

Program EVALPLOT  
(Version 2018-1)

by

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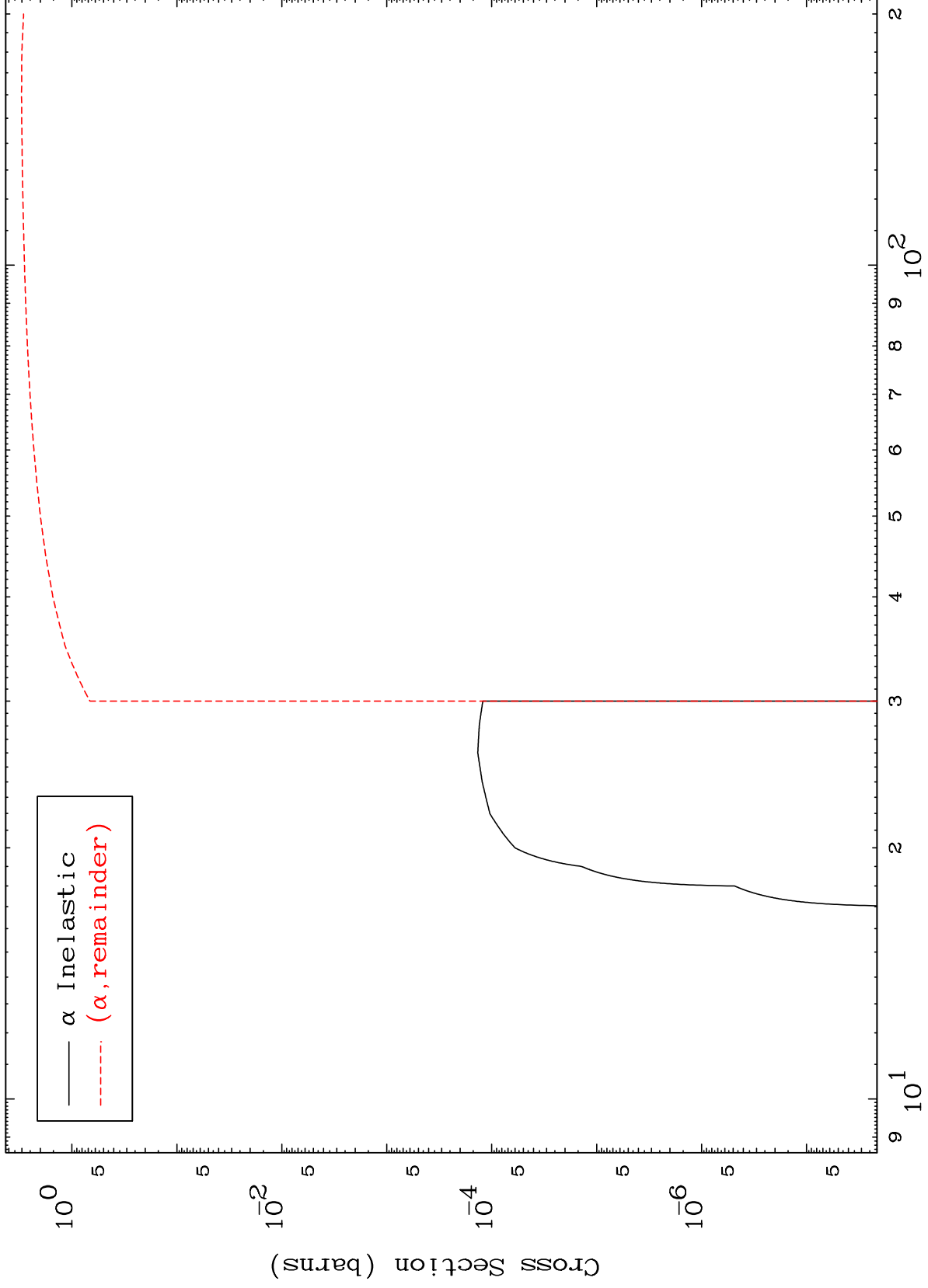
Press Mouse Button to Start

