

Program EVALPLOT  
(Version 2018-1)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

E.Mail: [redcullen1@comcast.net](mailto:redcullen1@comcast.net)

Web: [redcullen1.net/HOMEPAGE.NEW](http://redcullen1.net/HOMEPAGE.NEW)

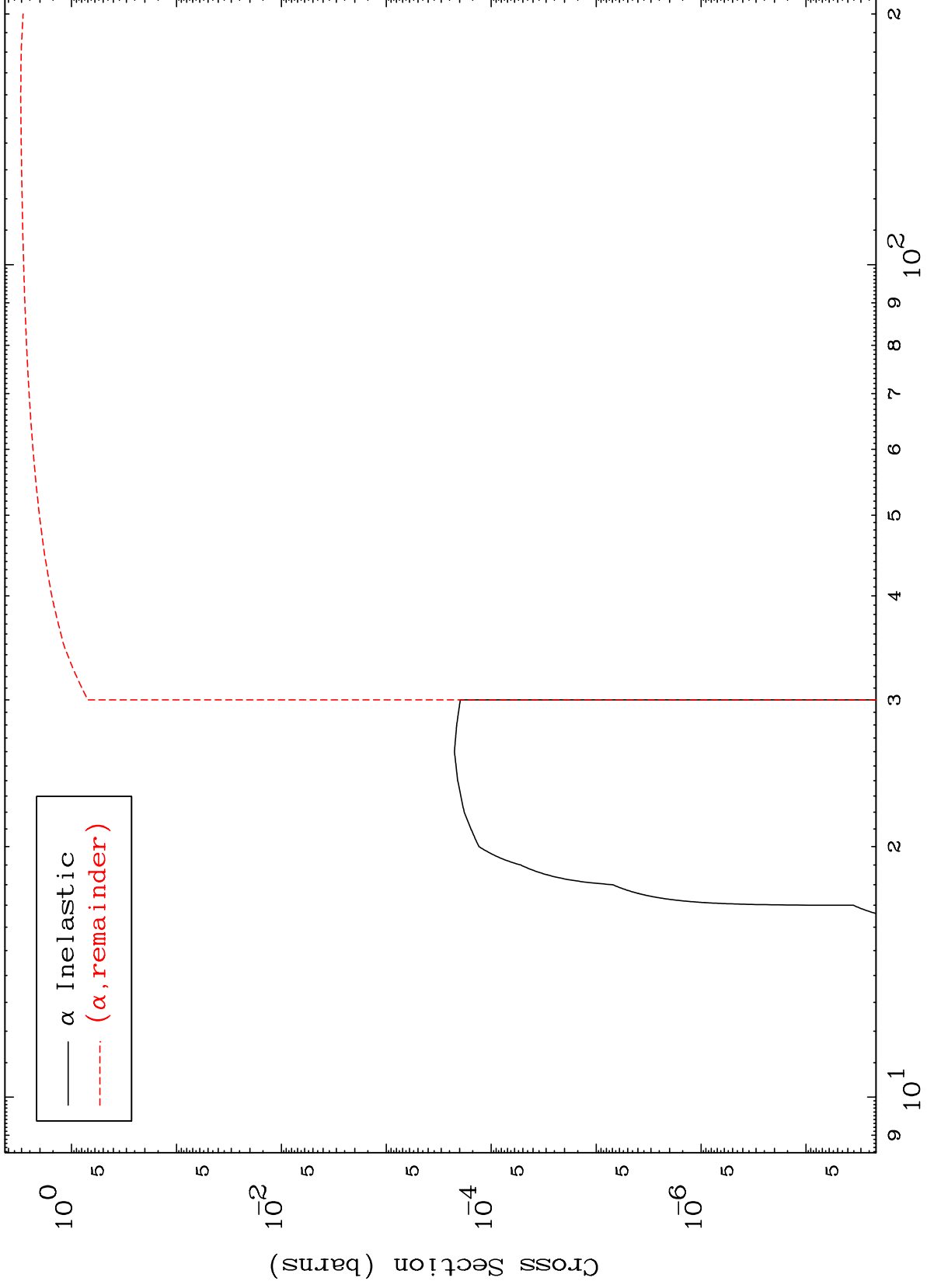
Press Mouse Button to Start

MAT 9951

$\alpha$  Major

101-Md-250

0 Kelvin Cross Sections



Incident Energy (MeV)

