

Program EVALPLOT  
(Version 2018-1)

by

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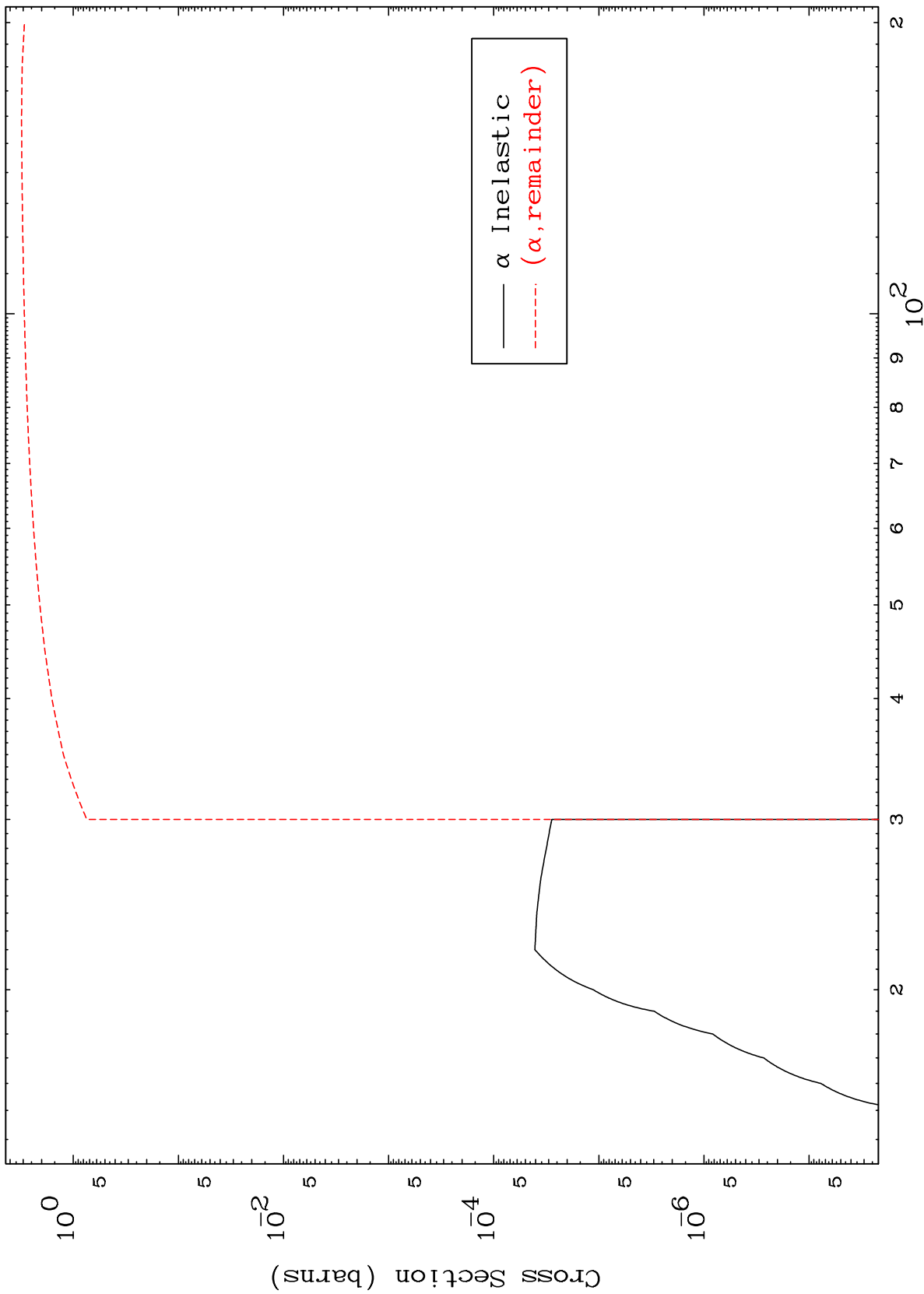
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9959

