

Program EVALPLOT  
(Version 2018-1)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

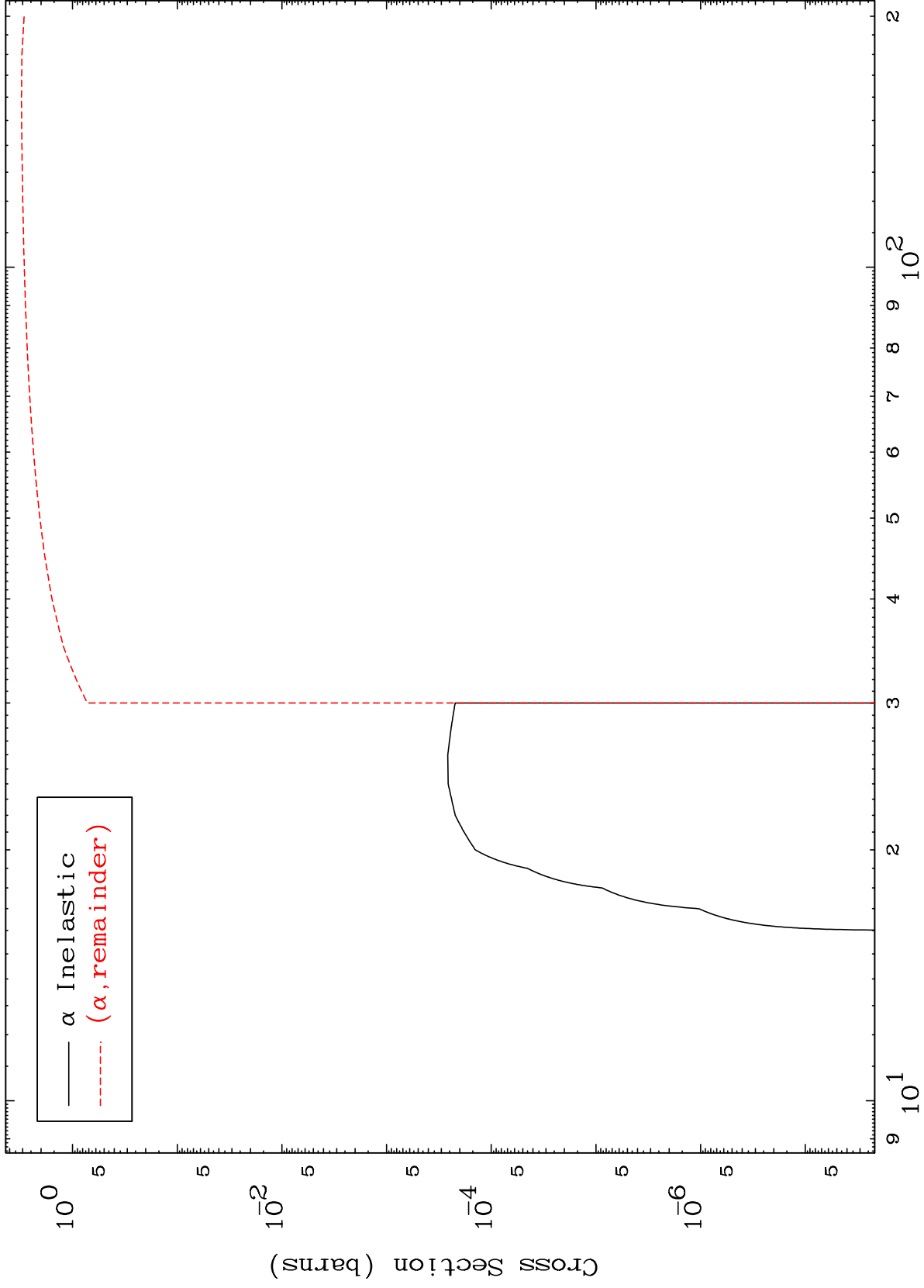
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9955