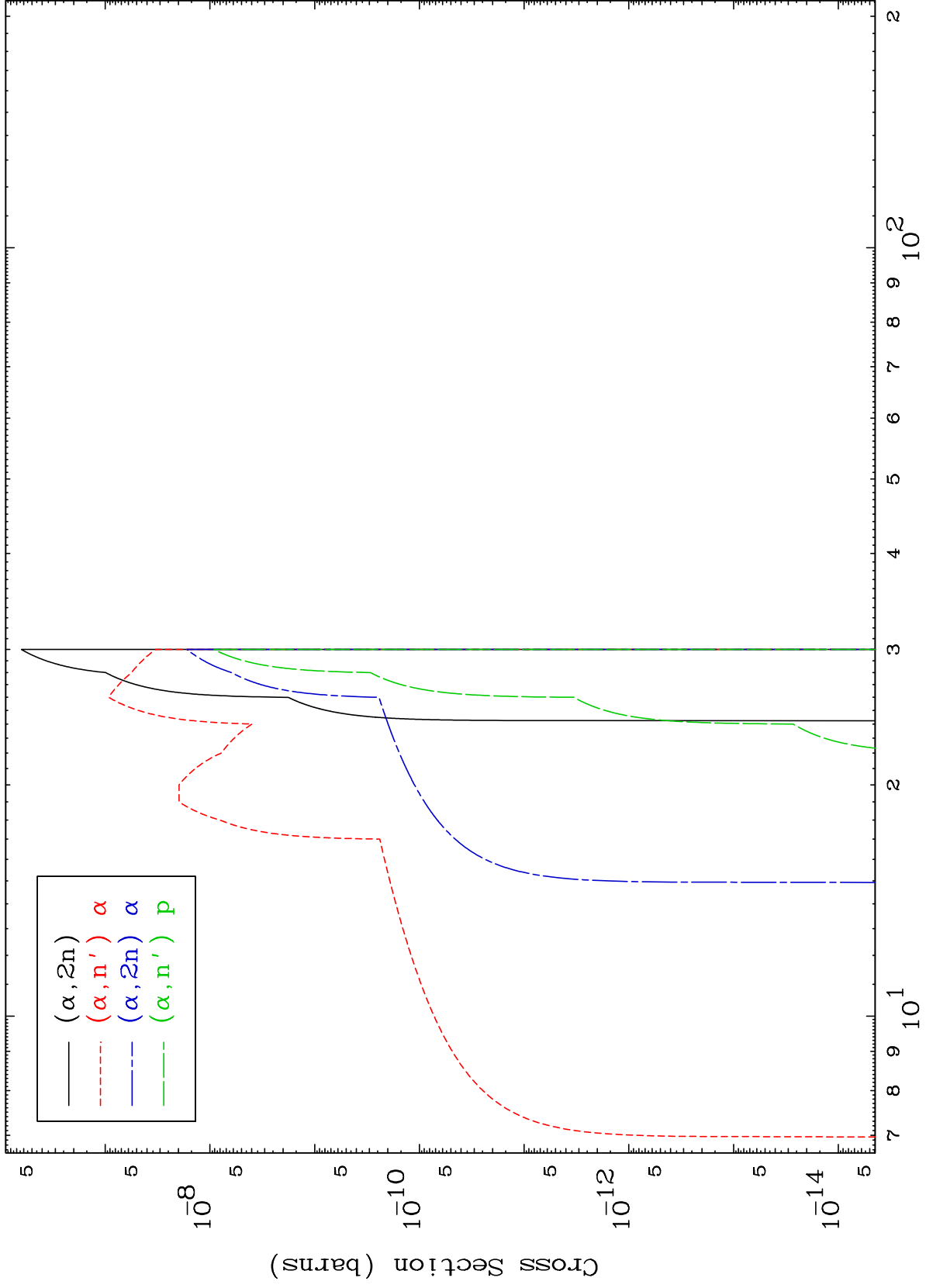
101-Md-250

1

MAT 9951

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

101-Md-250



2

Incident Energy (MeV)

