

Program EVALPLOT
(Version 2018-1)

by

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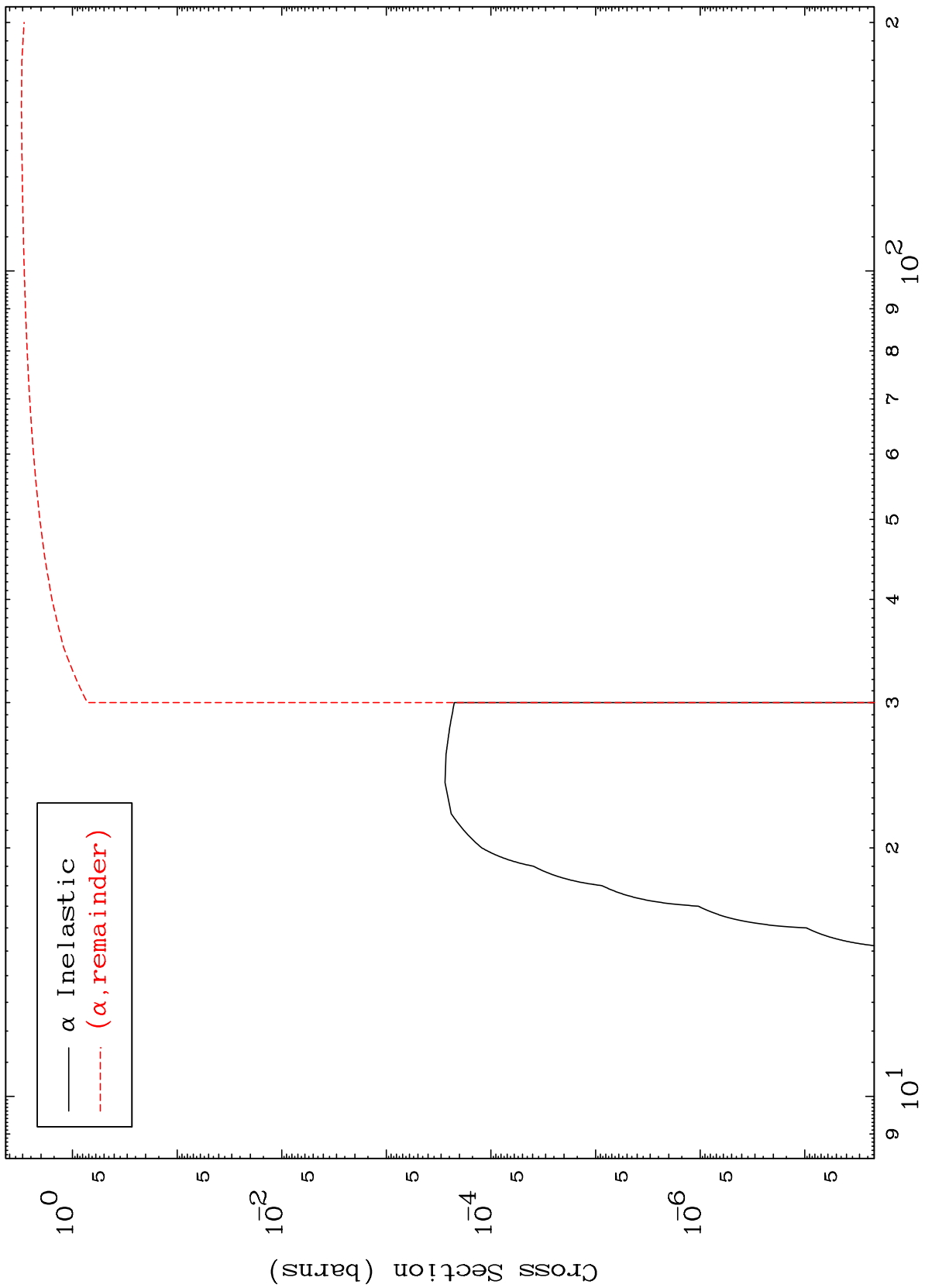
Press Mouse Button to Start