0 Kelvin  $\alpha$  Major  
Cross Sections

101-Md-254



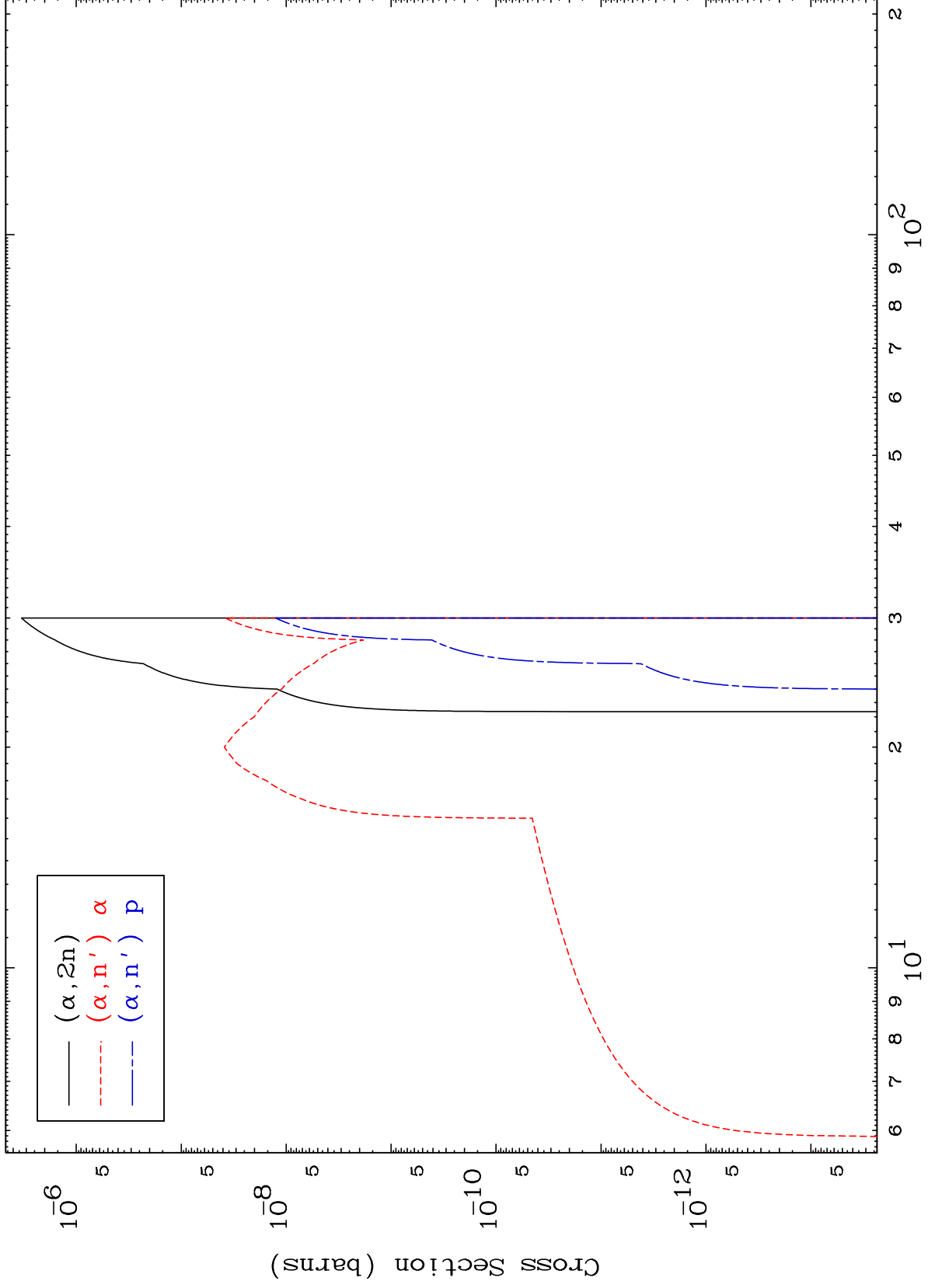
Incident Energy (MeV)

101-Md-254

MAT 9955

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

101-Md-254



( $\alpha, 2n$ )  
( $\alpha, n'$ )  $\alpha$   
( $\alpha, n'$ ) p

2

Incident Energy (MeV)

