

Program EVALPLOT  
(Version 2018-1)

by

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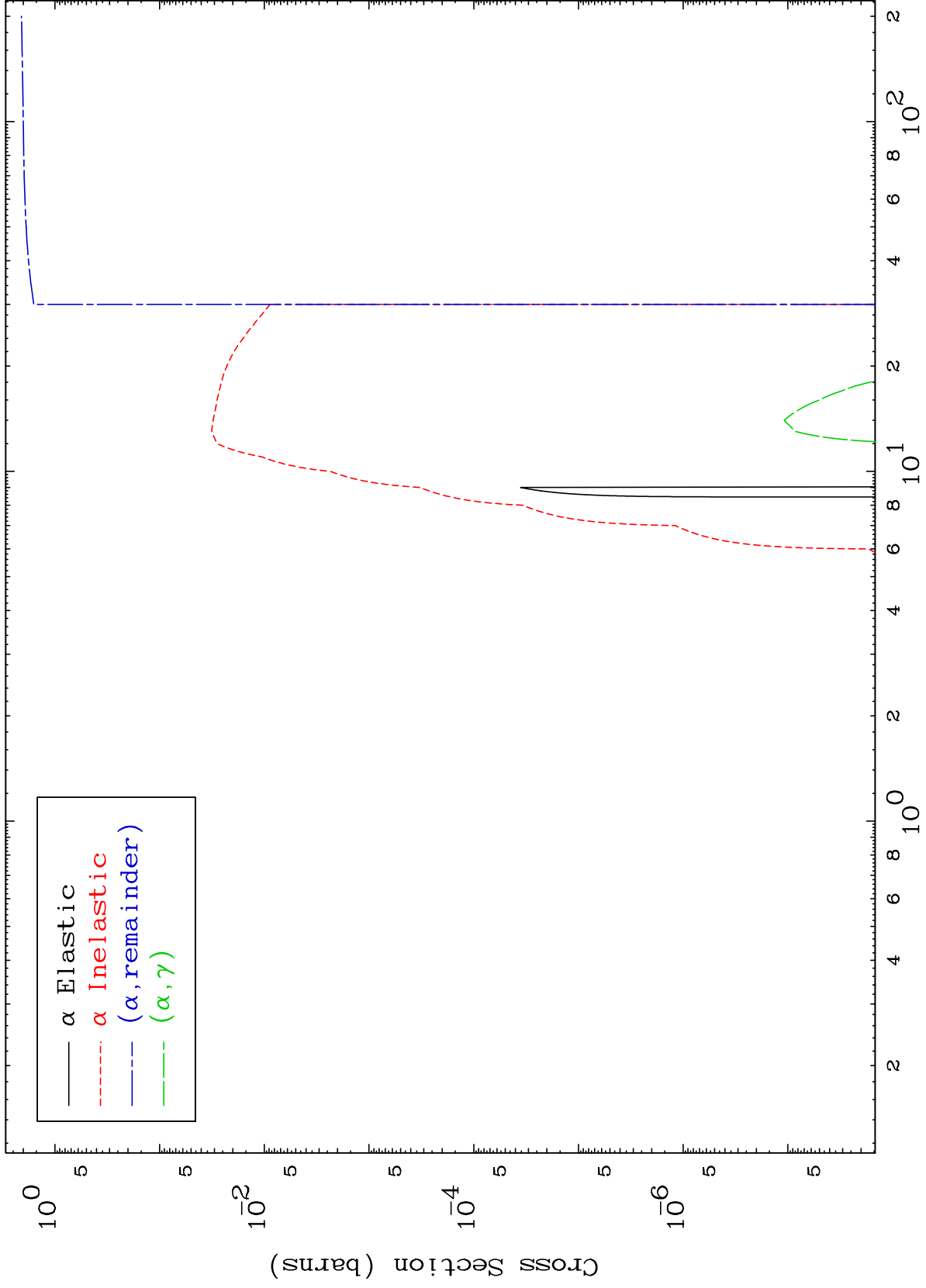
Press Mouse Button to Start

