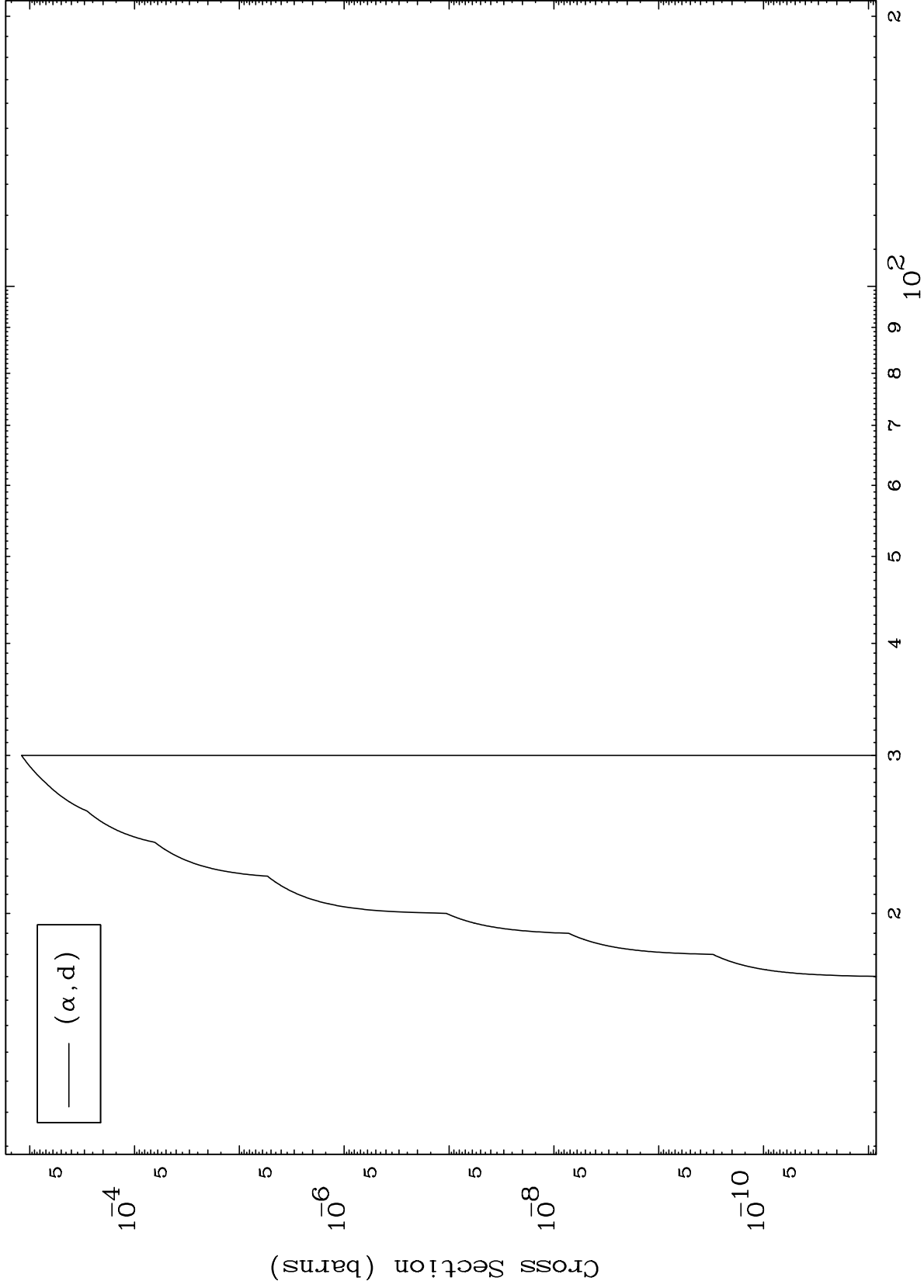
71-Lu-182

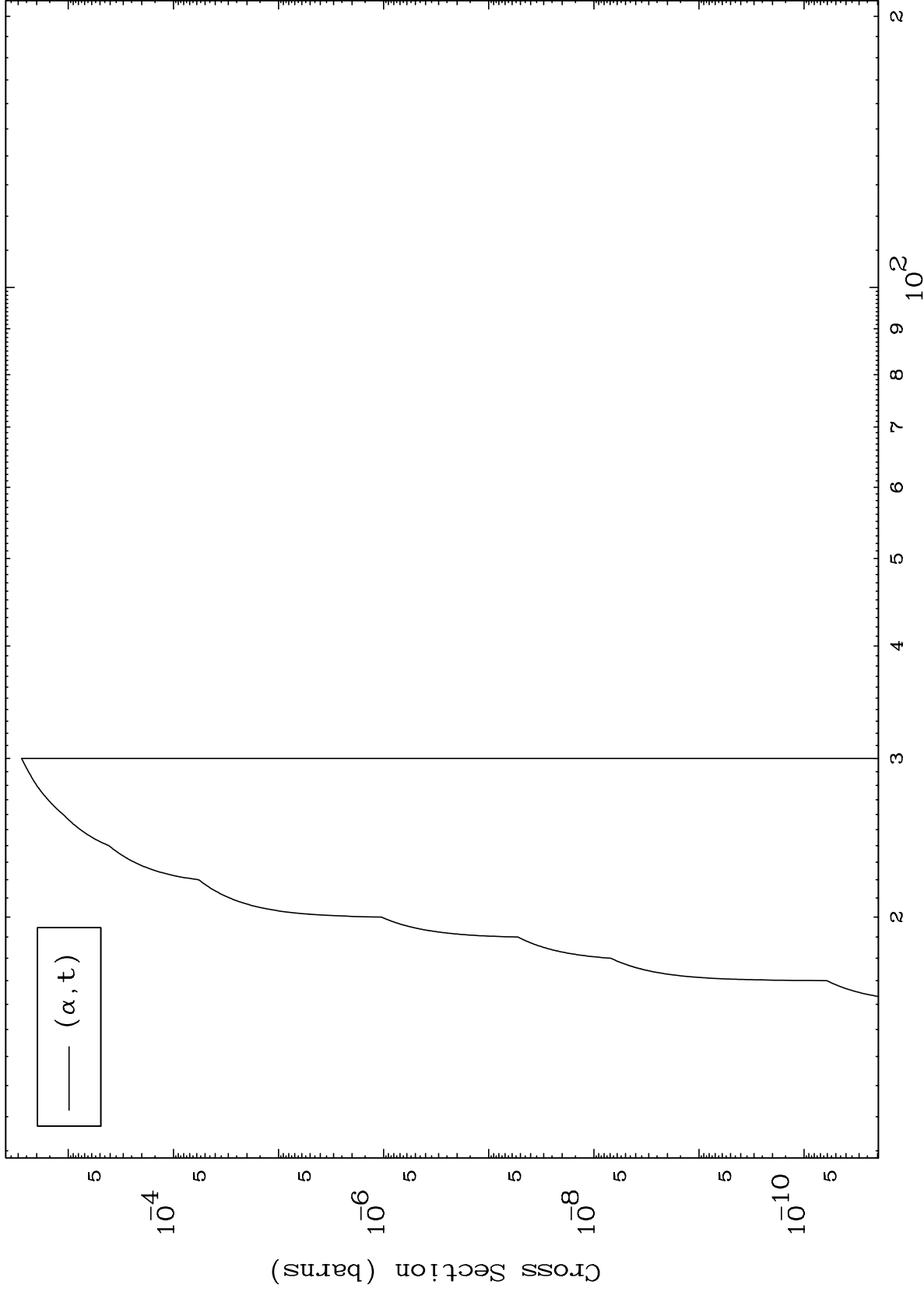