$\alpha$  Major  
0 Kelvin Cross Sections

101-Md-258



1

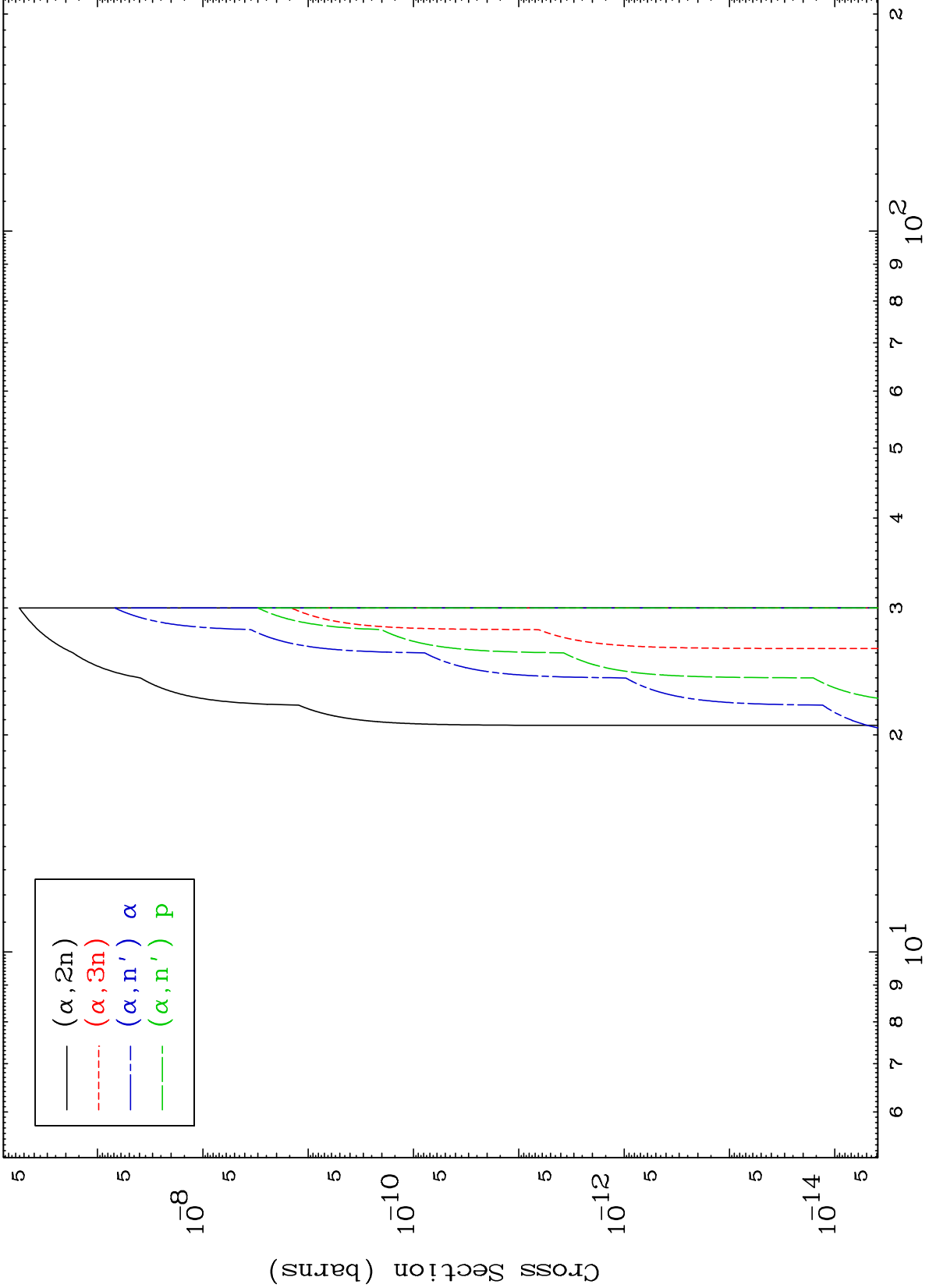
Incident Energy (MeV)