MAT 4161

$\alpha$  Major

41-Nb-105

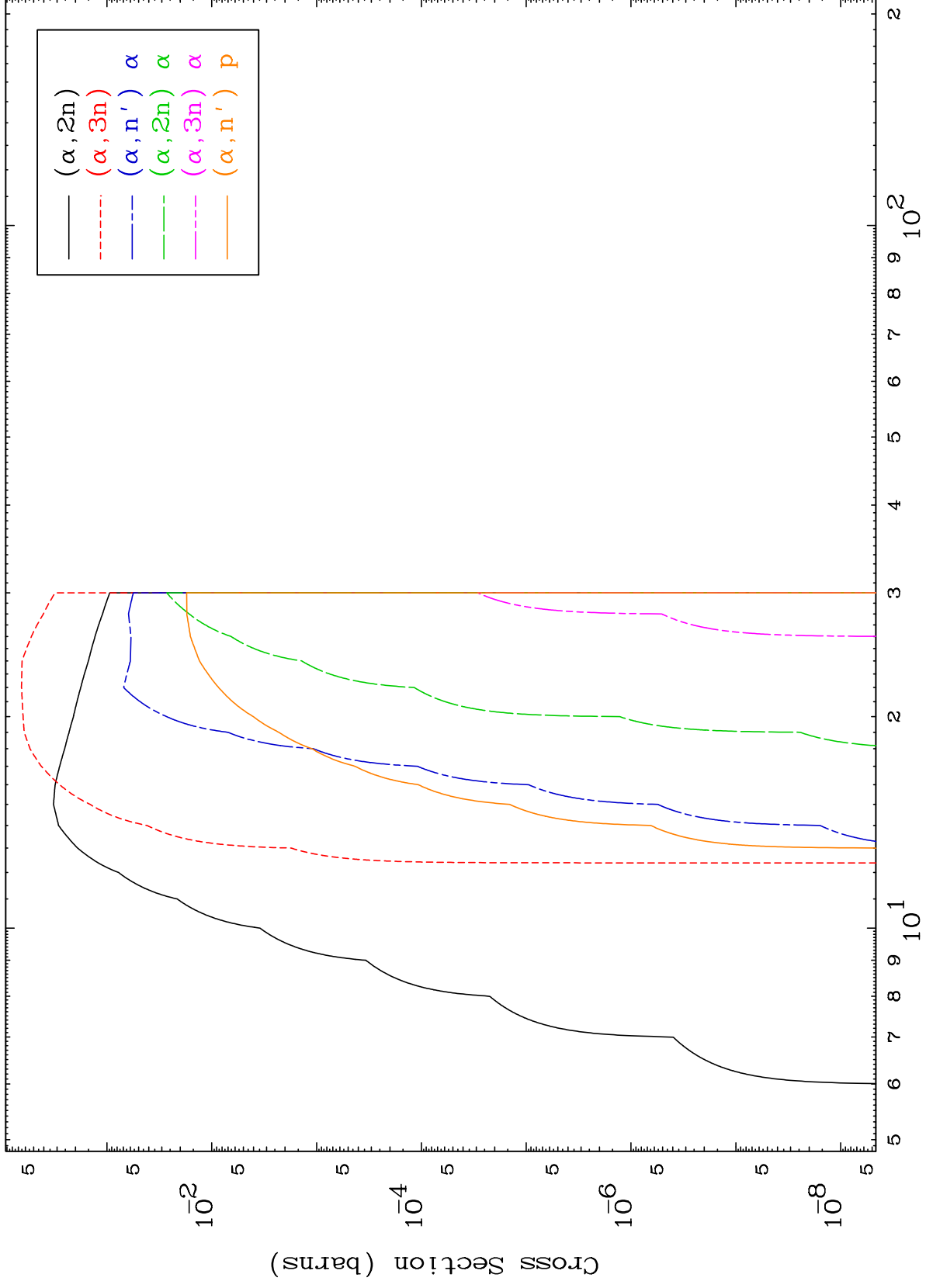
