

Program EVALPLOT  
(Version 2018-1)

by

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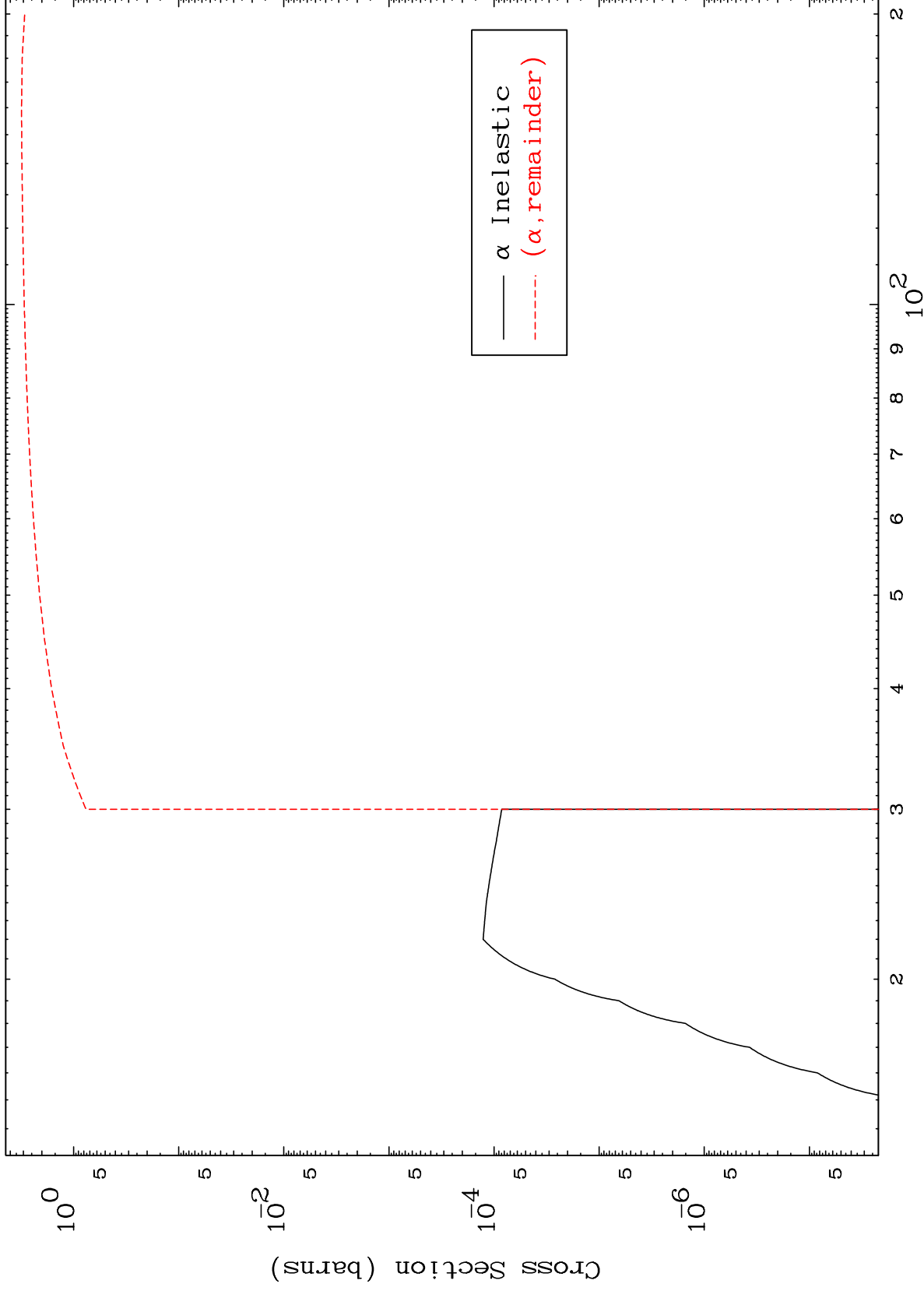
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9962