101-Md-258

MAT 9959

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

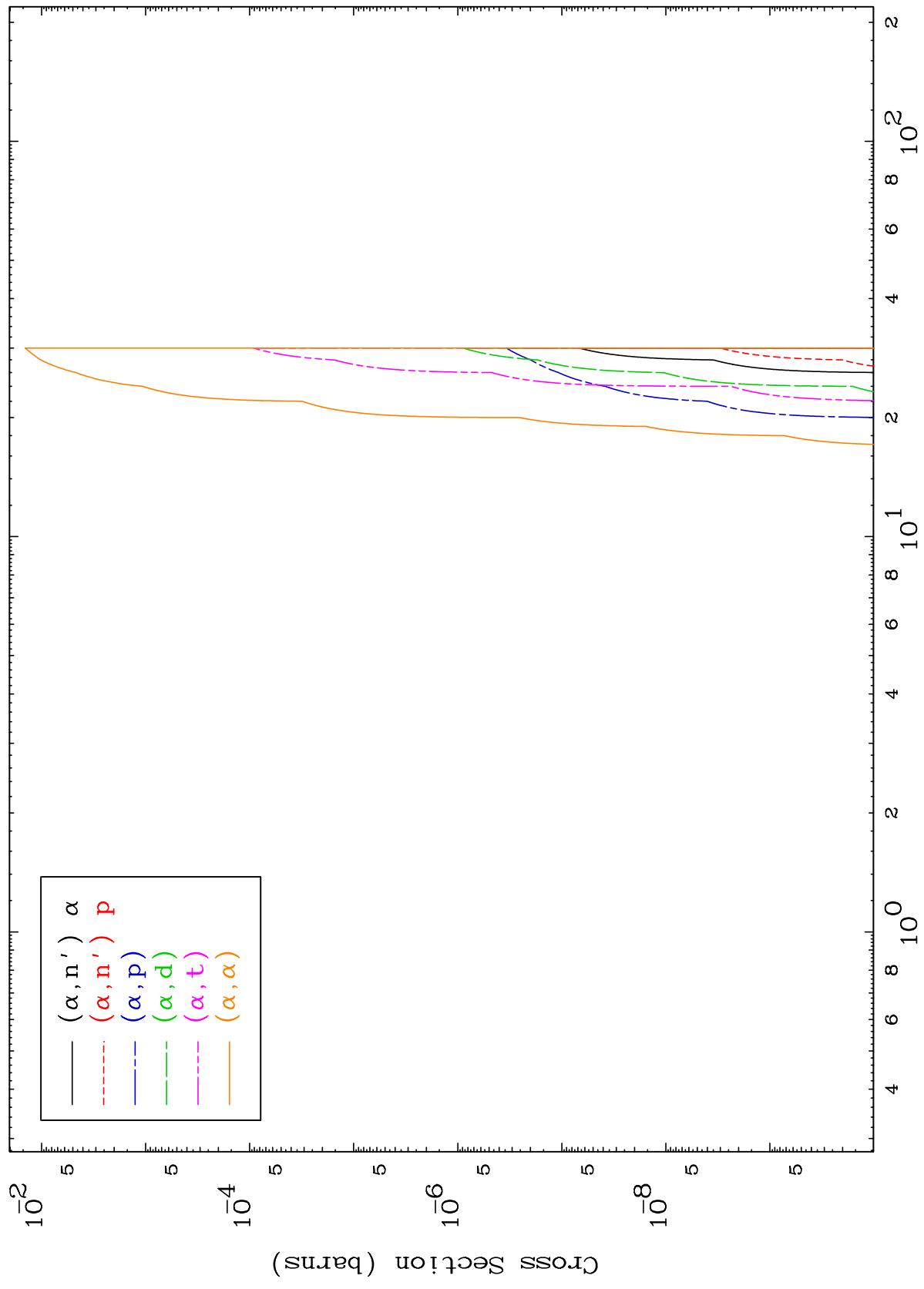
101-Md-258



MAT 9959

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