0 Kelvin Cross Sections



MAT 4161

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

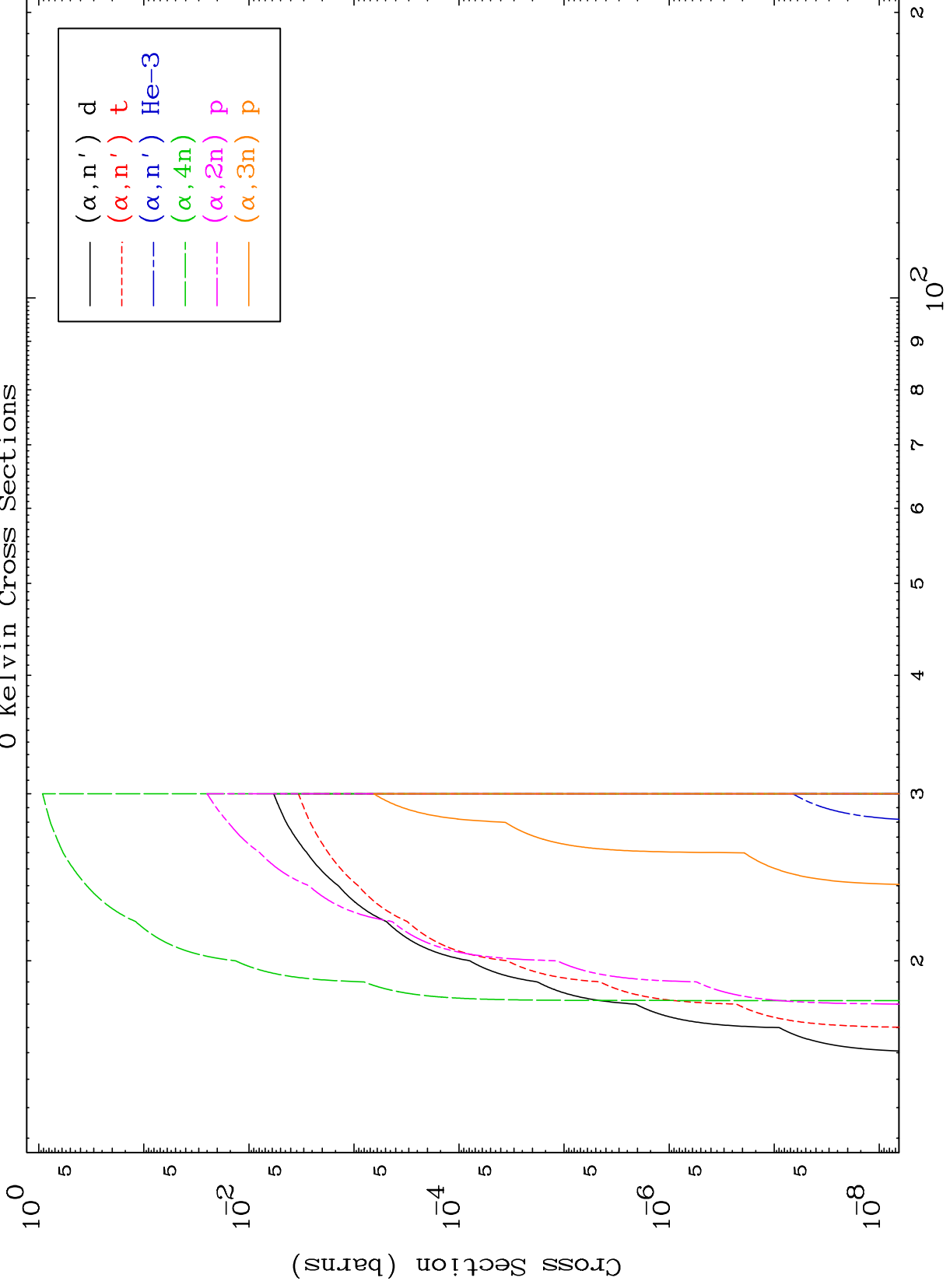
41-Nb-105

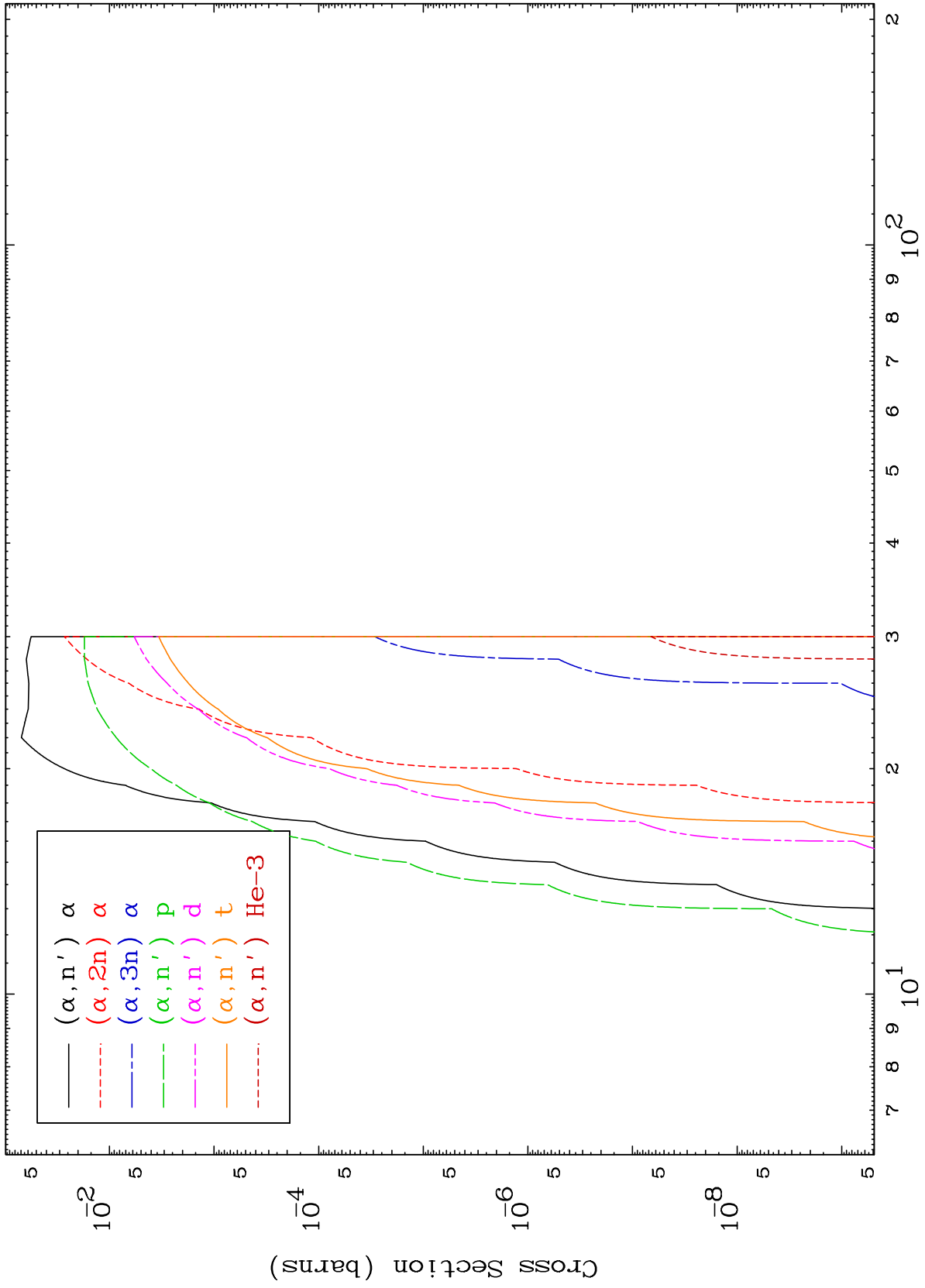