MAT 9960

α Major

101-Md-254

0 Kelvin Cross Sections



— α Inelastic
- - - α remainder

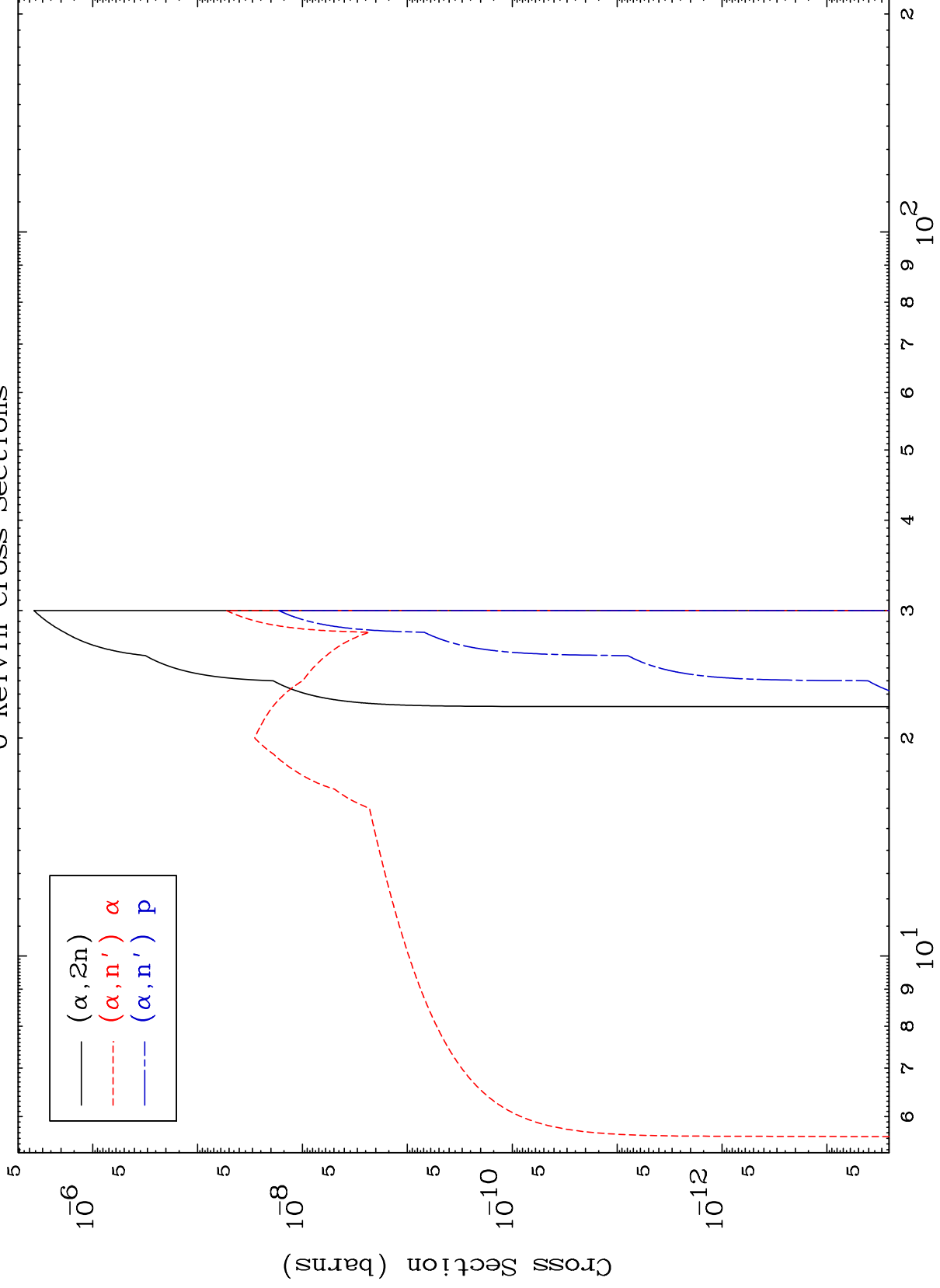
Incident Energy (MeV)