101-Md-254

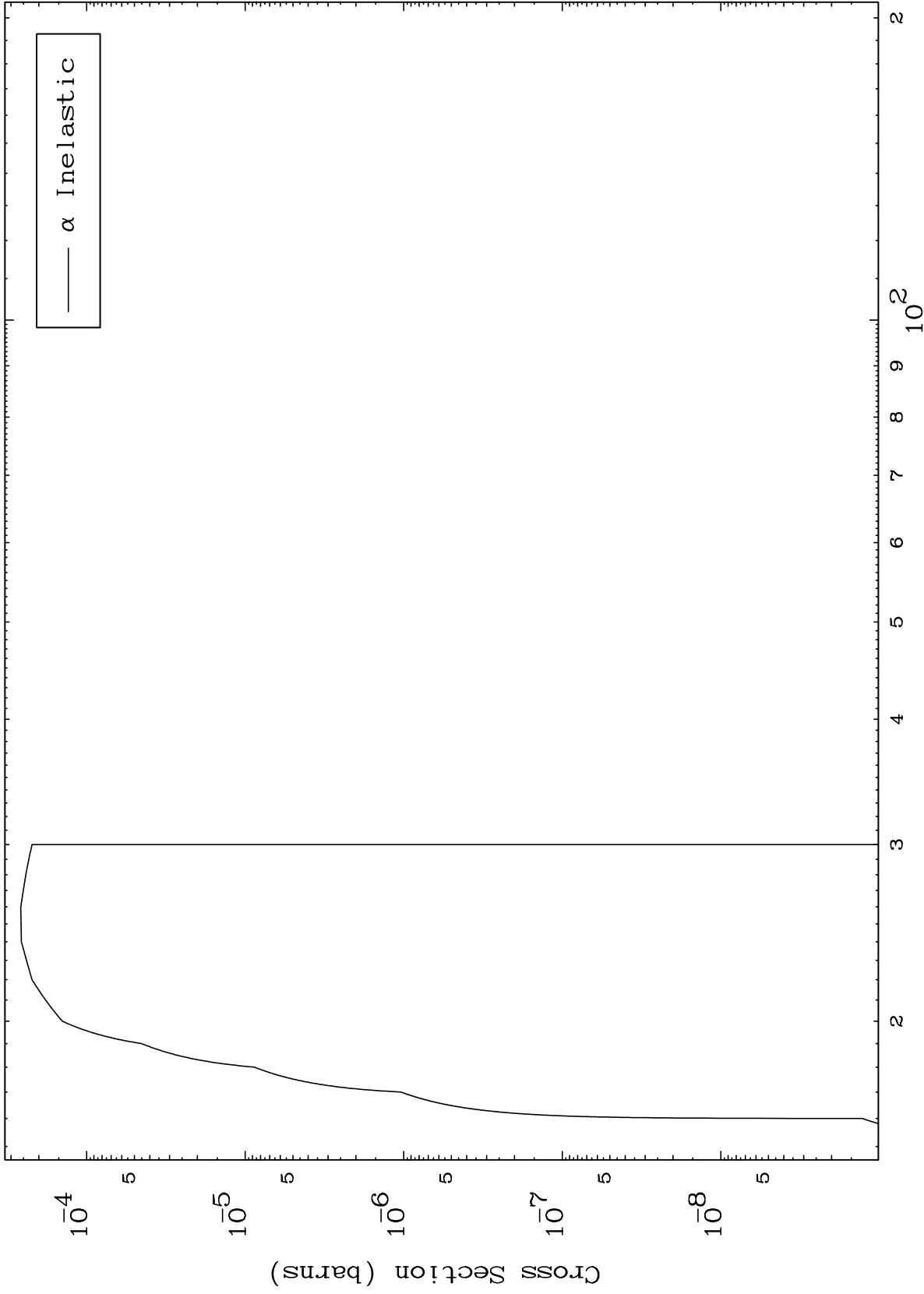


MAT 9955

( $\alpha, n'$ ) Level

101-Md-254

0 Kelvin Cross Sections



4

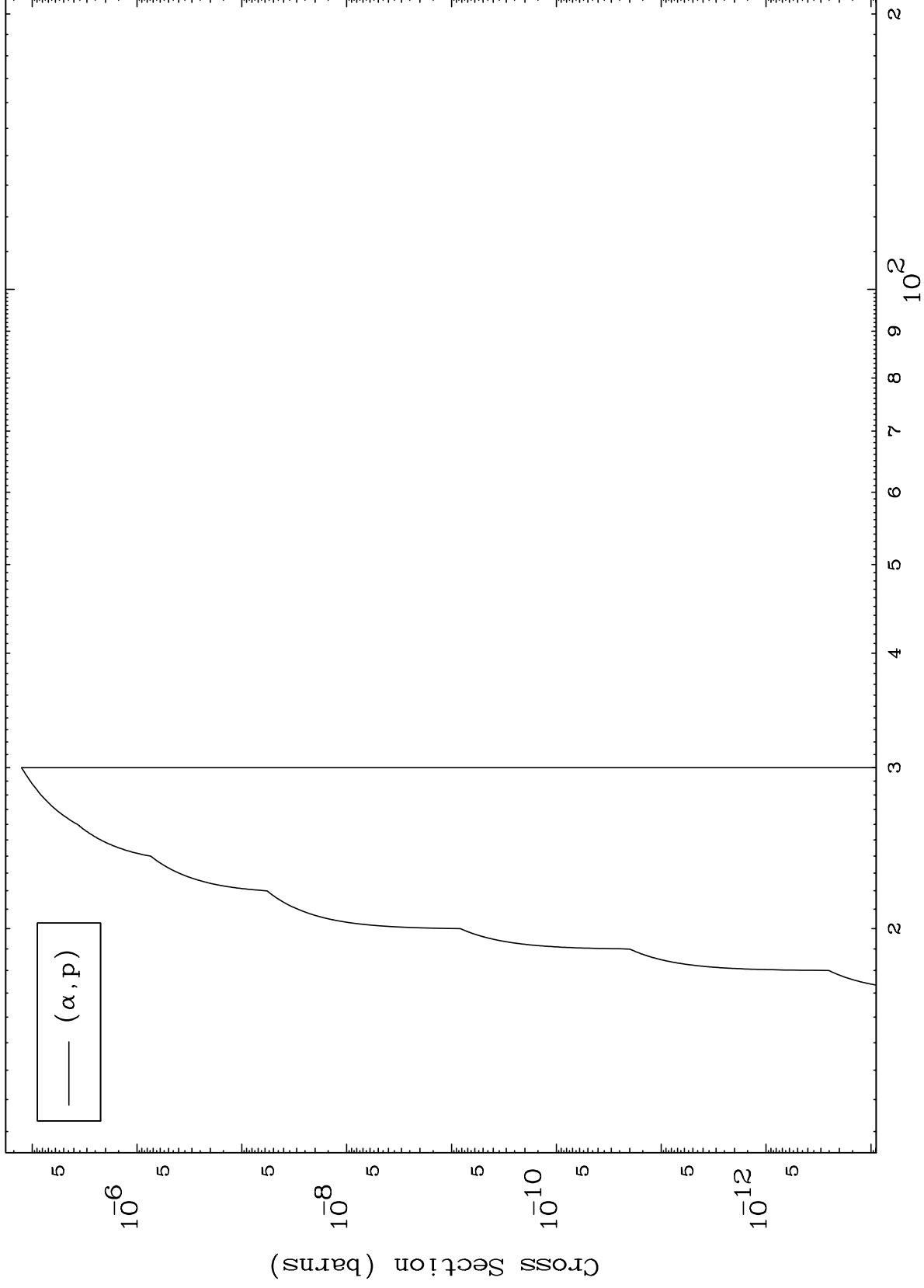