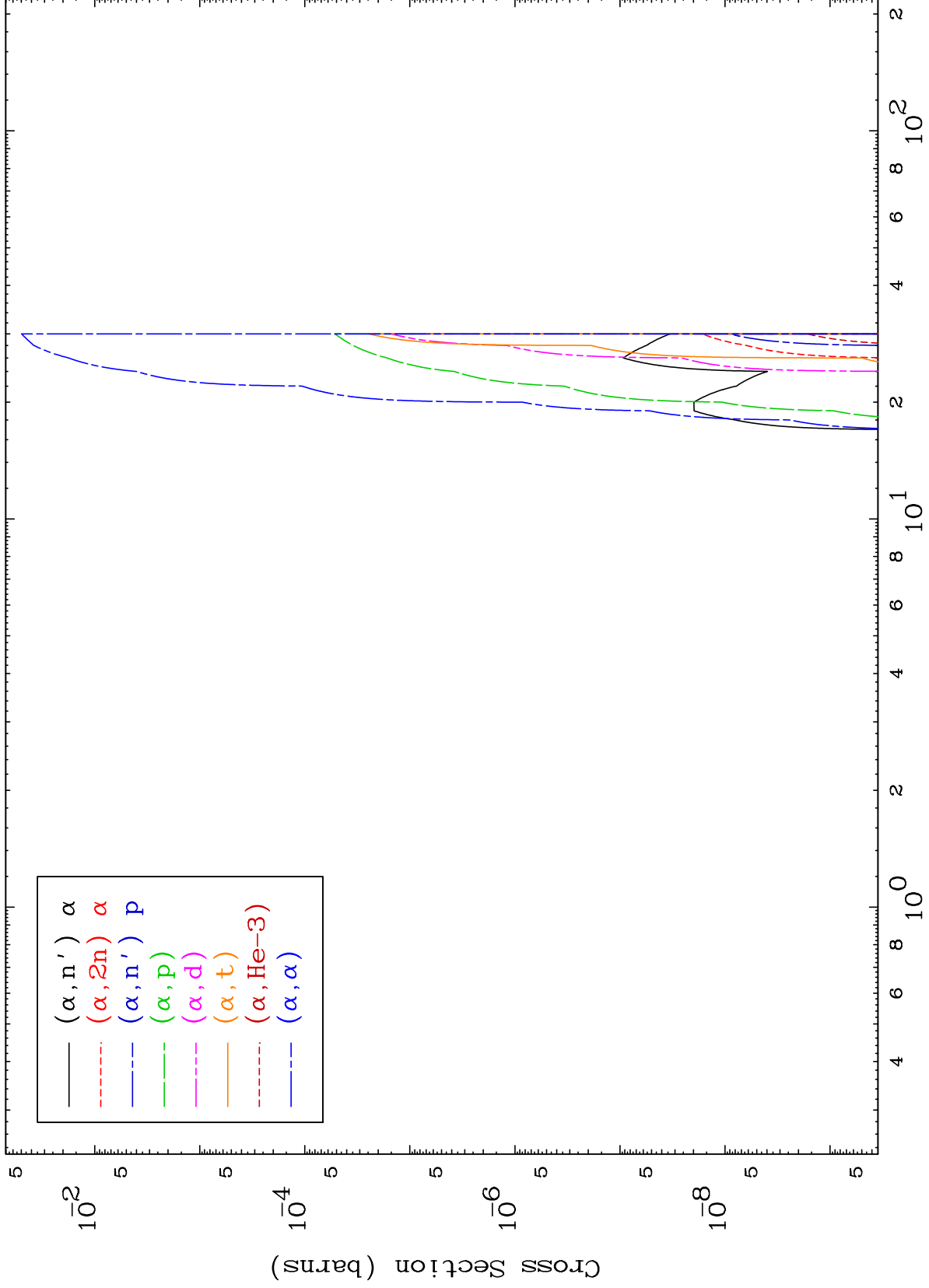
101-Md-250

MAT 9951

$\alpha$  Charged Particle

101-Md-250

0 Kelvin Cross Sections

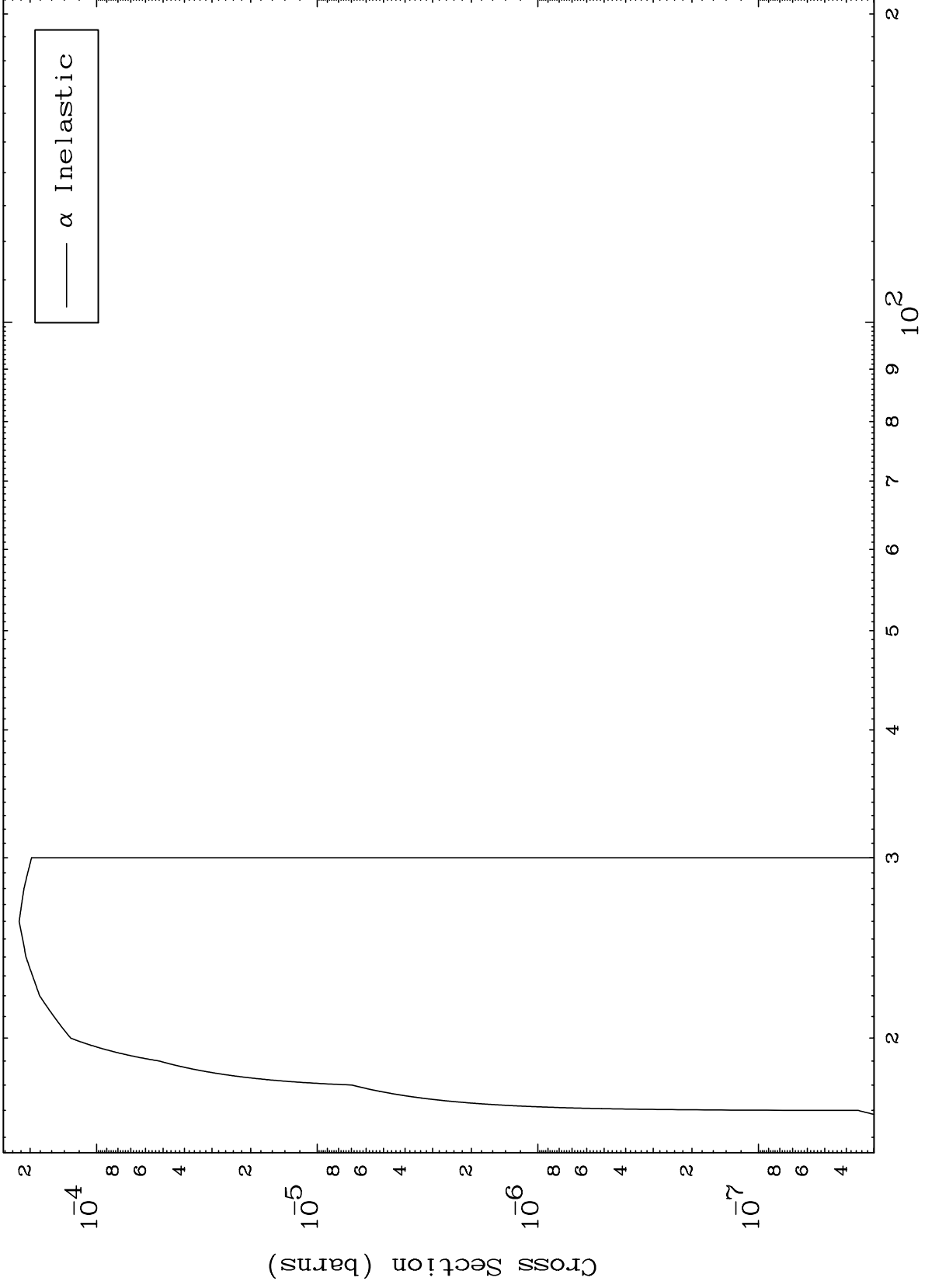


MAT 9951

( $\alpha, n'$ ) Level

101-Md-250

0 Kelvin Cross Sections



4

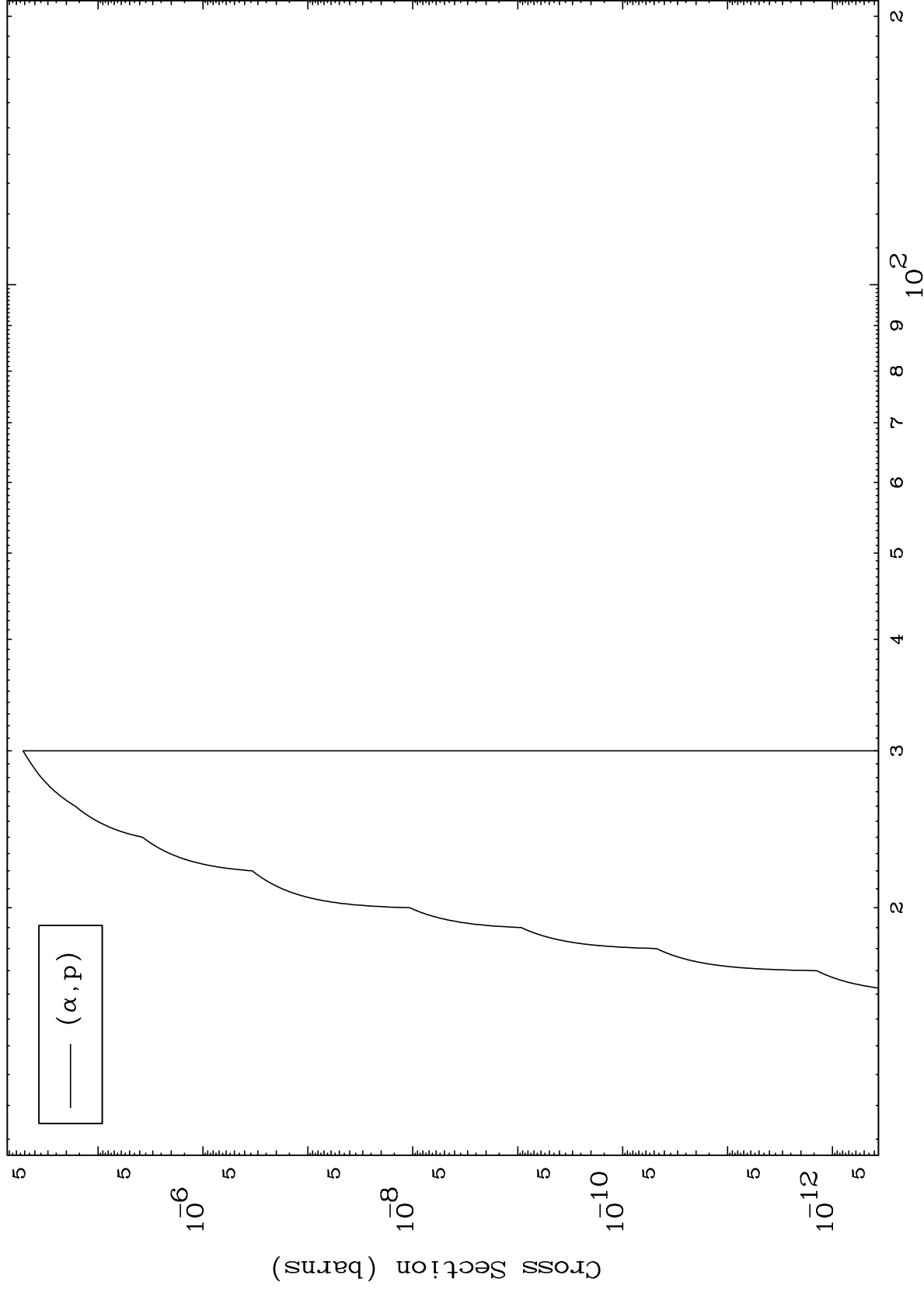
Incident Energy (MeV)