$\alpha$  Major  
0 Kelvin Cross Sections

101-Md-261



101-Md-261

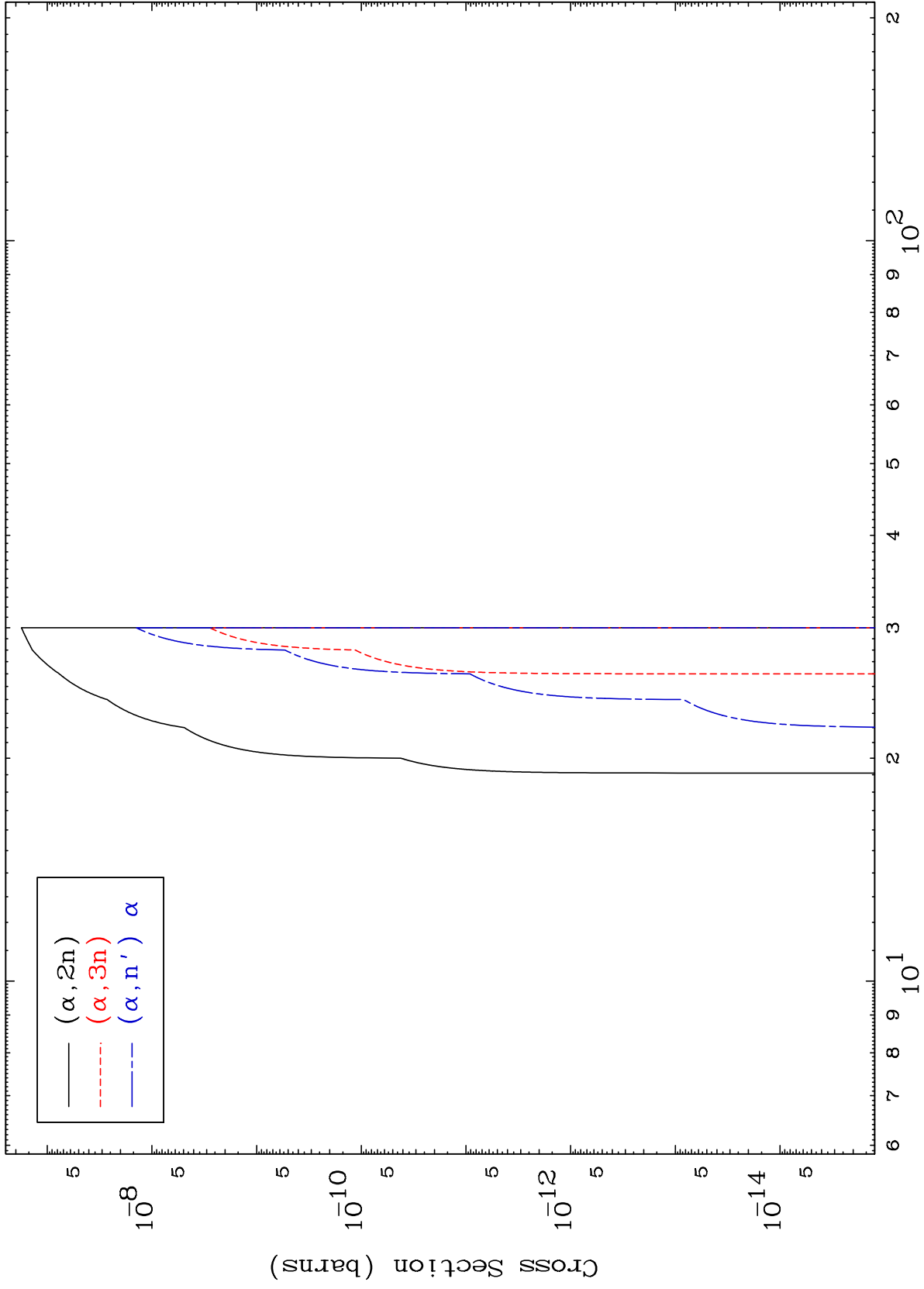
Incident Energy (MeV)