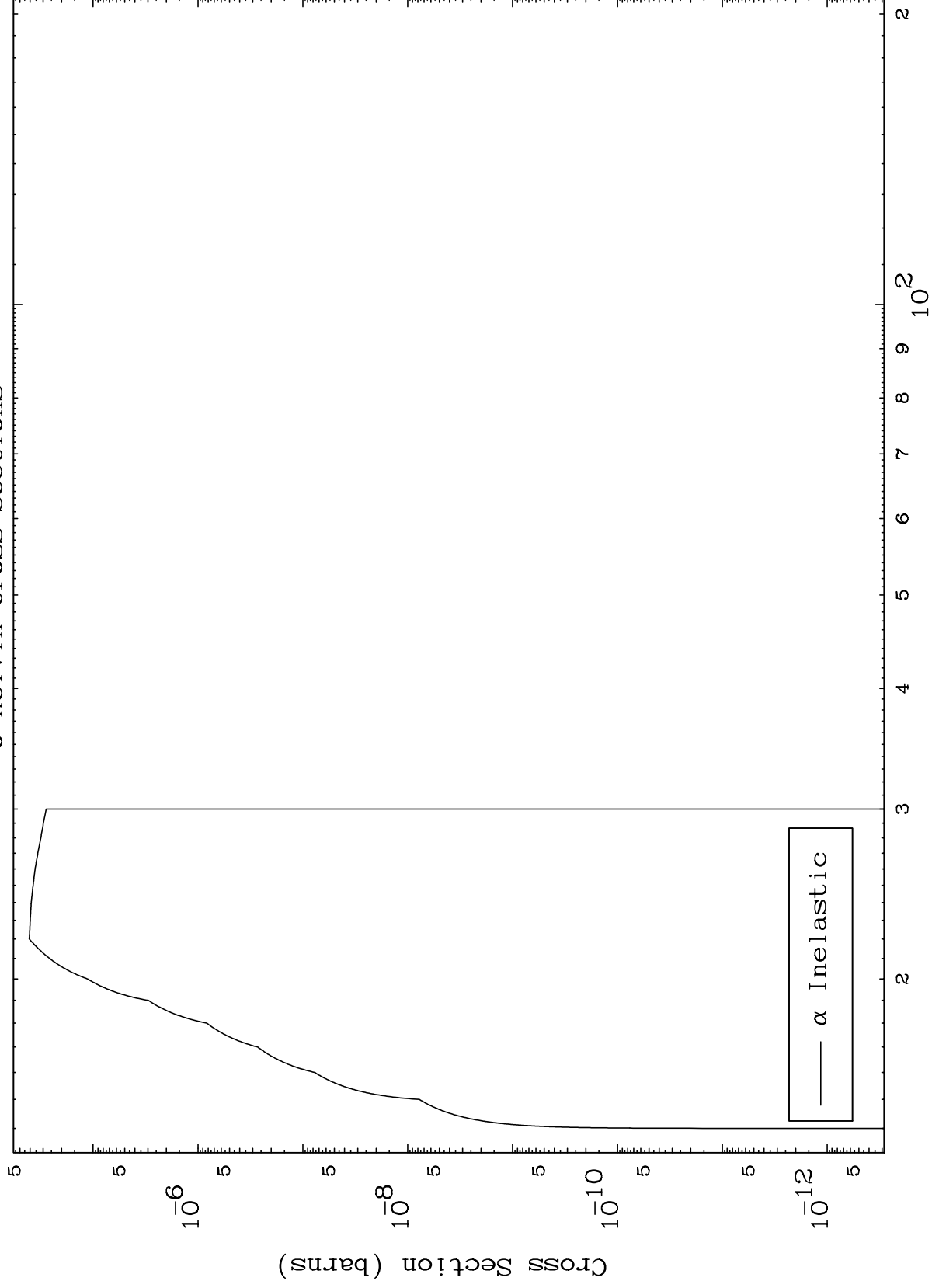
101-Md-258



MAT 9959

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

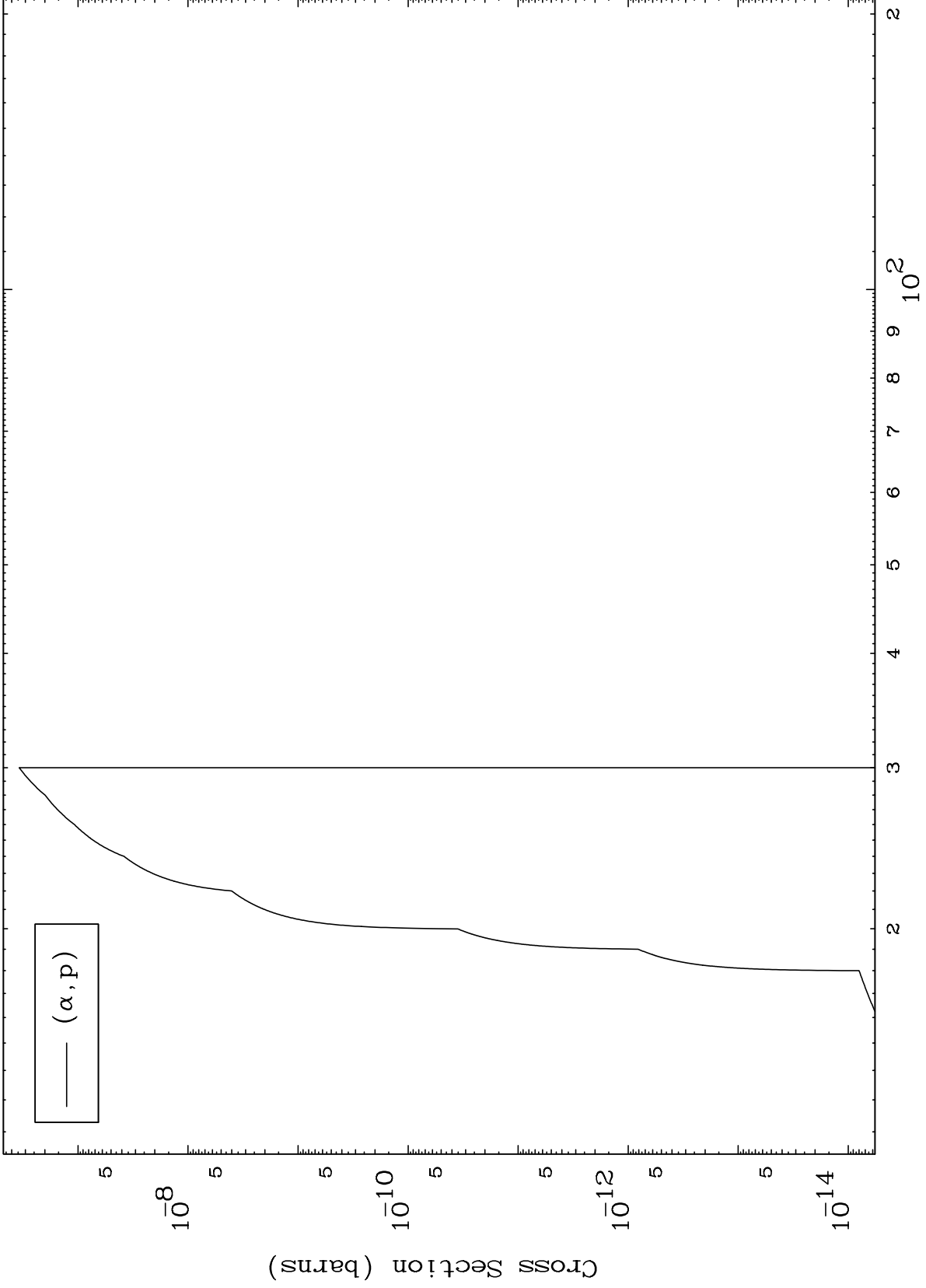
101-Md-258



4

Incident Energy (MeV)