101-Md-254

MAT 9960

α Neutron Production
0 Kelvin Cross Sections

101-Md-254



($\alpha, 2n$)
(α, n') α
(α, n') p

2

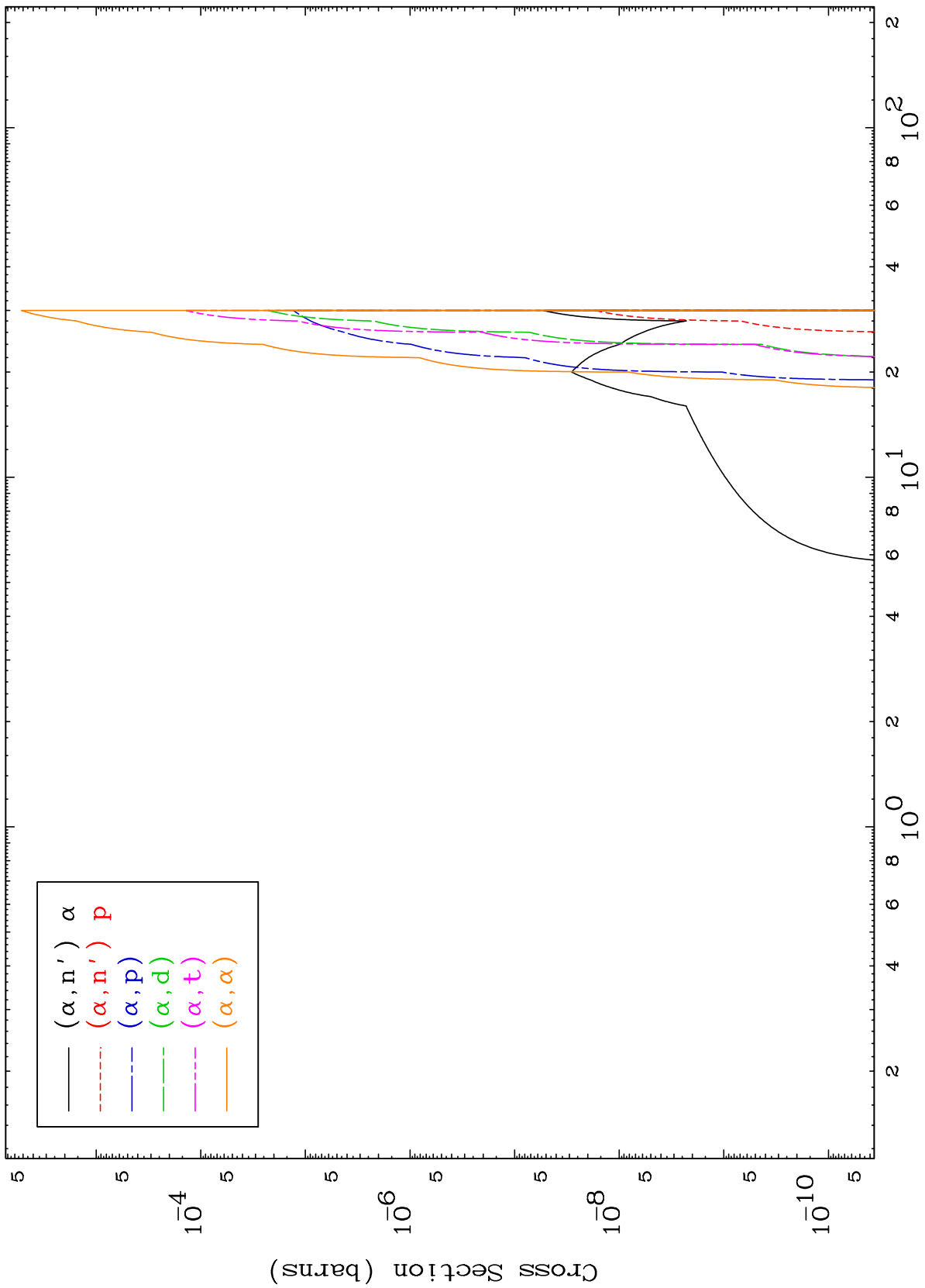
Incident Energy (MeV)