MAT 9952

$\alpha$  Major

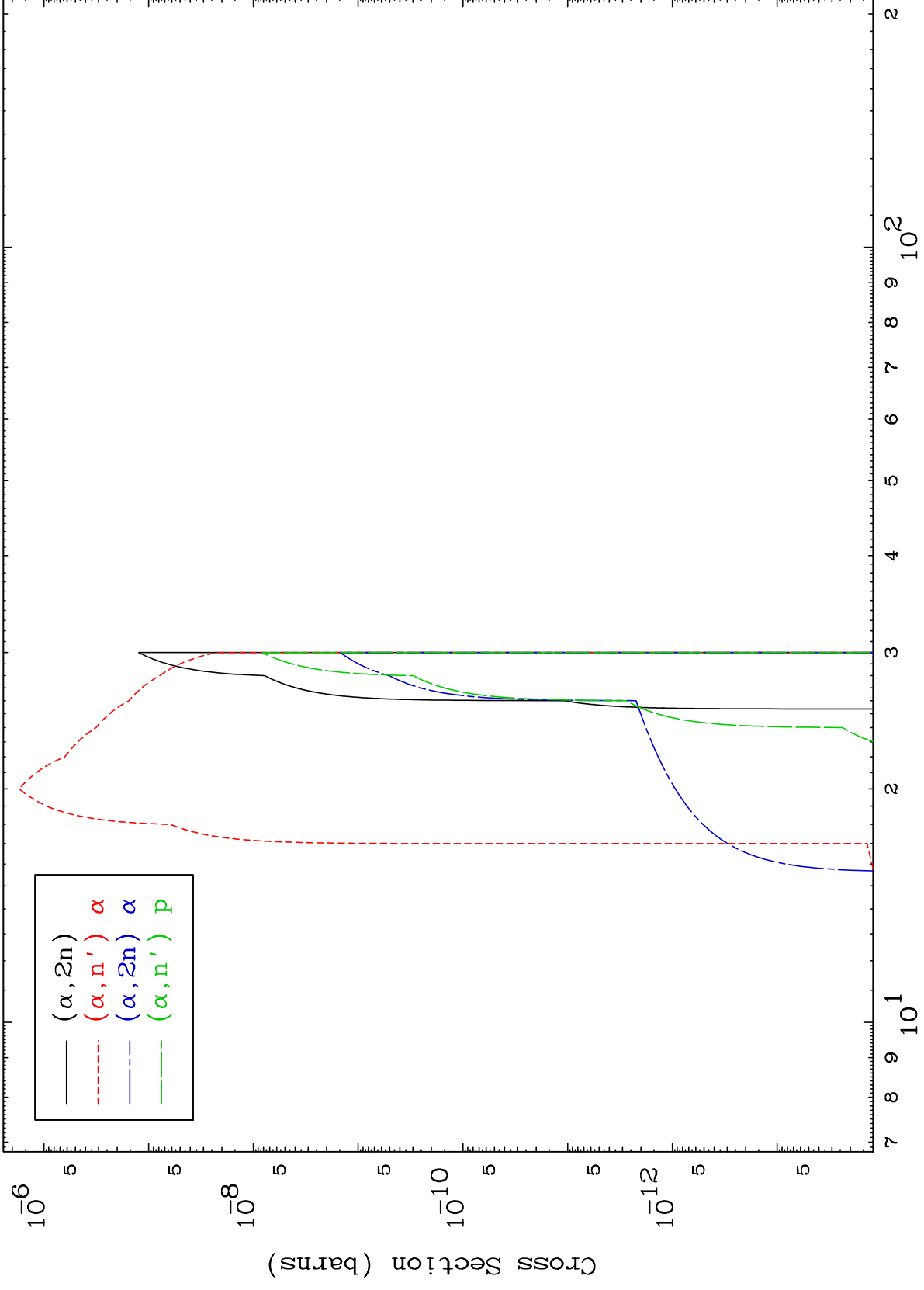
101-Md-246