101-Md-250

MAT 9951

101-Md-250

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections



101-Md-250

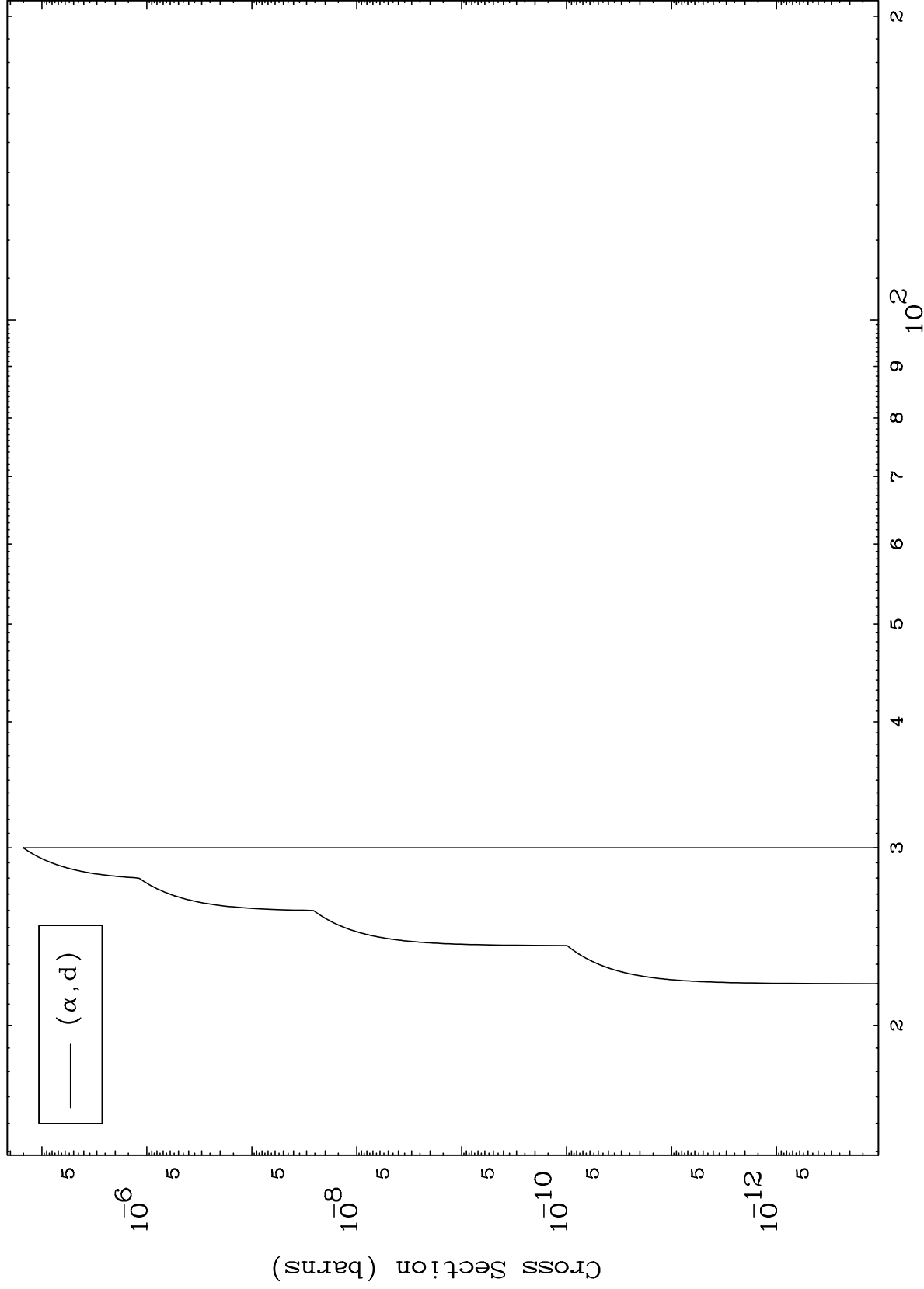
Incident Energy (MeV)