101-Md-254

MAT 9960

α Charged Particle
0 Kelvin Cross Sections

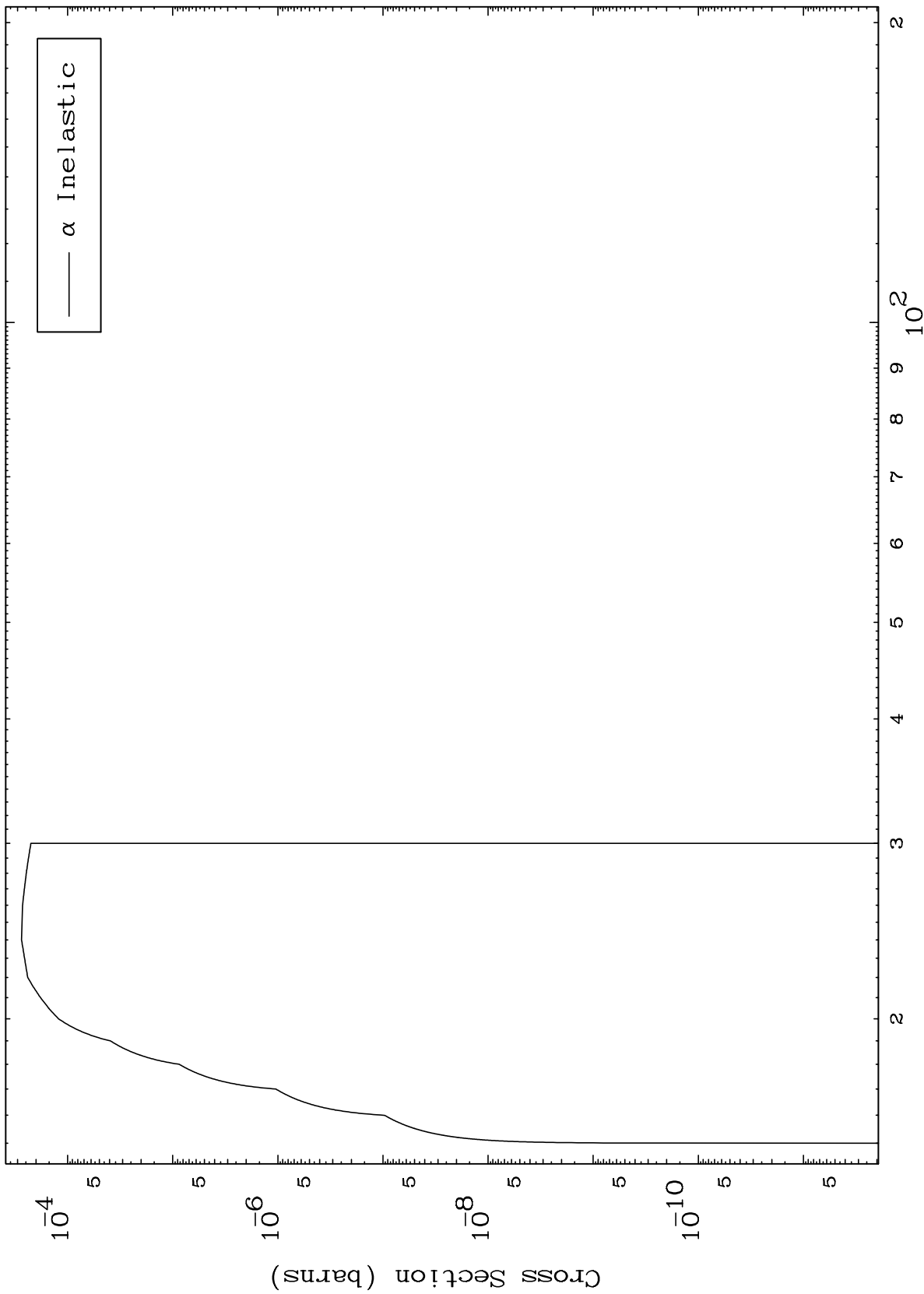
101-Md-254



MAT 9960

101-Md-254

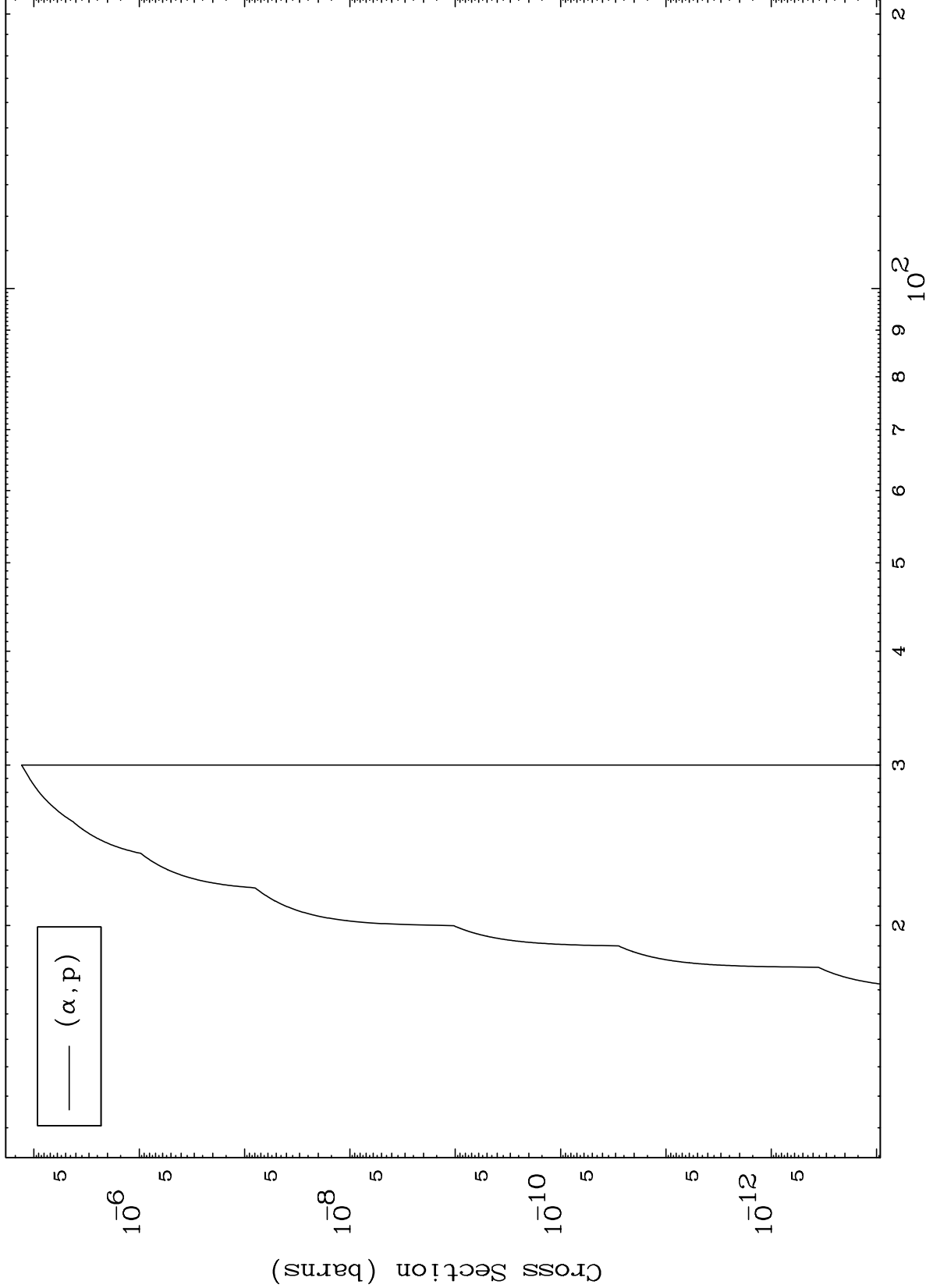
(α, n') Level
0 Kelvin Cross Sections



101-Md-254

Incident Energy (MeV)