Incident Energy (MeV)

101-Md-254

MAT 9955

101-Md-254

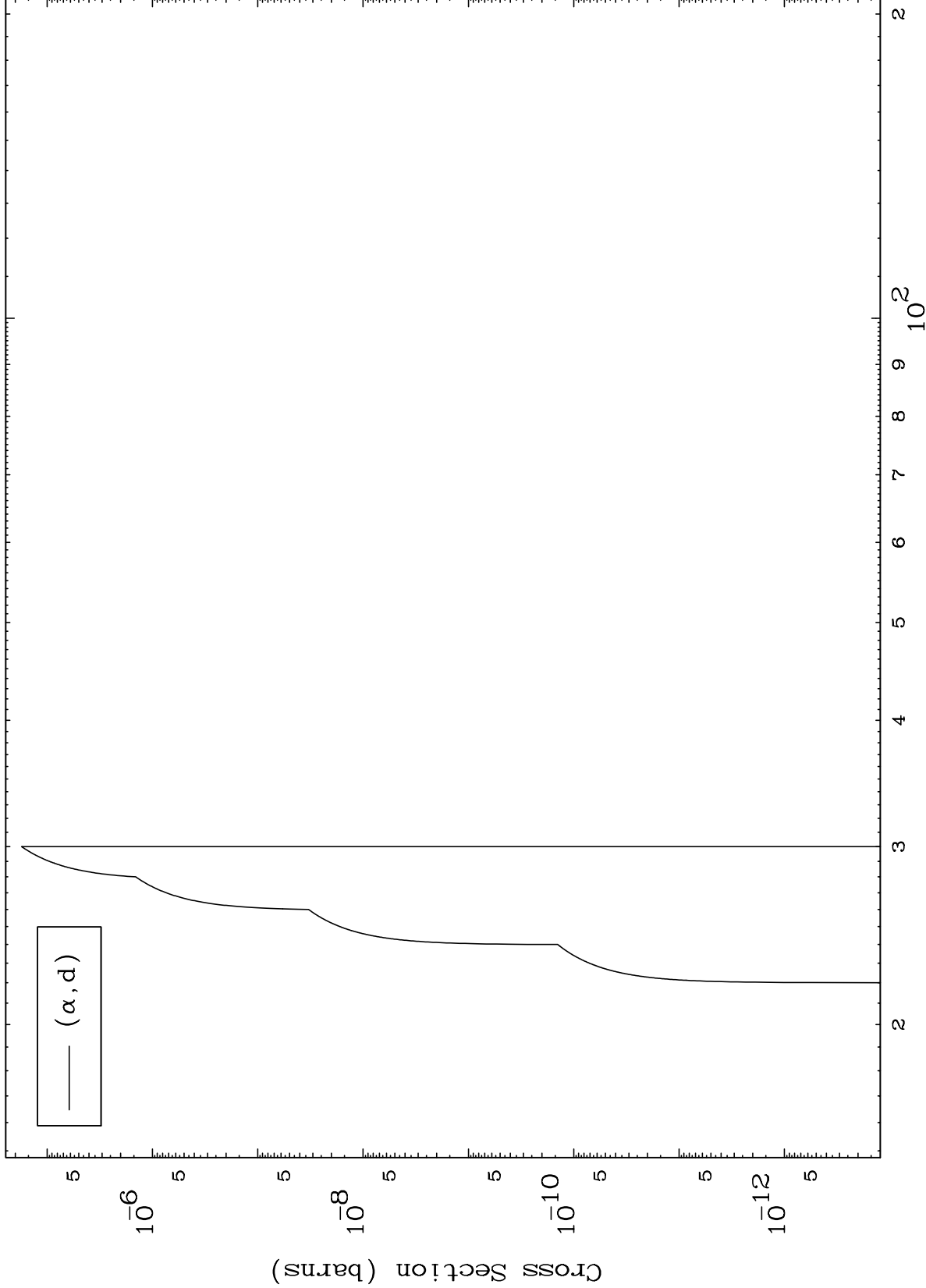
( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections