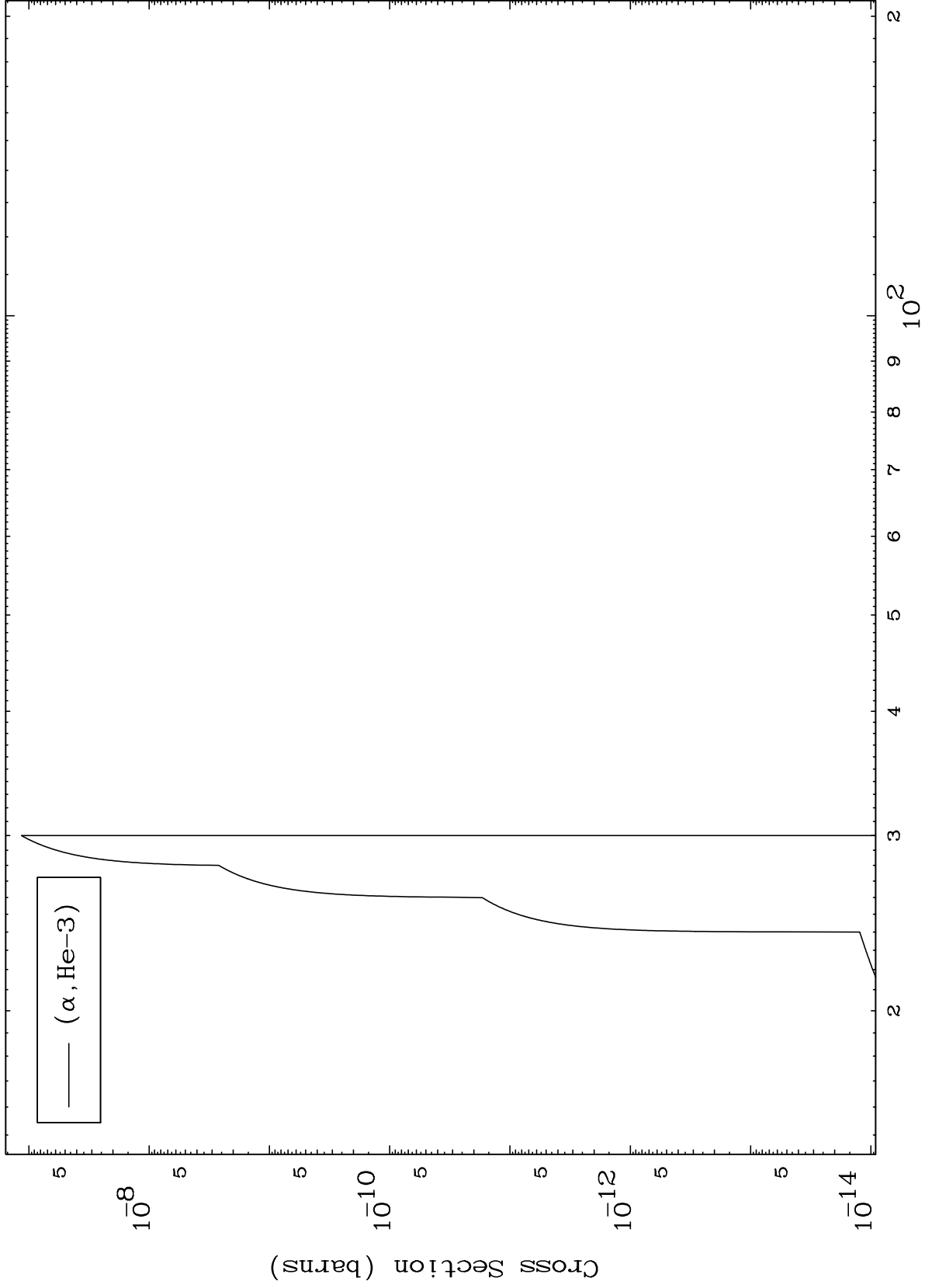
6

Incident Energy (MeV)