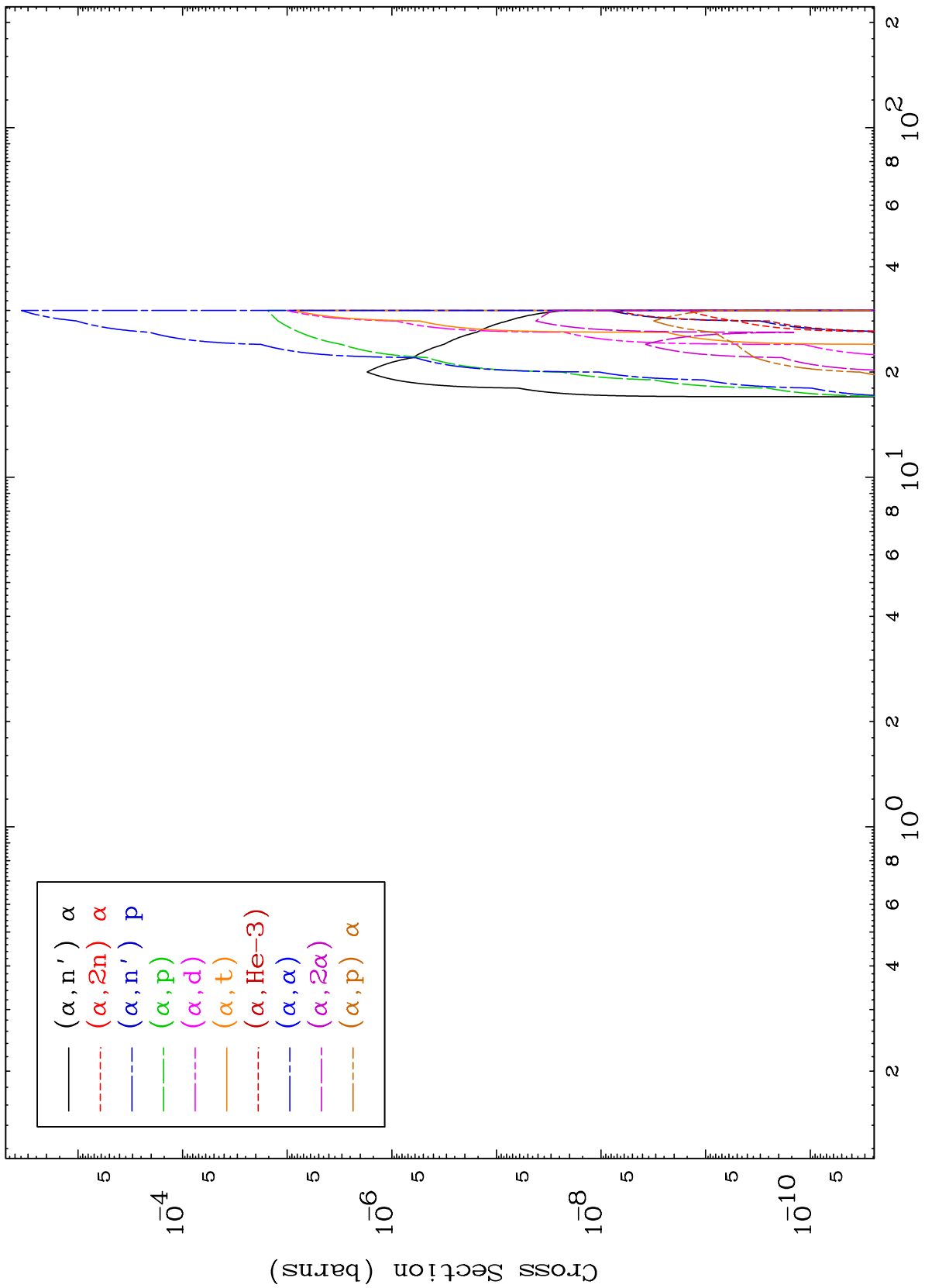
0 Kelvin Cross Sections



Incident Energy (MeV)