5

MAT 9951

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

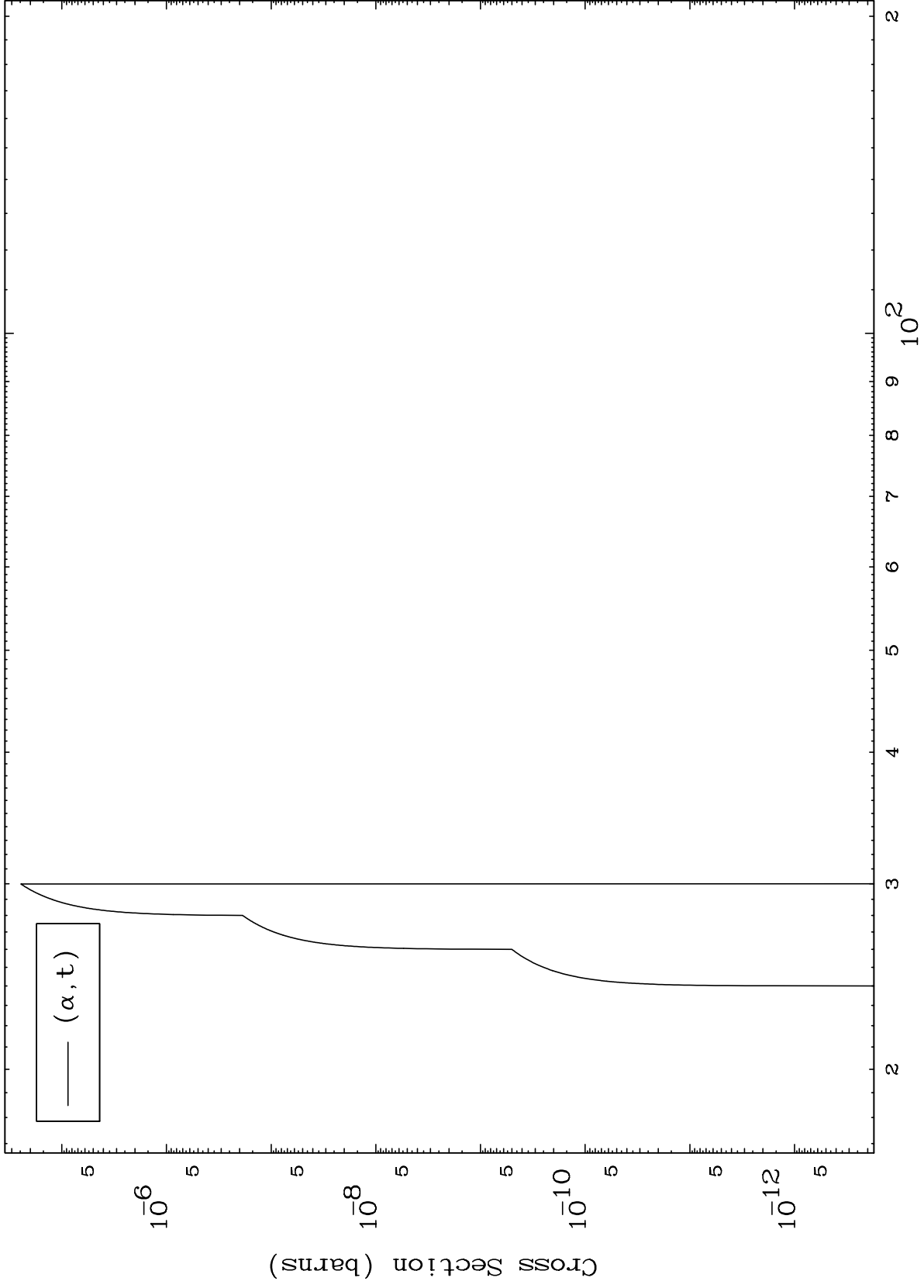
101-Md-250



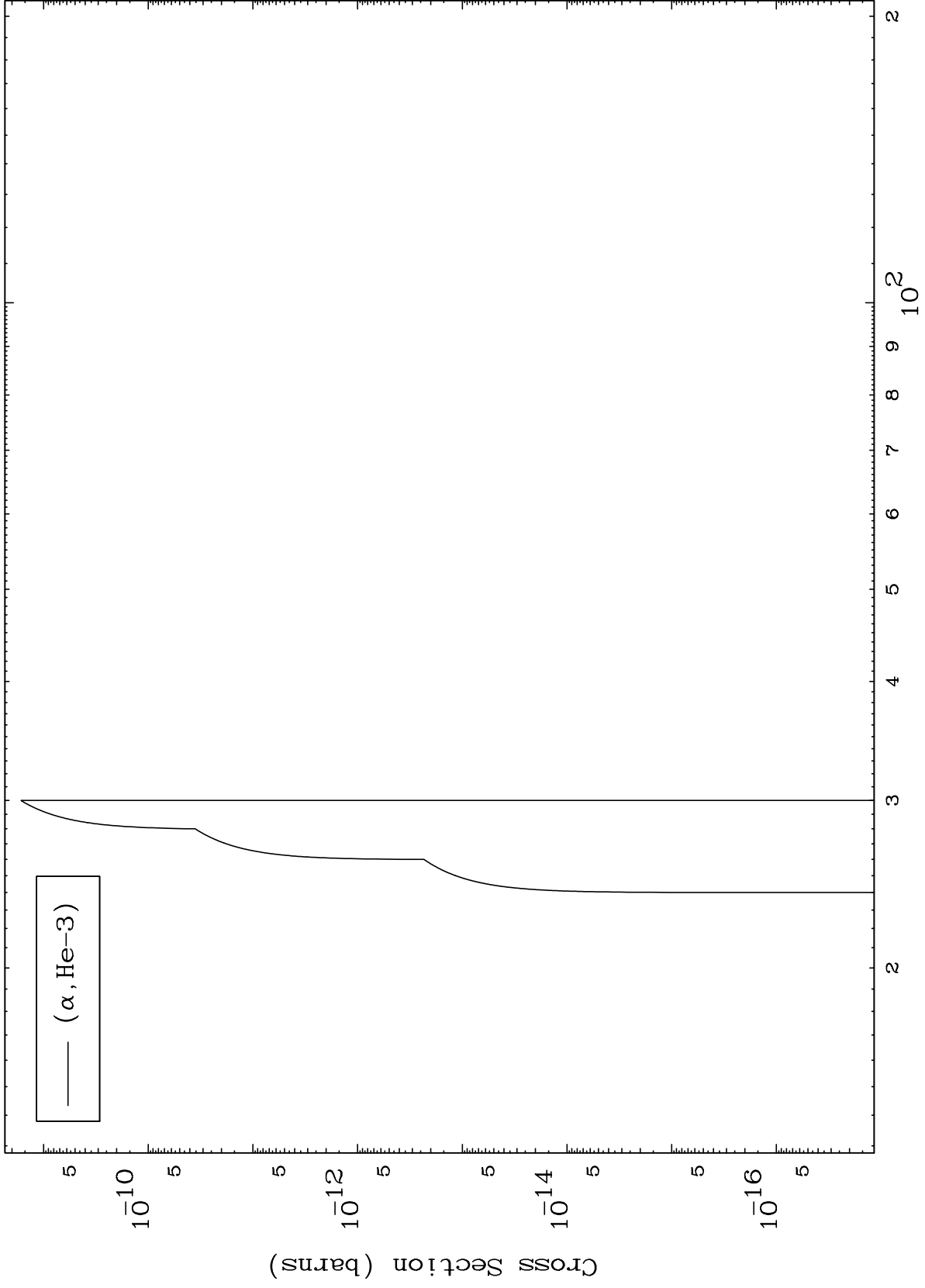
6

Incident Energy (MeV)

