

Program EVALPLOT
(Version 2018-1)

by

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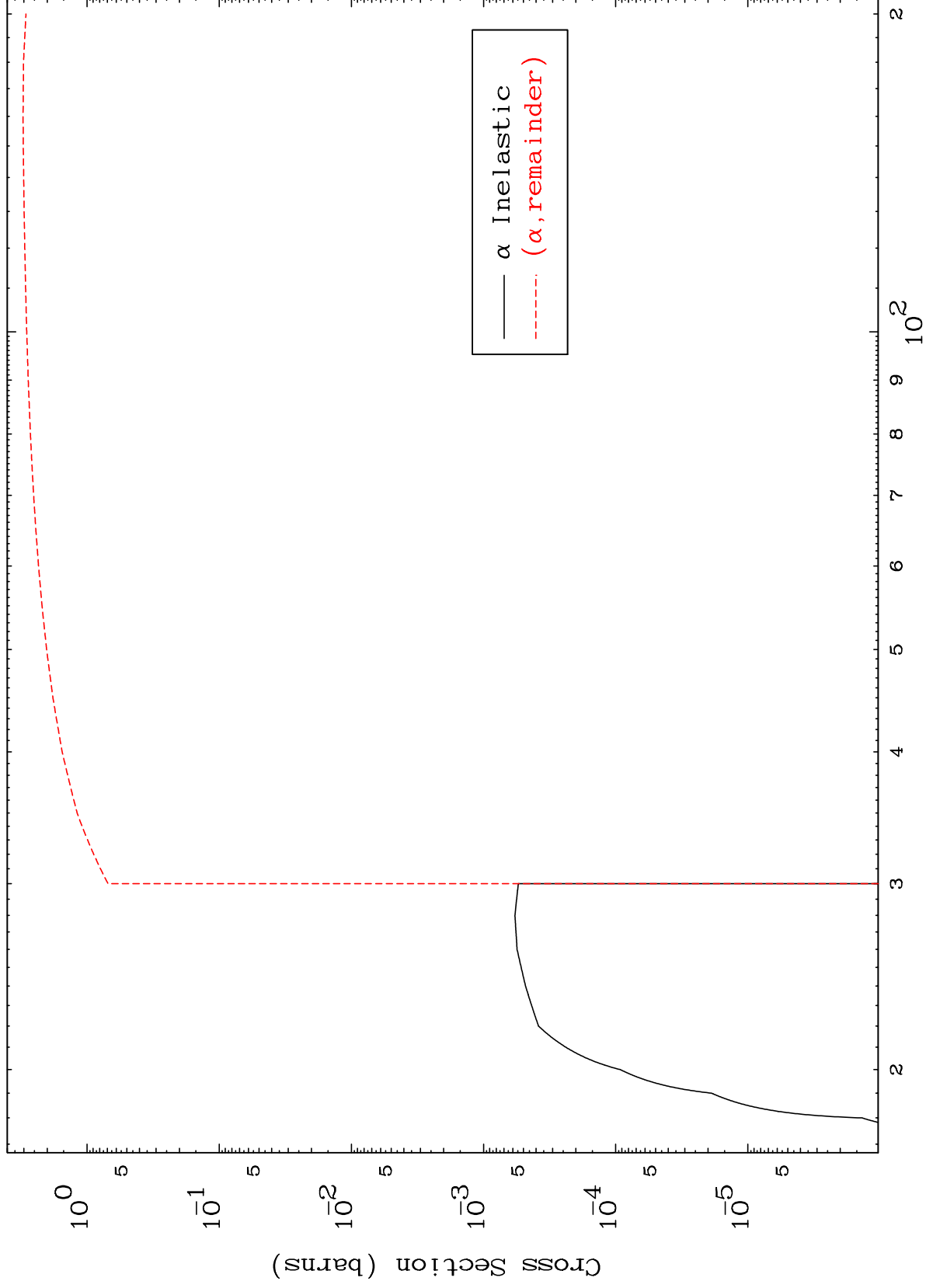
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9950

α Major
0 Kelvin Cross Sections

101-Md-249



1

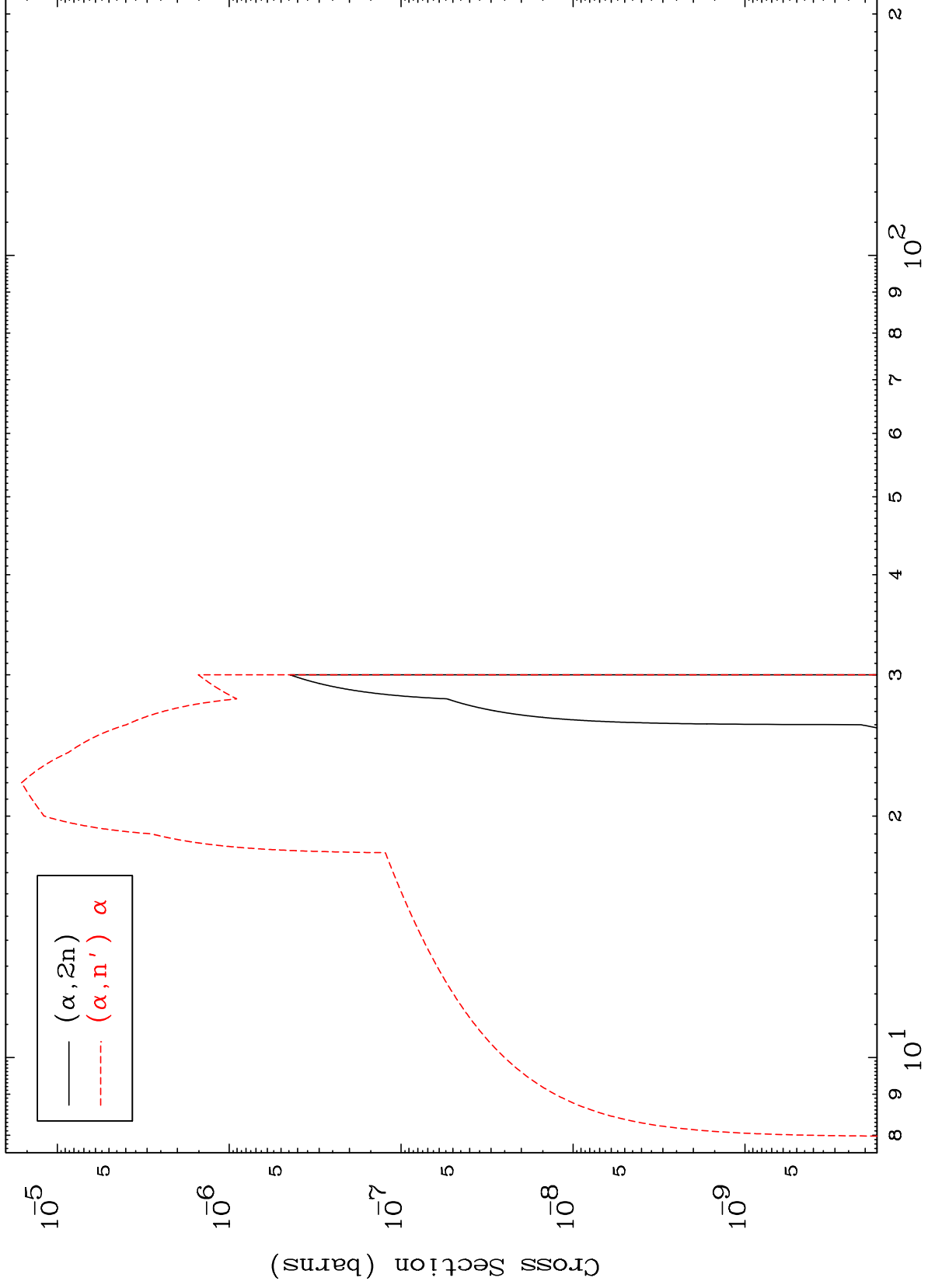
Incident Energy (MeV)