101-Md-246



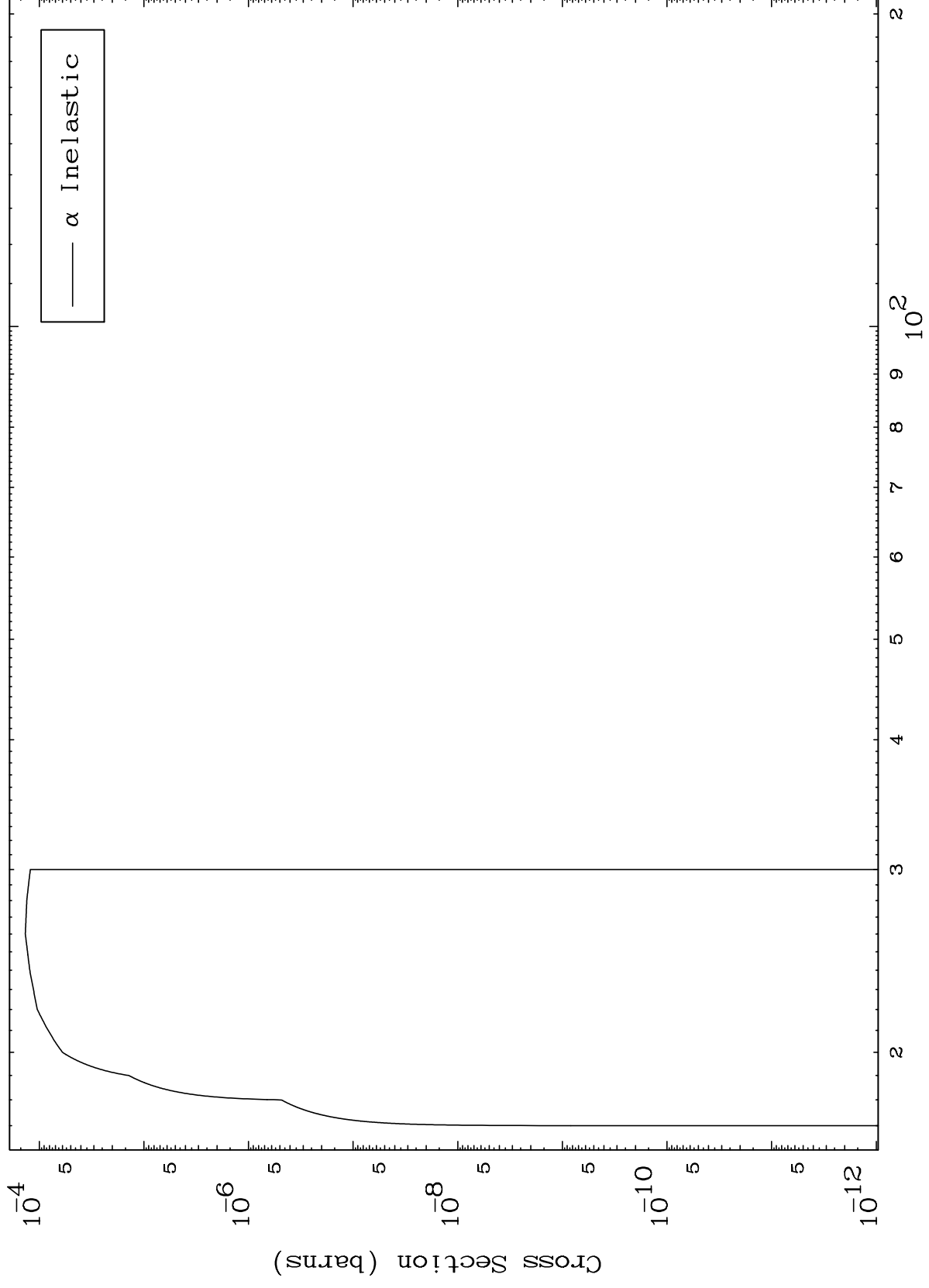


MAT 9952

( $\alpha, n'$ ) Level

101-Md-246

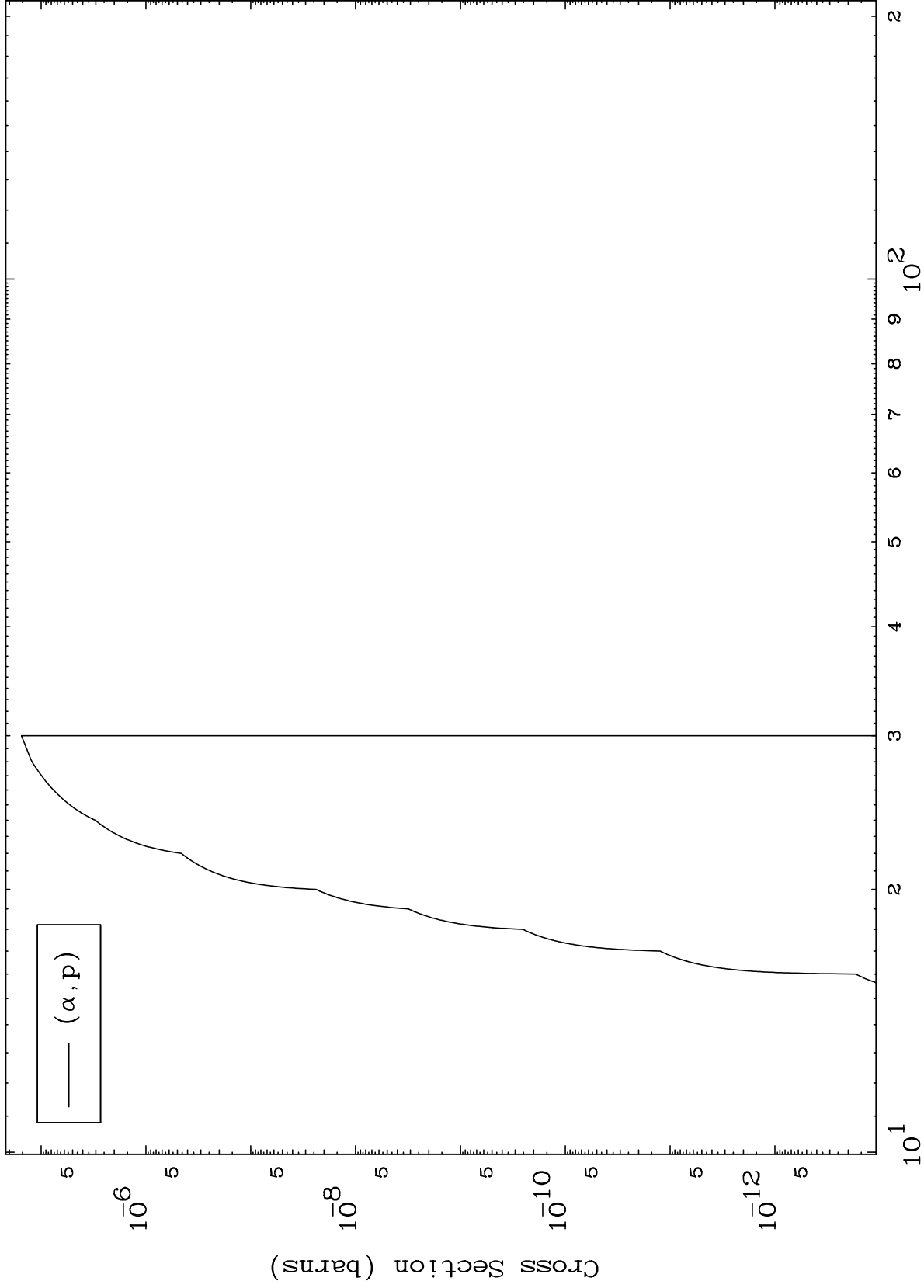