101-Md-254

Incident Energy (MeV)

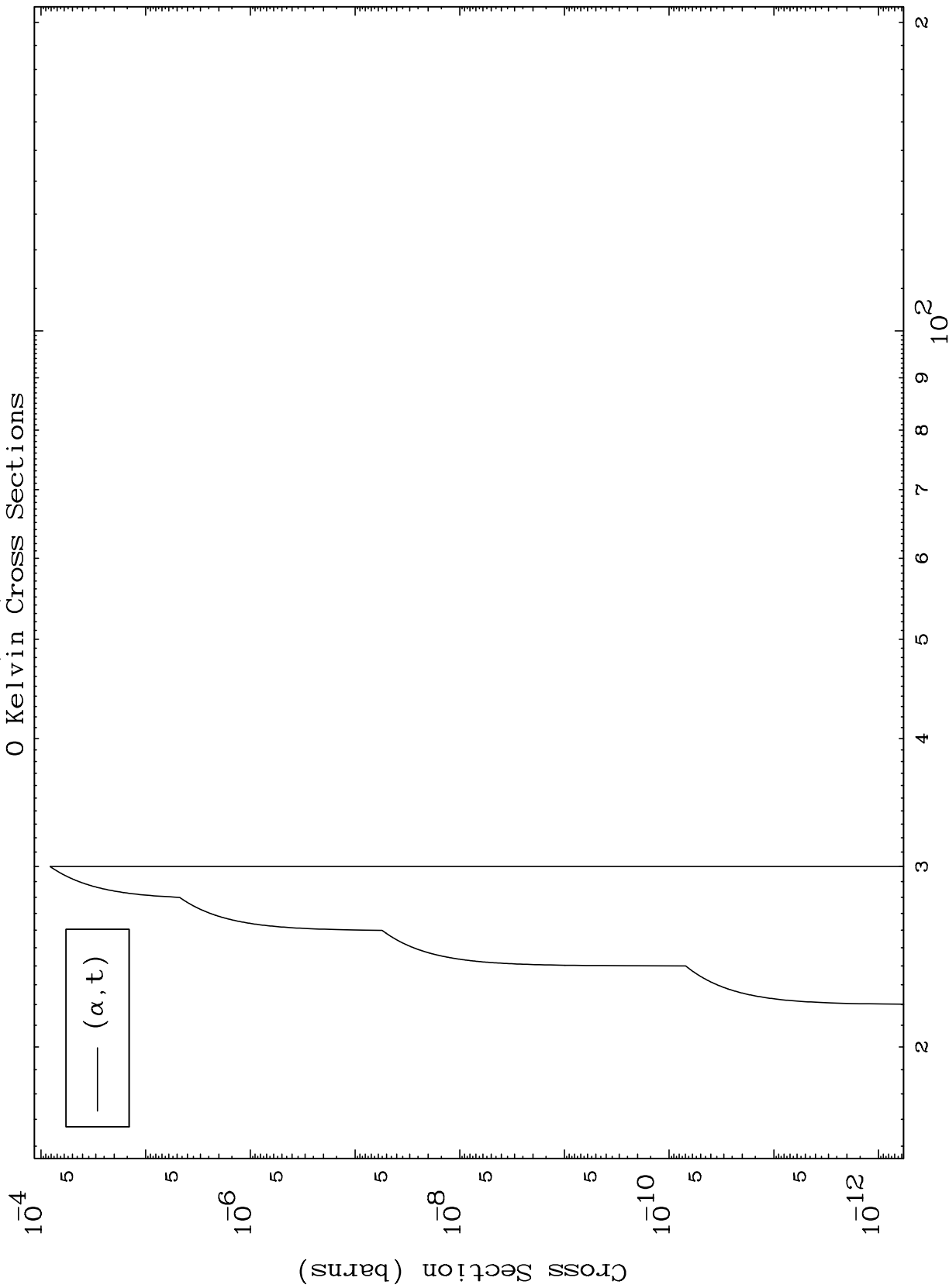
5



MAT 9955

101-Md-254

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections



7

Incident Energy (MeV)