101-Md-249

MAT 9950

α Neutron Production
0 Kelvin Cross Sections

101-Md-249



— $(\alpha, 2n)$
- - - (α, n') α

Incident Energy (MeV)

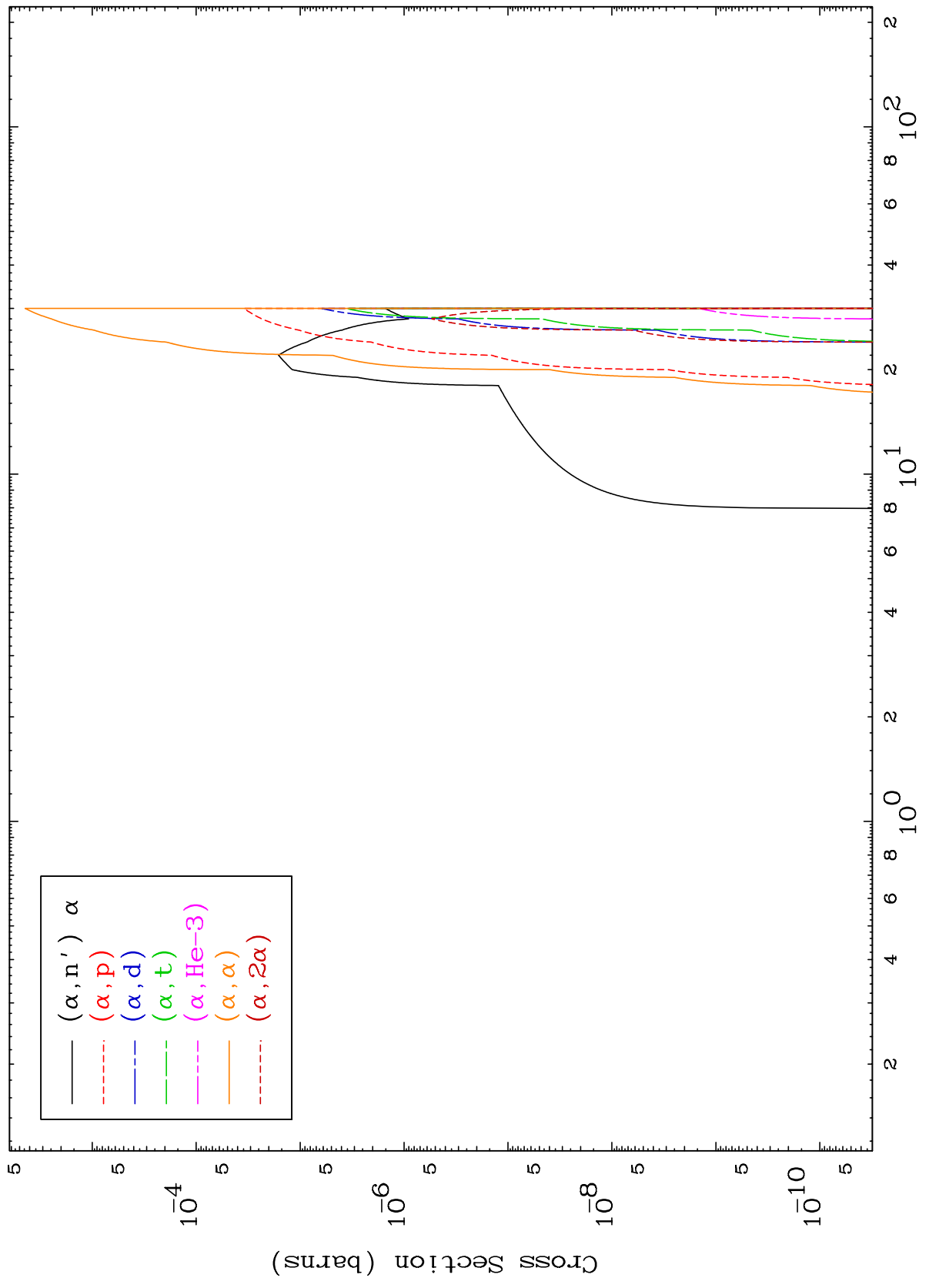
101-Md-249

2

MAT 9950

α Charged Particle