0 Kelvin Cross Sections



MAT 9952

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

101-Md-246



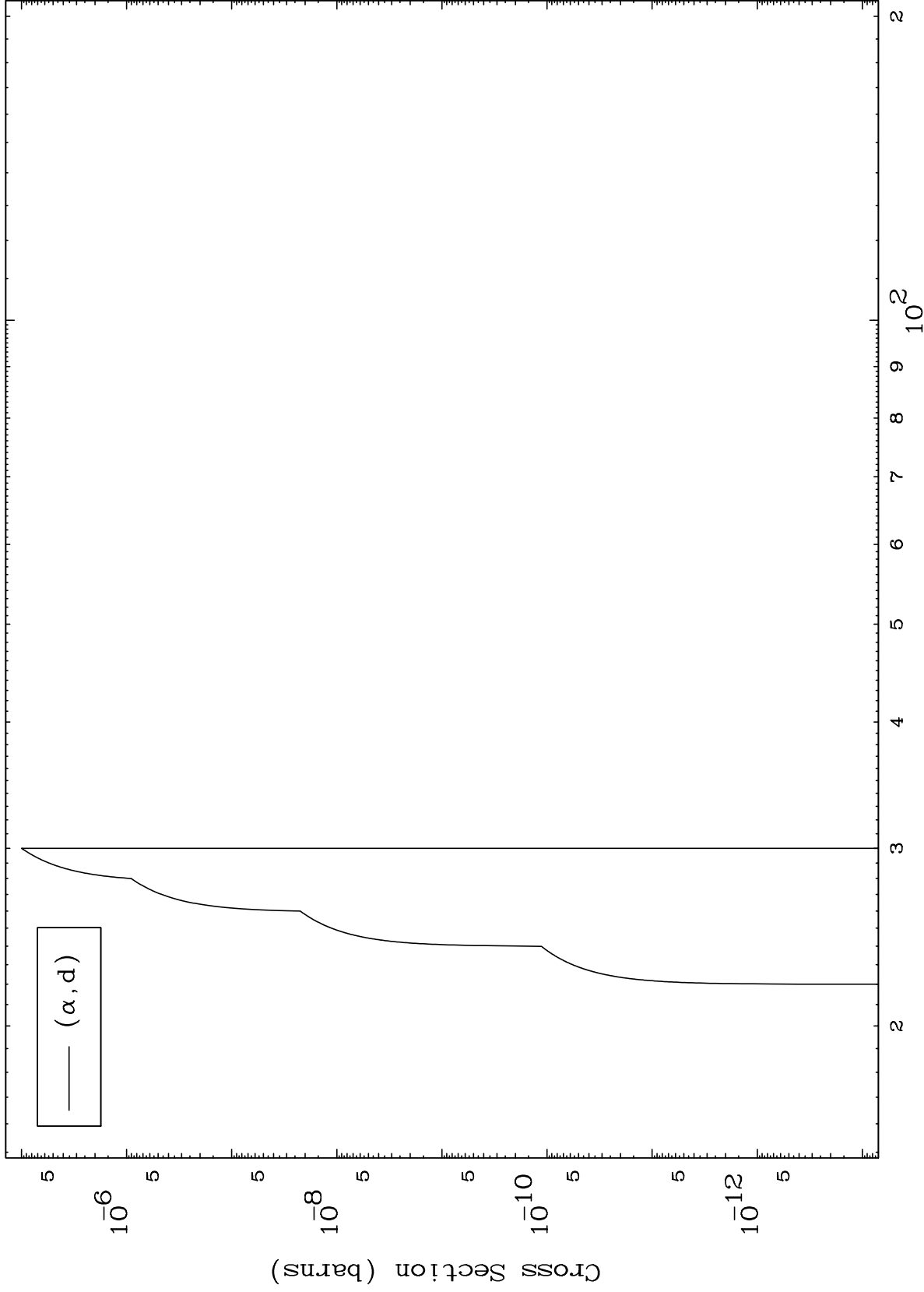
Incident Energy (MeV)