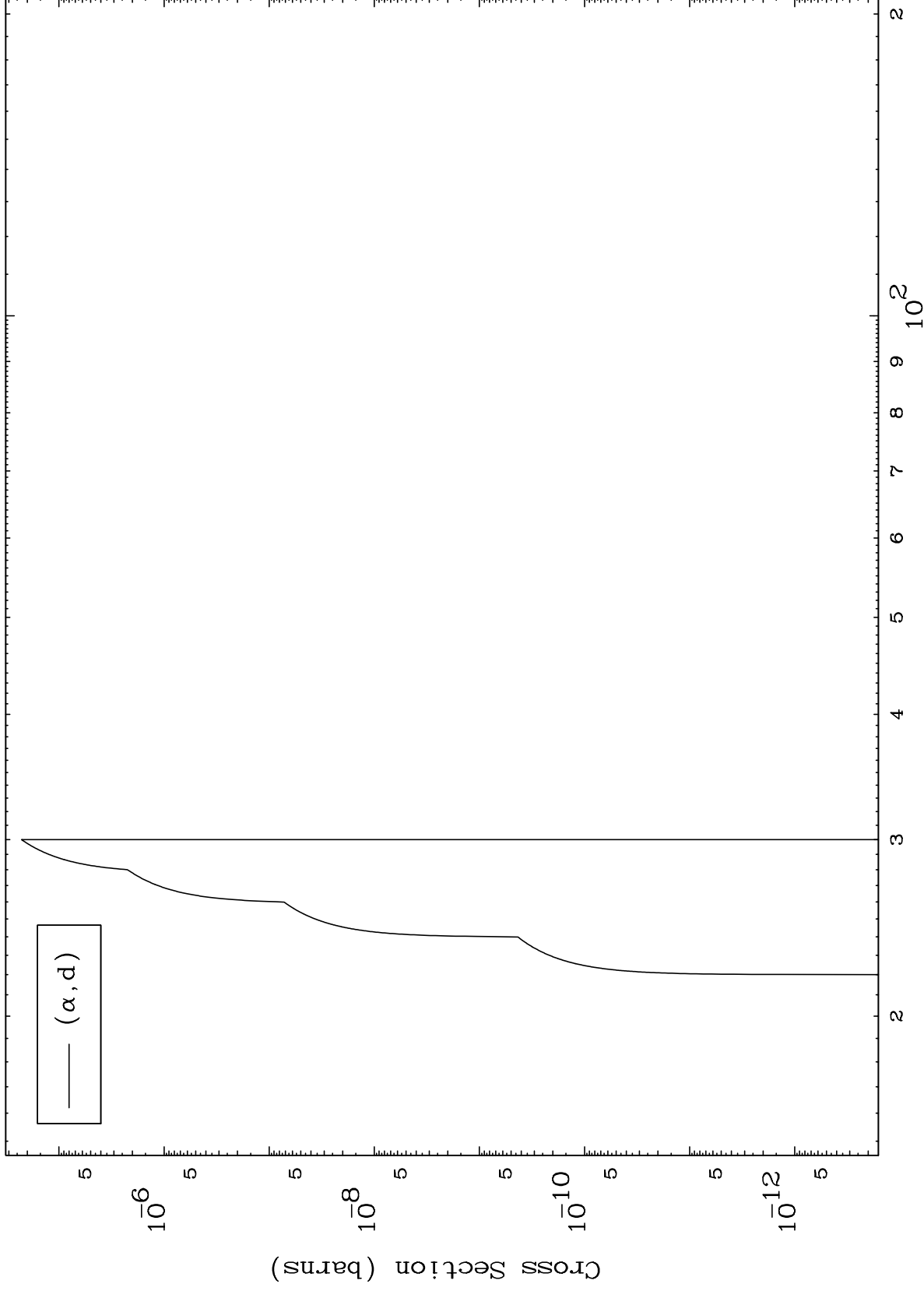
4



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(α, d) Levels
0 Kelvin Cross Sections