2

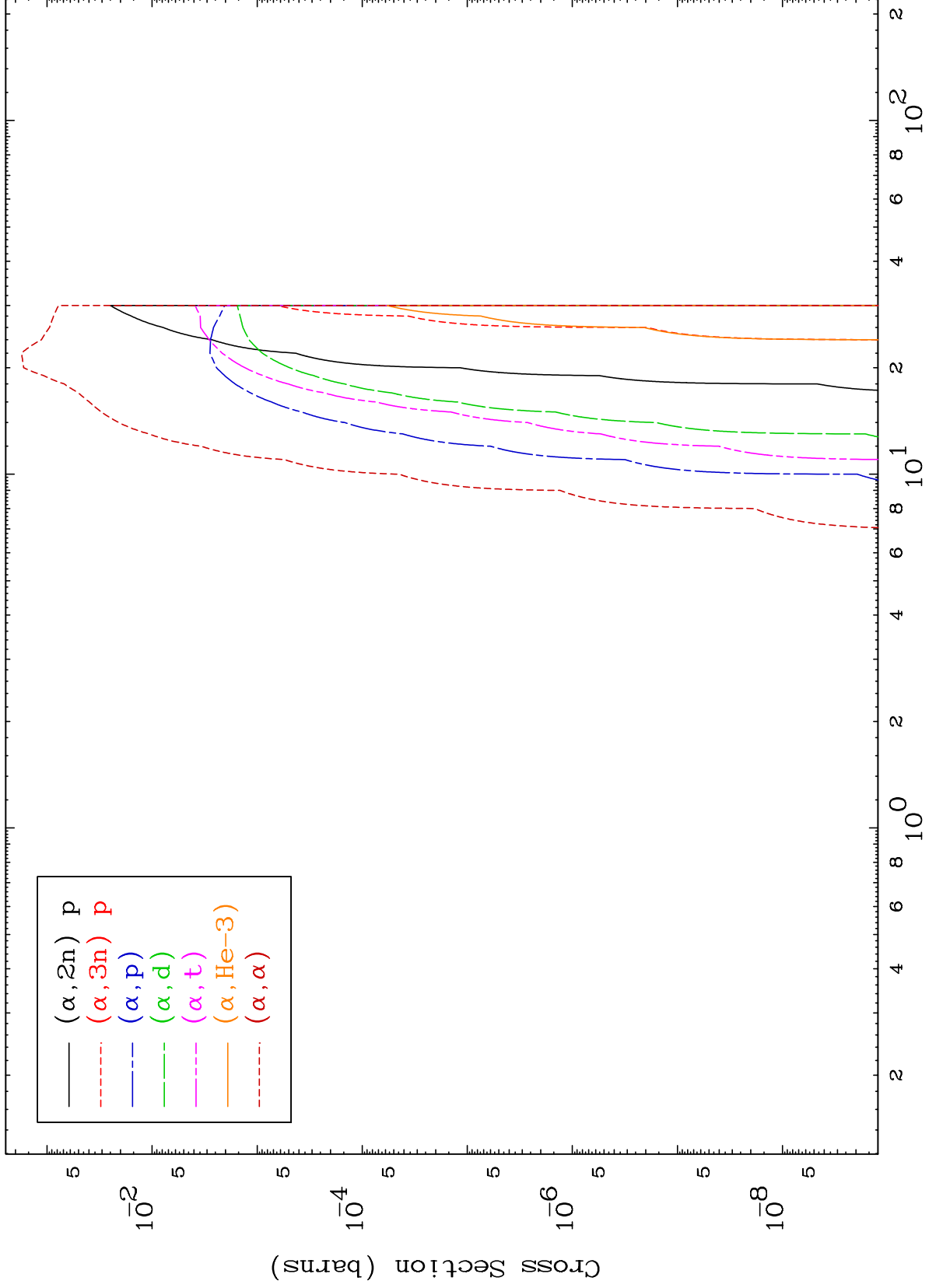
Incident Energy (MeV)