71-Lu-182





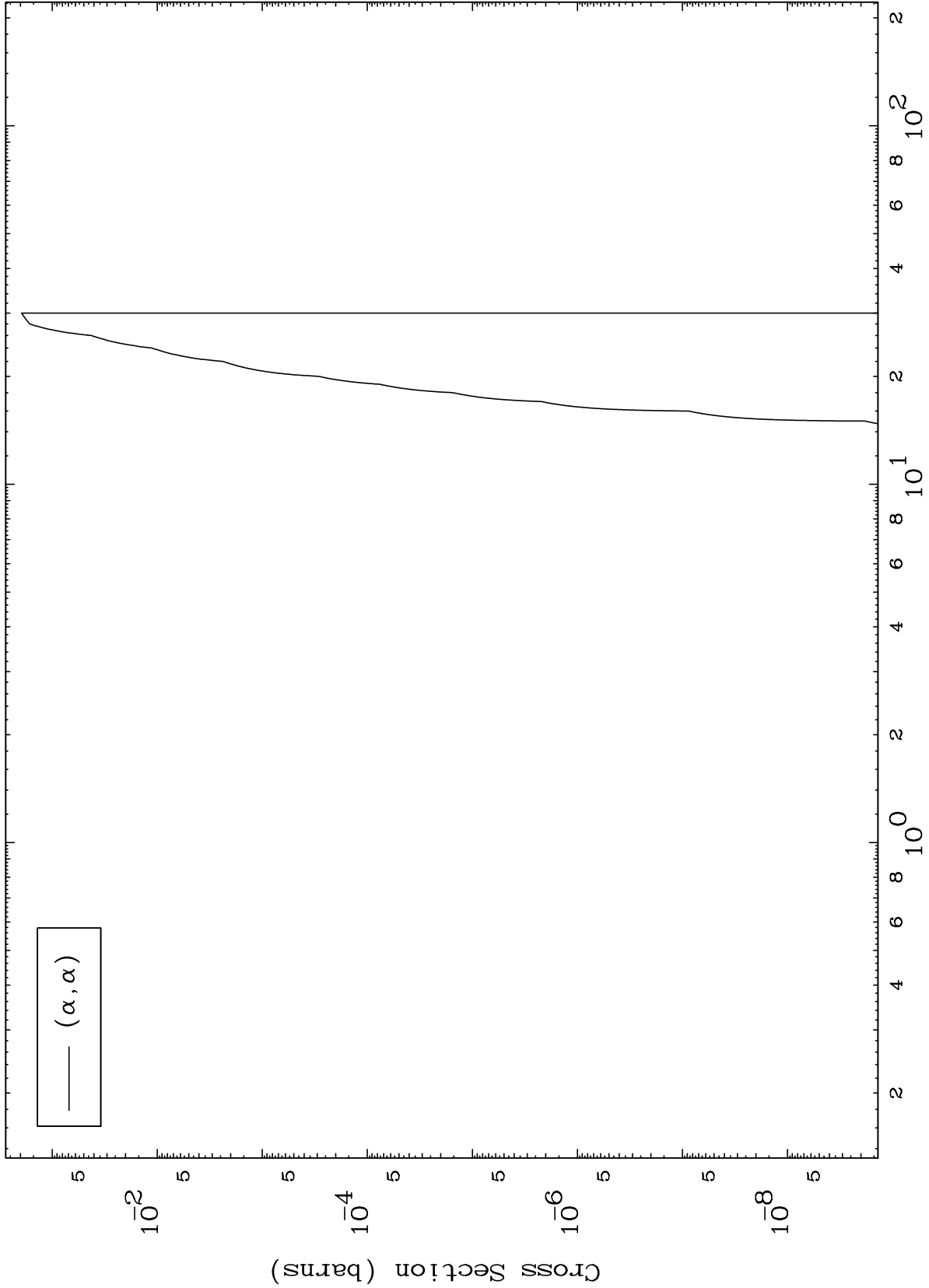


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(α, α) Levels