101-Md-254



6

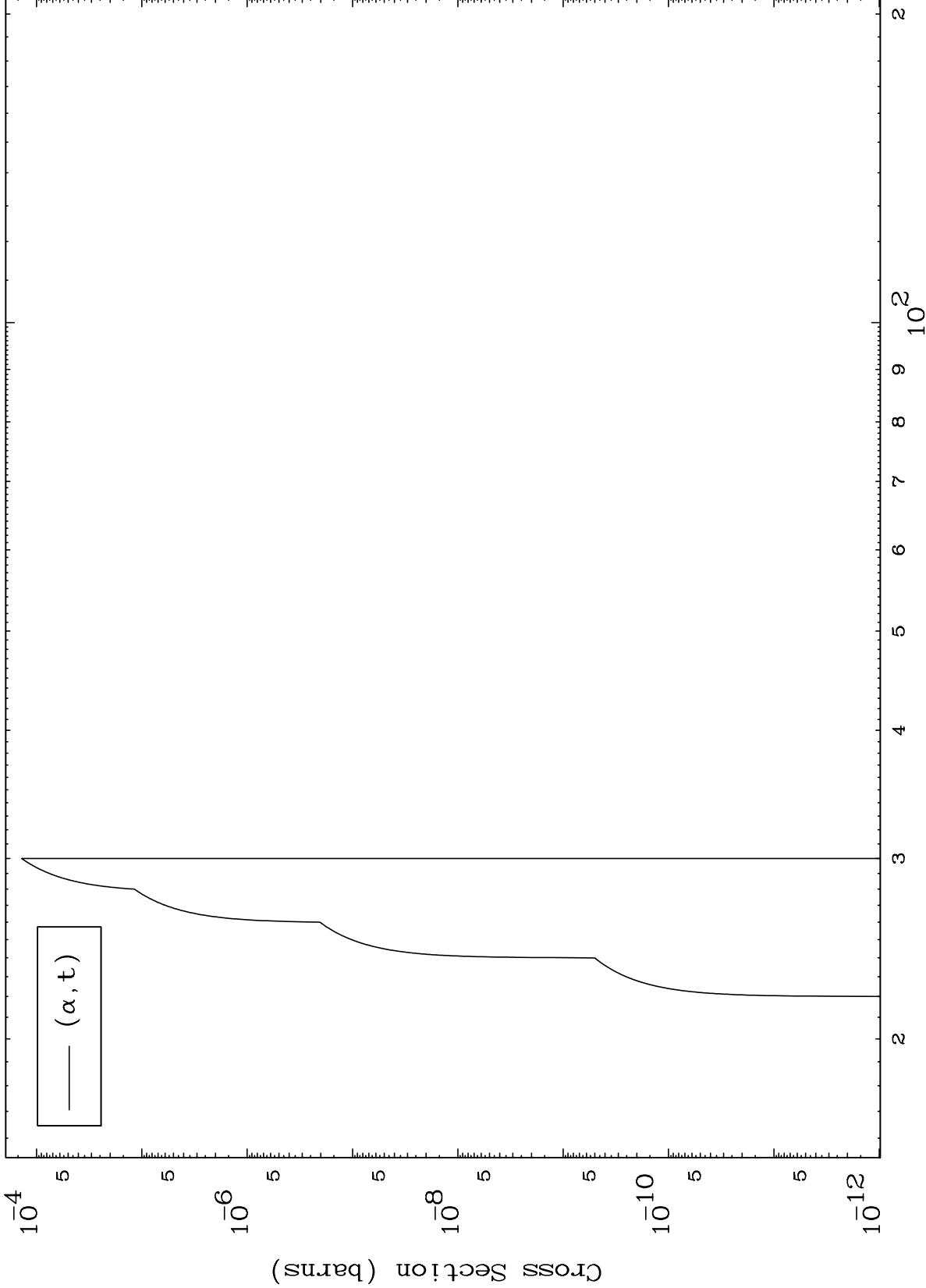
Incident Energy (MeV)