0 Kelvin Cross Sections

101-Md-249

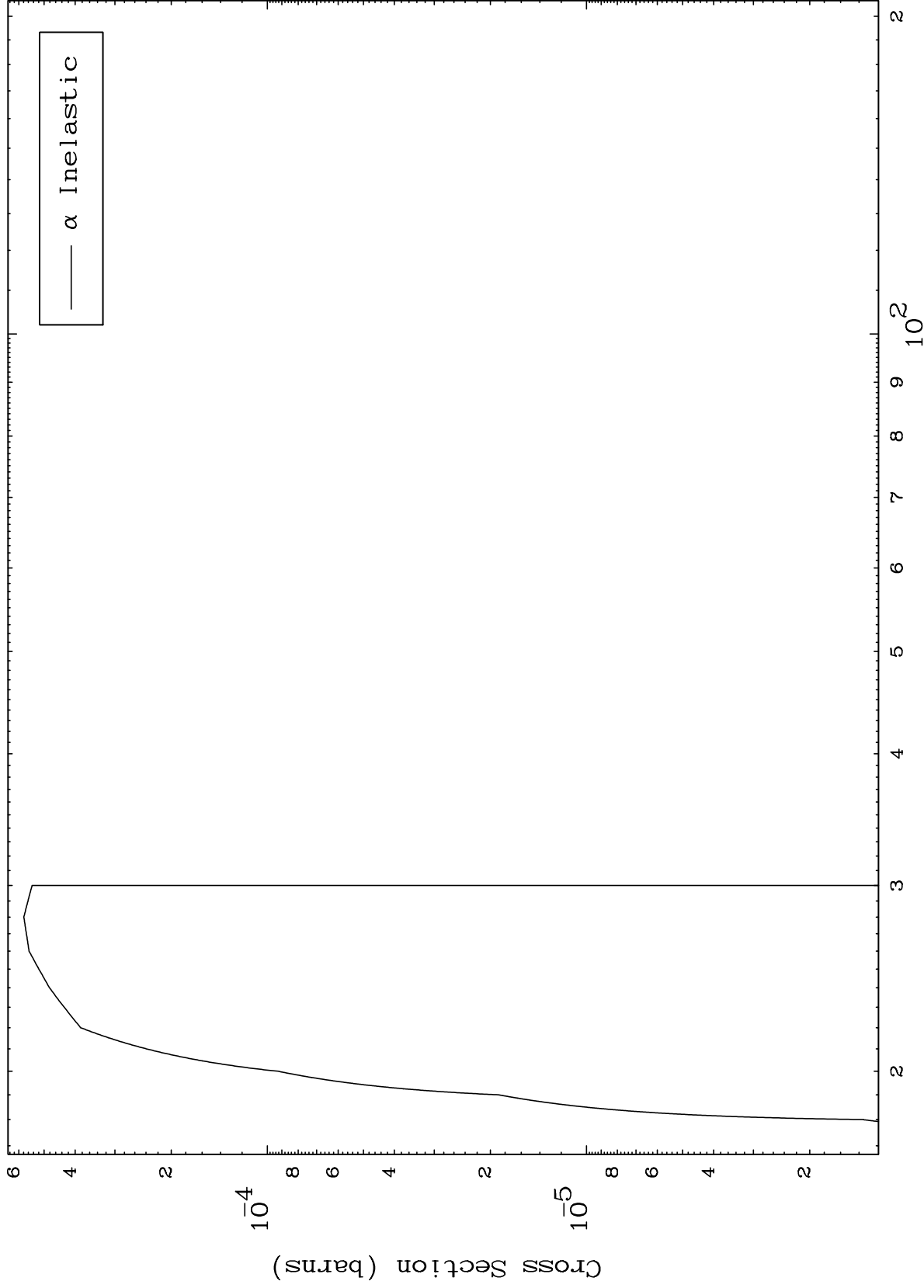


MAT 9950

(α, n') Level

101-Md-249

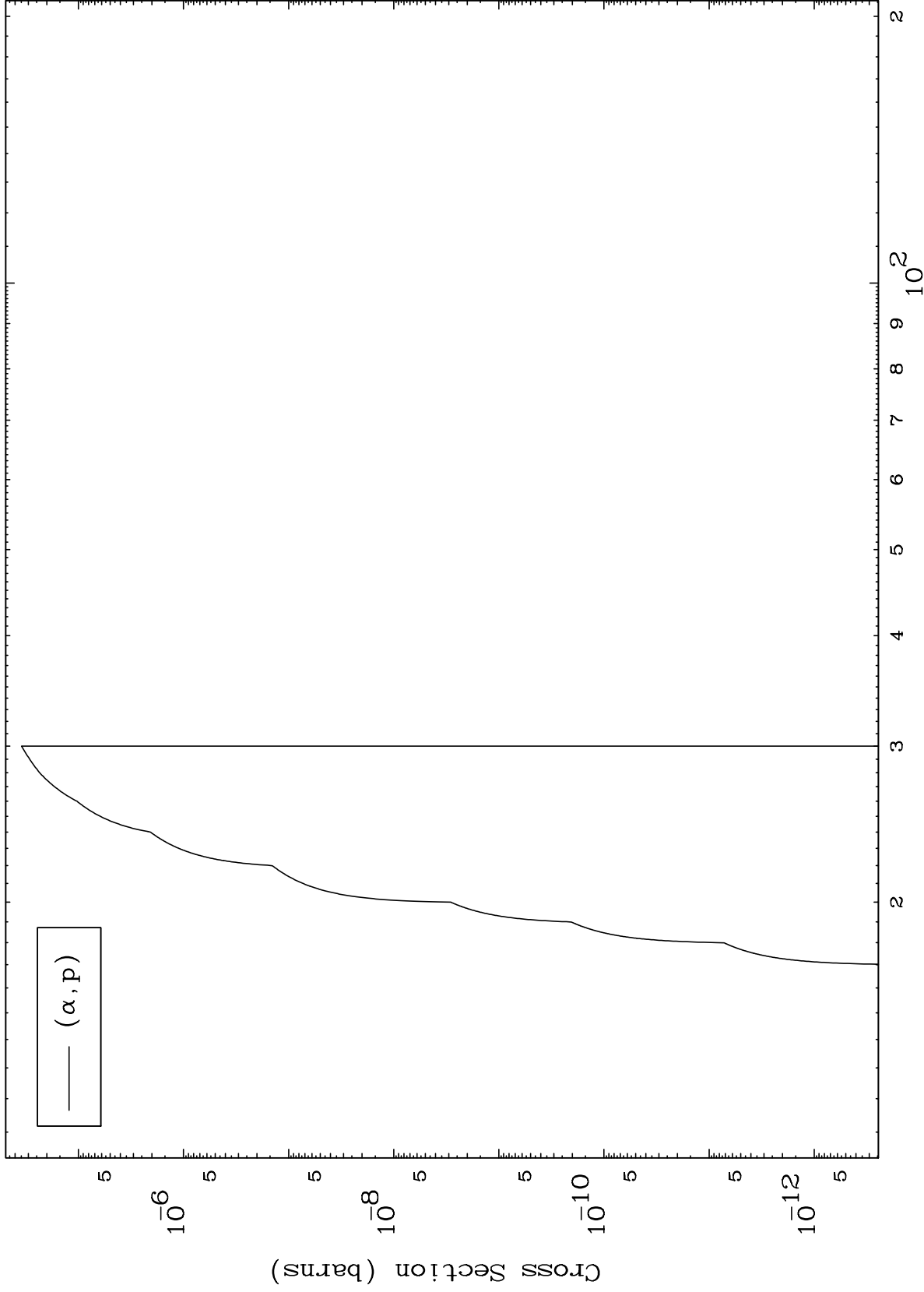
0 Kelvin Cross Sections



4

Incident Energy (MeV)