1

MAT 9962

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

101-Md-261

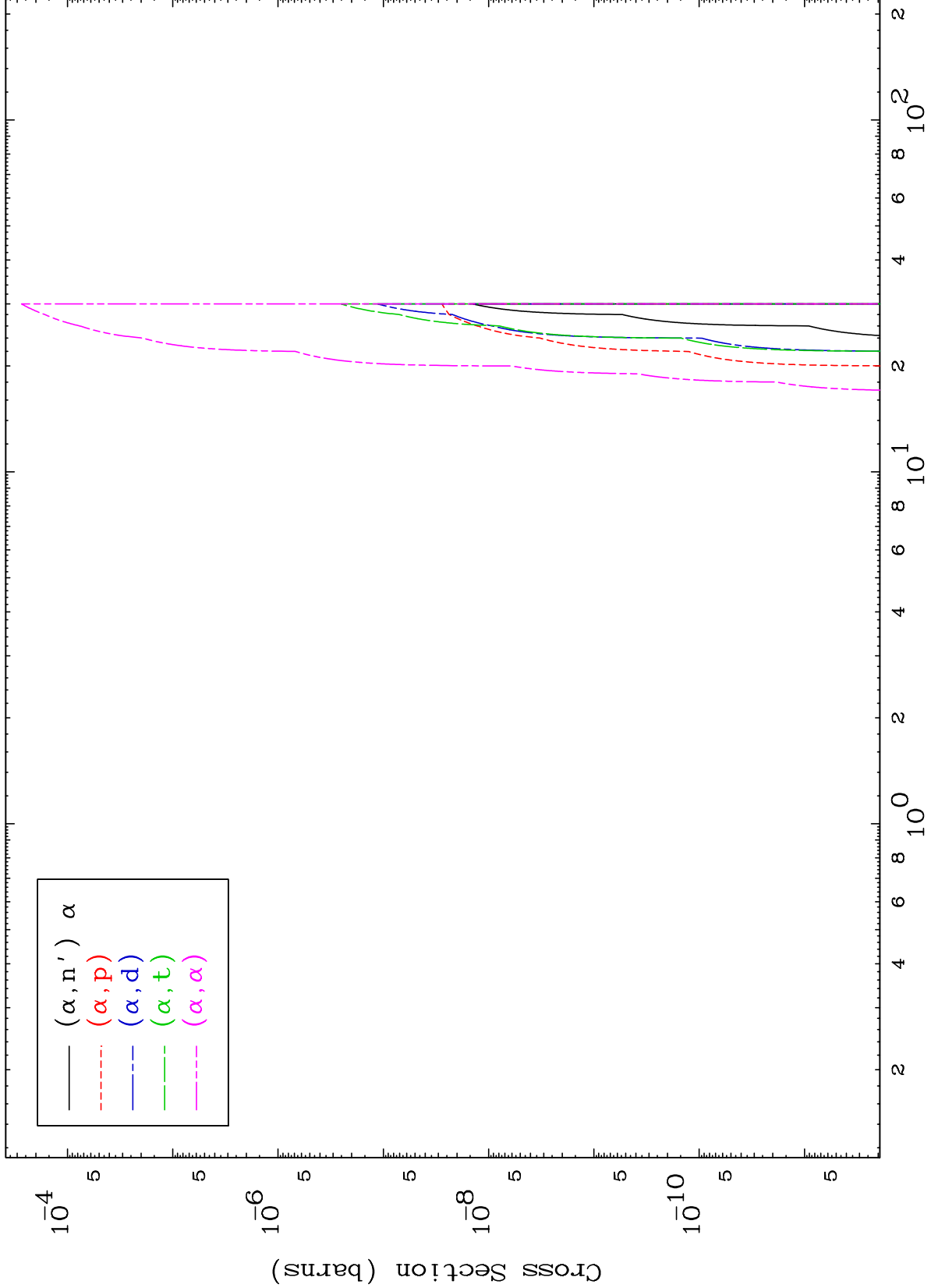


( $\alpha, 2n$ )  
( $\alpha, 3n$ )  
( $\alpha, n'$ )  $\alpha$

2

Incident Energy (MeV)