101-Md-246

MAT 9952

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

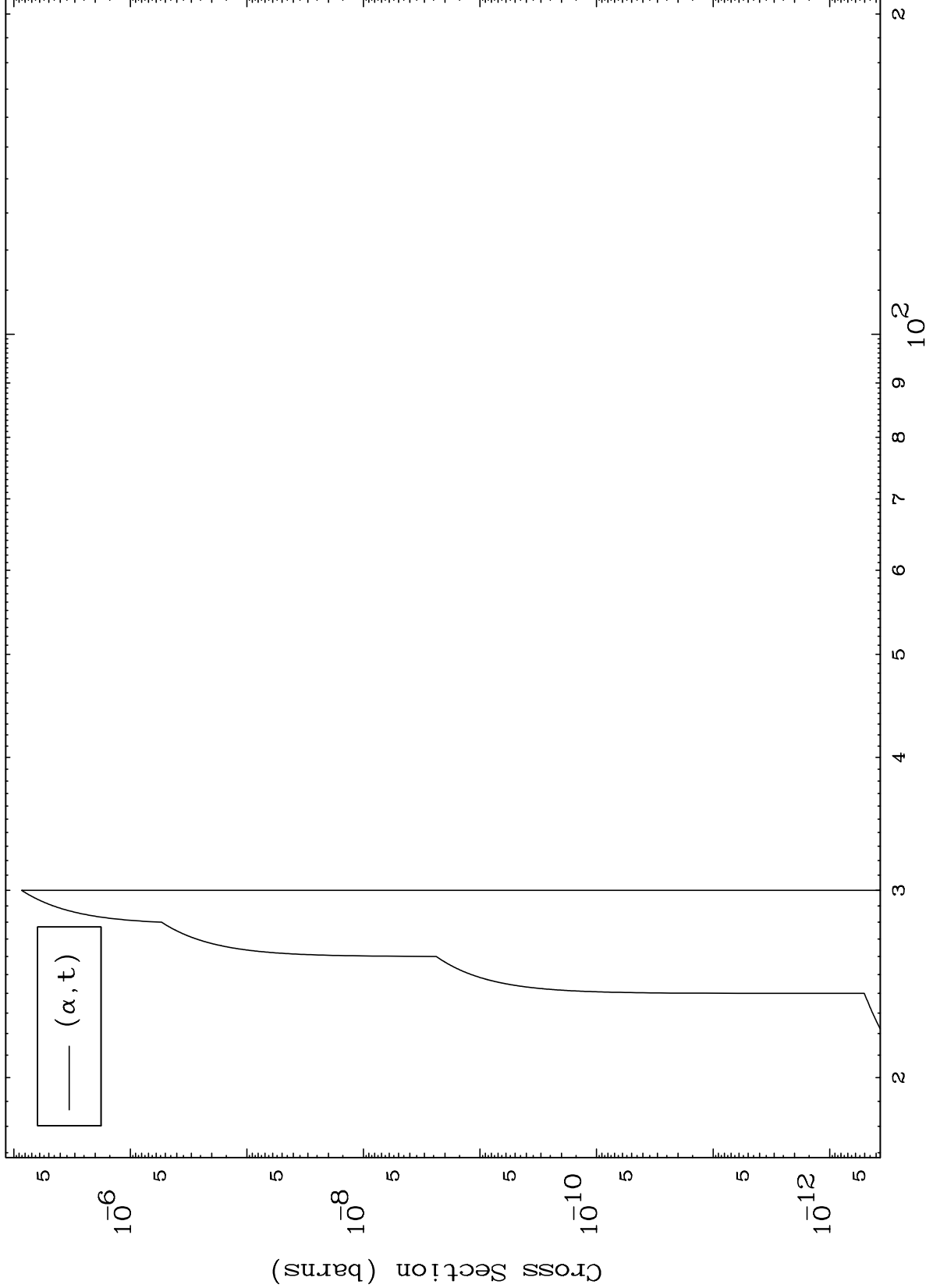
101-Md-246



6

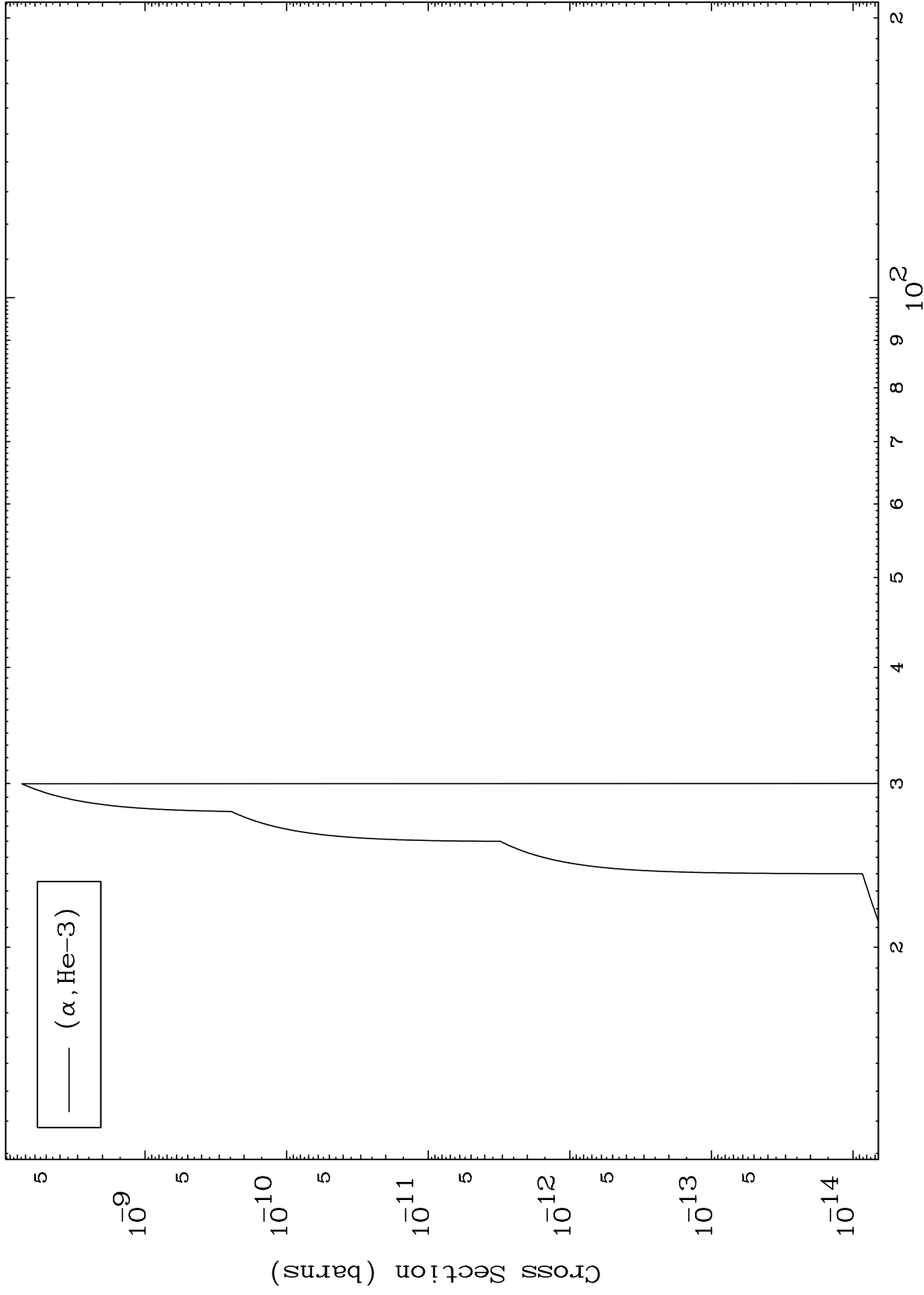
Incident Energy (MeV)