71-Lu-182

0 Kelvin Cross Sections

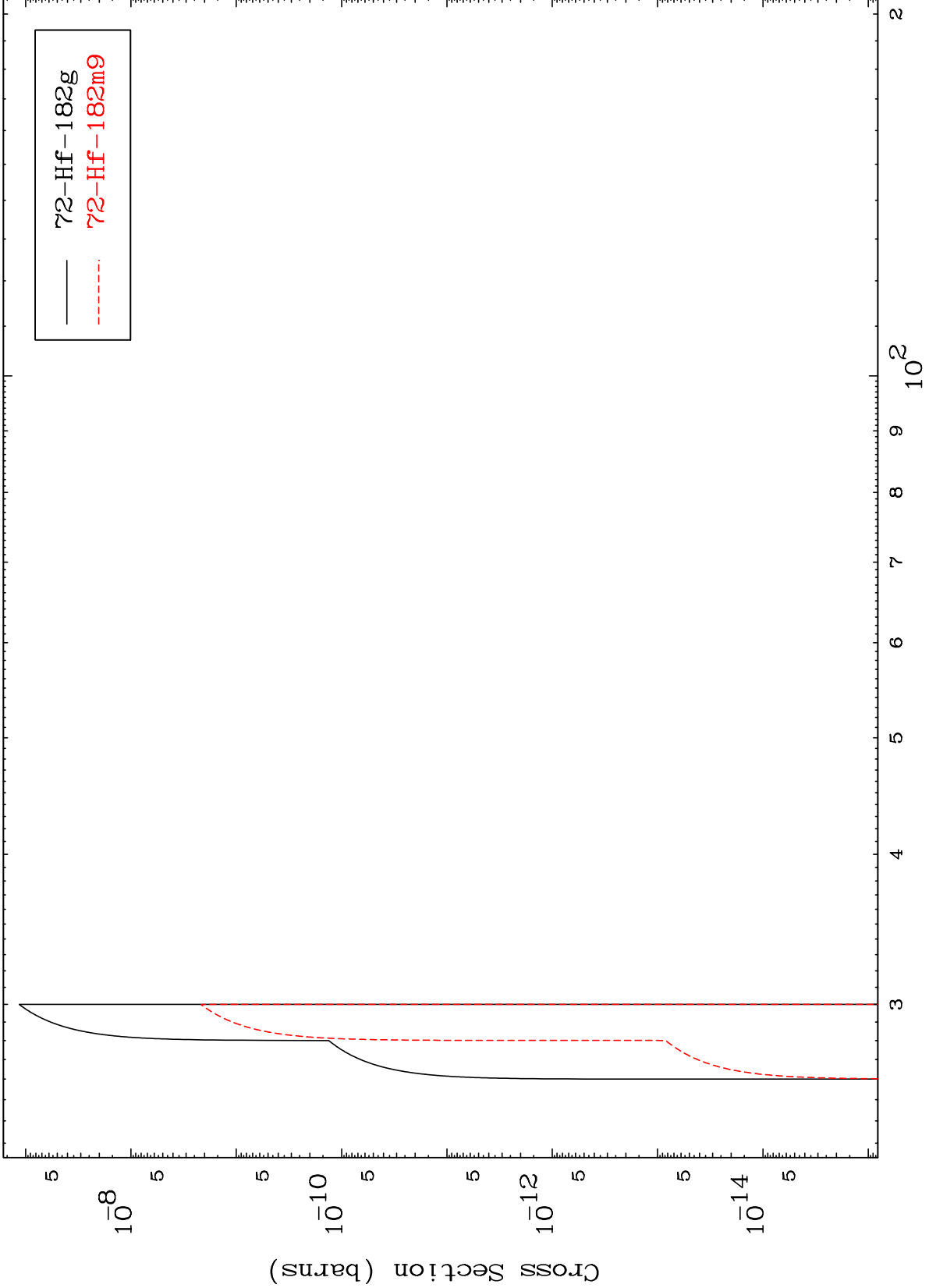


10

Incident Energy (MeV)