41-Nb-105





0 Kelvin Cross Sections

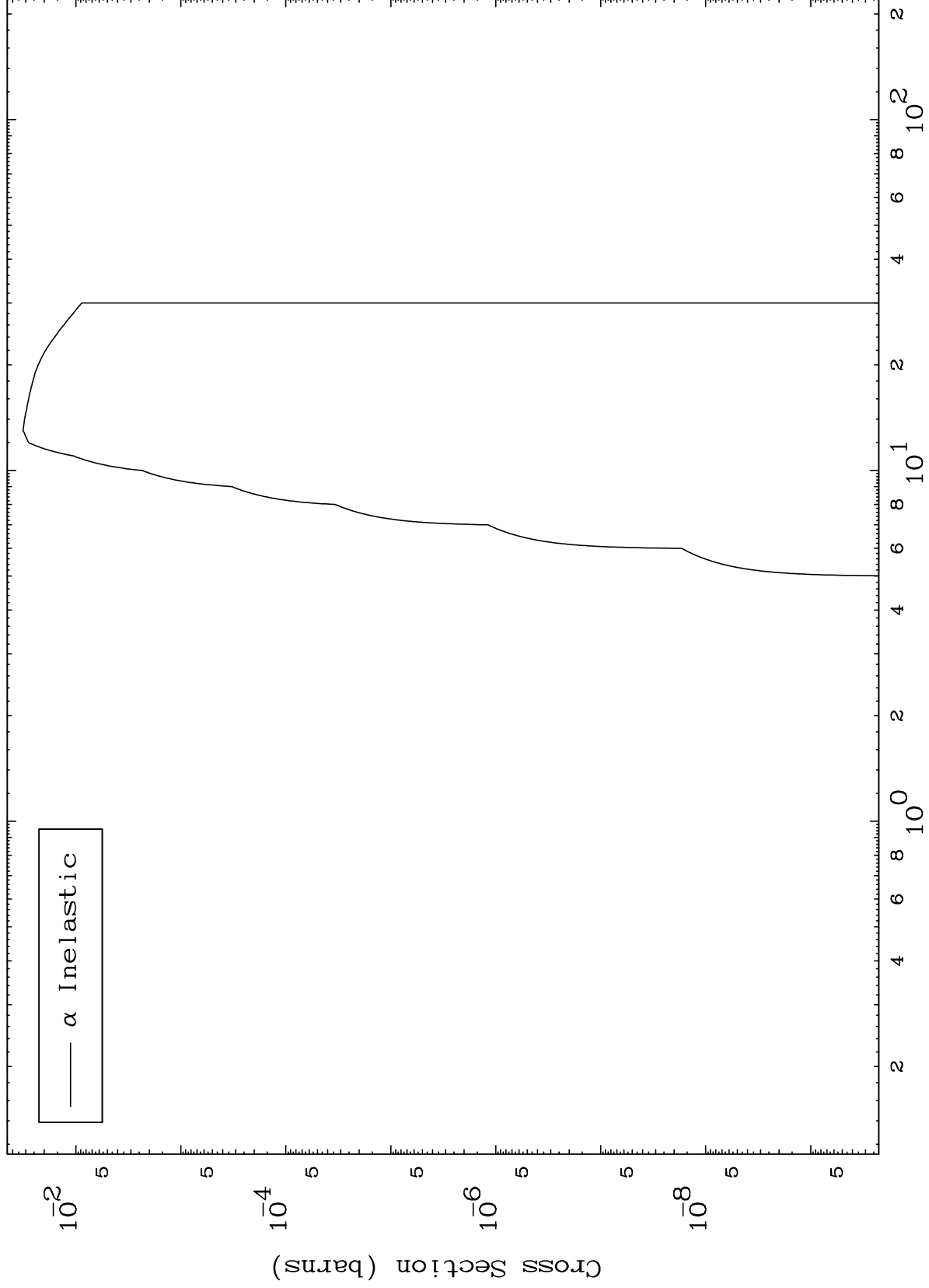


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( $\alpha, n'$ ) Level

41-Nb-105

0 Kelvin Cross Sections



6

Incident Energy (MeV)

