

Program EVALPLOT  
(Version 2018-1)

by

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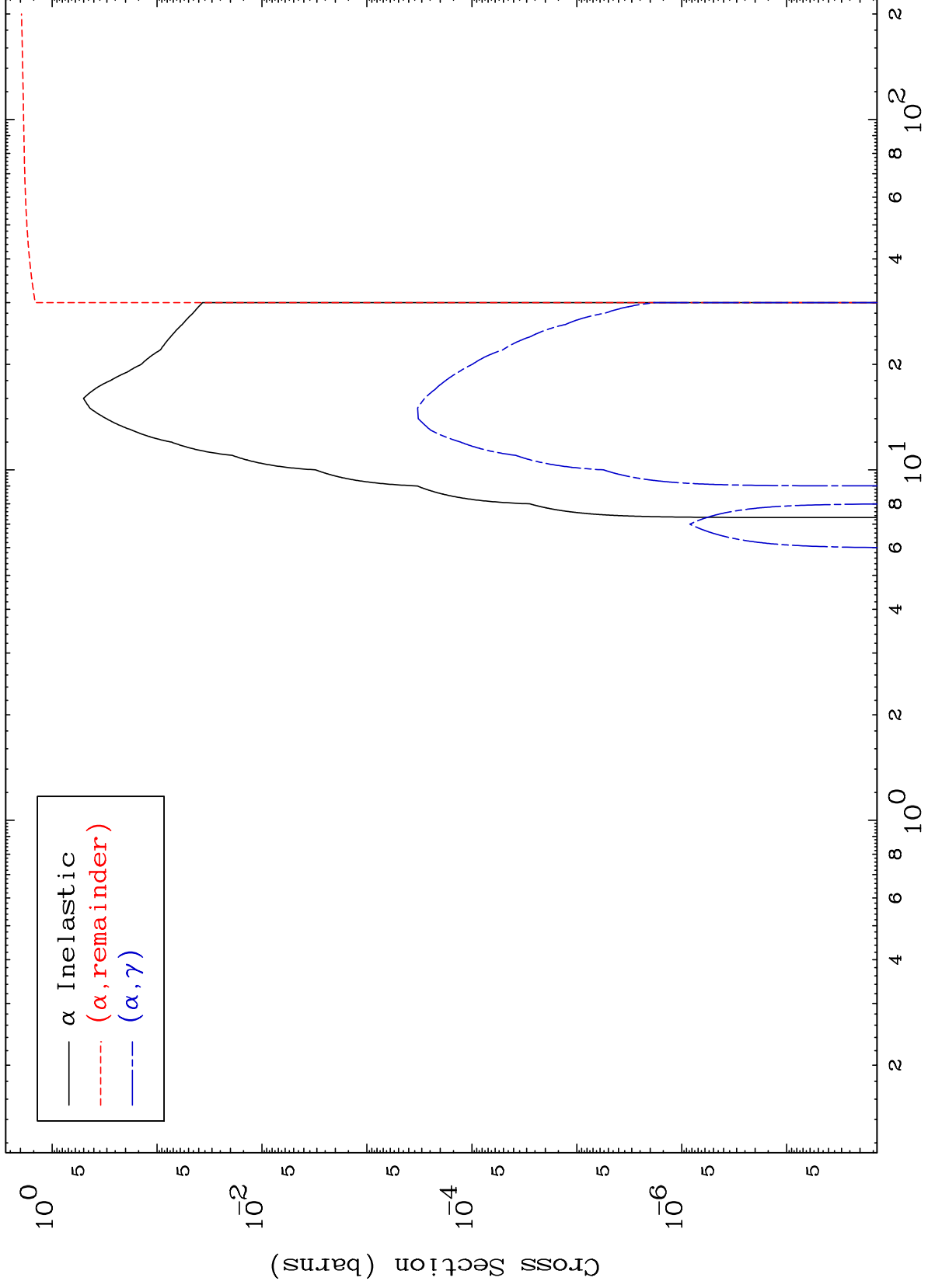
Press Mouse Button to Start

MAT 4126

$\alpha$  Major

41-Nb-93

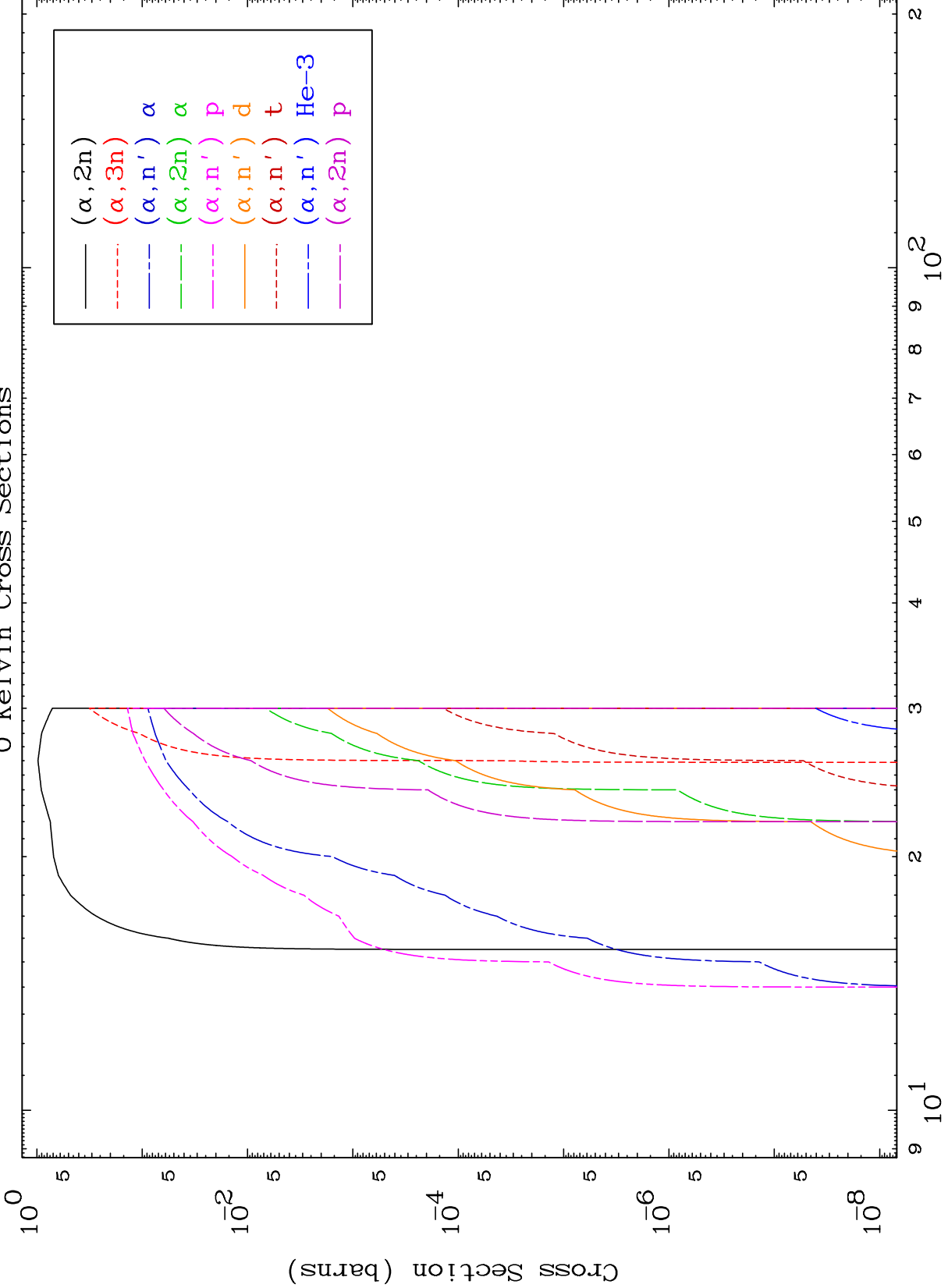
0 Kelvin Cross Sections



MAT 4126

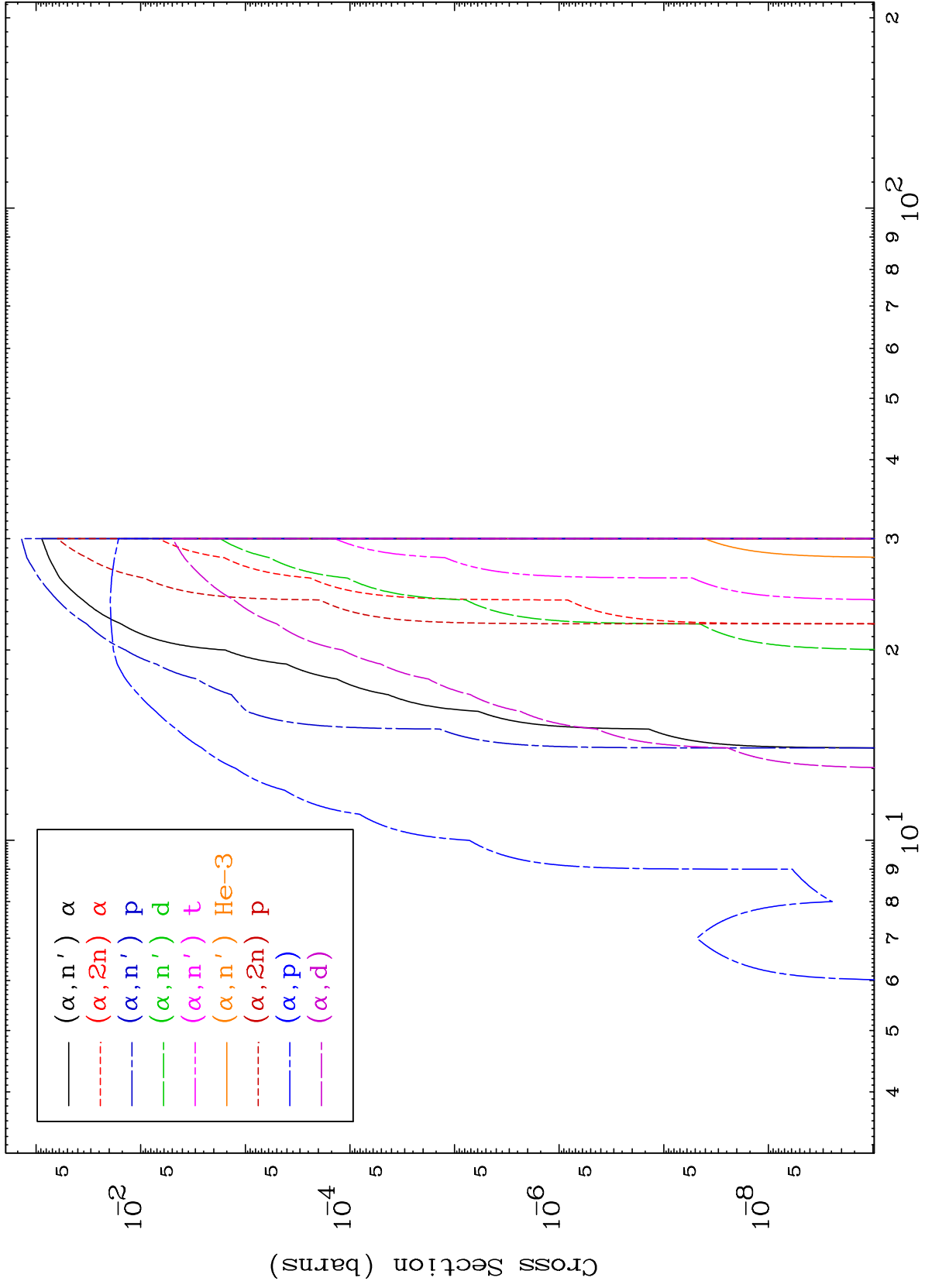
$\alpha$  Neutron Production  
0 Kelvin Cross Sections