71-Lu-182

Radionuclide Production Cross Section

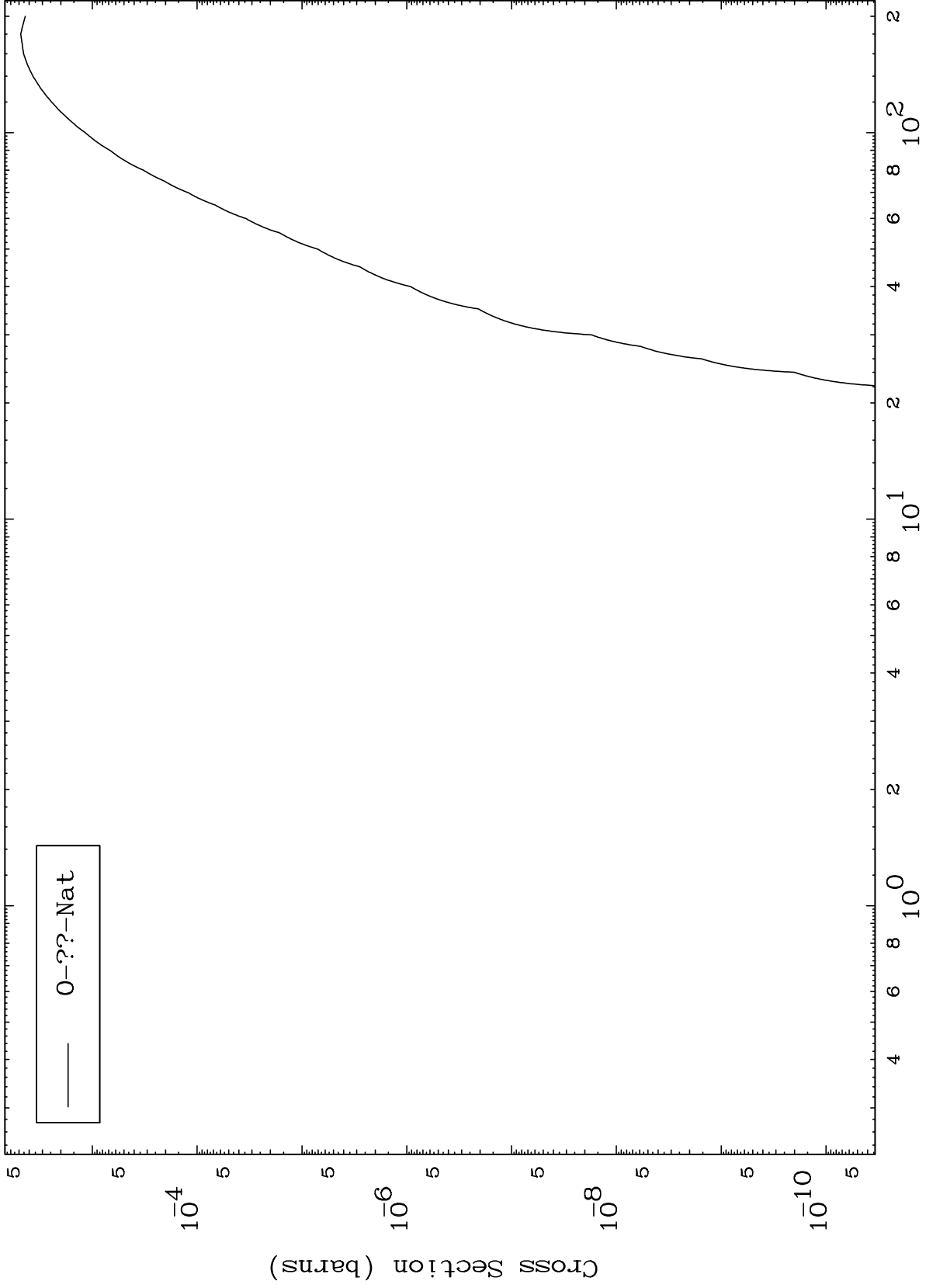


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α Fission