101-Md-250





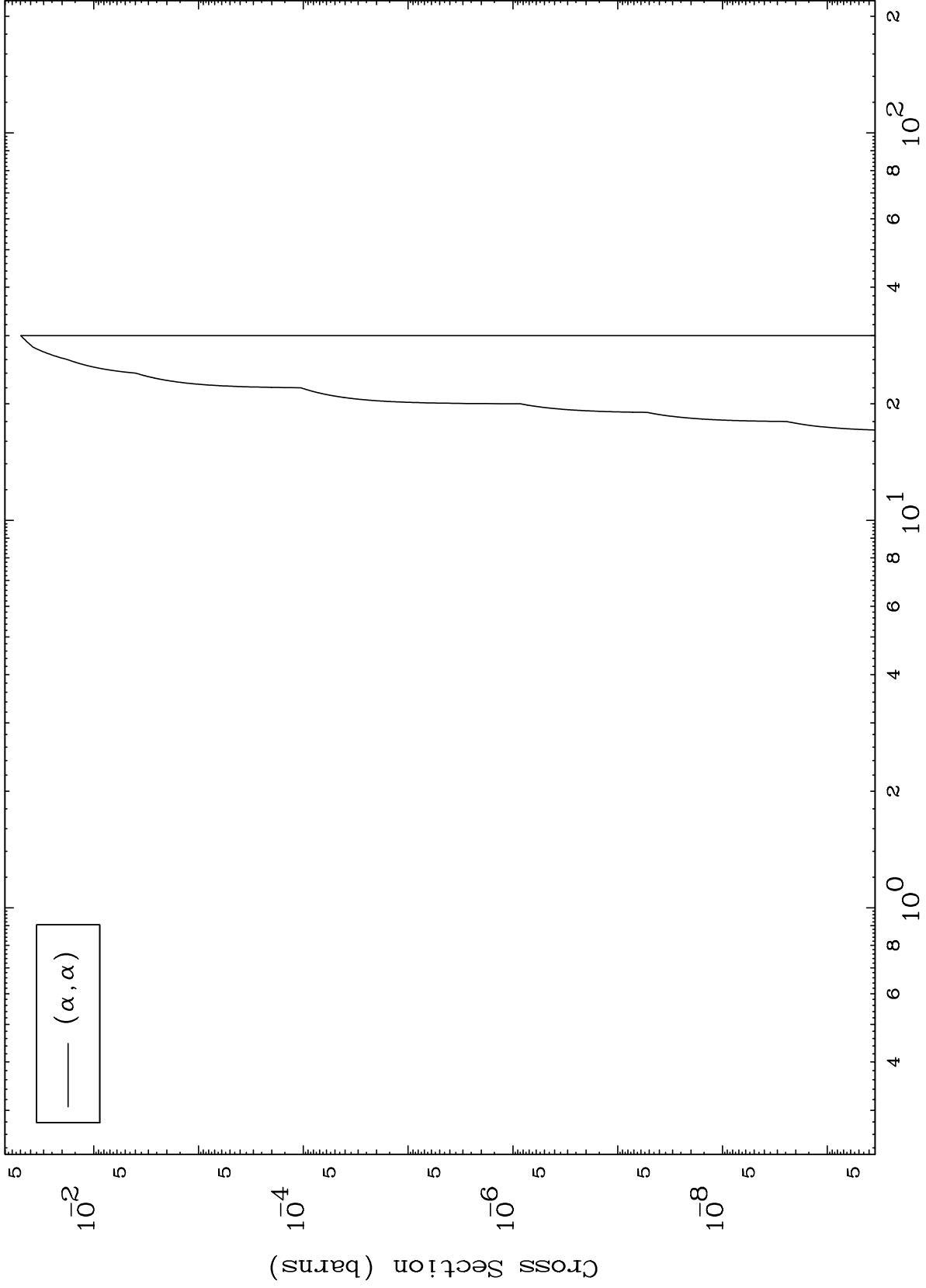


MAT 9951

( $\alpha, \alpha$ ) Levels

101-Md-250

0 Kelvin Cross Sections

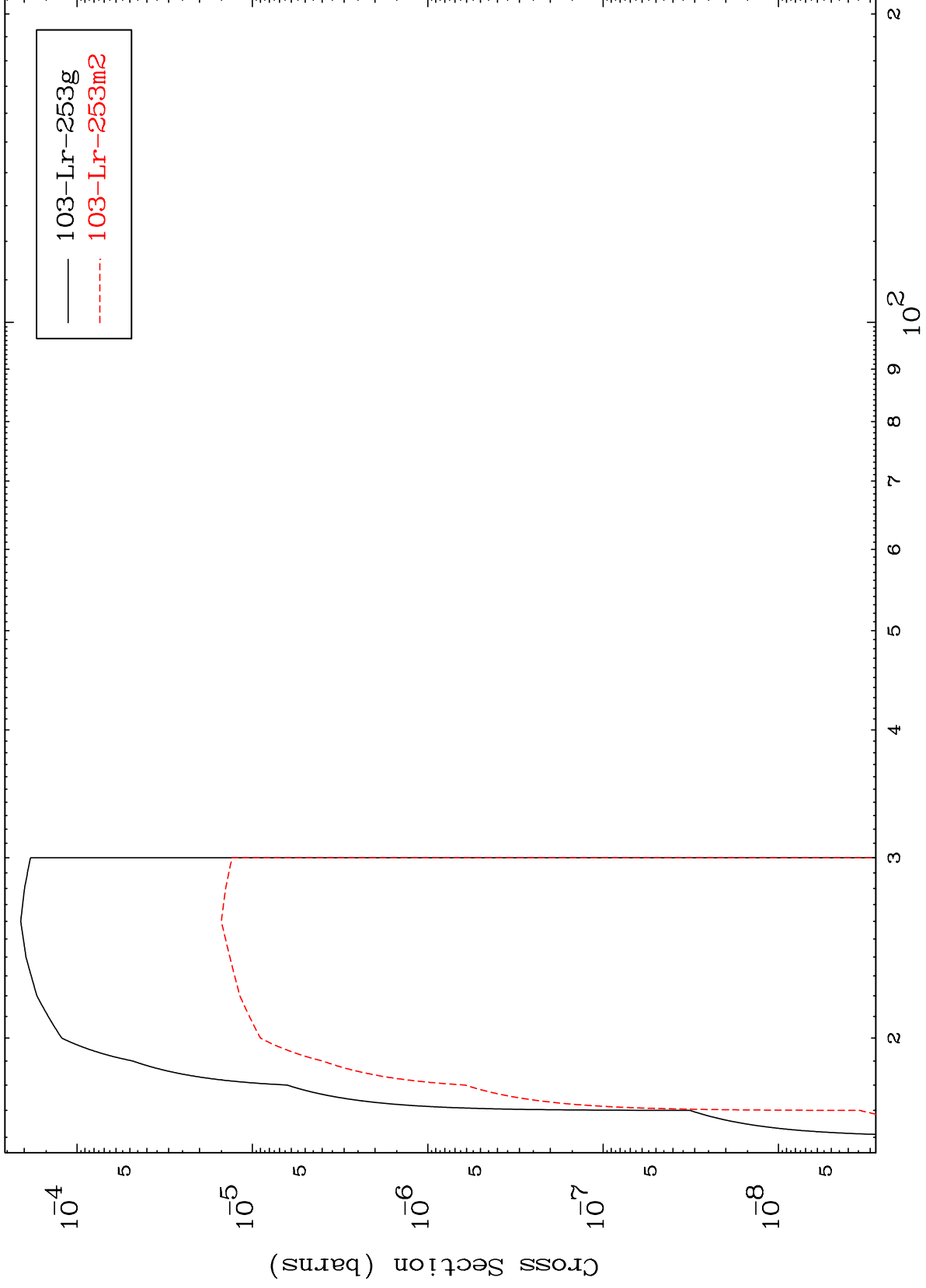