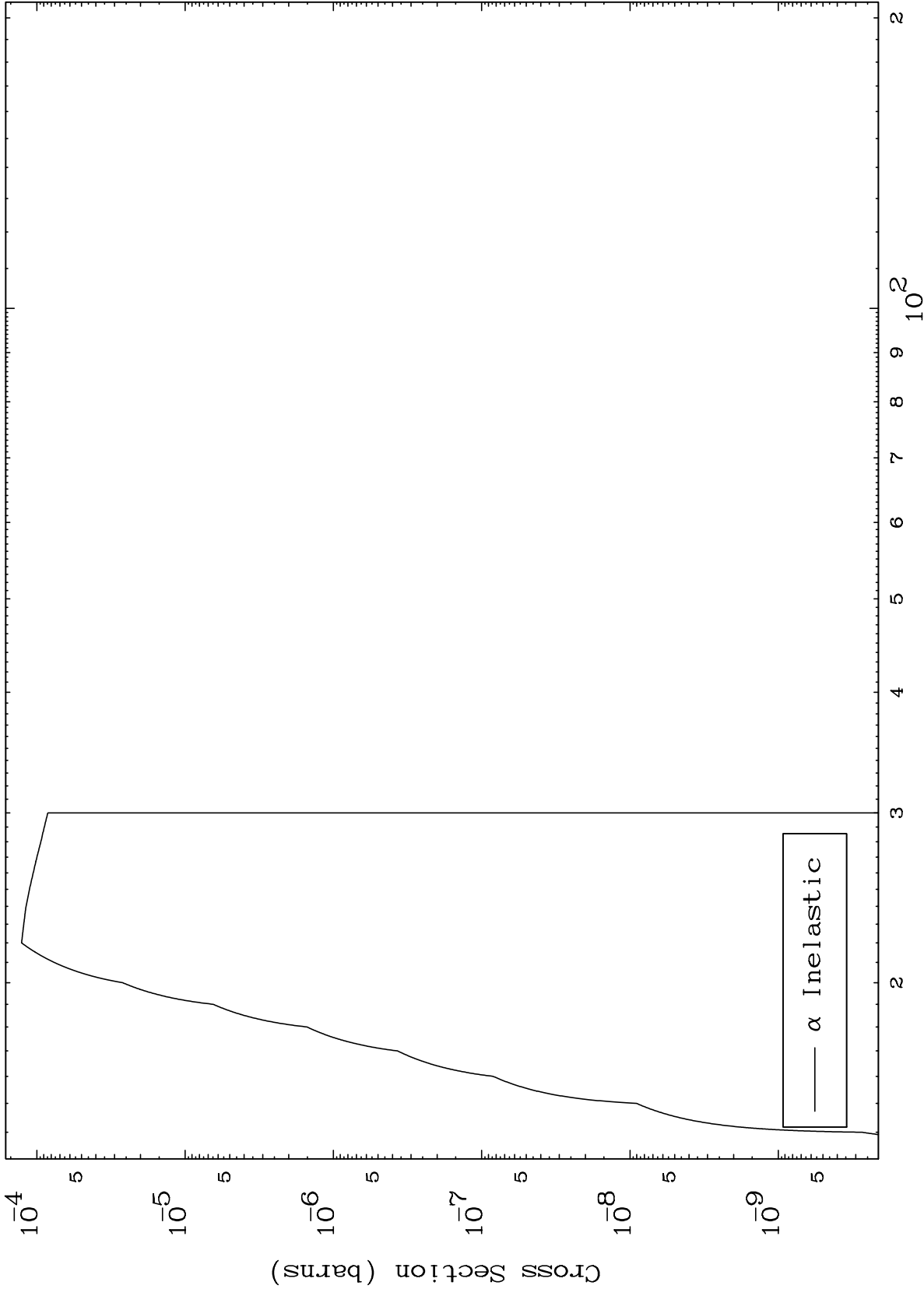
101-Md-261



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101-Md-261