101-Md-254

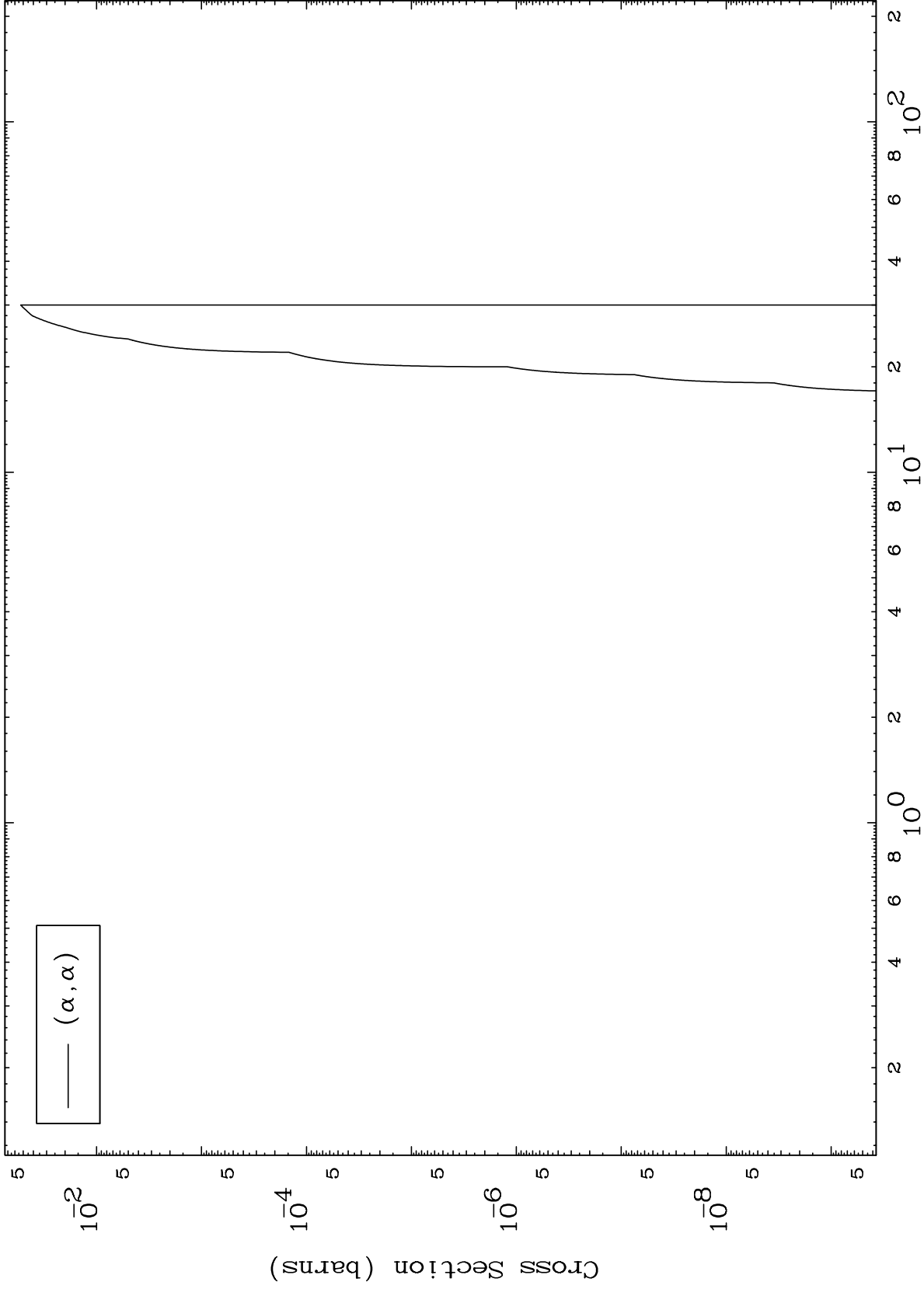


MAT 9955

( $\alpha, \alpha$ ) Levels