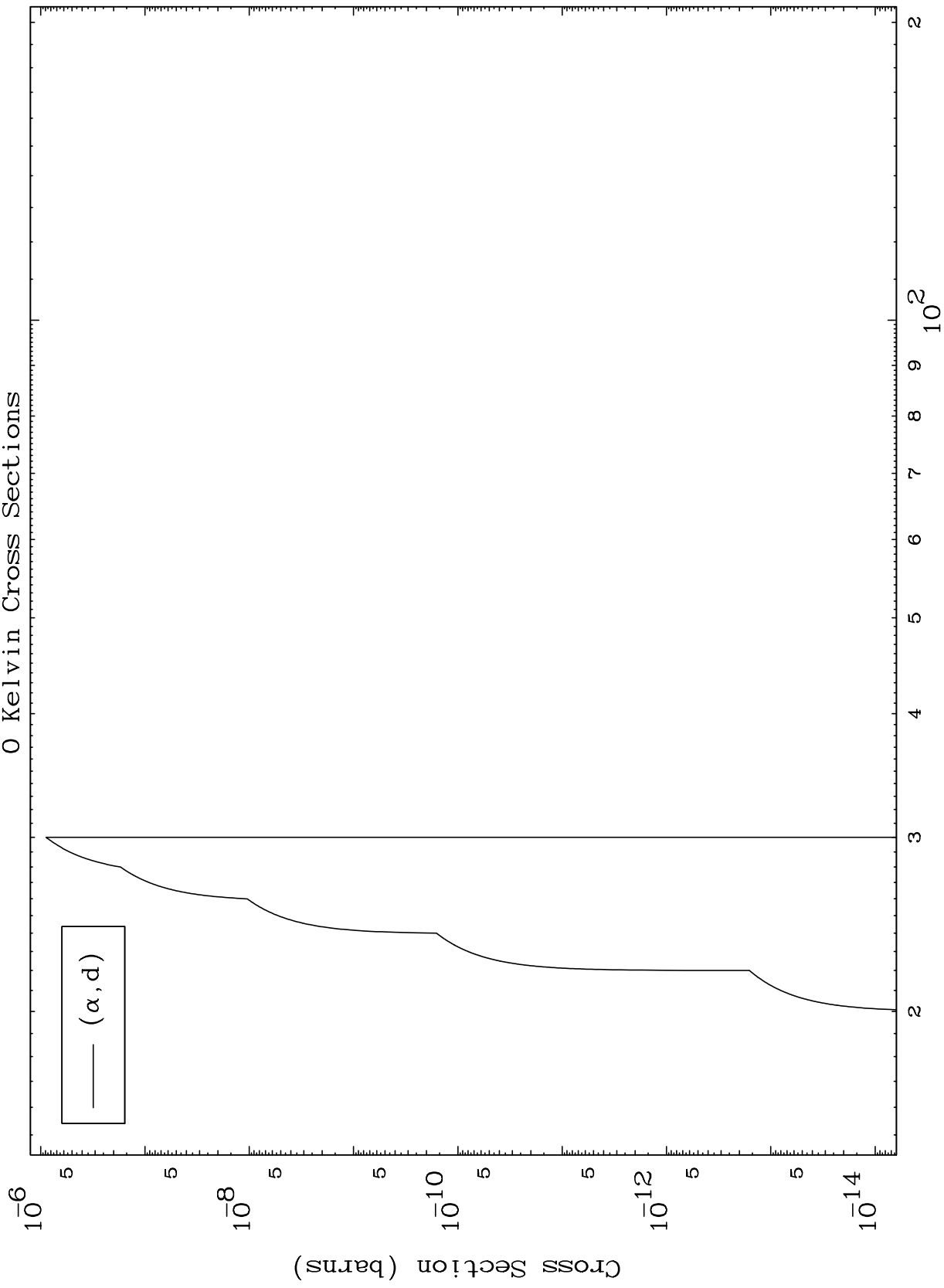
101-Md-258



MAT 9959

101-Md-258

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections



101-Md-258

Incident Energy (MeV)