101-Md-254

MAT 9960

(α, t) Levels
0 Kelvin Cross Sections

101-Md-254

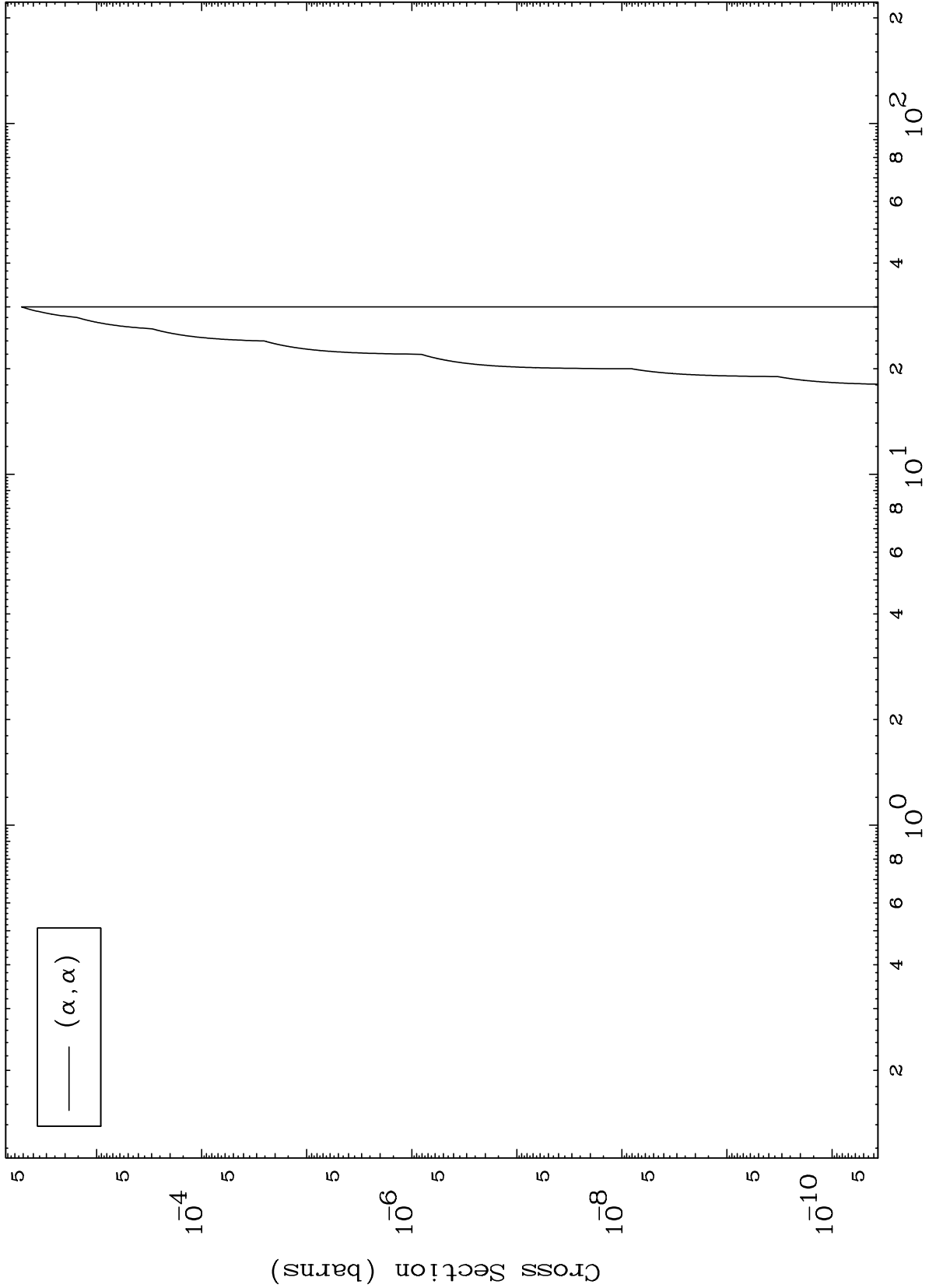


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(α, α) Levels

101-Md-254

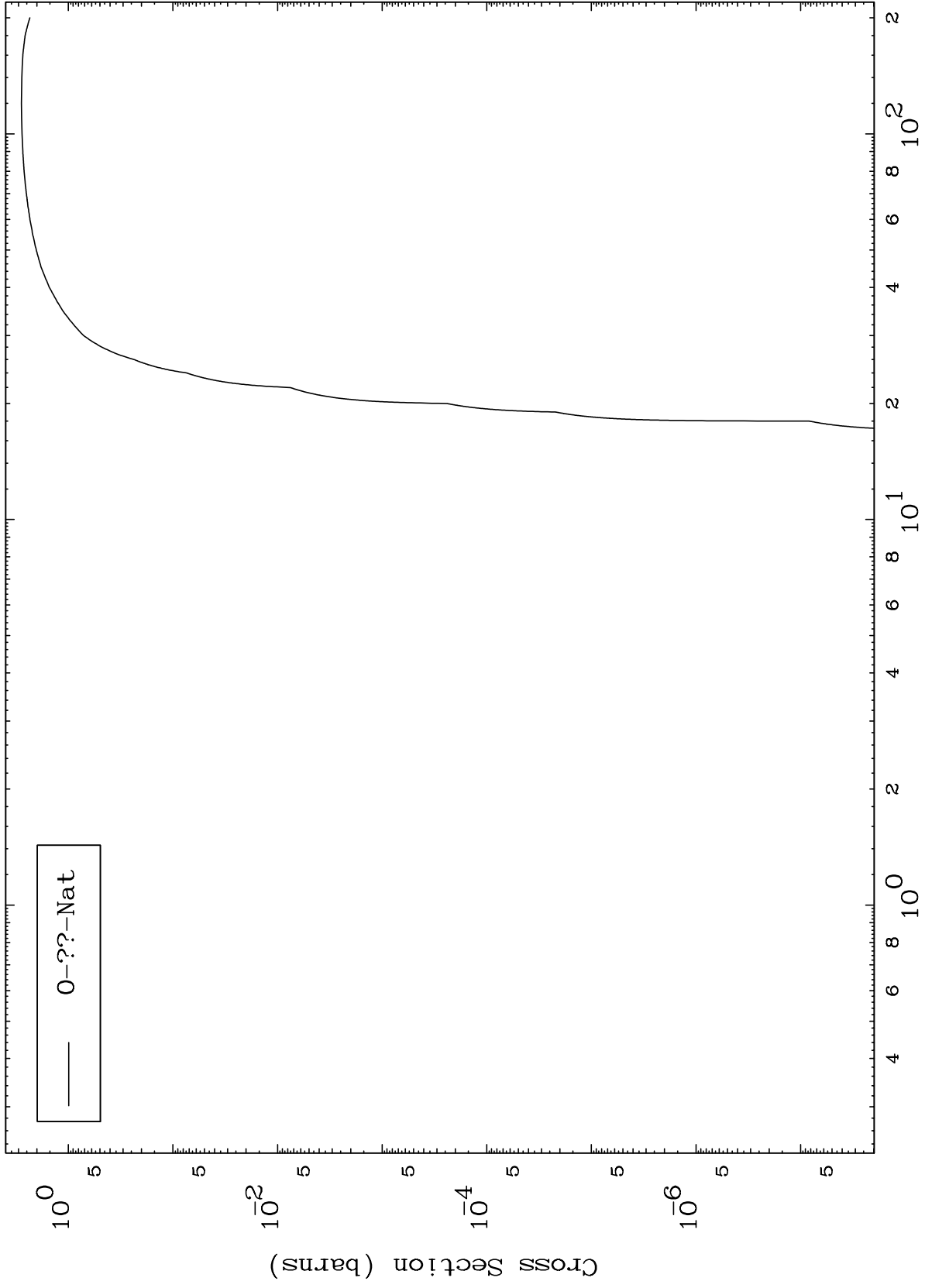