101-Md-254

0 Kelvin Cross Sections



8

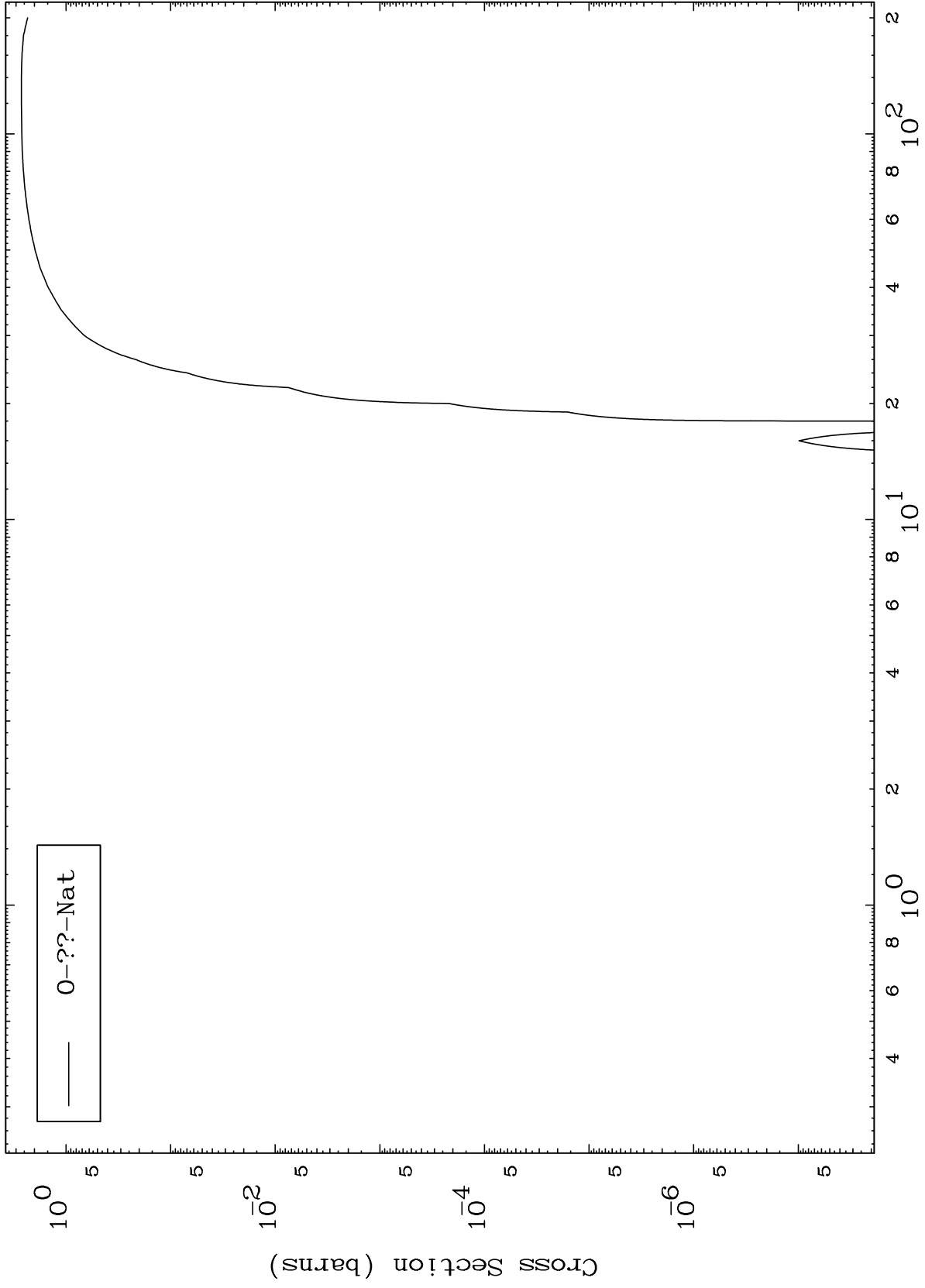
Incident Energy (MeV)