71-Lu-182

Radionuclide Production Cross Section

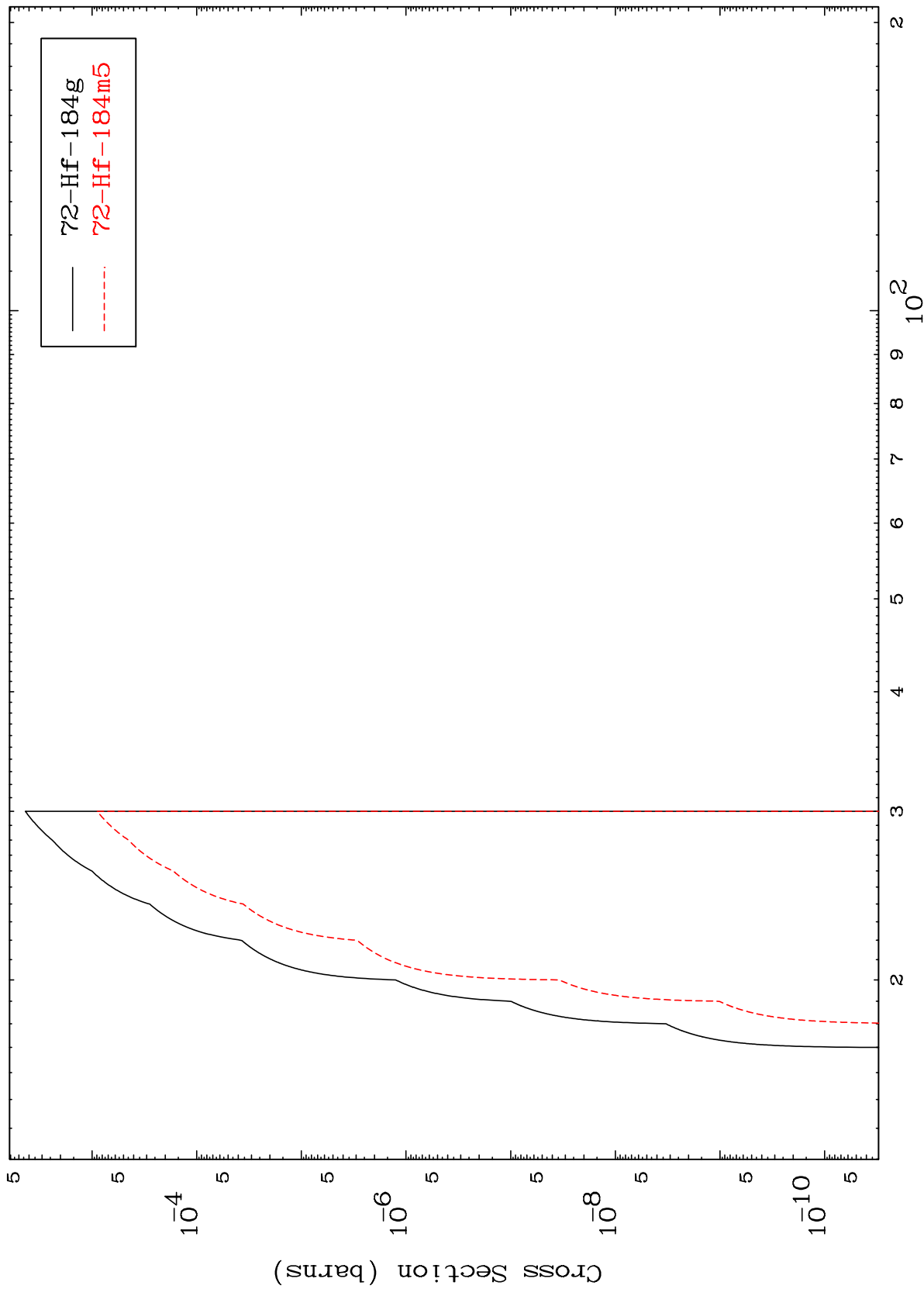


12

Incident Energy (MeV)