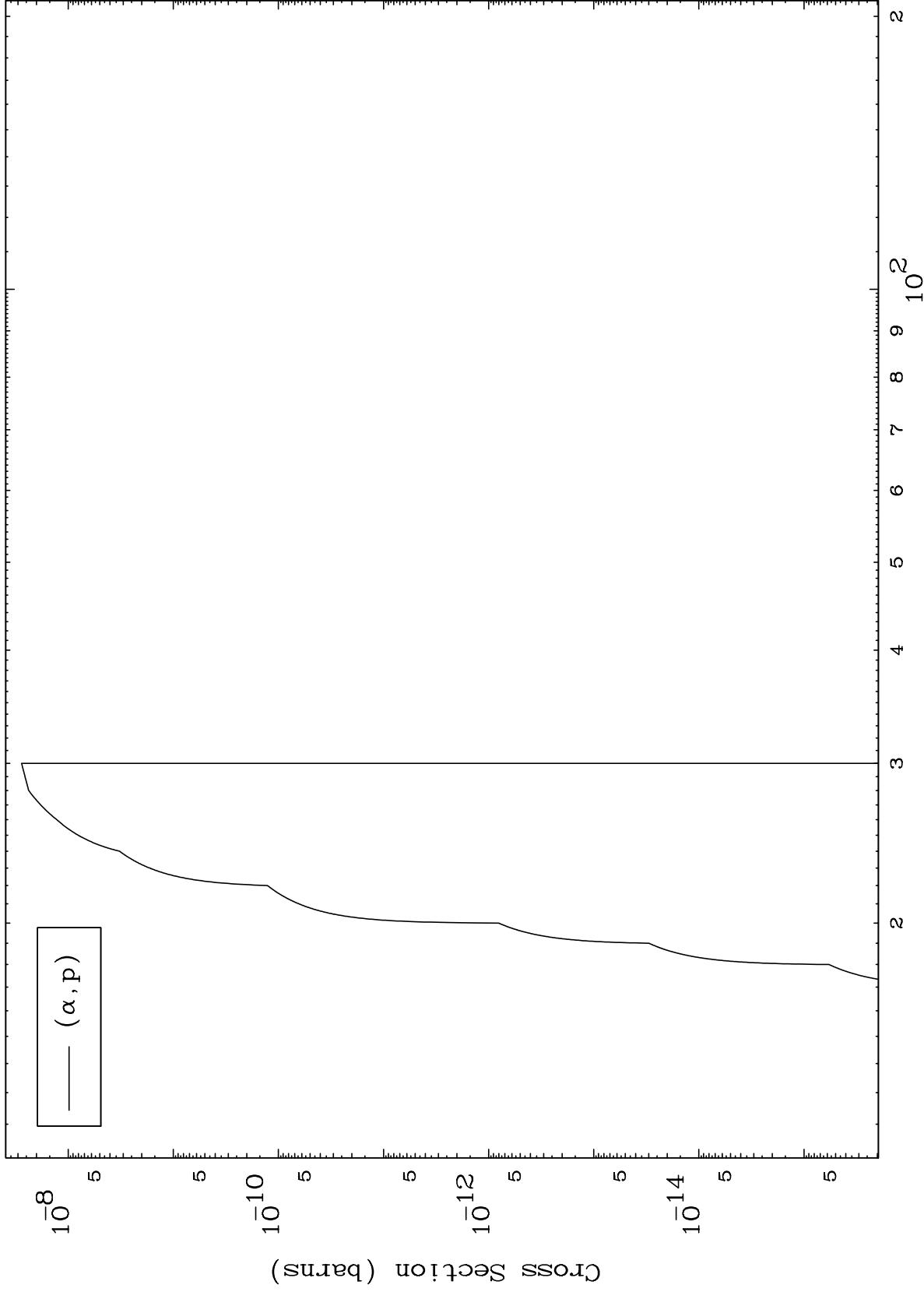
( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