101-Md-254

MAT 9955

101-Md-254

$\alpha$  Fission  
Radionuclide Production Cross Section



101-Md-254

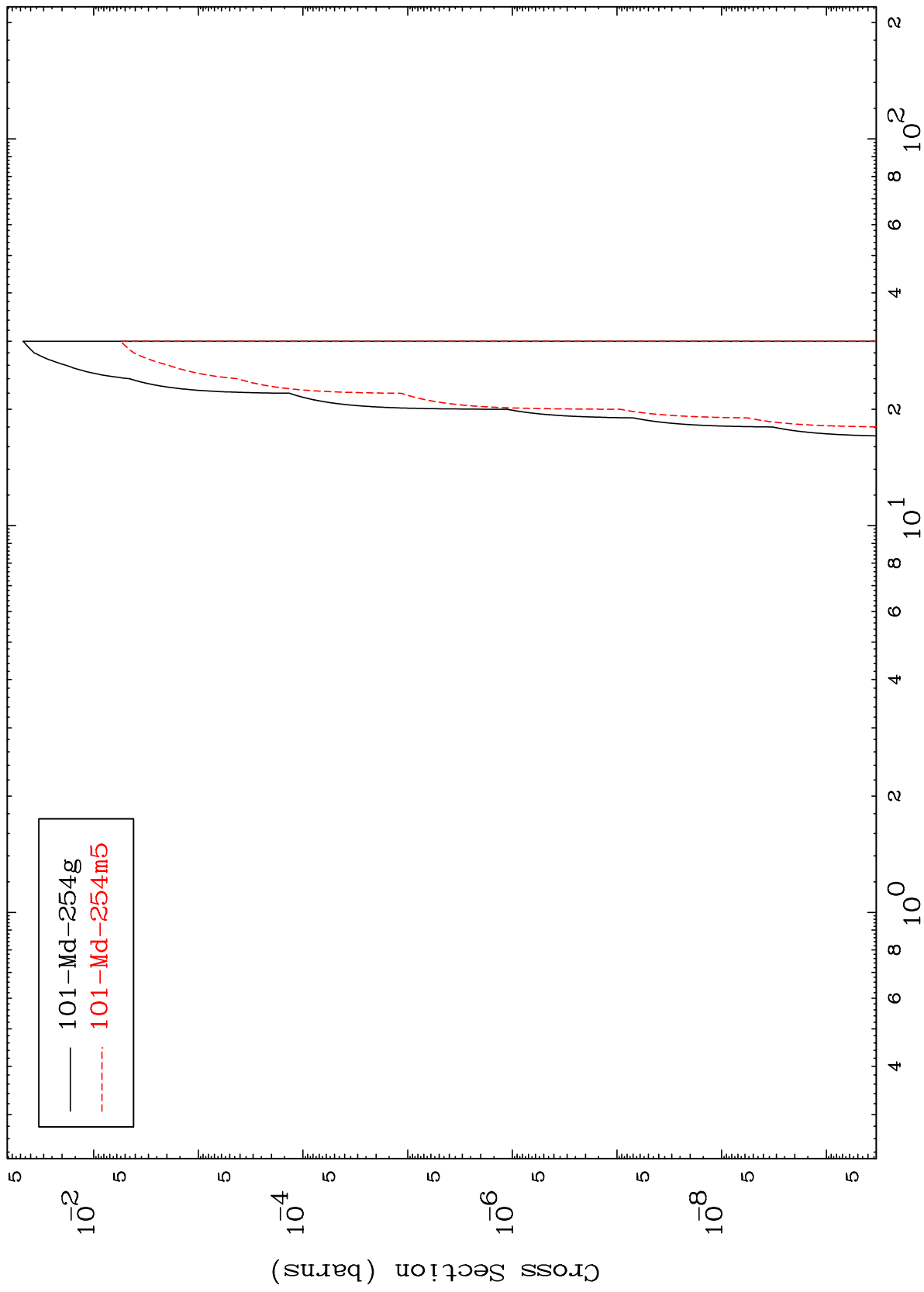
Incident Energy (MeV)

MAT 9955

( $\alpha, \alpha$ )

101-Md-254

Radionuclide Production Cross Section



10

Incident Energy (MeV)

101-Md-254