71-Lu-182

Radionuclide Production Cross Section

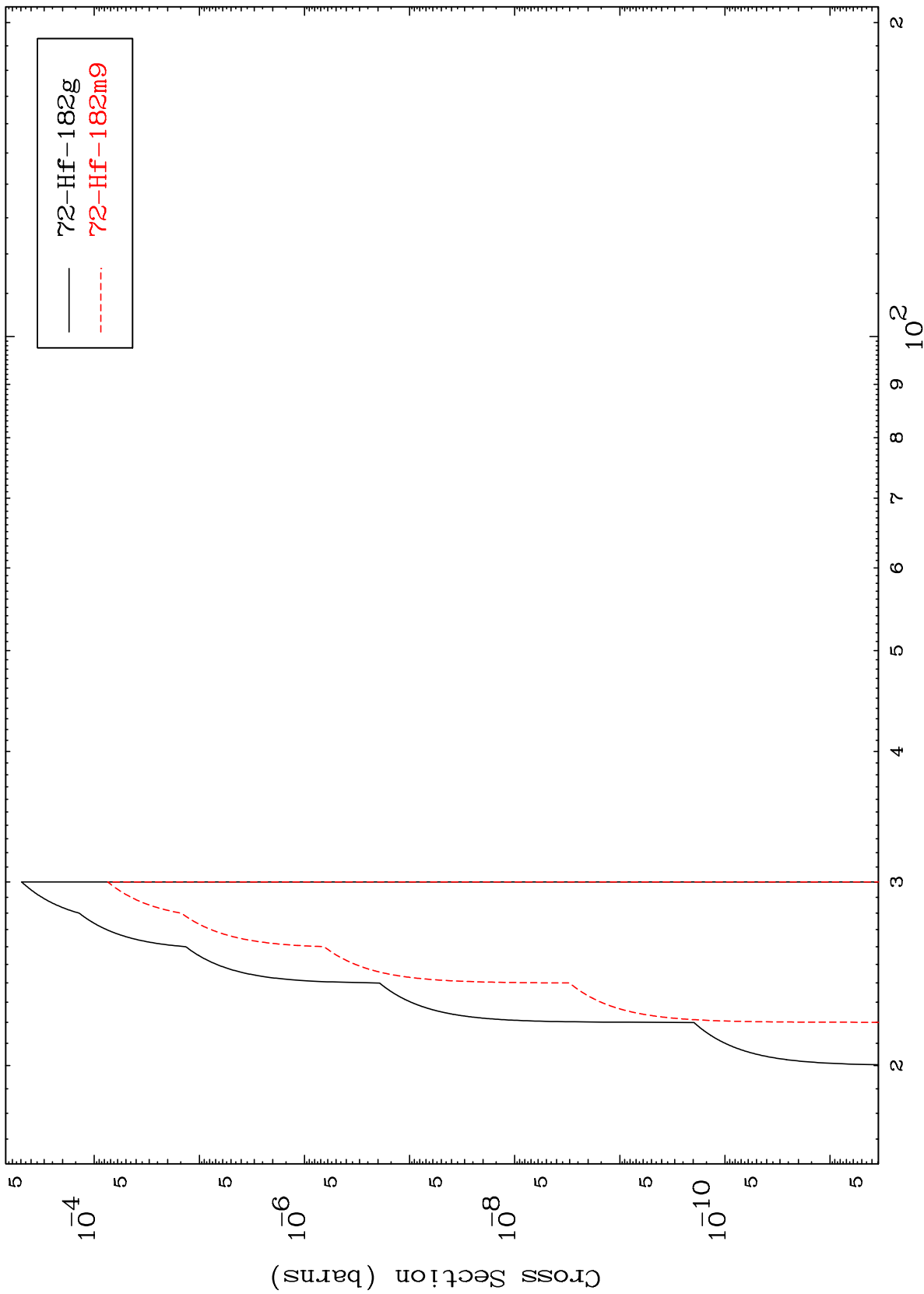


MAT 7146

(α, n') t

71-Lu-182

Radionuclide Production Cross Section

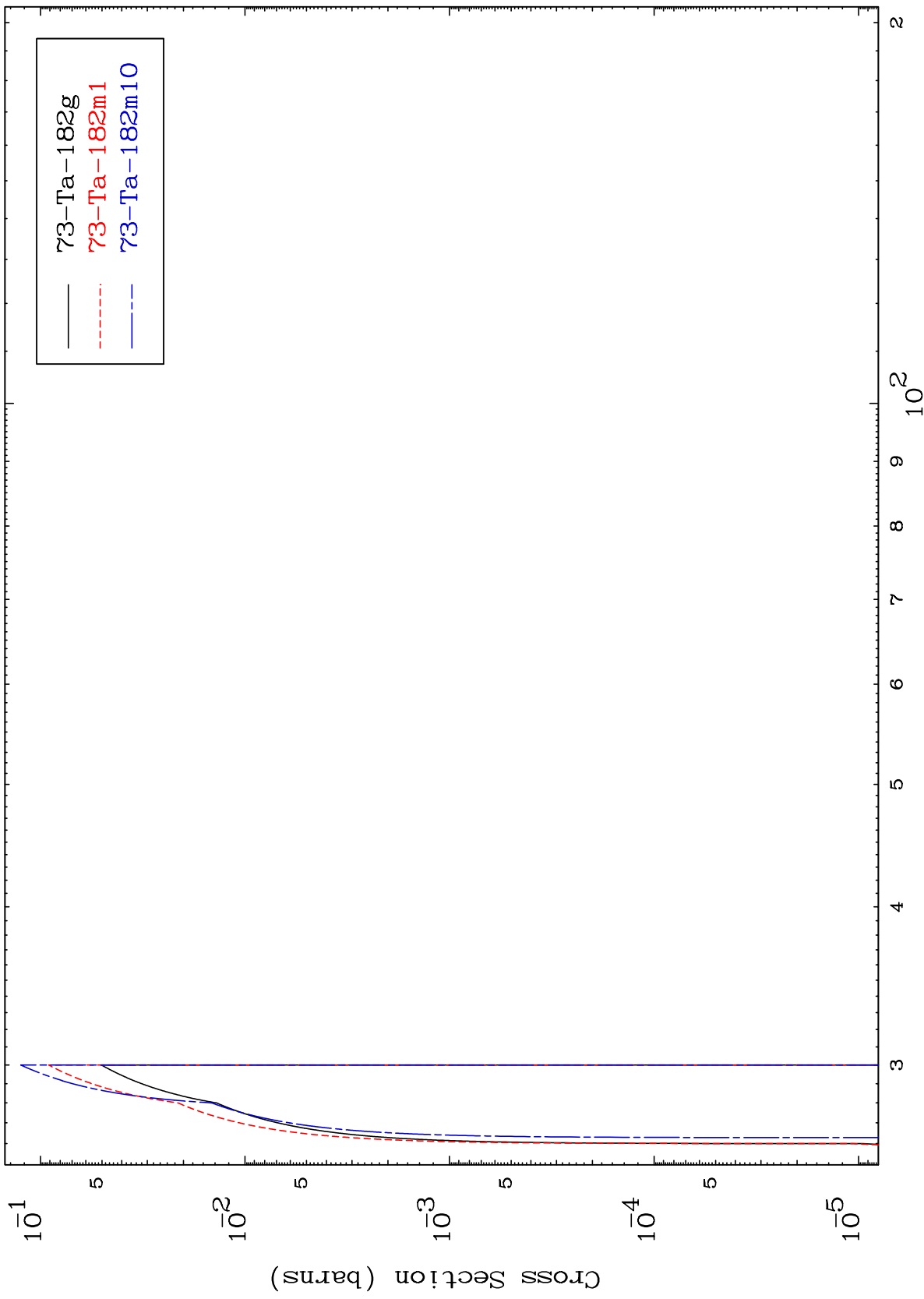


14

Incident Energy (MeV)

71-Lu-182

($\alpha, 4n$)
Radionuclide Production Cross Section