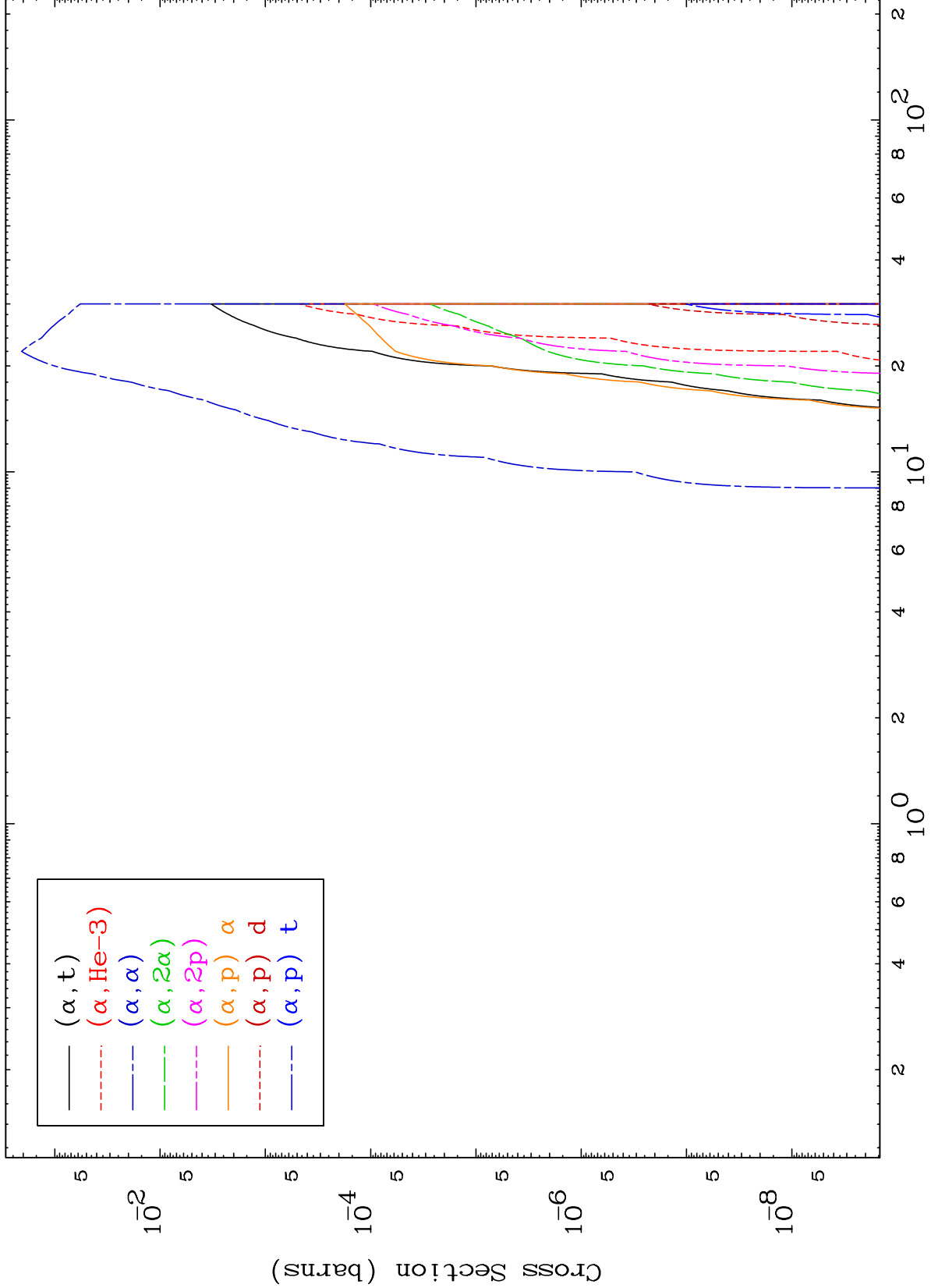
41-Nb-93



Incident Energy (MeV)