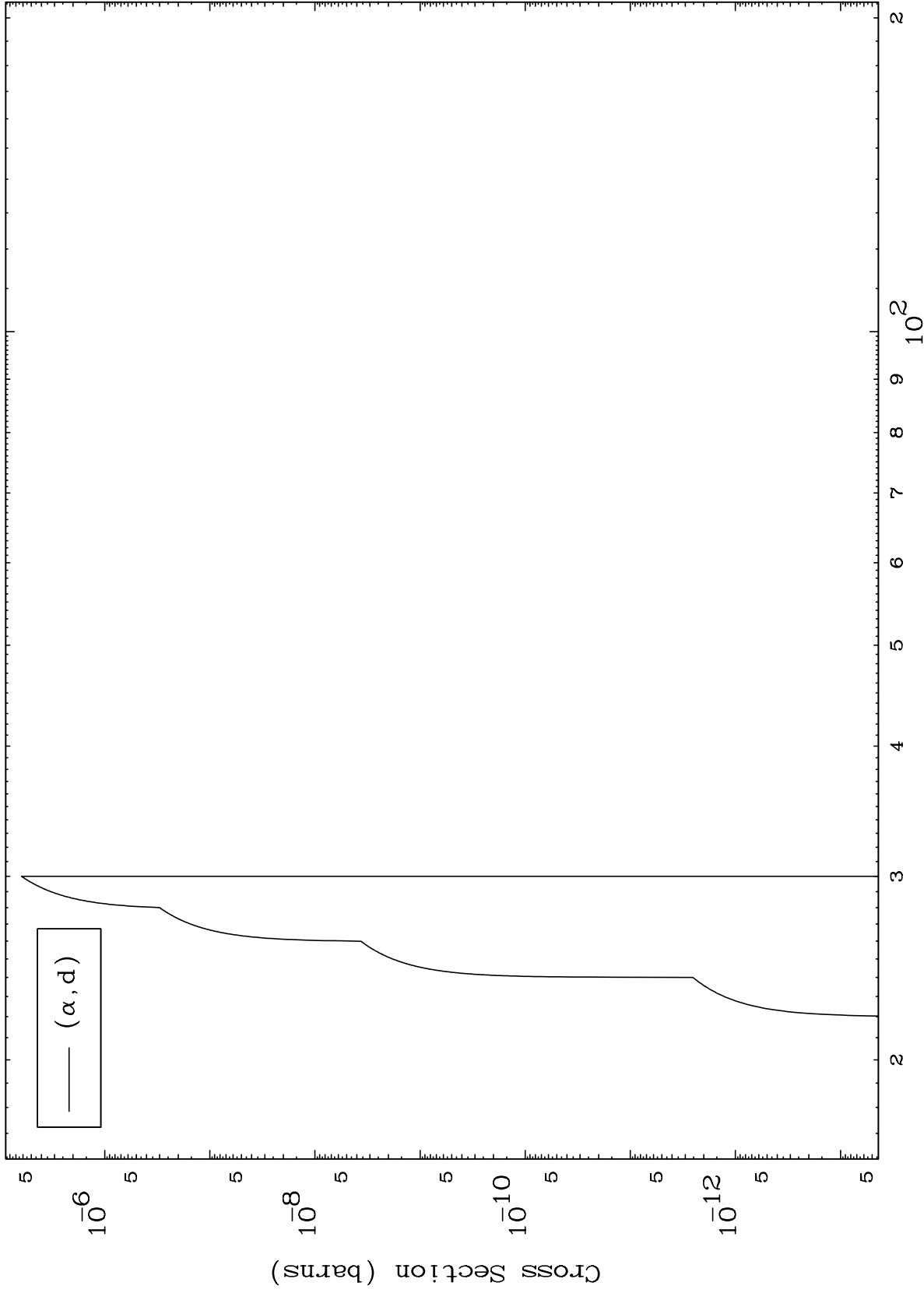
101-Md-249



MAT 9950

(α, d) Levels
0 Kelvin Cross Sections

101-Md-249



6

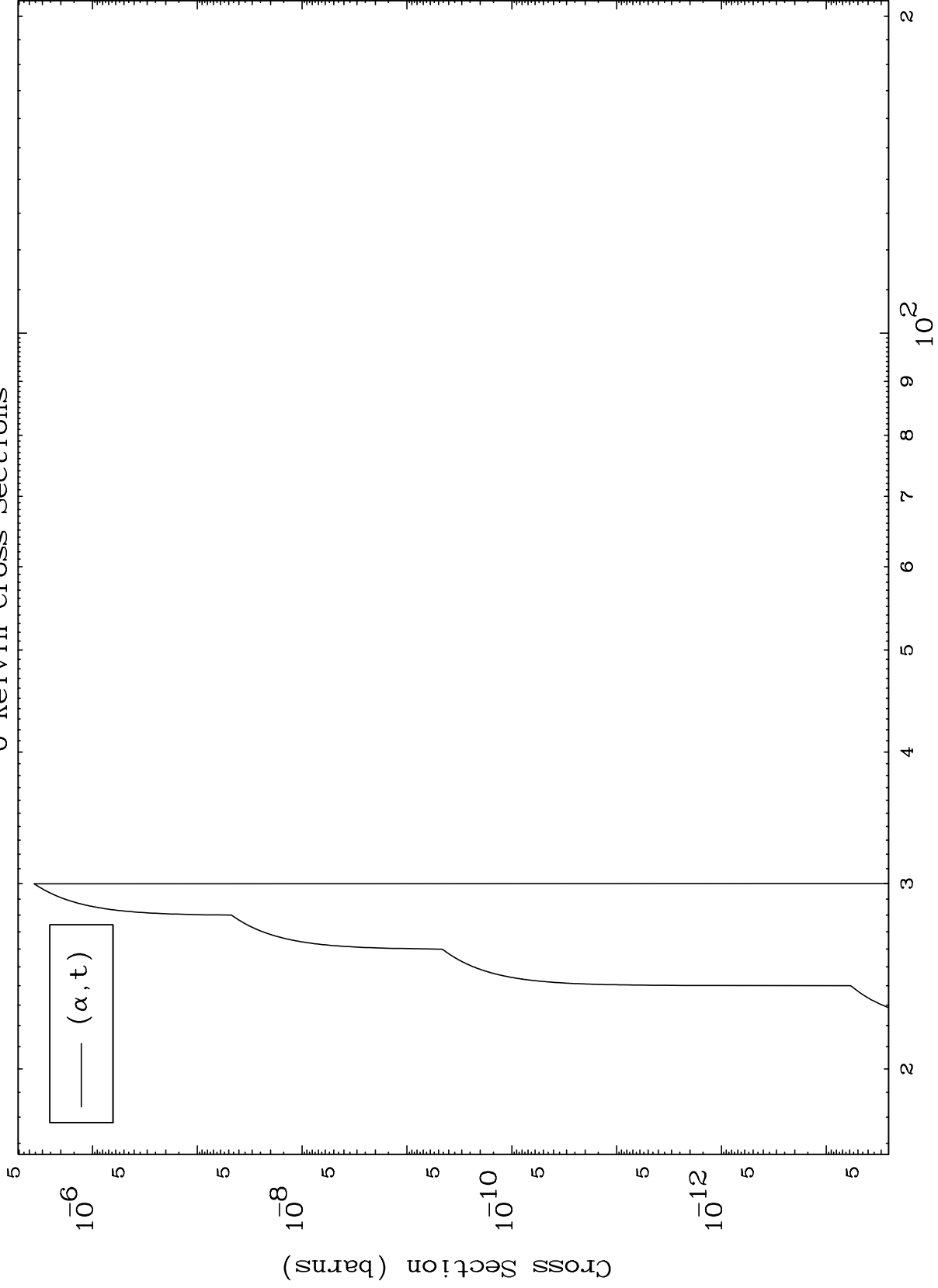
Incident Energy (MeV)