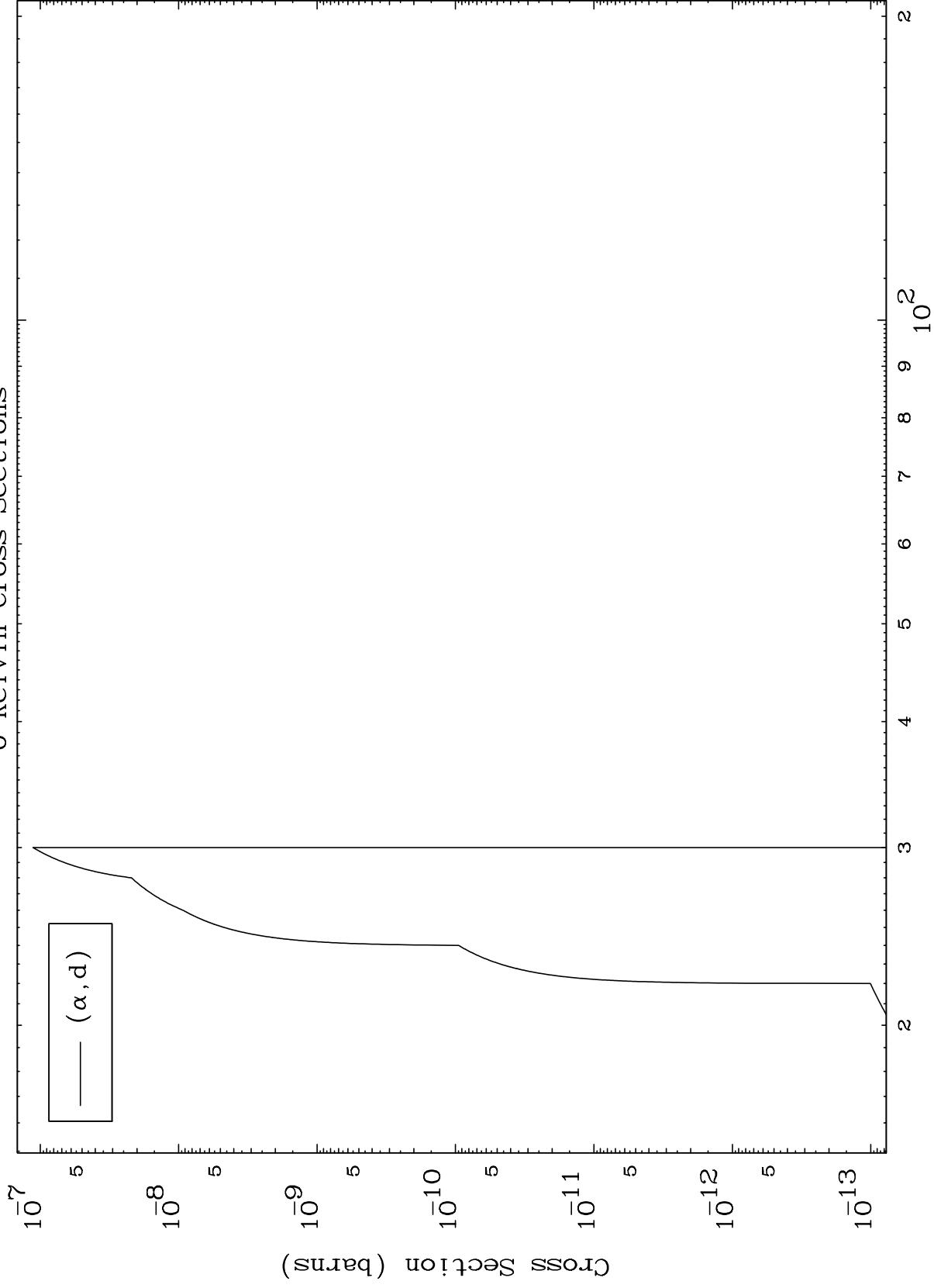