41-Nb-93



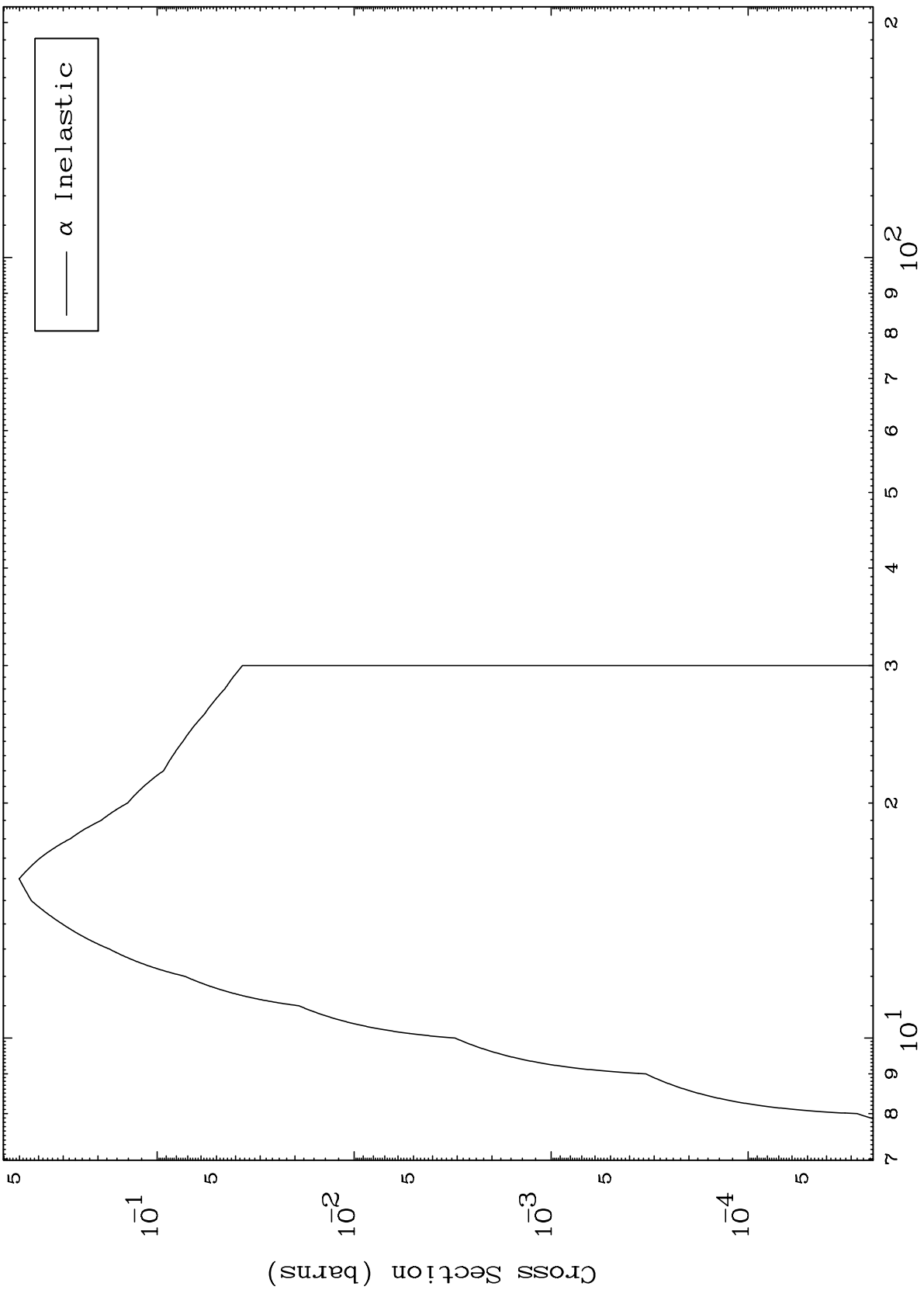


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( $\alpha, n'$ ) Level

41-Nb-93

0 Kelvin Cross Sections

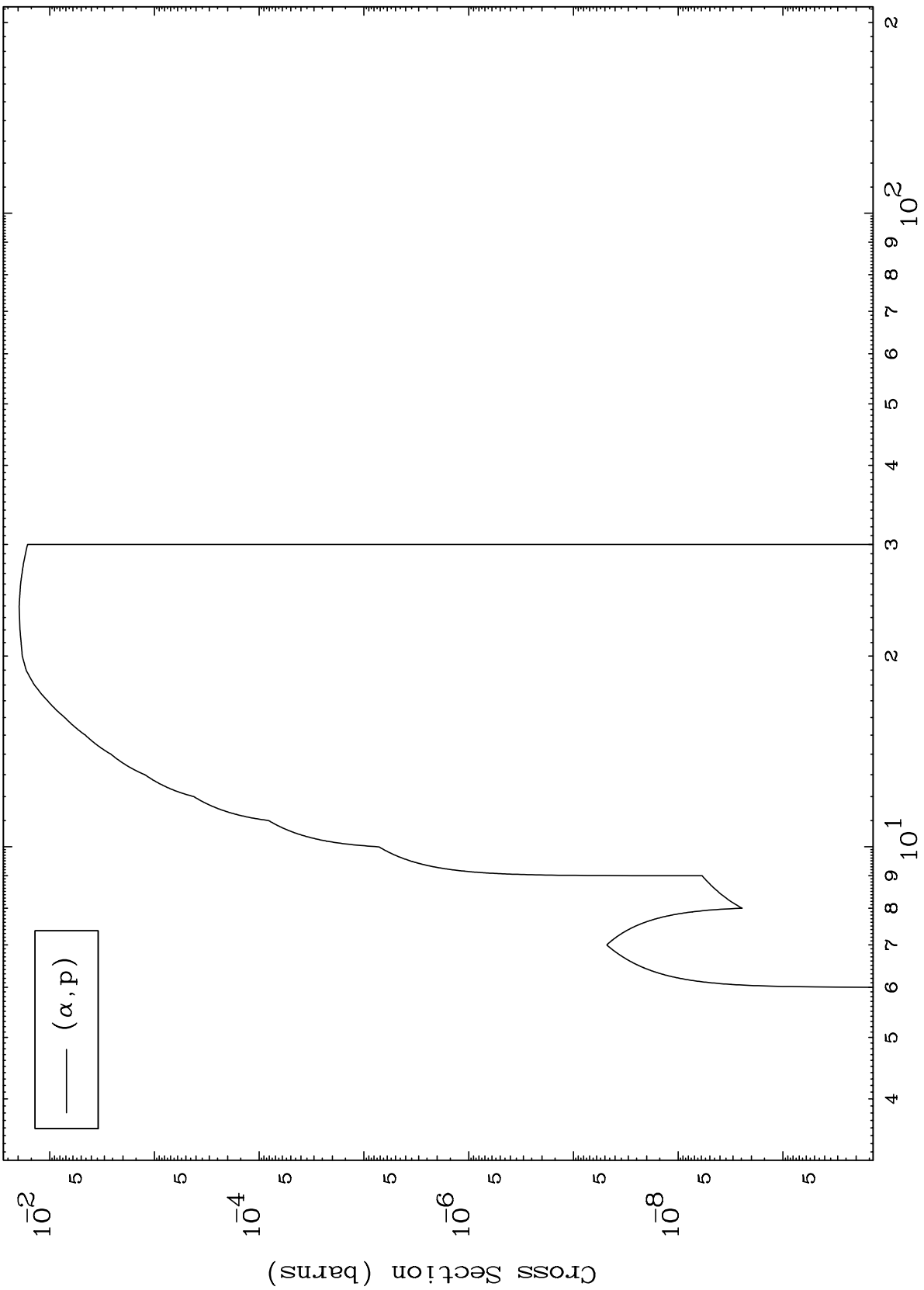


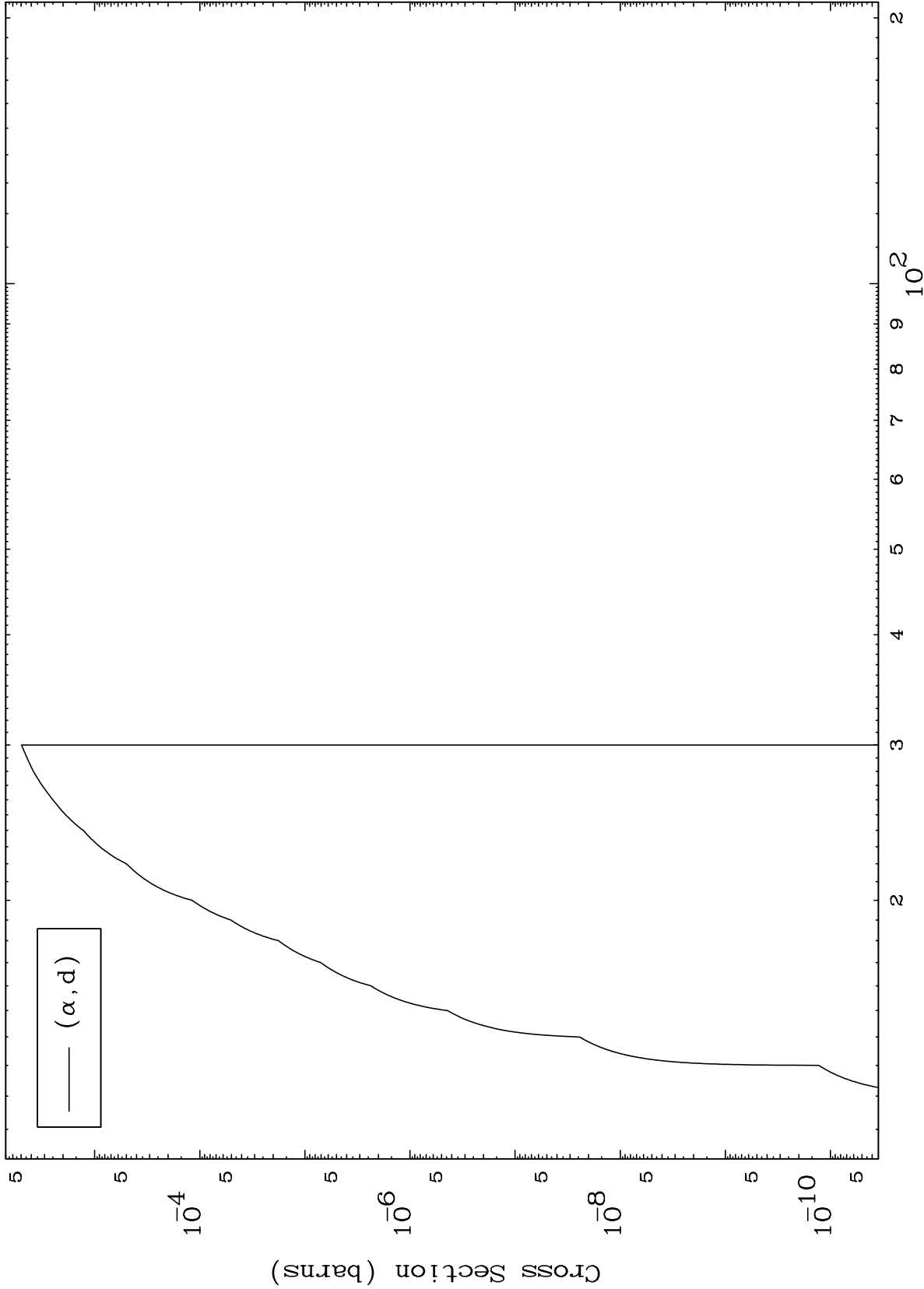
5

Incident Energy (MeV)

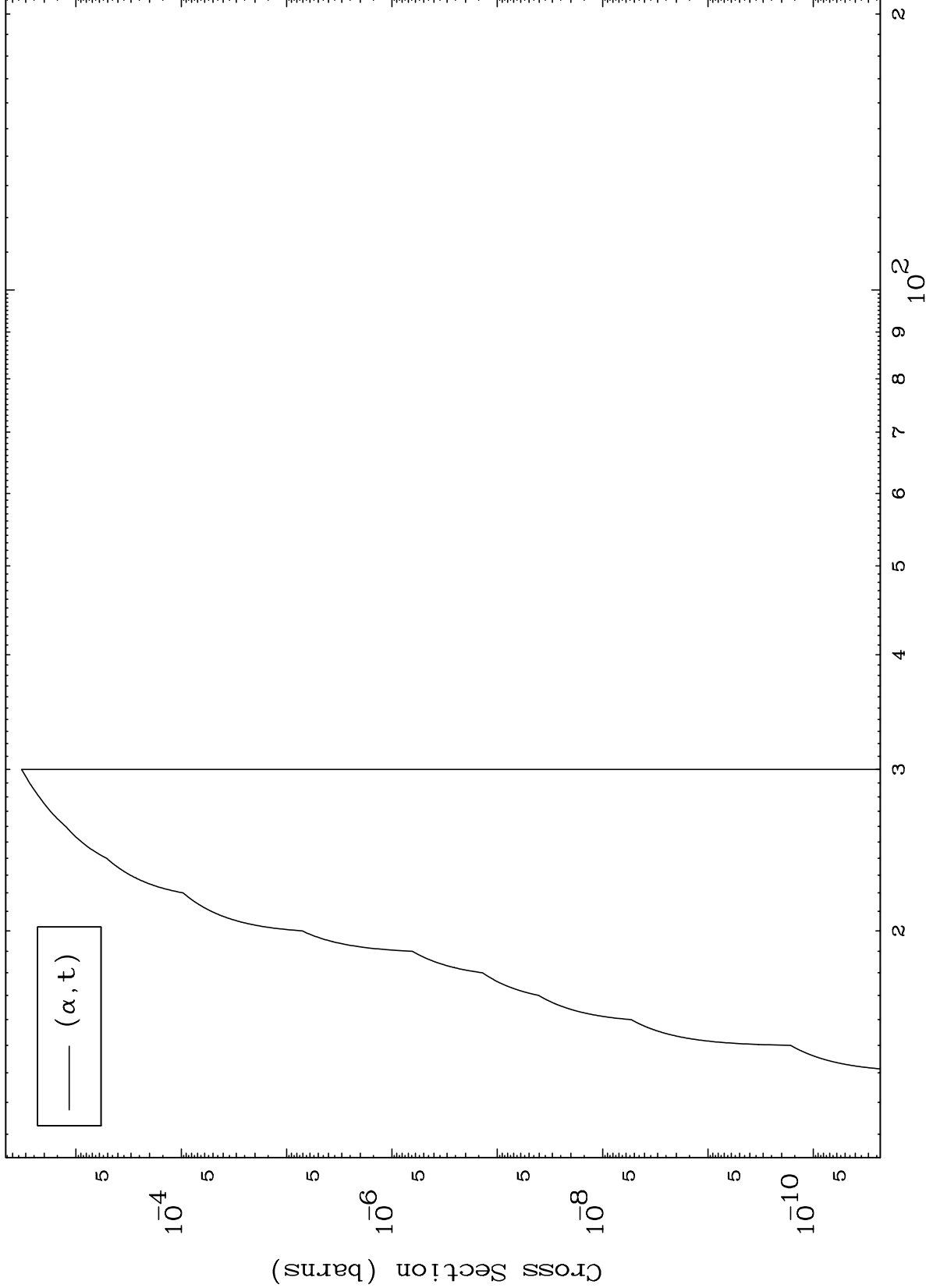
41-Nb-93

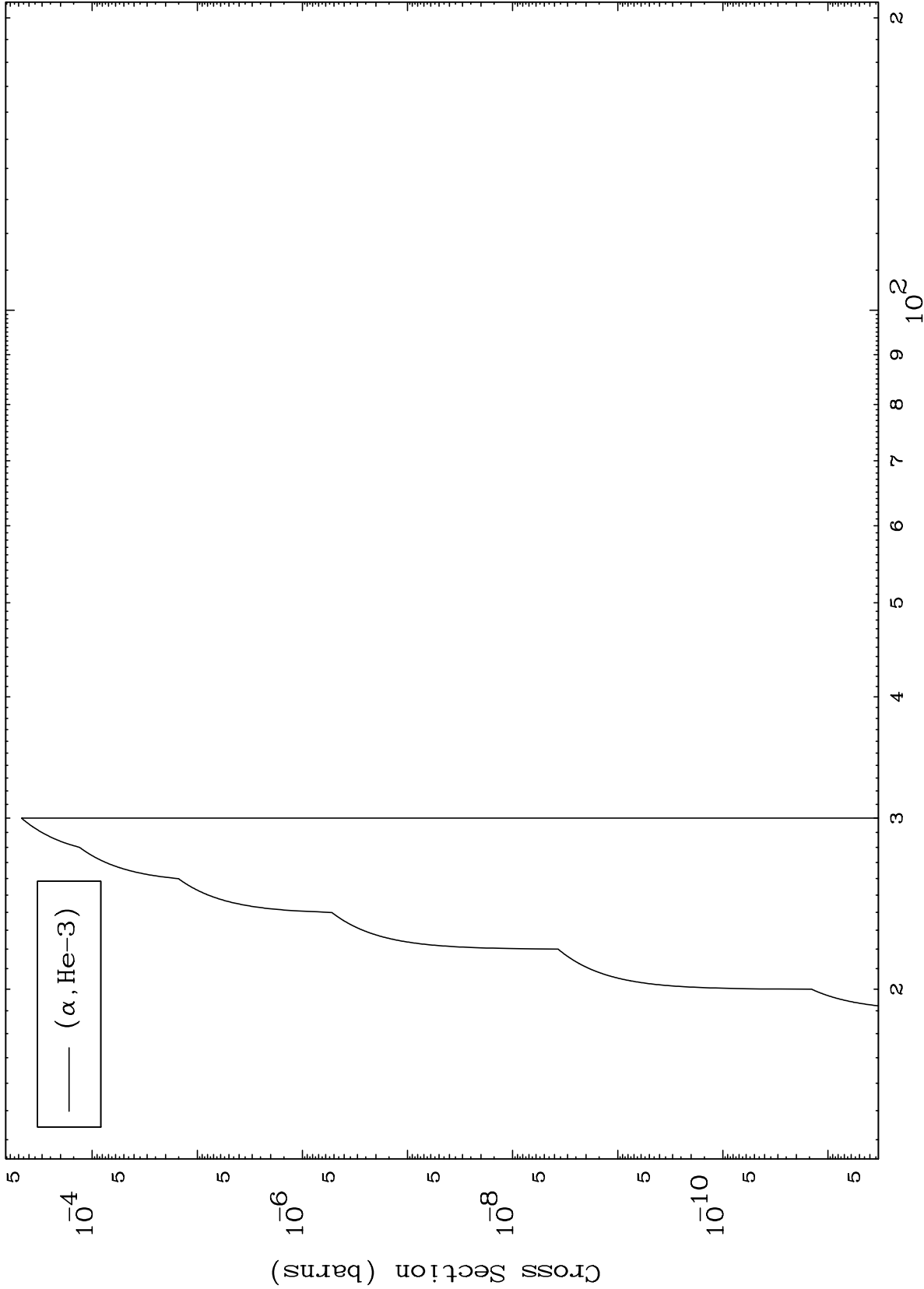
( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections









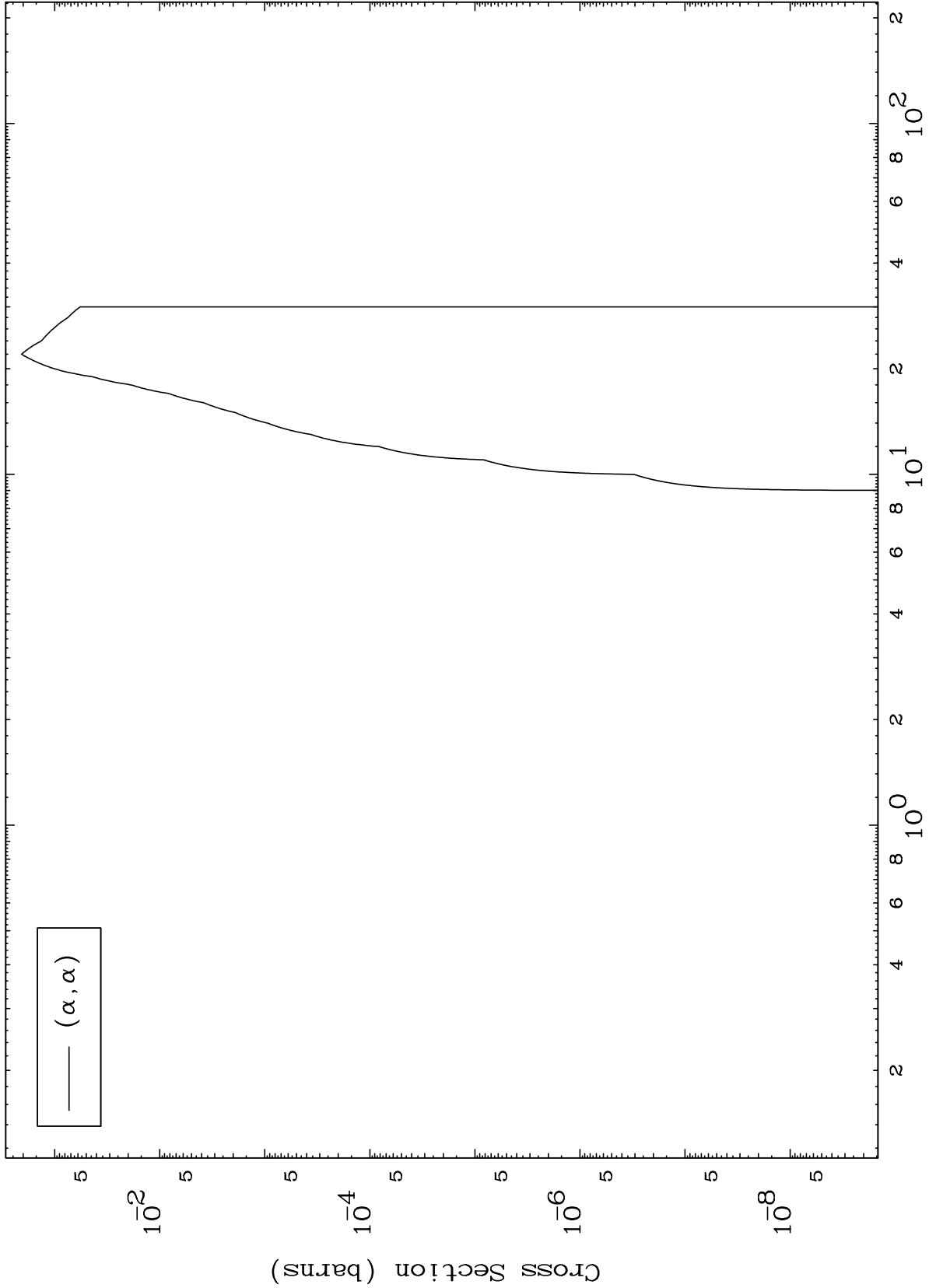


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( $\alpha, \alpha$ ) Levels

41-Nb-93

0 Kelvin Cross Sections



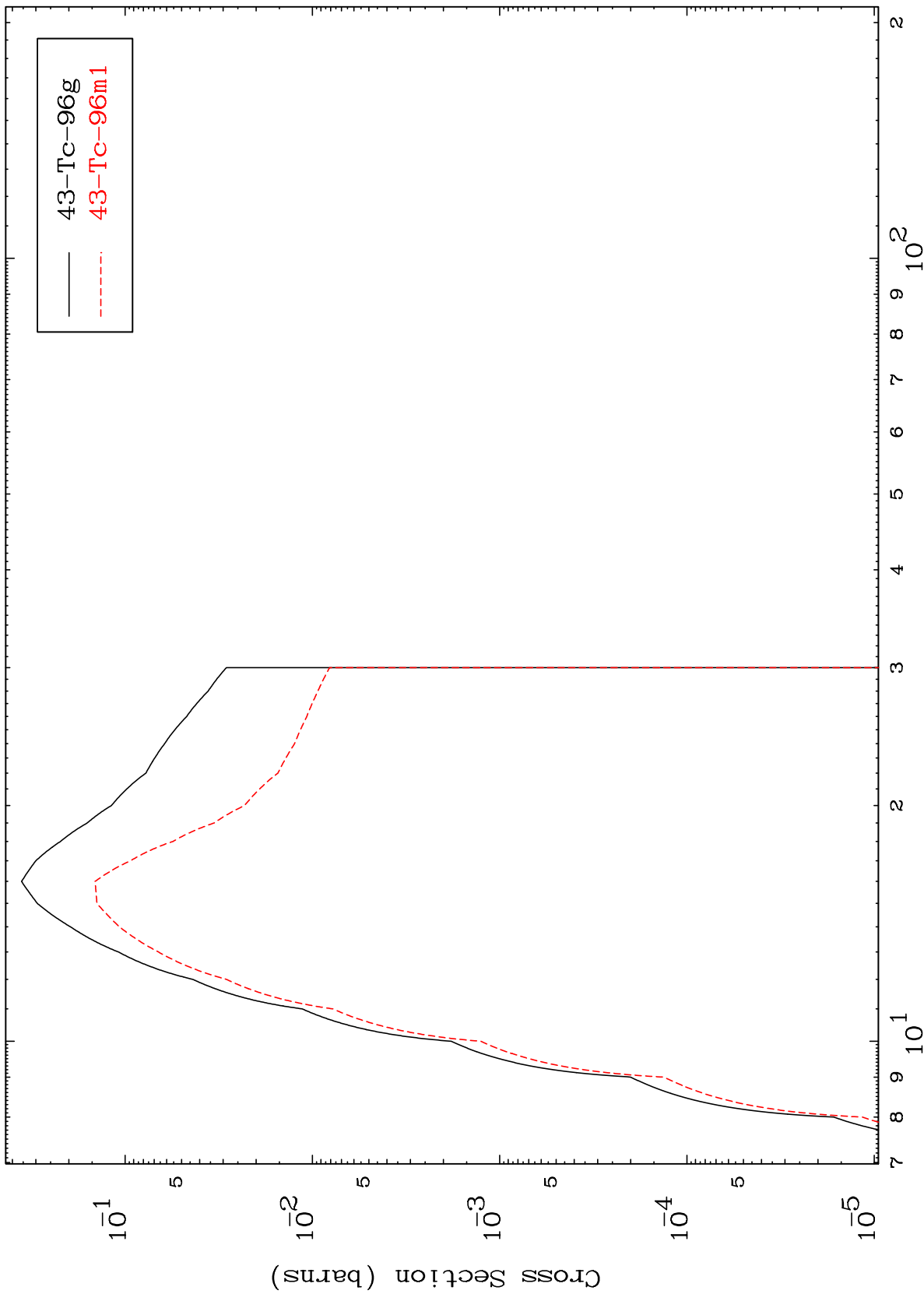
( $\alpha, \alpha$ )

10

Incident Energy (MeV)

41-Nb-93

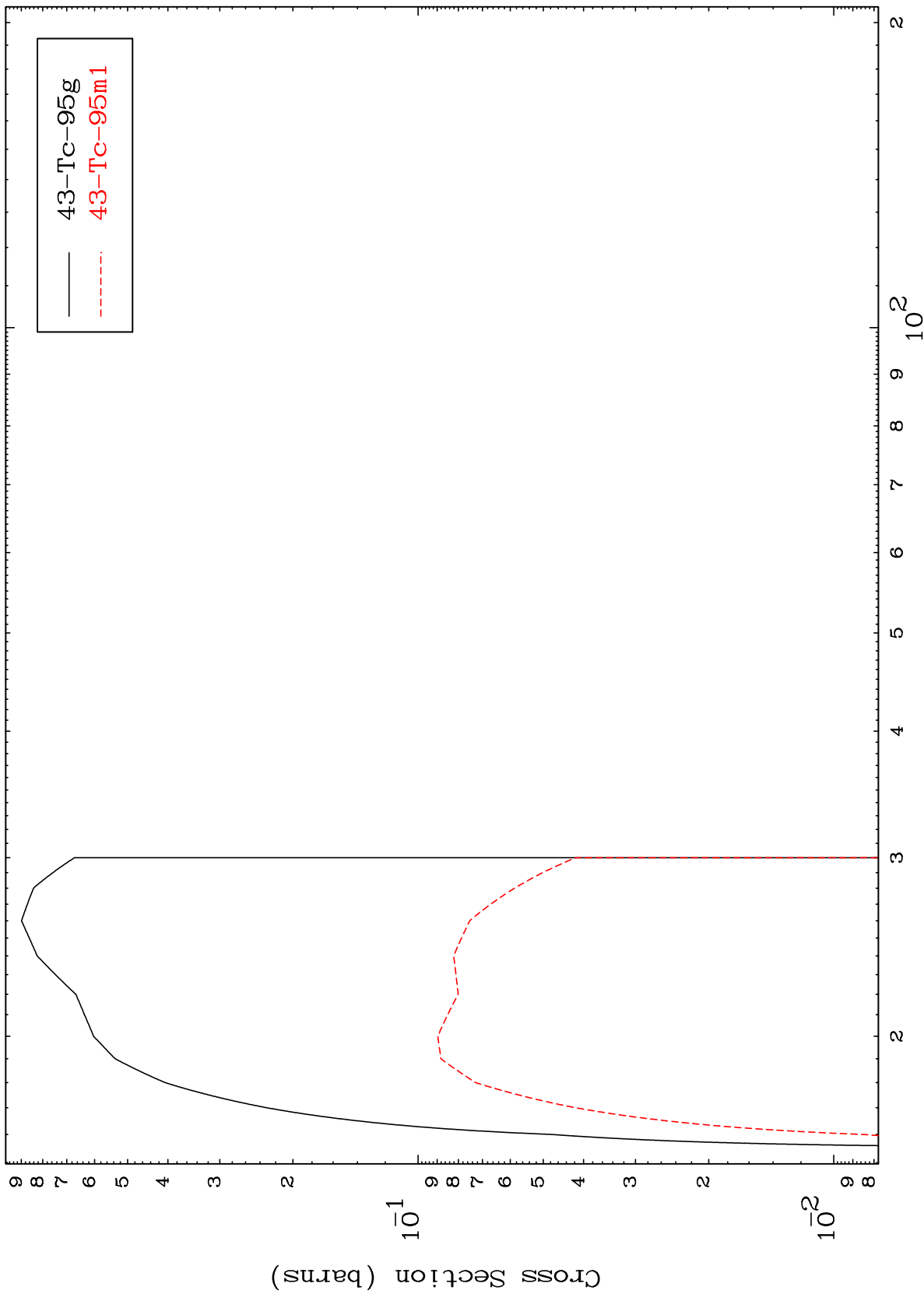
Radionuclide  $\alpha$  Inelastic Production Cross Section