101-Md-249

MAT 9950

(α, t) Levels
0 Kelvin Cross Sections

101-Md-249

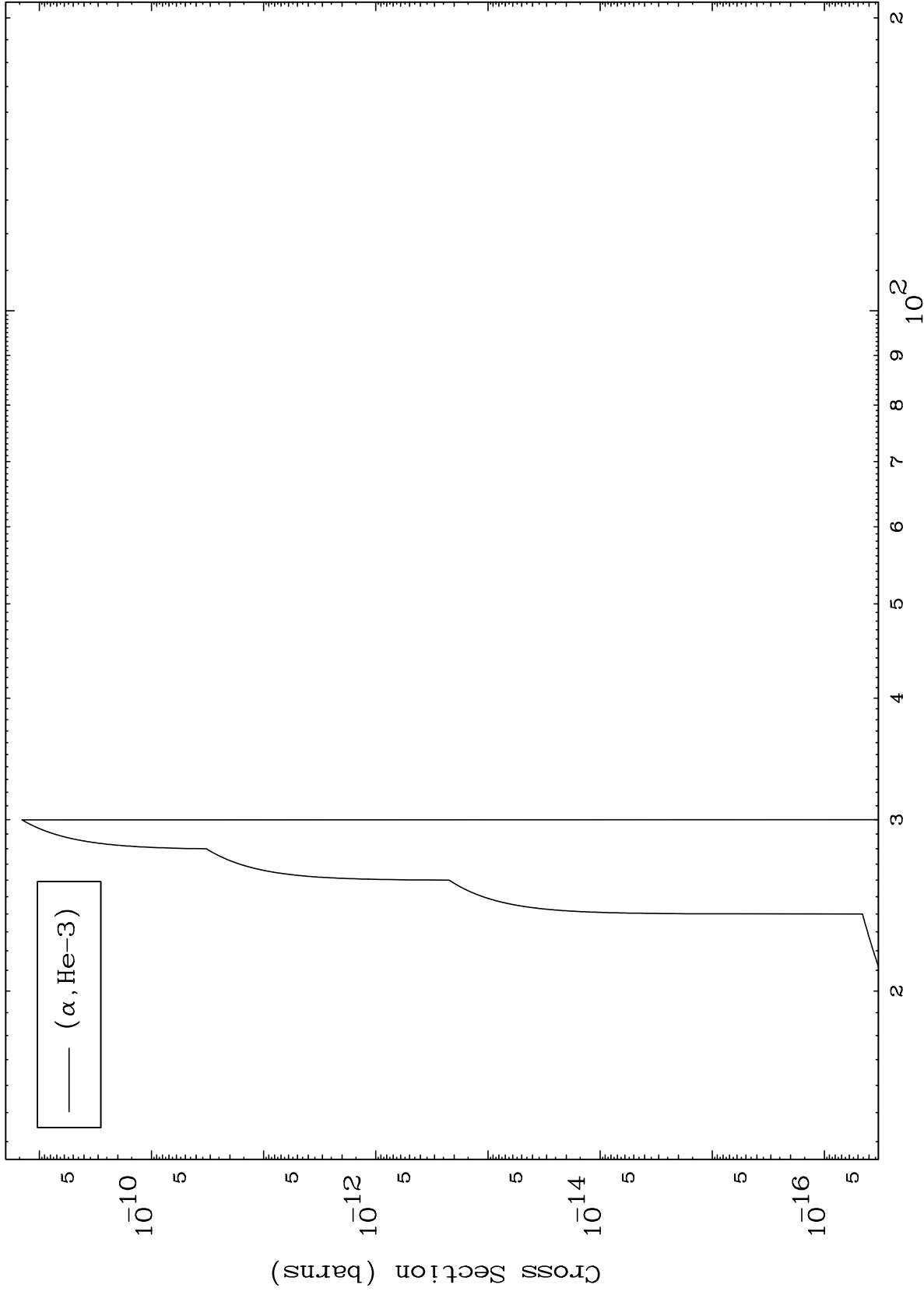


7

101-Md-249

101-Md-249

($\alpha, \text{He}3$) Levels
0 Kelvin Cross Sections