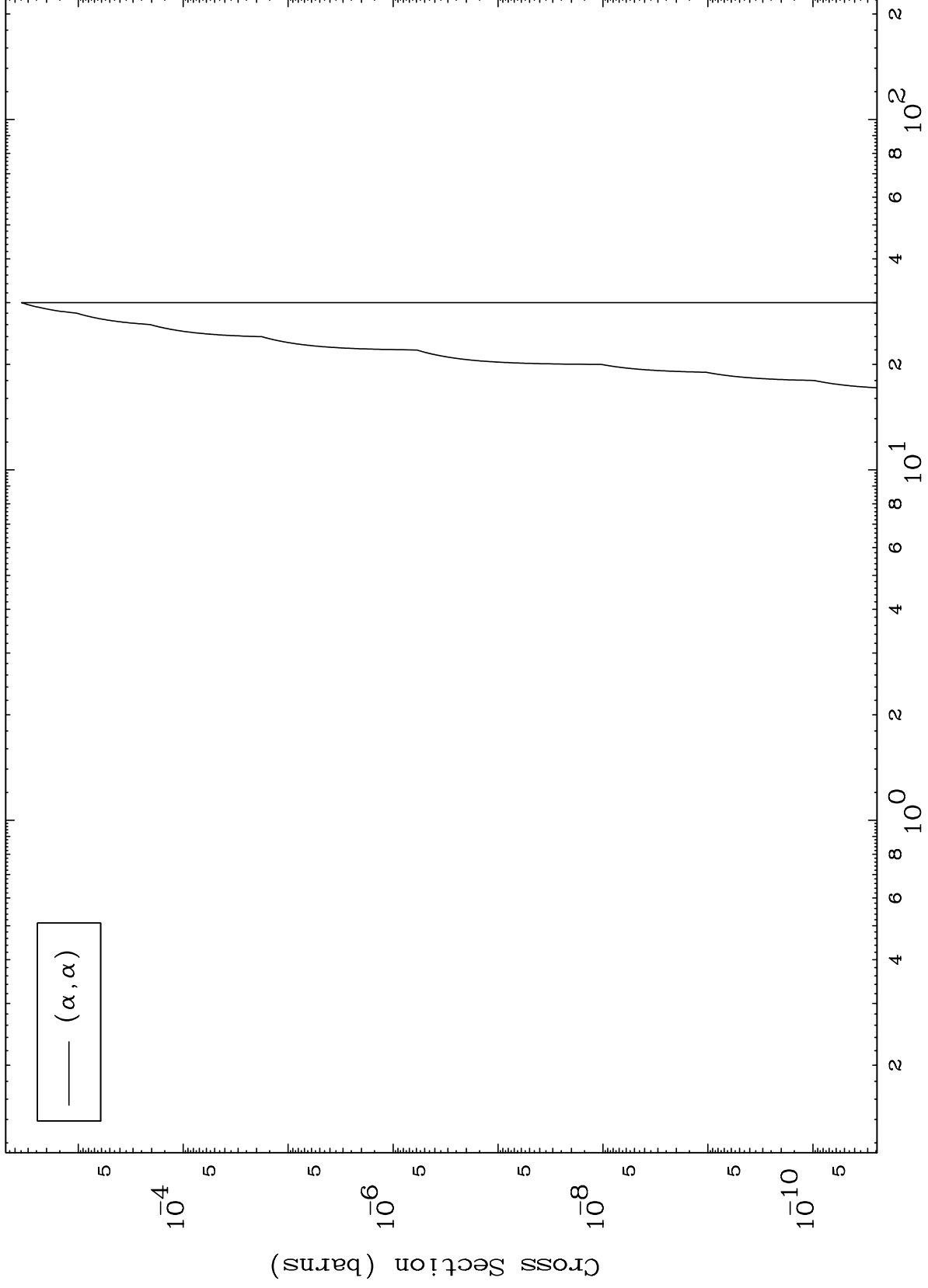
101-Md-246





( $\alpha$ ,He3) Levels  
0 Kelvin Cross Sections

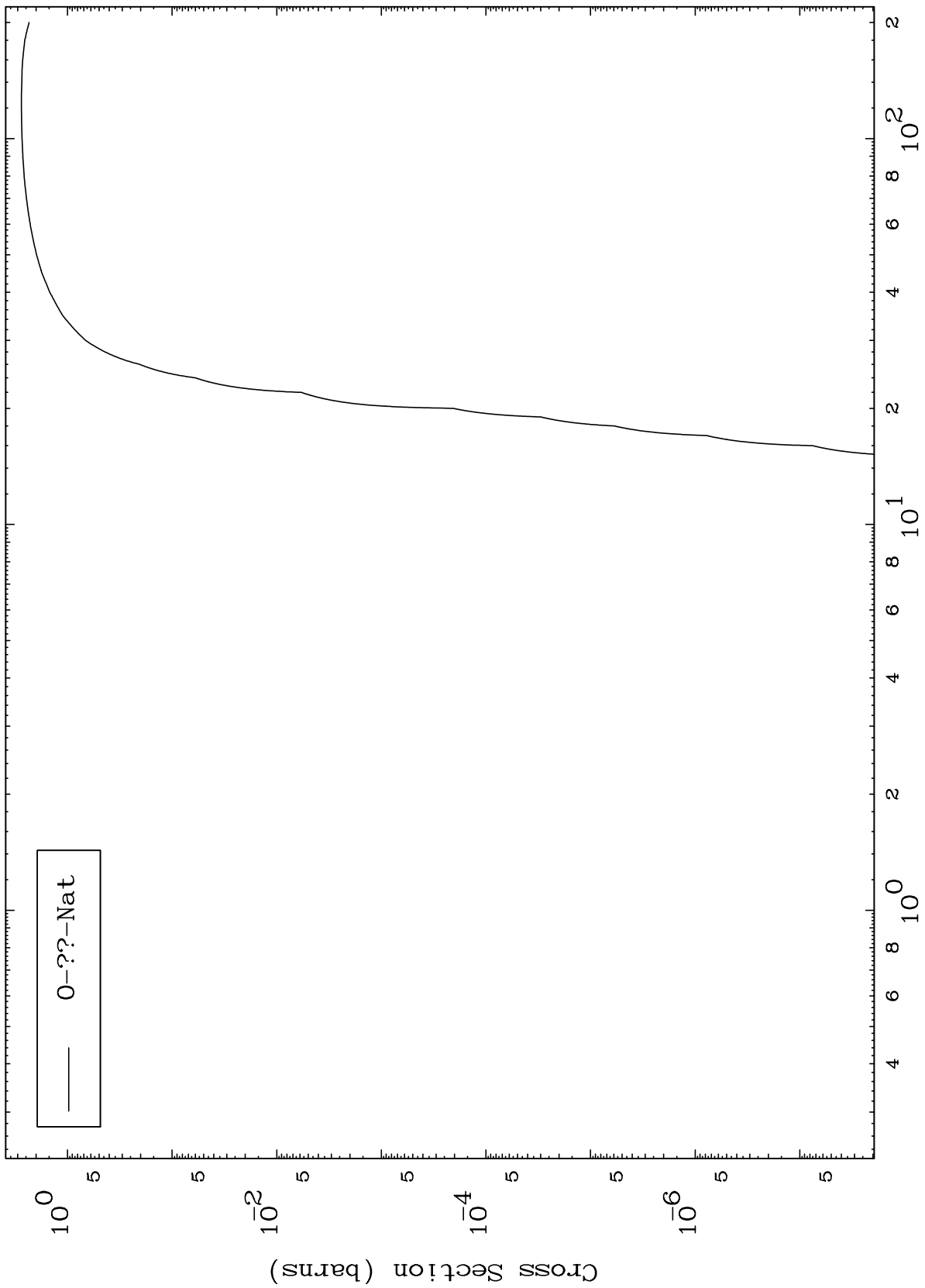




MAT 9952

101-Md-246

$\alpha$  Fission  
Radionuclide Production Cross Section



101-Md-246

Incident Energy (MeV)

10

MAT 9952

101-Md-246

( $\alpha, \alpha$ )

Radionuclide Production Cross Section