0 Kelvin Cross Sections



MAT 9960

101-Md-254

α Fission
Radionuclide Production Cross Section

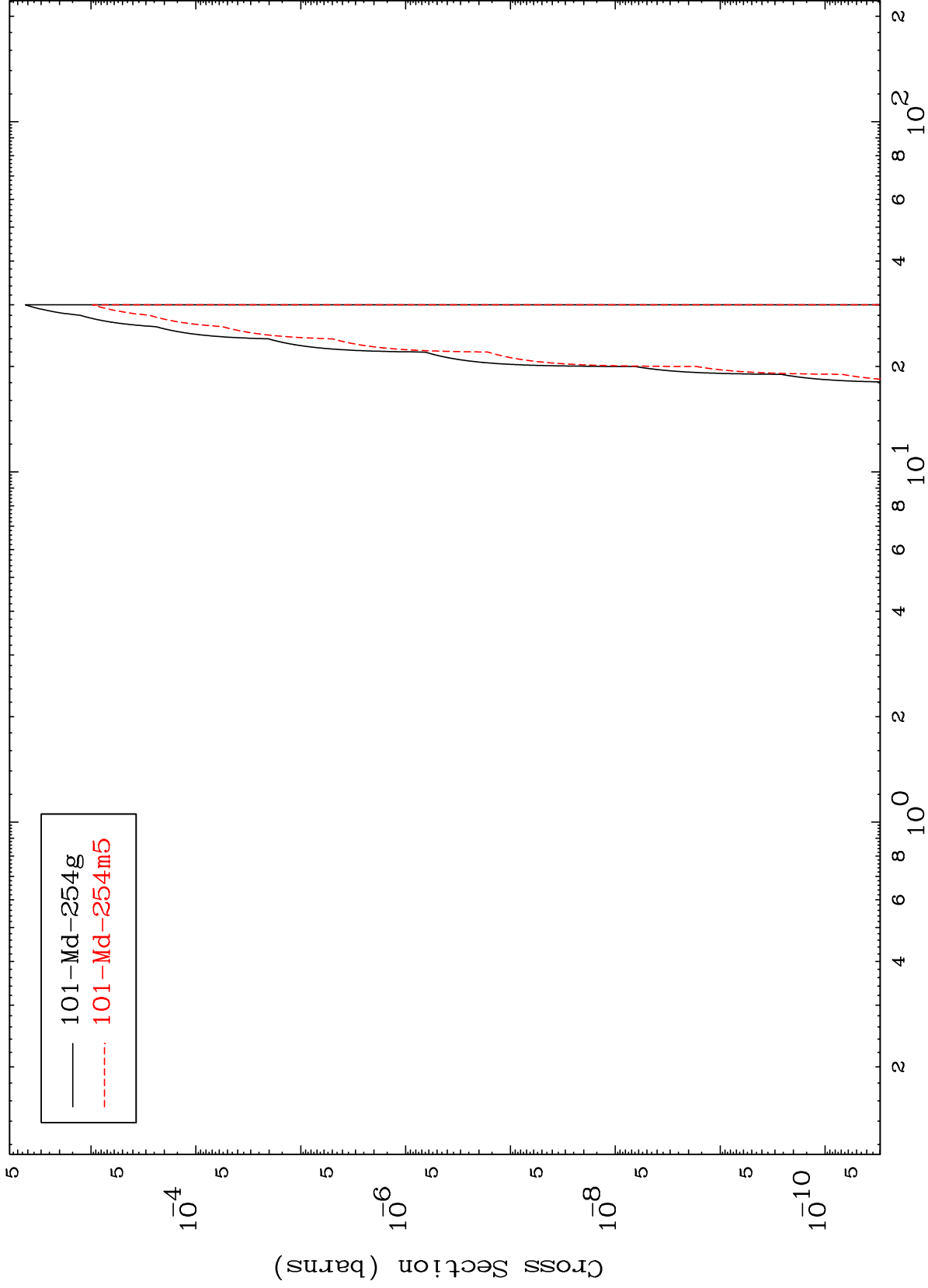


MAT 9960

(α, α)

101-Md-254

Radionuclide Production Cross Section



10

Incident Energy (MeV)

101-Md-254