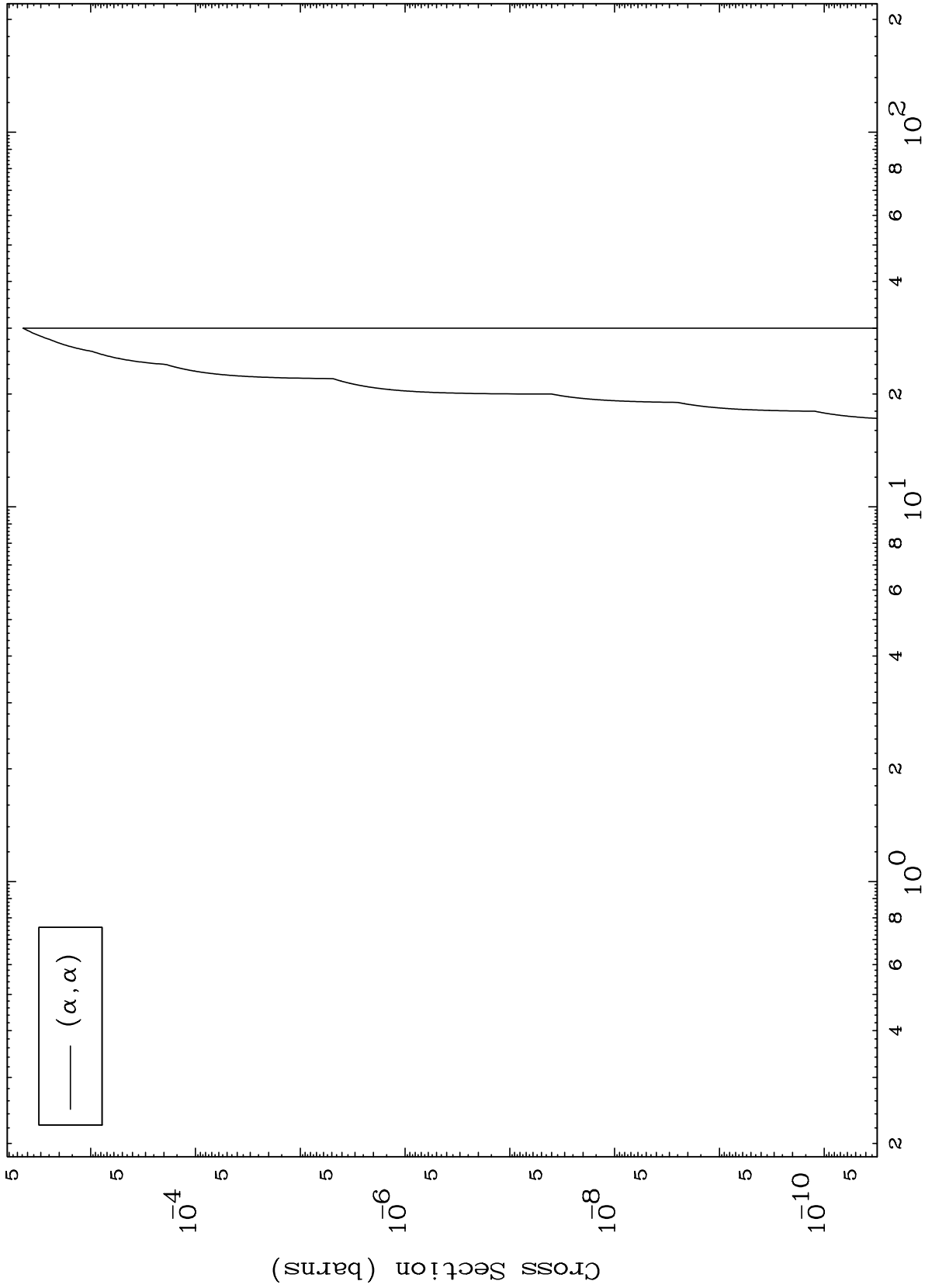


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(α, α) Levels

101-Md-249

0 Kelvin Cross Sections



9

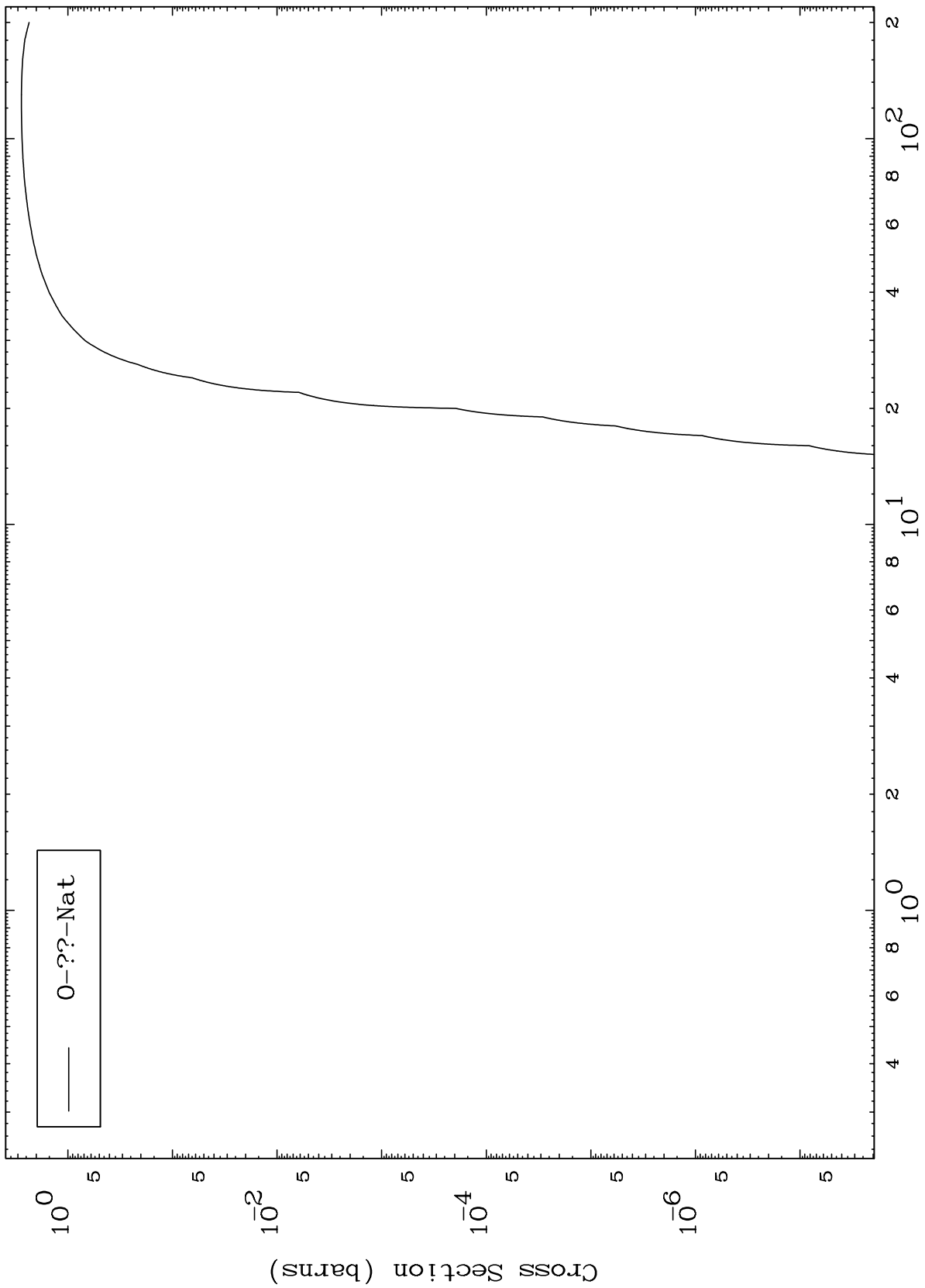
Incident Energy (MeV)