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41-Nb-93

( $\alpha, 2n$ )  
Radionuclide Production Cross Section



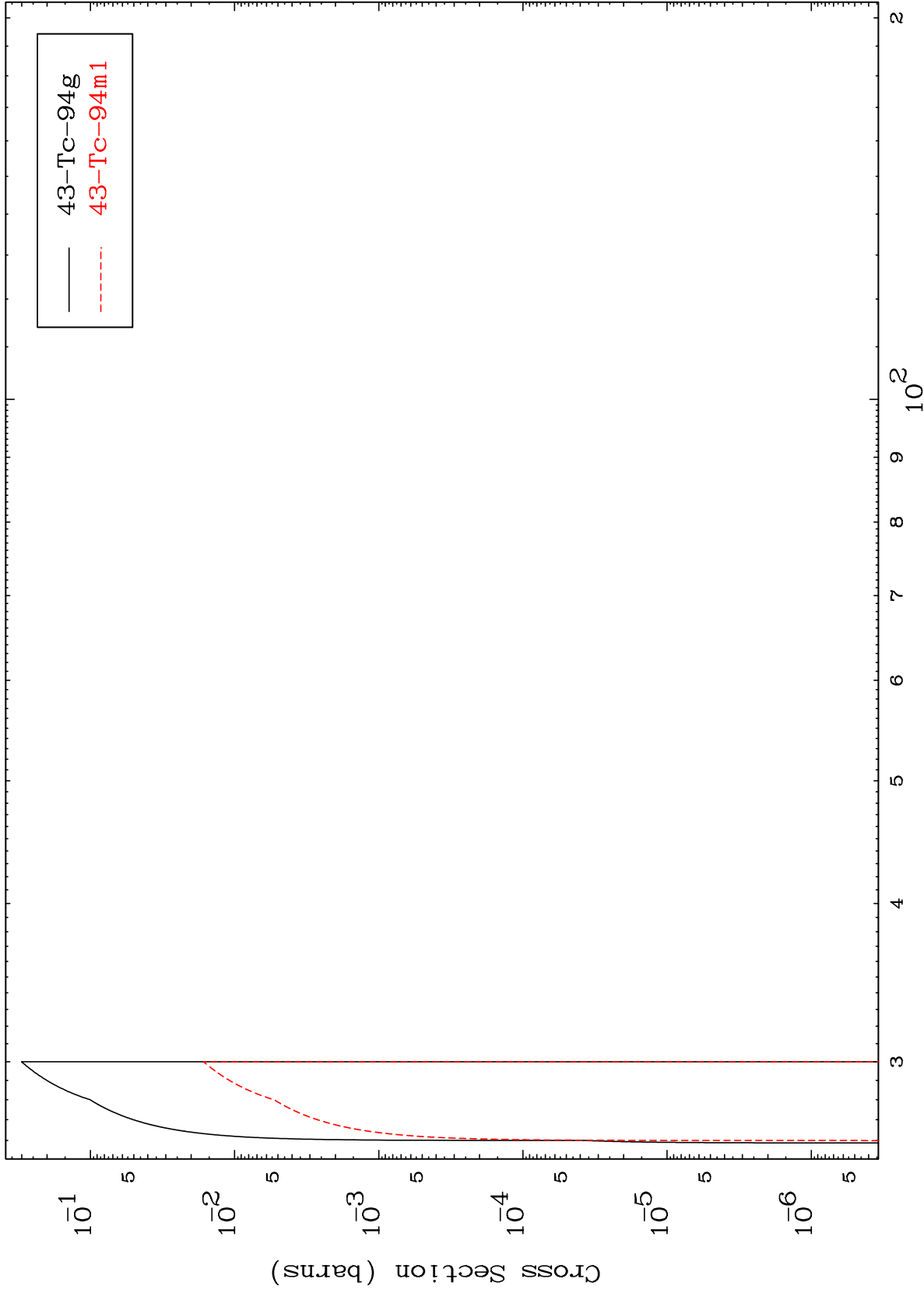
43-Tc-95g  
43-Tc-95m1

12

Incident Energy (MeV)

41-Nb-93

Radionuclide Production Cross Section

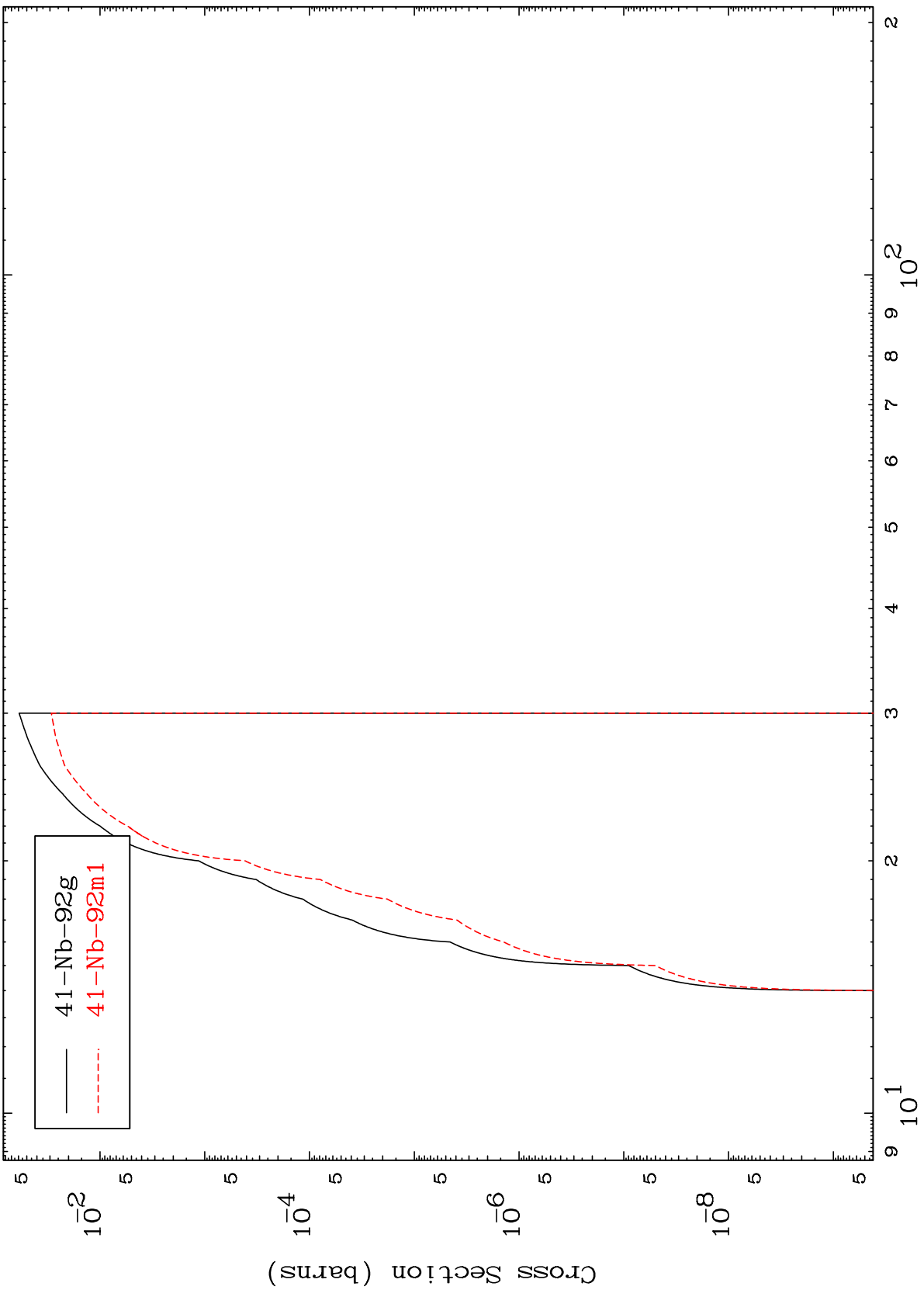


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( $\alpha, n'$ )  $\alpha$

41-Nb-93

Radionuclide Production Cross Section



14

Incident Energy (MeV)

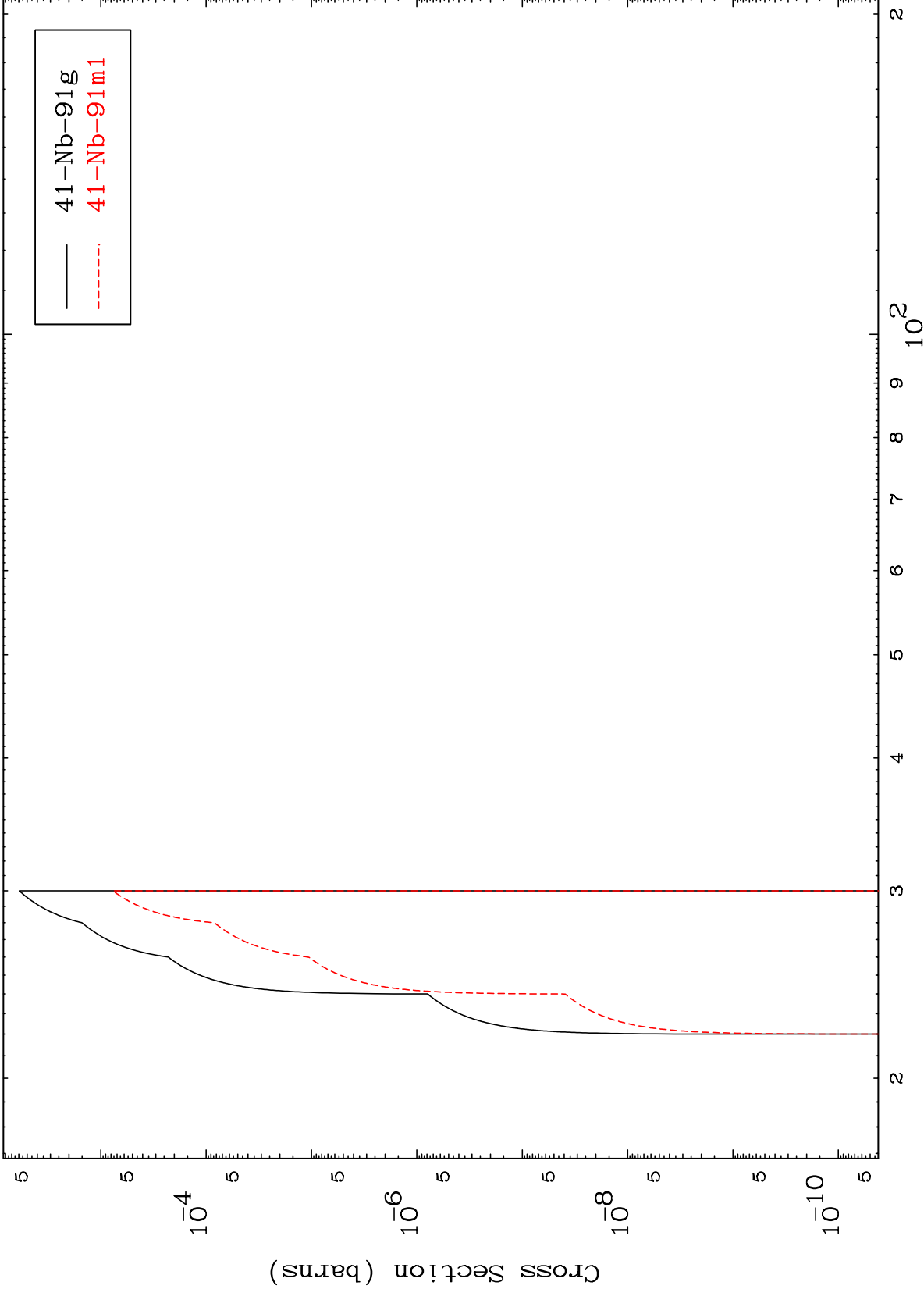
41-Nb-93

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$(\alpha, 2n)$   $\alpha$

41-Nb-93

Radionuclide Production Cross Section



15

Incident Energy (MeV)