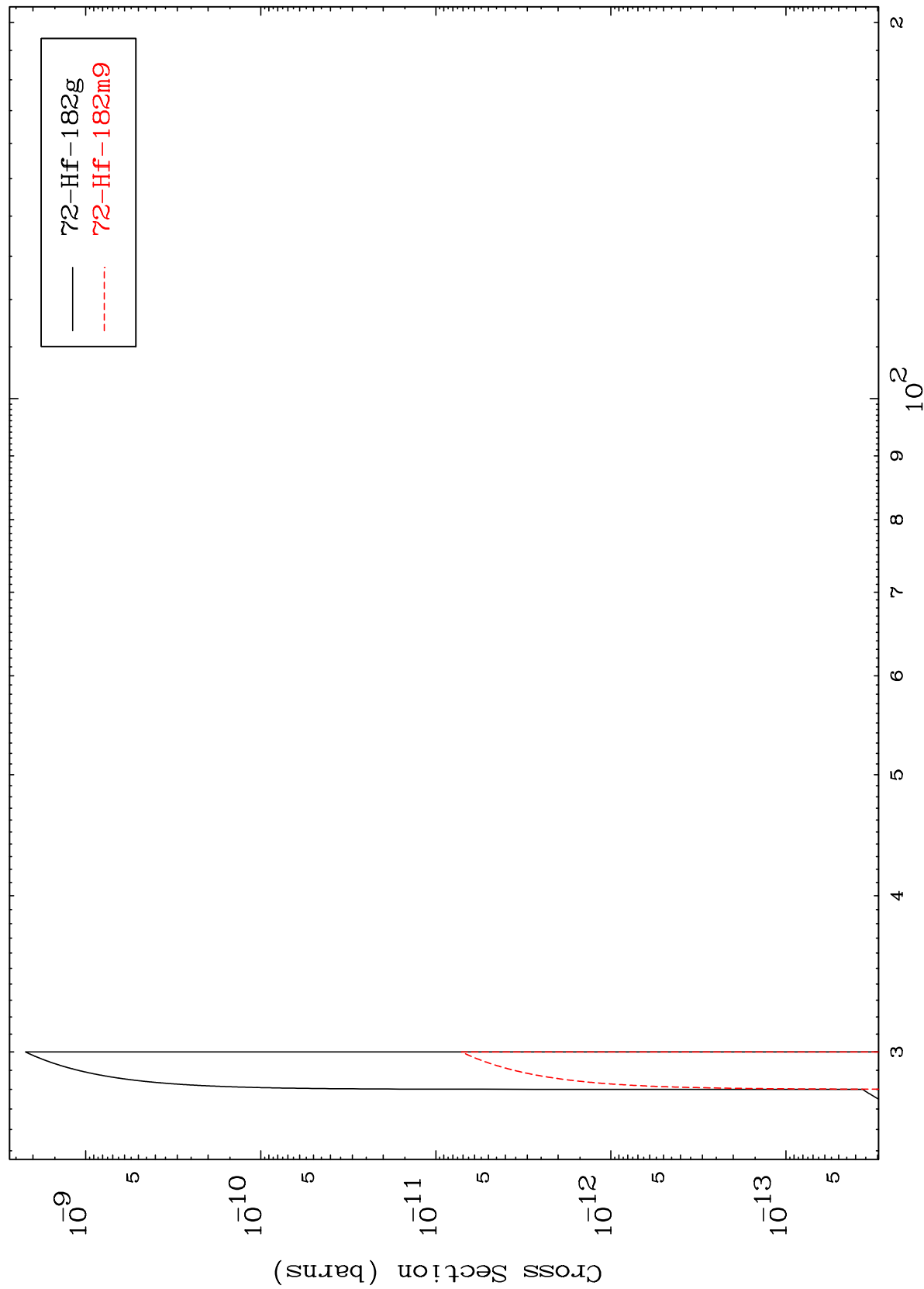


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($\alpha, 3n$) p

71-Lu-182

Radionuclide Production Cross Section



72-Hf-182g
72-Hf-182m9

16

Incident Energy (MeV)

71-Lu-182

Radionuclide Production Cross Section