101-Md-249

MAT 9950

101-Md-249

α Fission
Radionuclide Production Cross Section

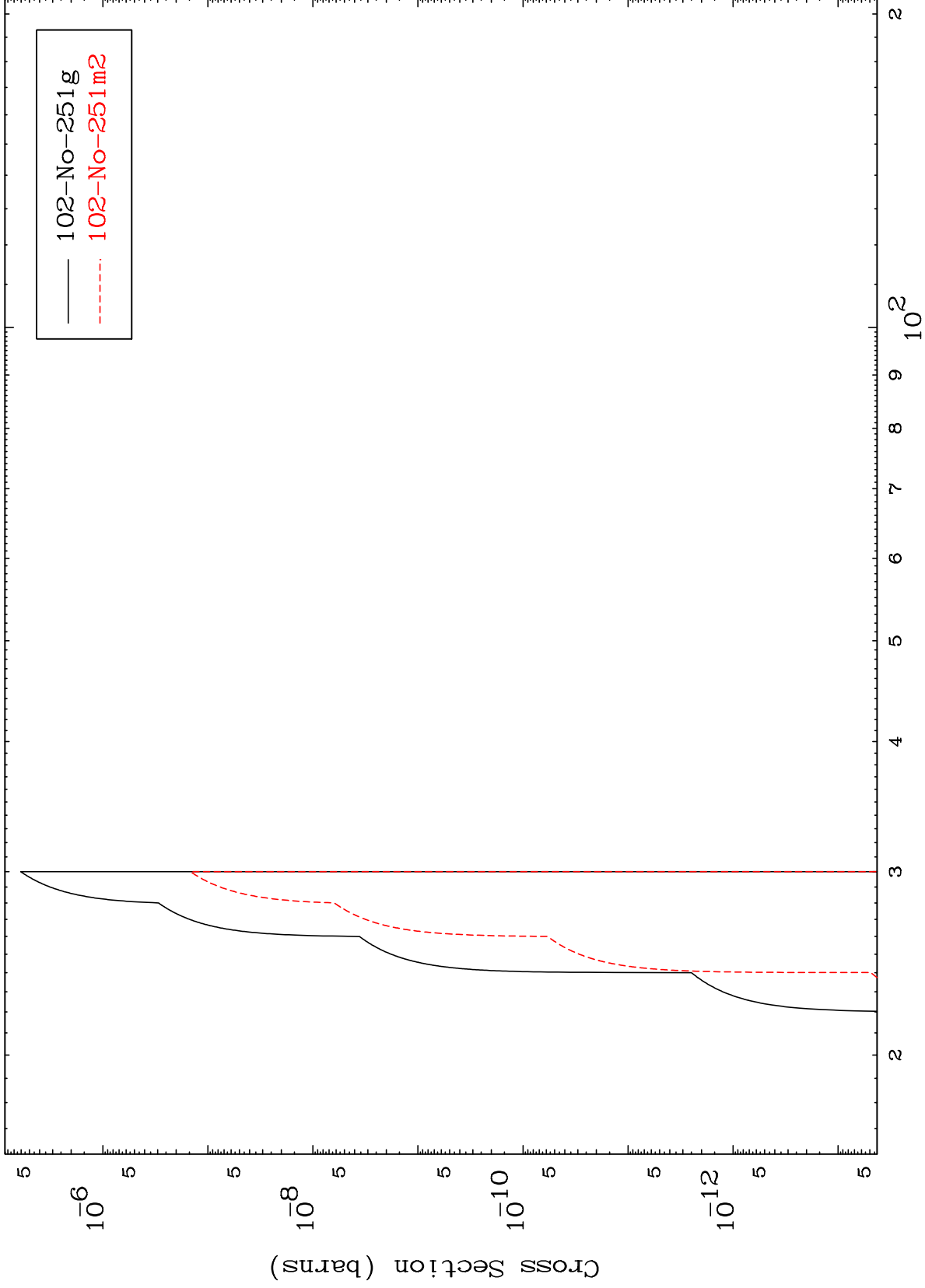


101-Md-249

Incident Energy (MeV)