41-Nb-93

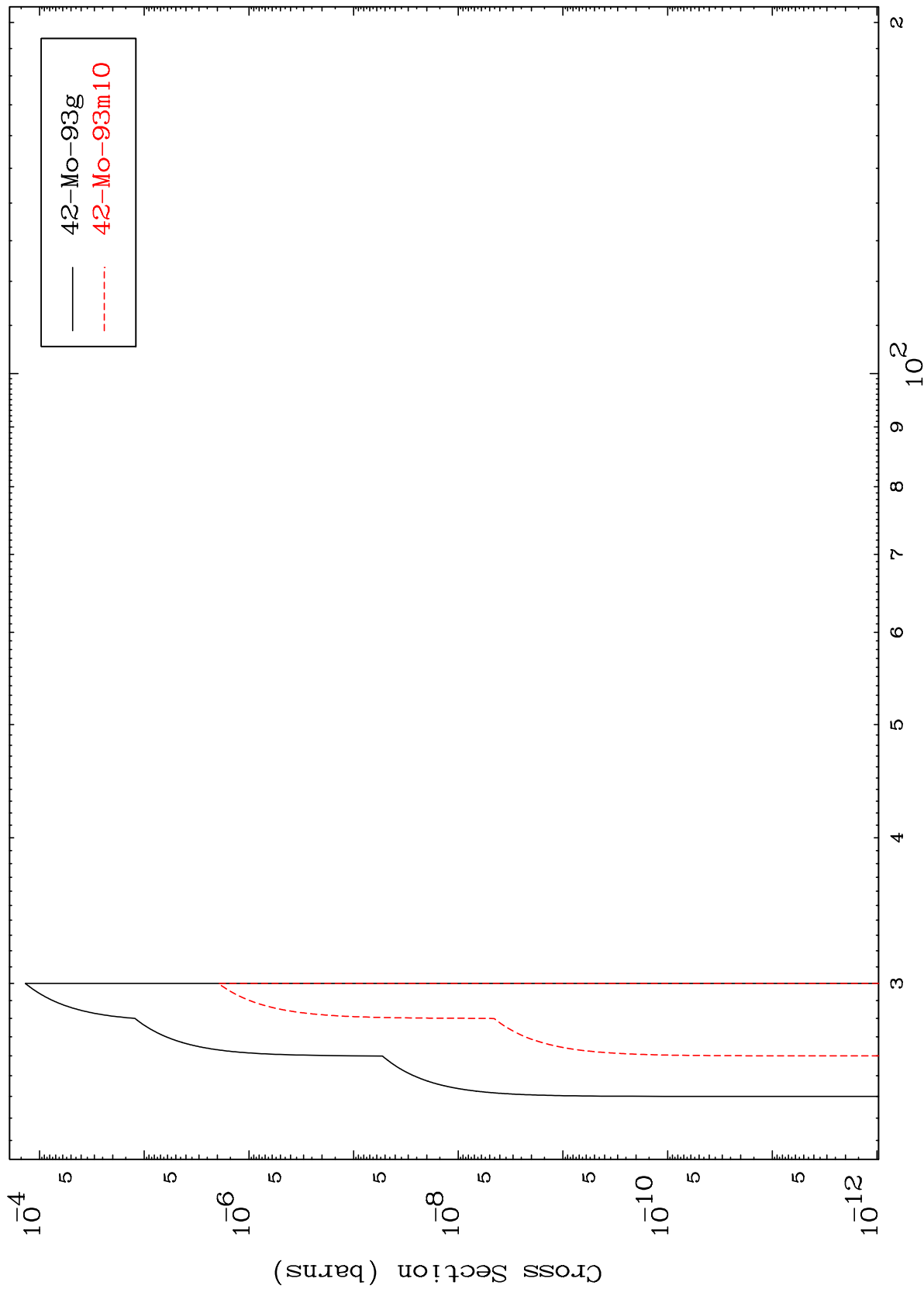


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$(\alpha, n')$  t

41-Nb-93

Radionuclide Production Cross Section

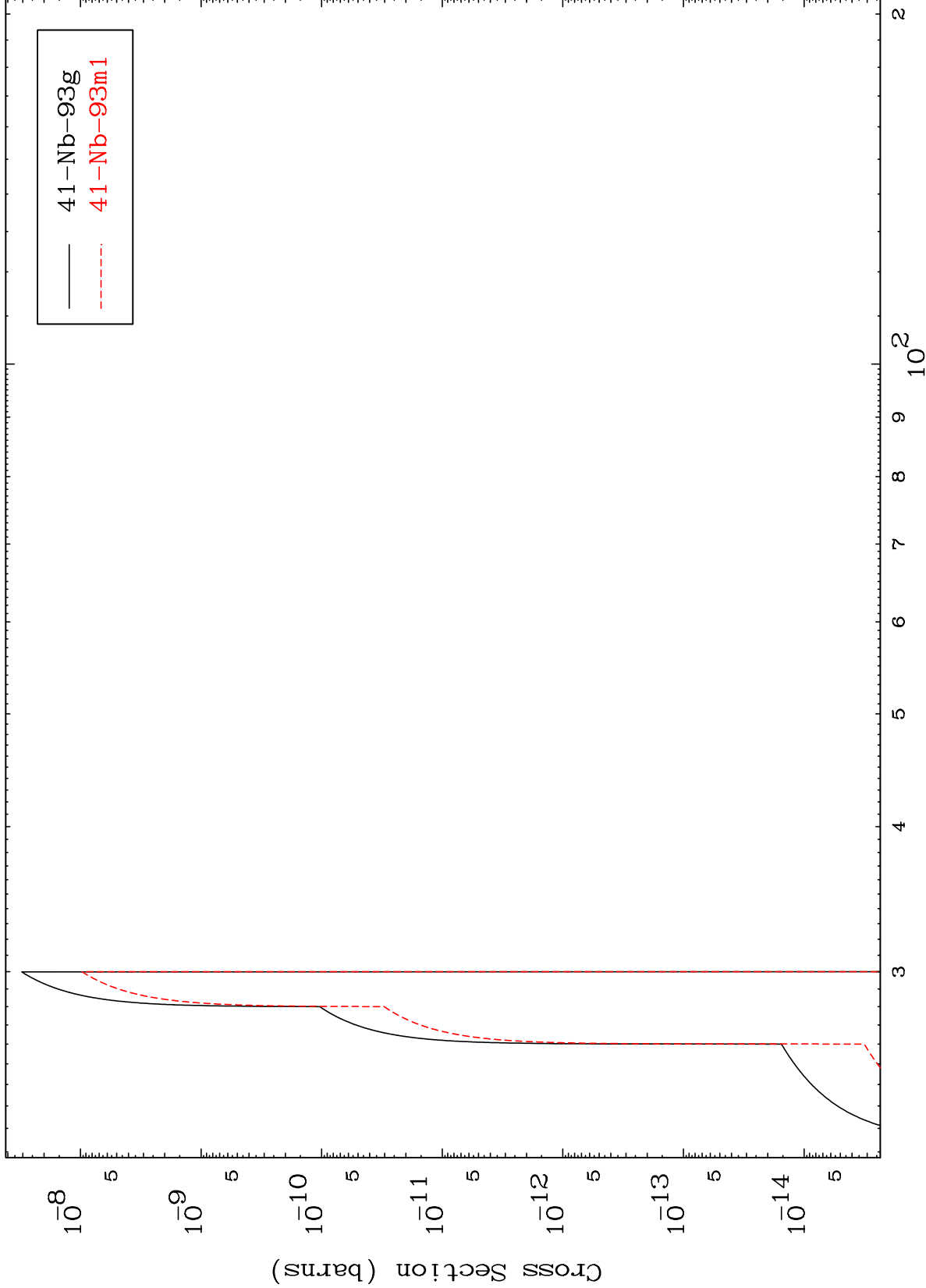


16

Incident Energy (MeV)