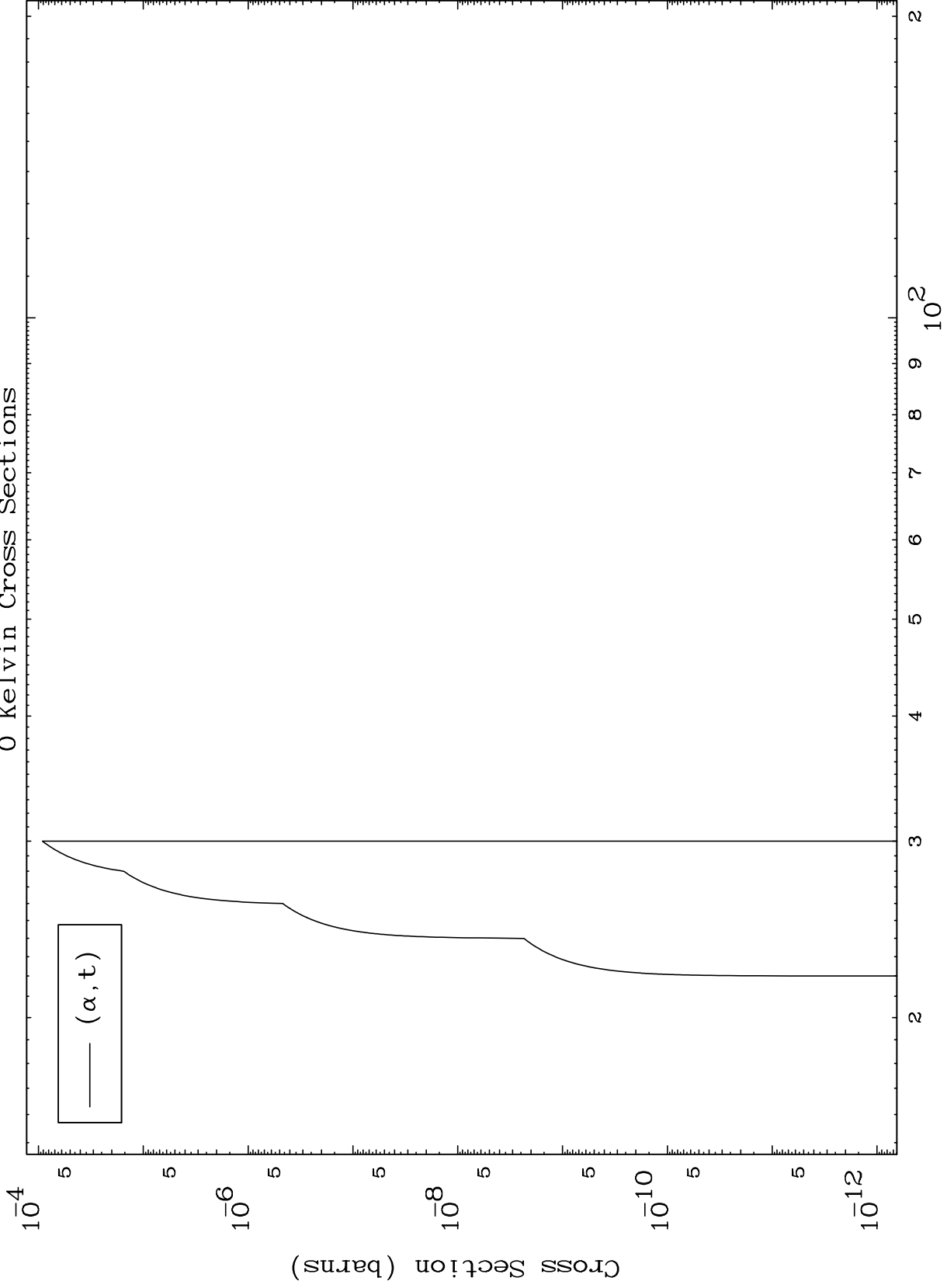
6

MAT 9959

( $\alpha, t$ ) Levels

101-Md-258

0 Kelvin Cross Sections



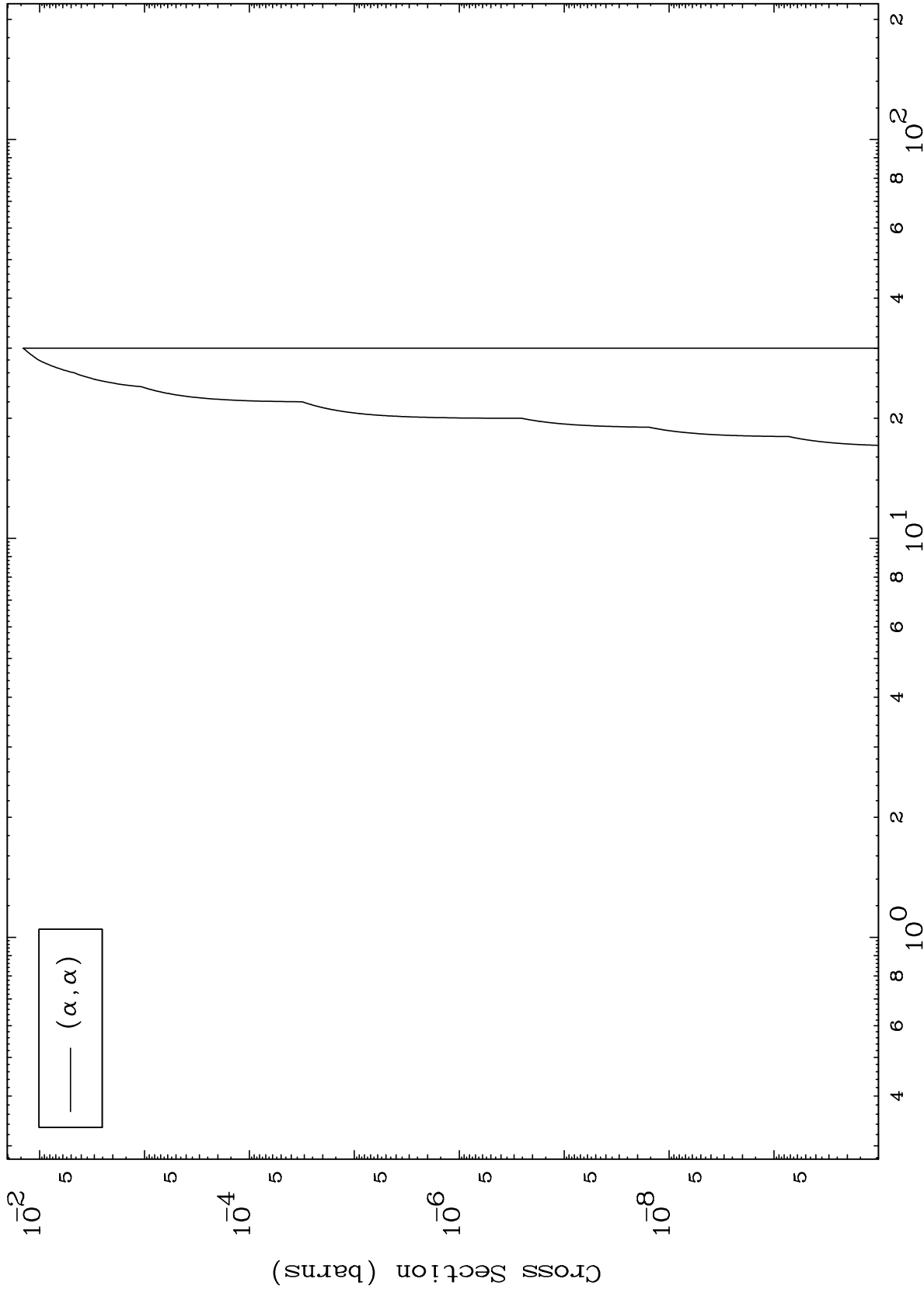