10

Radionuclide Production Cross Section

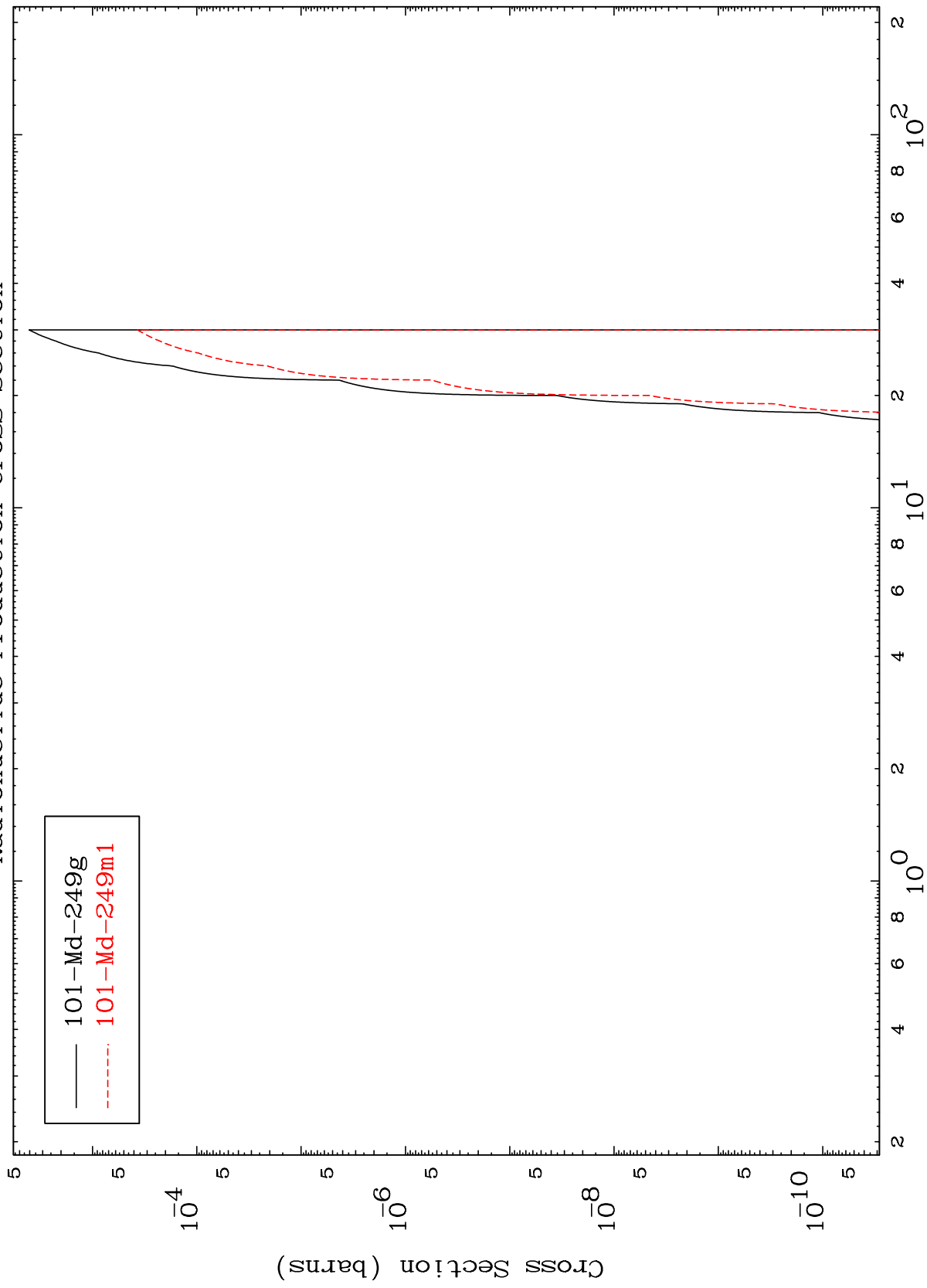


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101-Md-249

(α, α)

Radionuclide Production Cross Section



101-Md-249

Incident Energy (MeV)

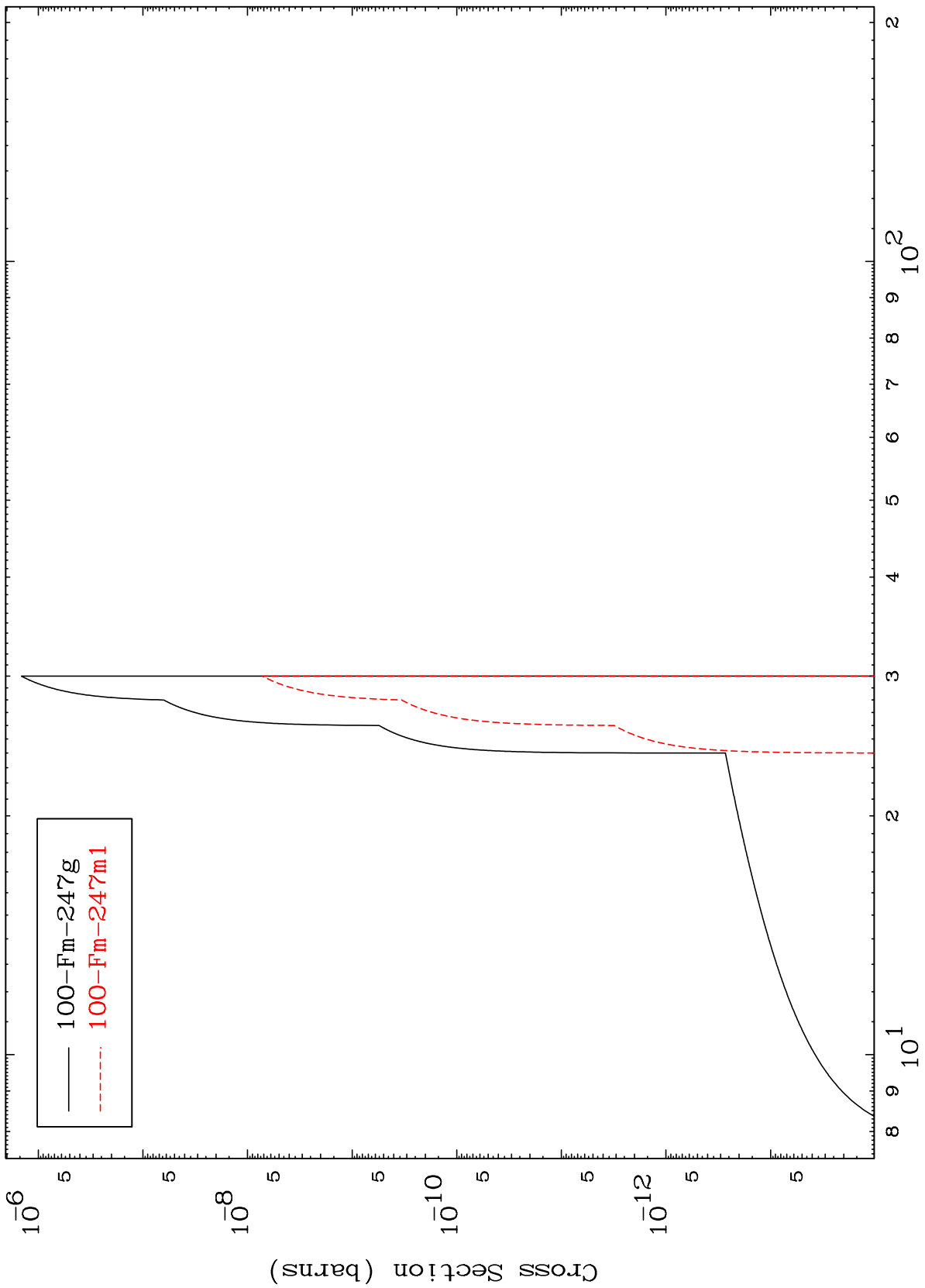
12

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(α, d) α

101-Md-249

Radionuclide Production Cross Section



13

Incident Energy (MeV)

101-Md-249