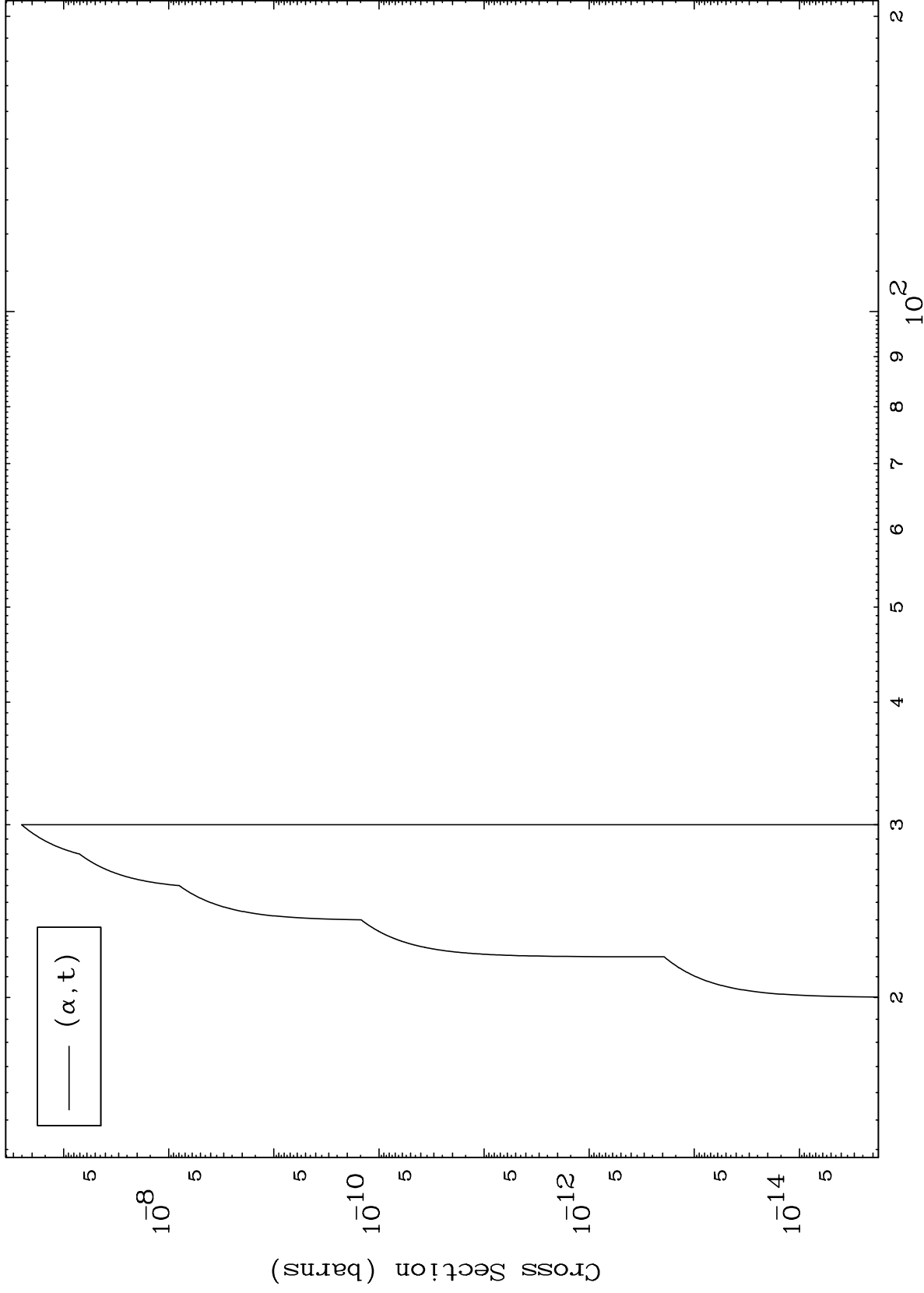
101-Md-261

Incident Energy (MeV)