MAT 9959

( $\alpha, \alpha$ ) Levels

101-Md-258

0 Kelvin Cross Sections



8

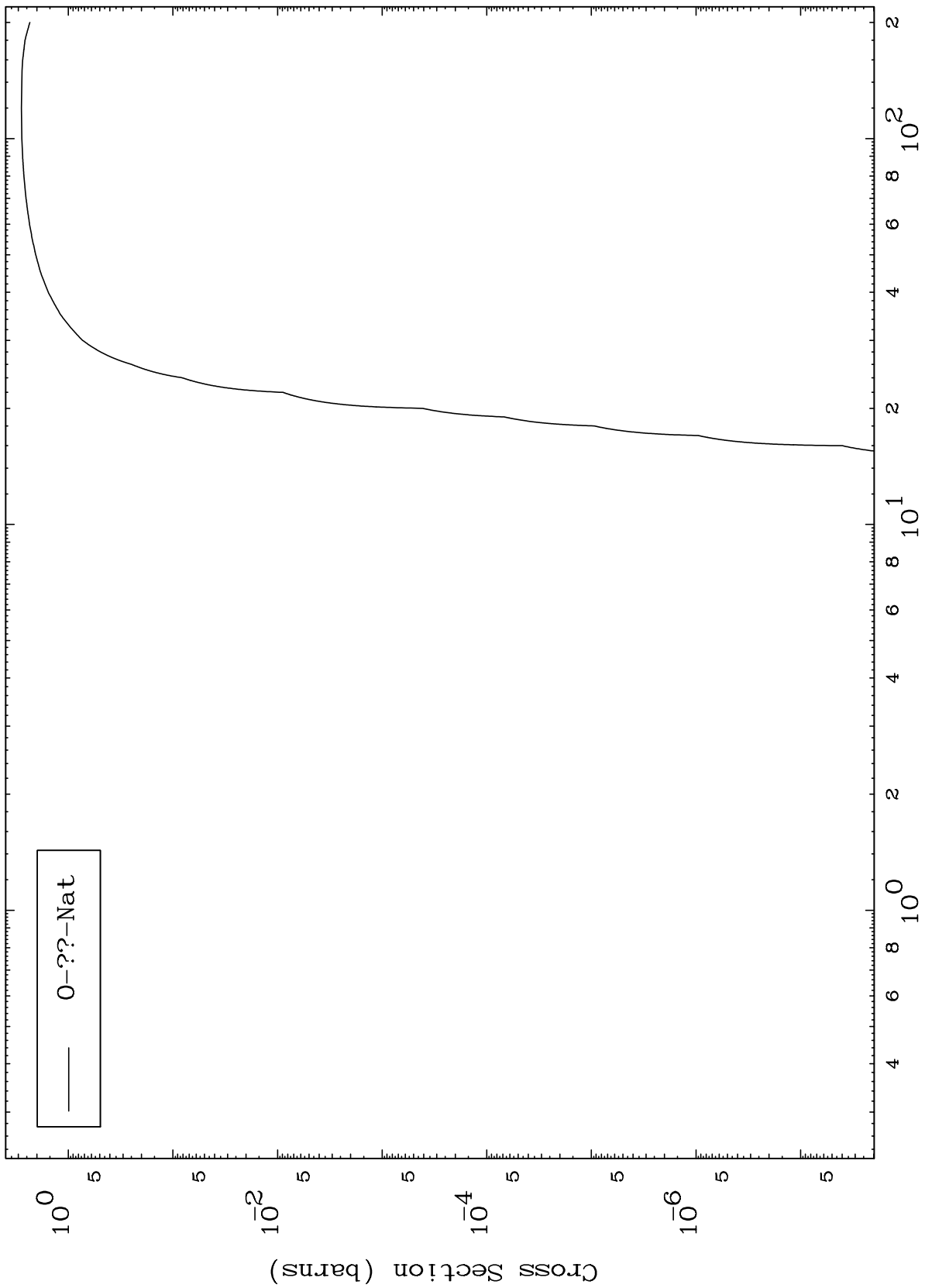
Incident Energy (MeV)