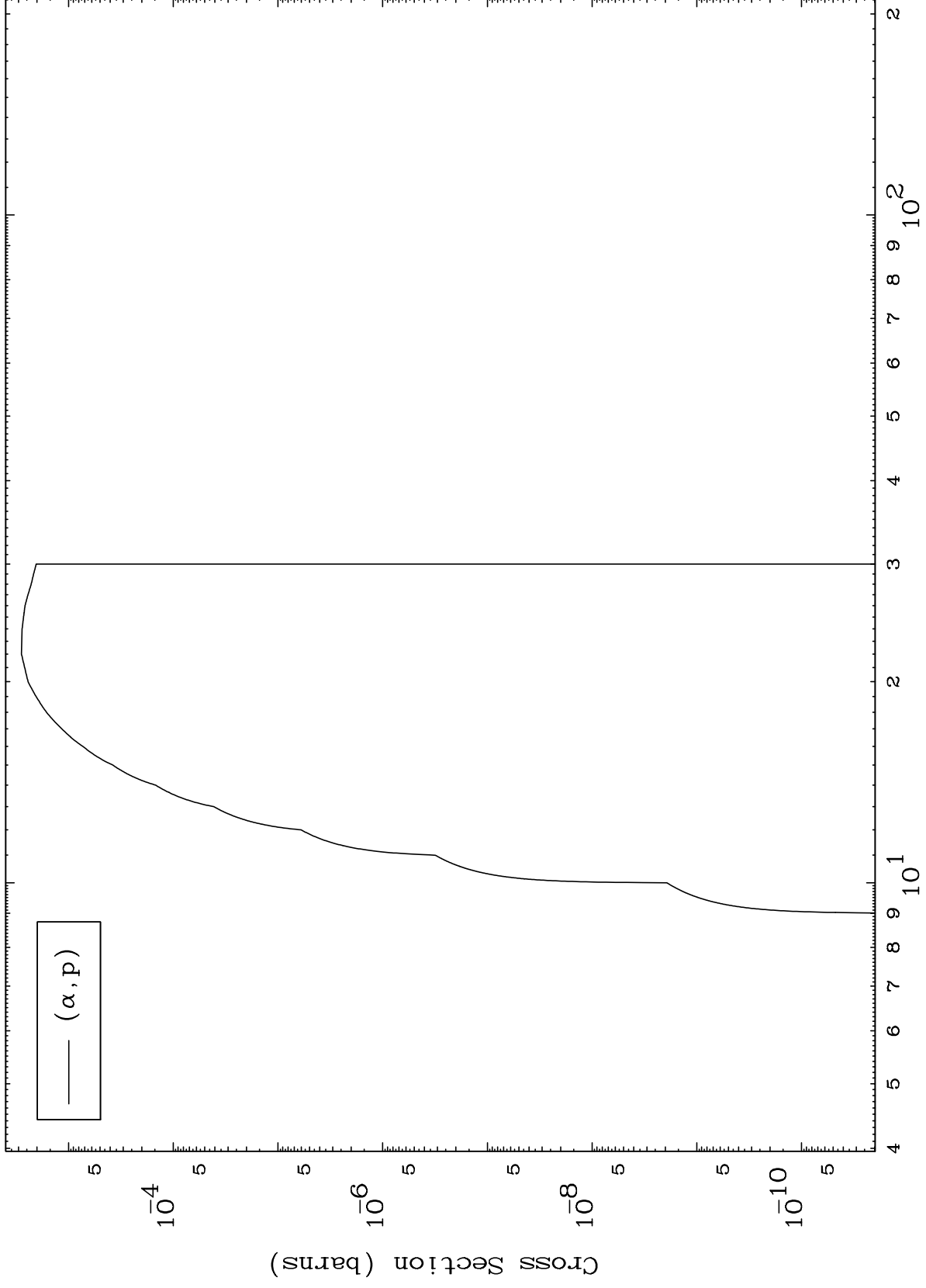
41-Nb-105

MAT 4161

( $\alpha, p$ ) Levels

41-Nb-105

0 Kelvin Cross Sections



Incident Energy (MeV)