MAT 9951

$\alpha$  Inelastic

101-Md-250

Radionuclide Production Cross Section



10

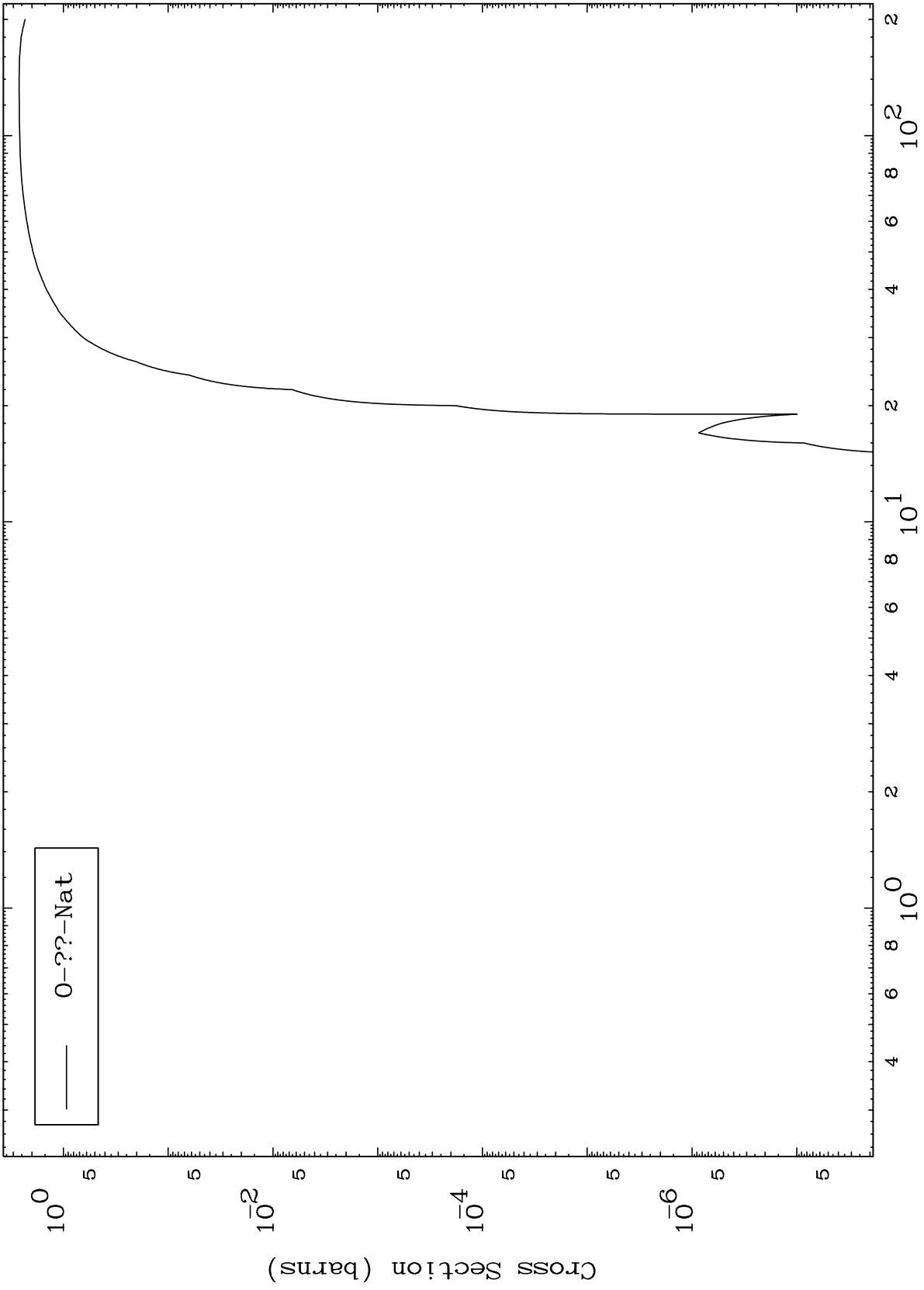
Incident Energy (MeV)