101-Md-258

MAT 9959

101-Md-258

$\alpha$  Fission  
Radionuclide Production Cross Section

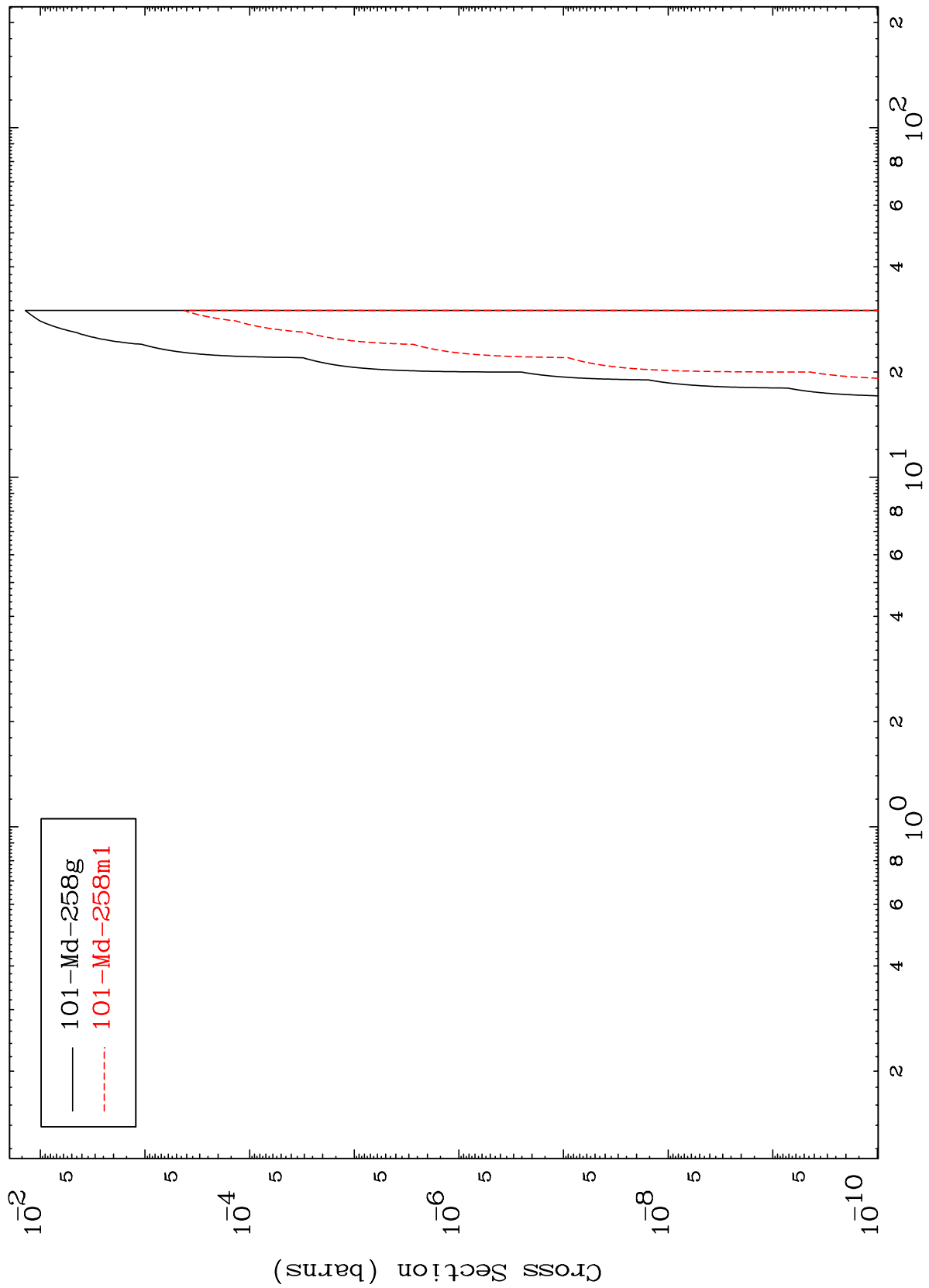


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( $\alpha, \alpha$ )

101-Md-258

Radionuclide Production Cross Section



101-Md-258g  
101-Md-258m1

10

Incident Energy (MeV)

101-Md-258