41-Nb-93

Radionuclide Production Cross Section

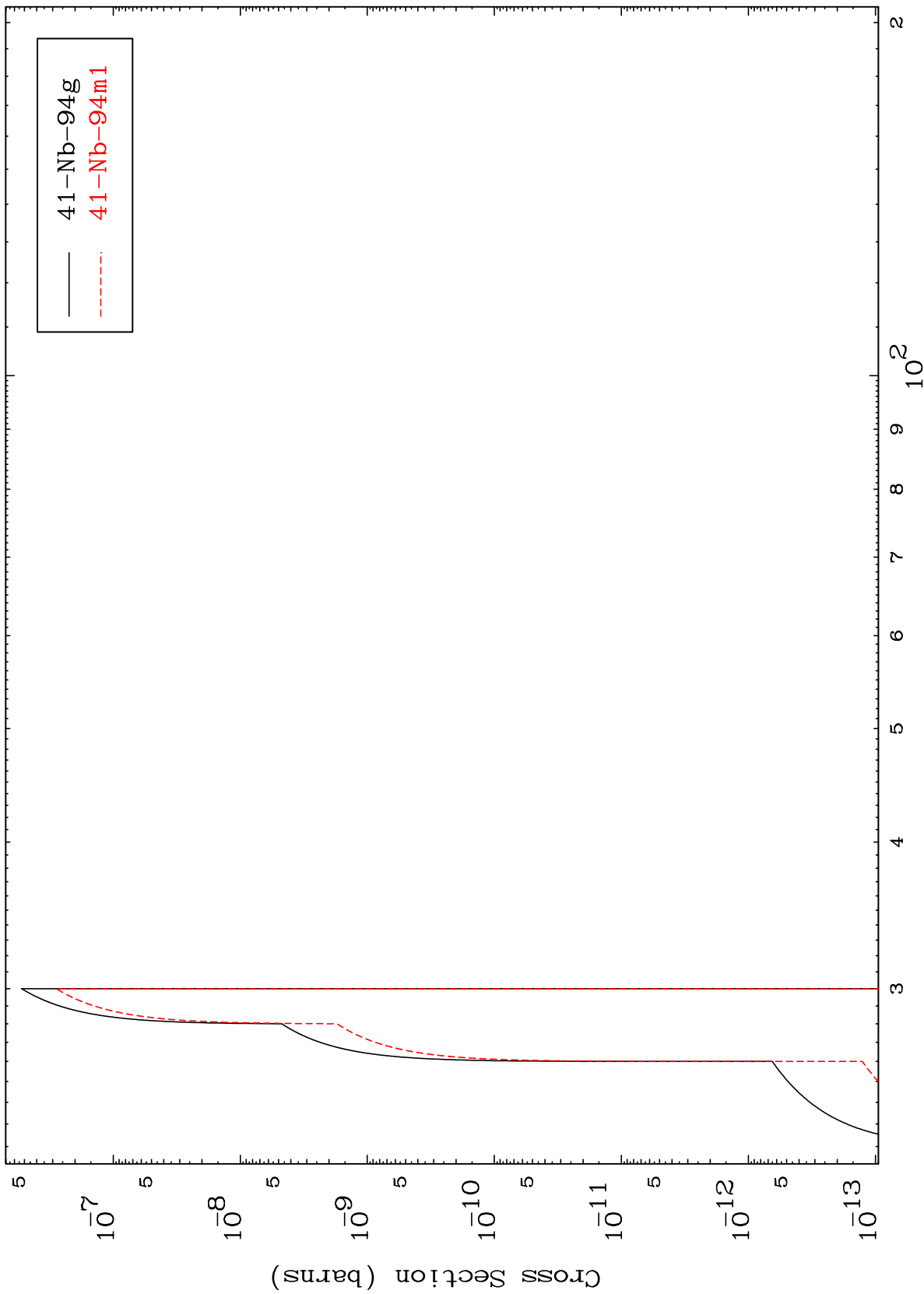


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( $\alpha, 2n$ ) p

41-Nb-93

Radionuclide Production Cross Section



18

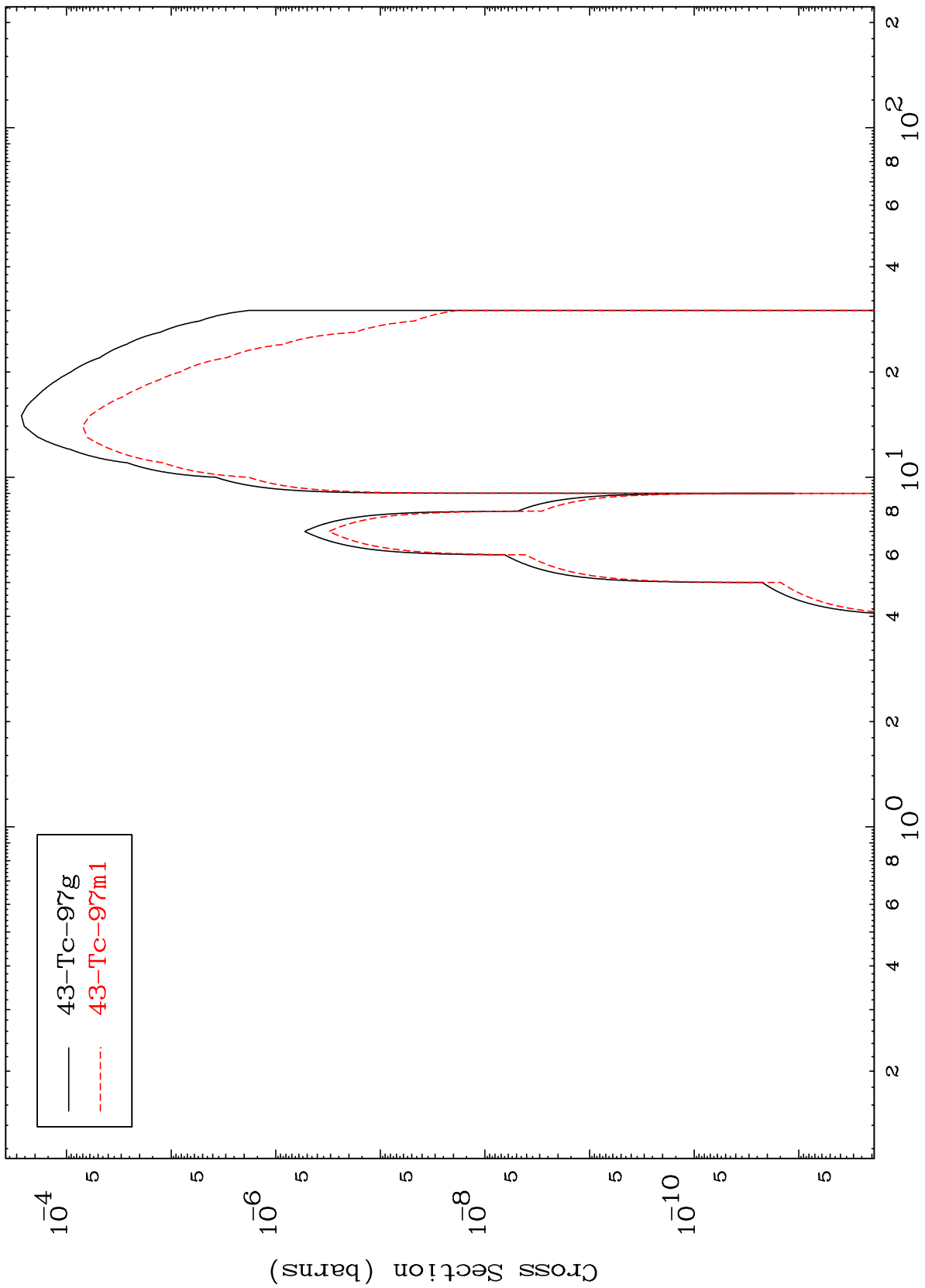
Incident Energy (MeV)

41-Nb-93

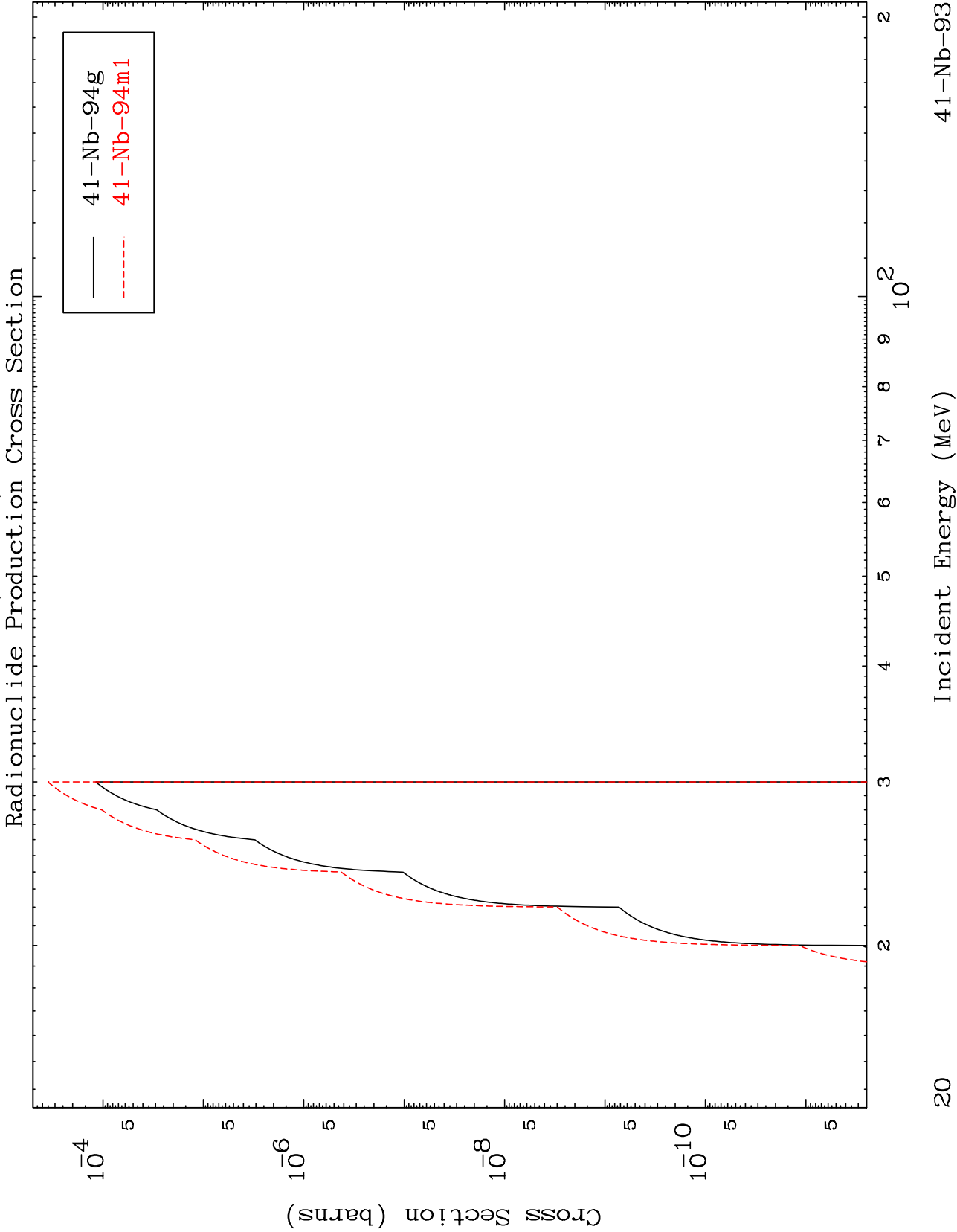
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41-Nb-93

$(\alpha, \gamma)$   
Radionuclide Production Cross Section



— 43-Tc-97g  
- - - 43-Tc-97m1

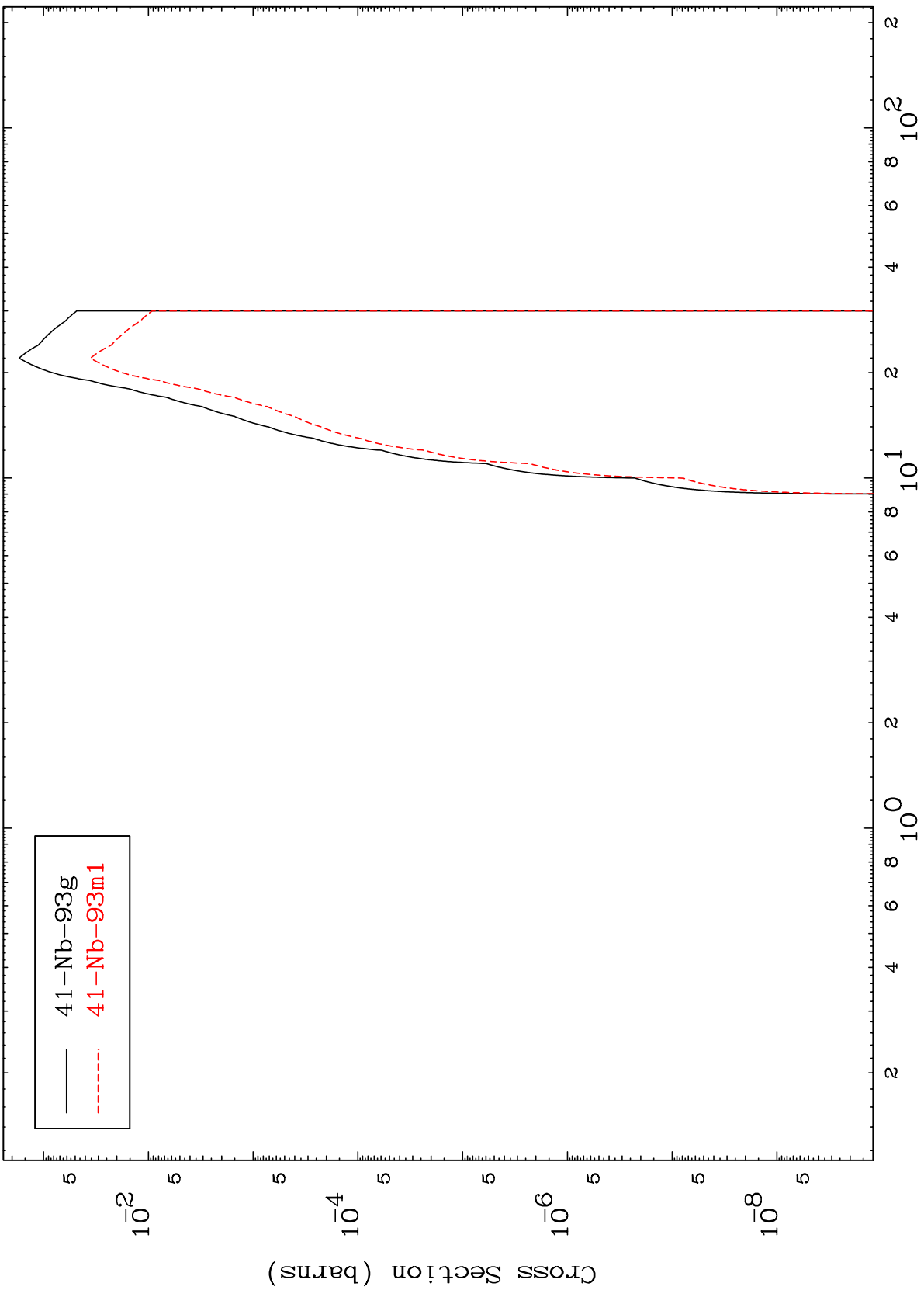


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( $\alpha, \alpha$ )