101-Md-250

MAT 9951

101-Md-250

$\alpha$  Fission  
Radionuclide Production Cross Section

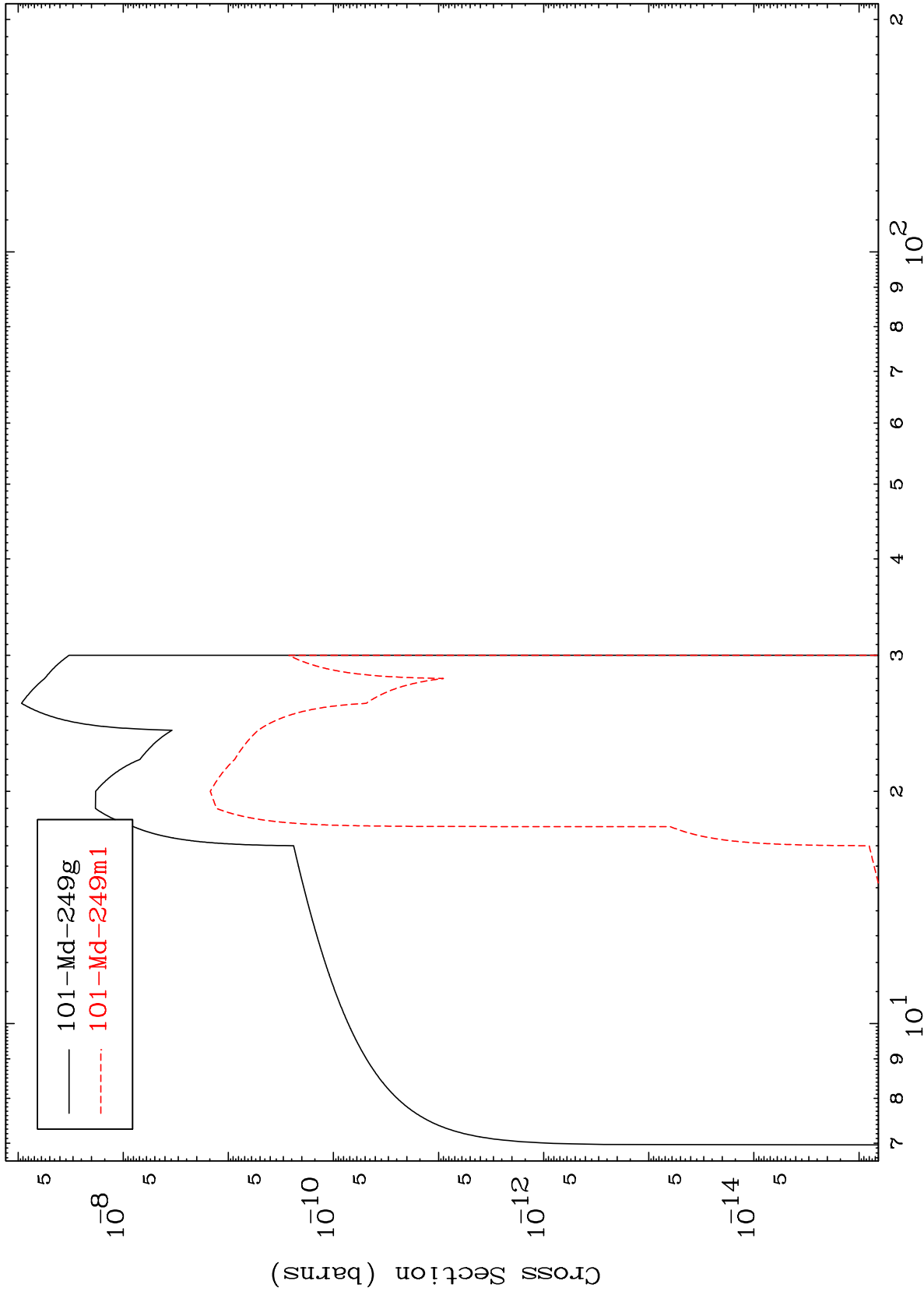


MAT 9951

101-Md-250

$(\alpha, n')$   $\alpha$

Radionuclide Production Cross Section



12

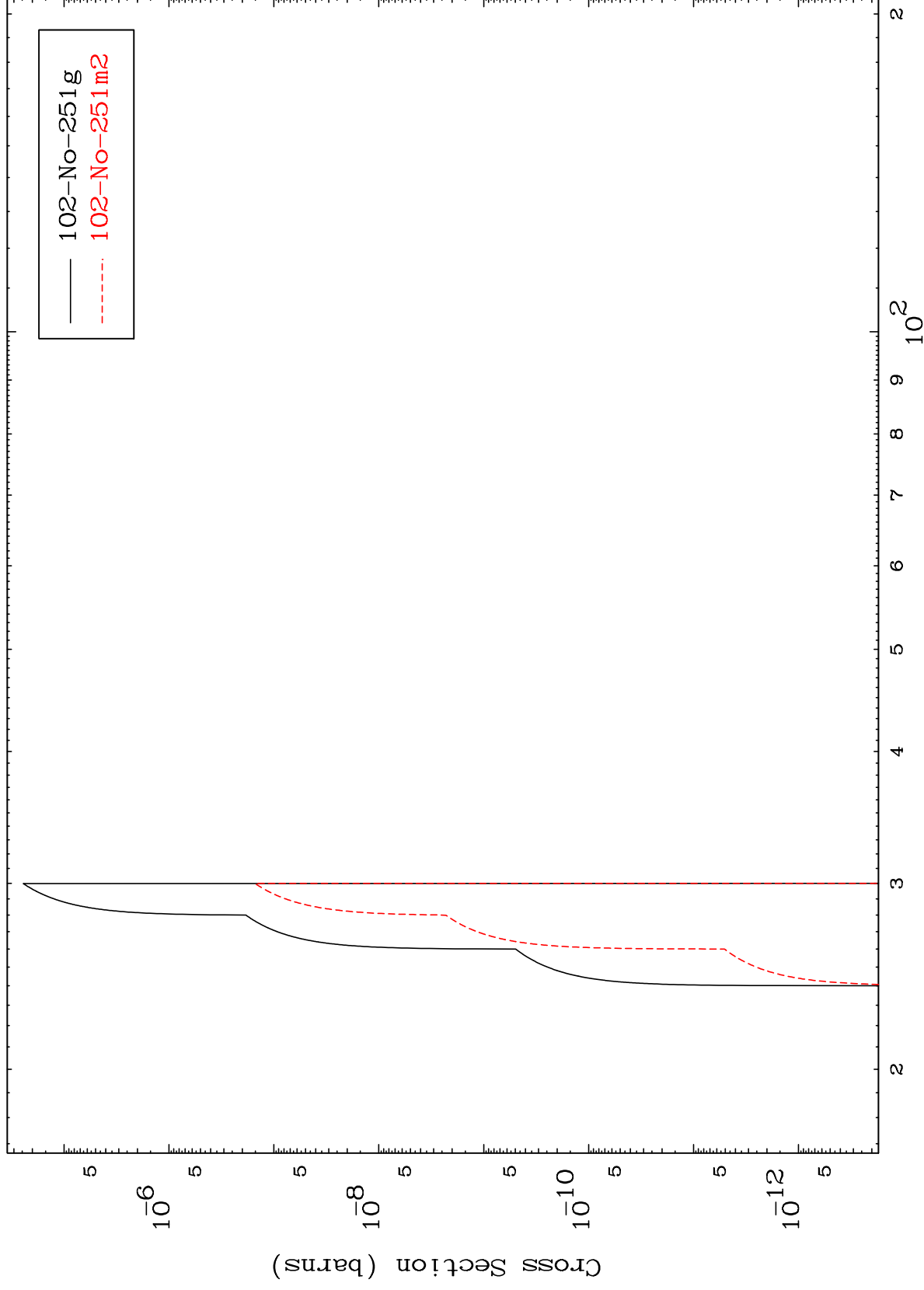
Incident Energy (MeV)

101-Md-250

MAT 9951

101-Md-250

( $\alpha, t$ )  
Radionuclide Production Cross Section



13

101-Md-250

Incident Energy (MeV)