4





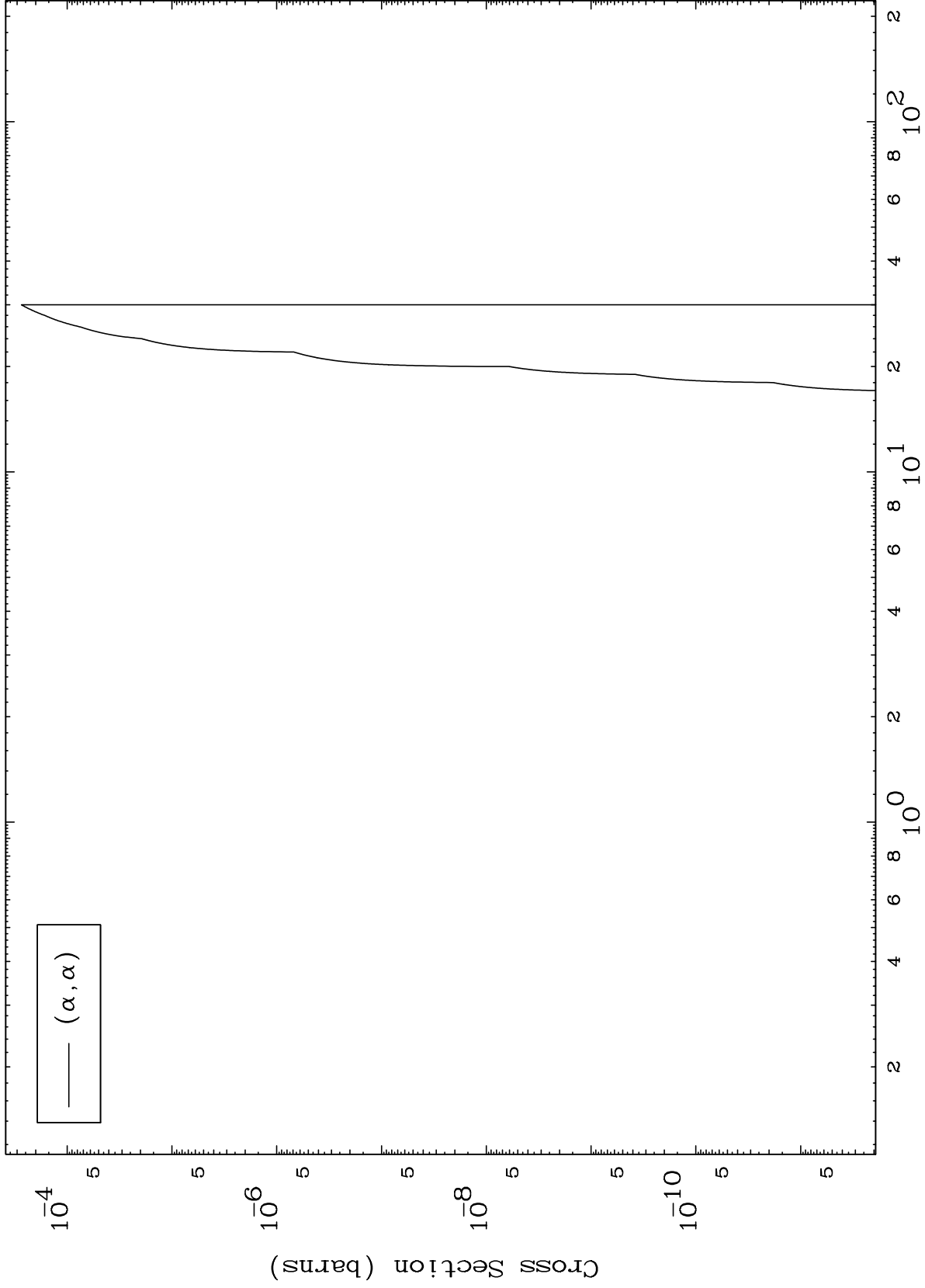




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( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

101-Md-261



MAT 9962

101-Md-261

$\alpha$  Fission  
Radionuclide Production Cross Section