41-Nb-93

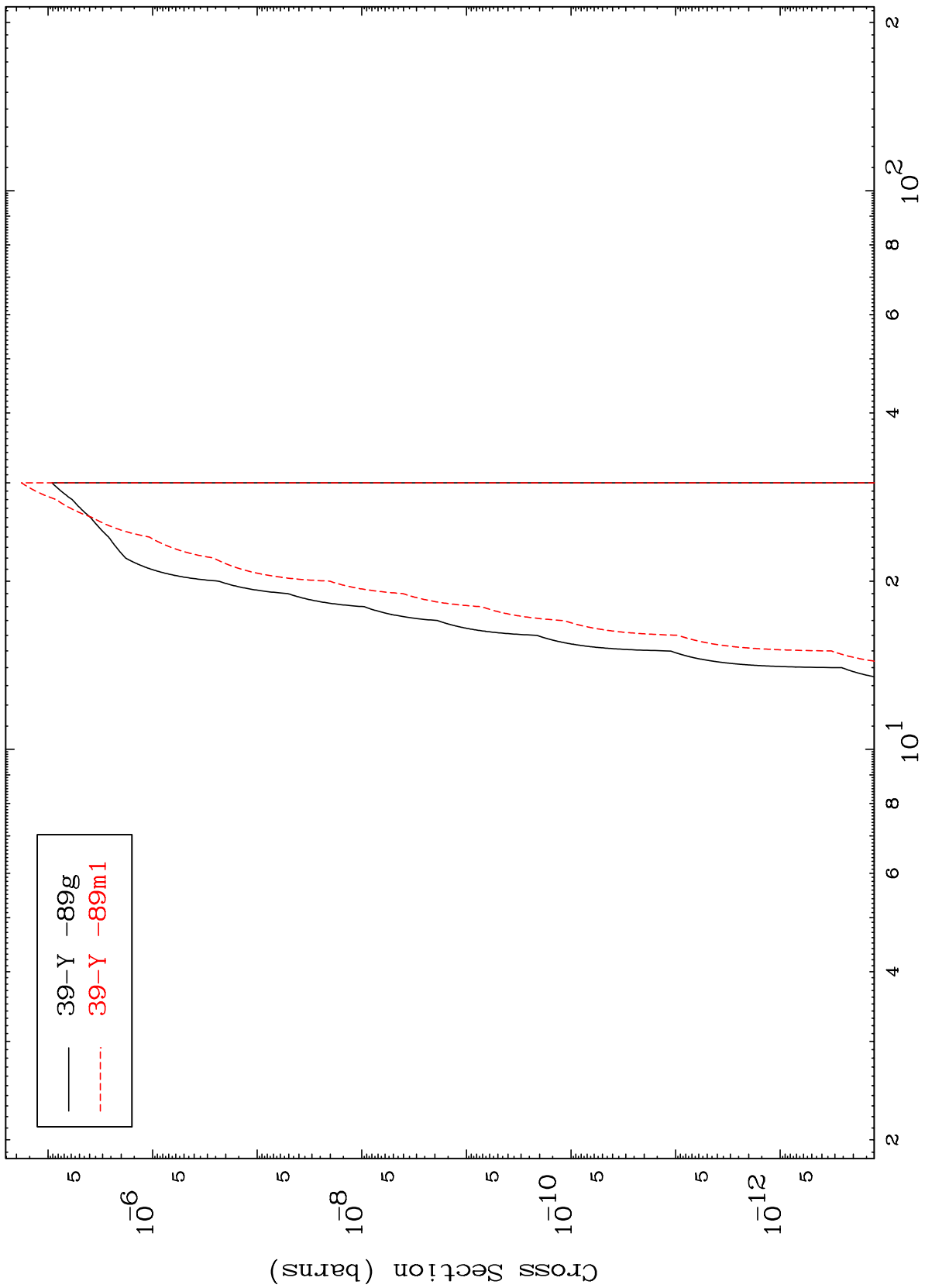
Radionuclide Production Cross Section



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41-Nb-93

( $\alpha, 2\alpha$ )  
Radionuclide Production Cross Section



22

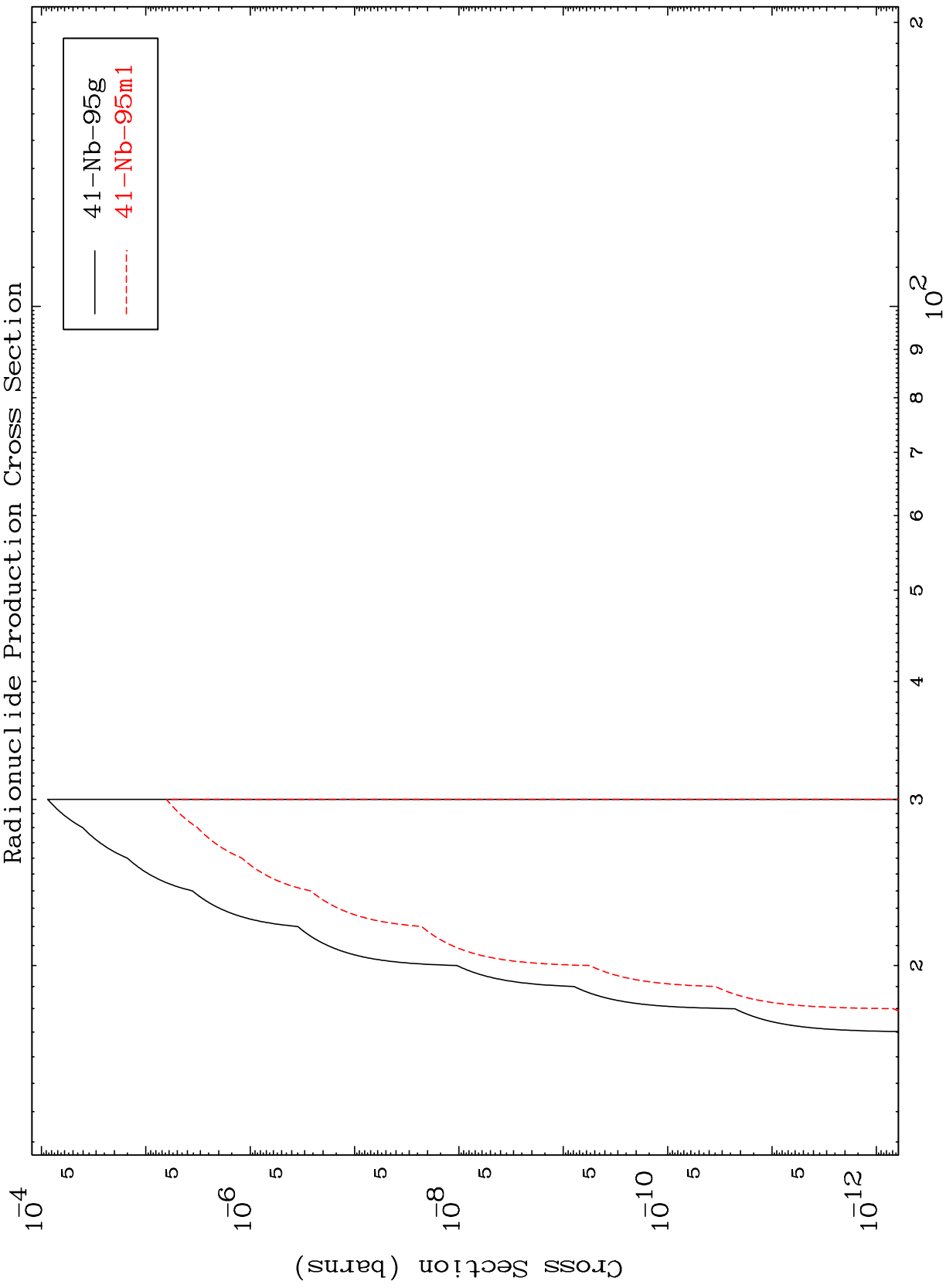
41-Nb-93

Incident Energy (MeV)

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41-Nb-93

( $\alpha, 2p$ )  
Radionuclide Production Cross Section



23

41-Nb-93

Incident Energy (MeV)

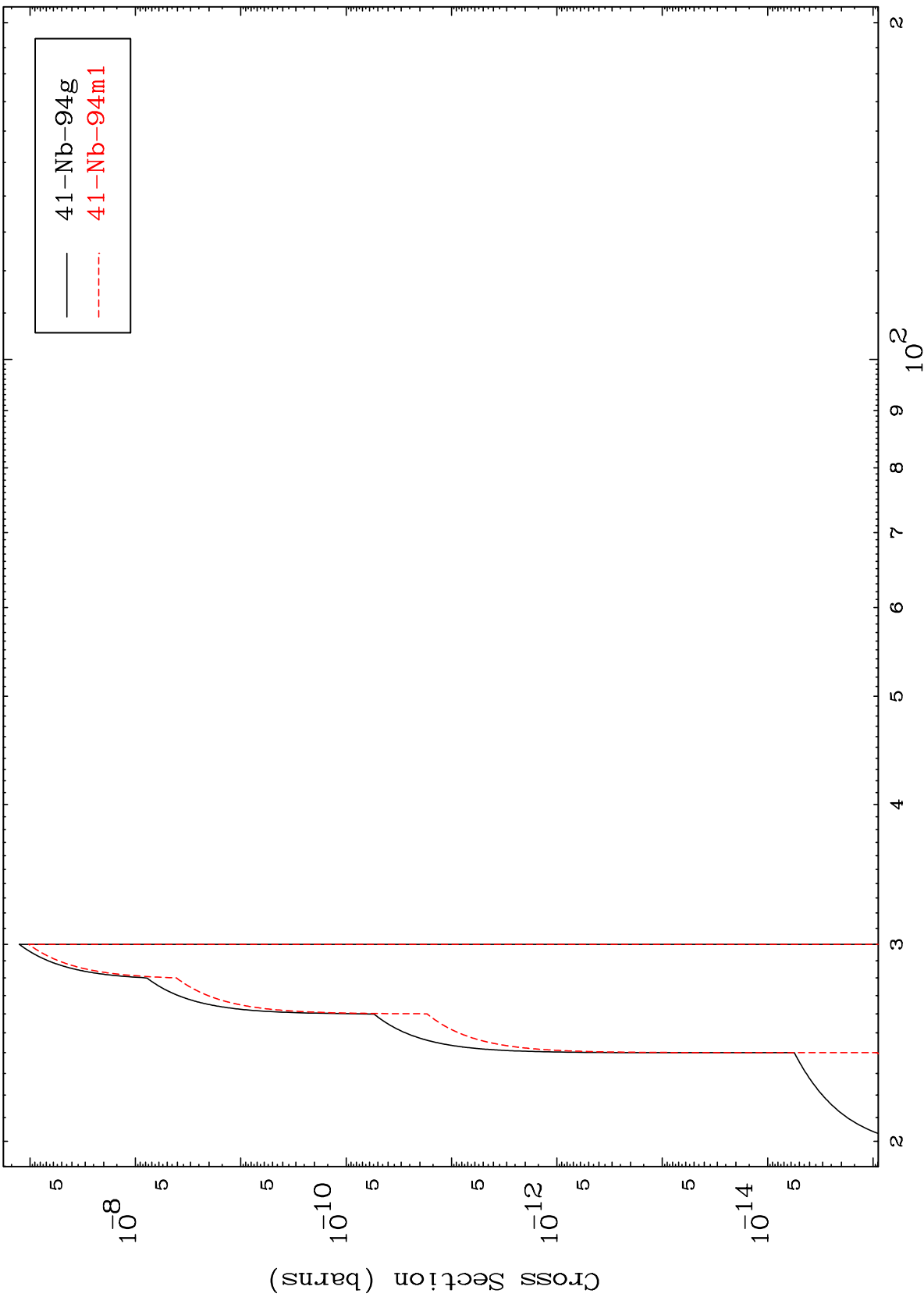


MAT 4126

( $\alpha, p$ ) d

41-Nb-93

Radionuclide Production Cross Section



24

Incident Energy (MeV)

41-Nb-93

Radionuclide Production Cross Section