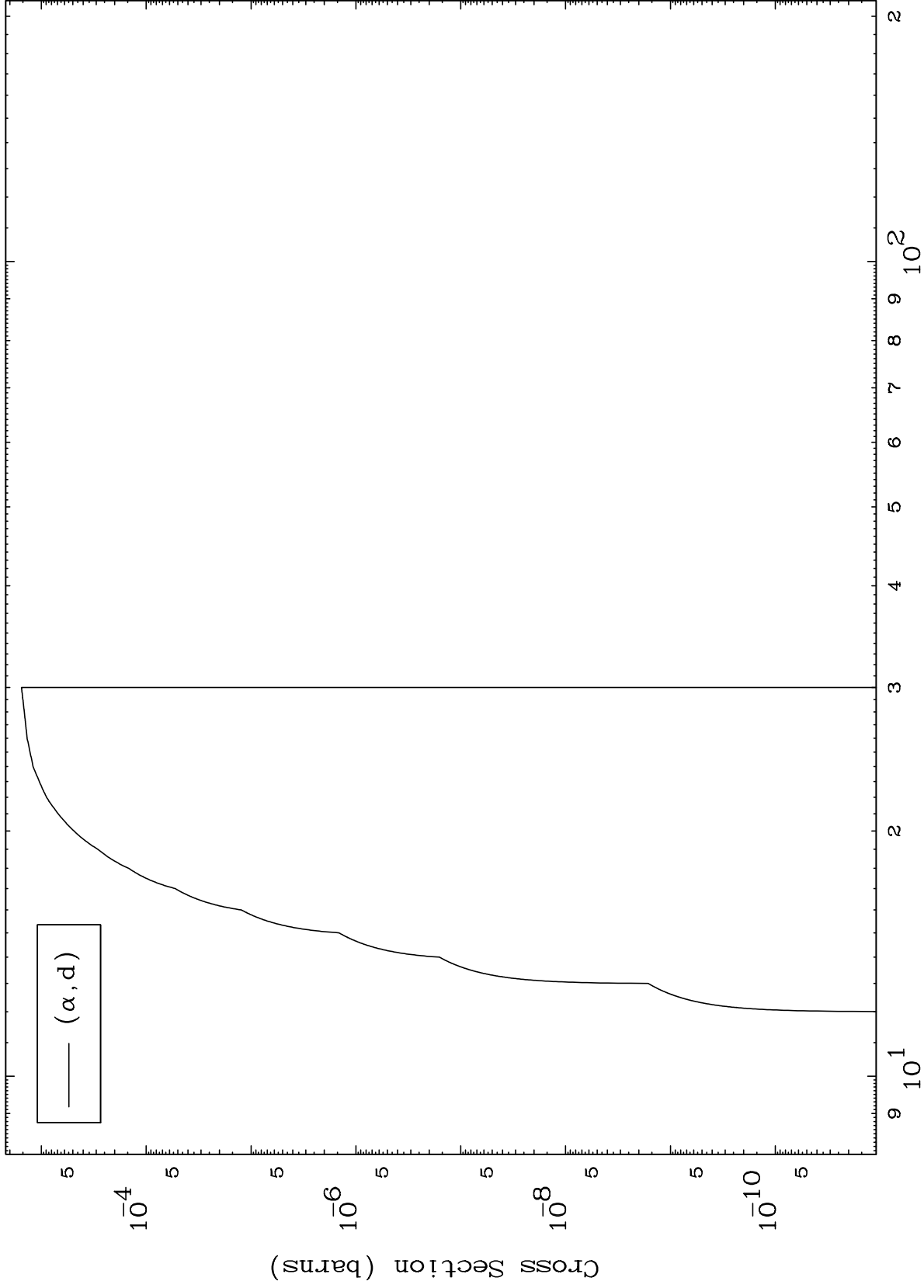
41-Nb-105



MAT 4161

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

41-Nb-105



8

Incident Energy (MeV)

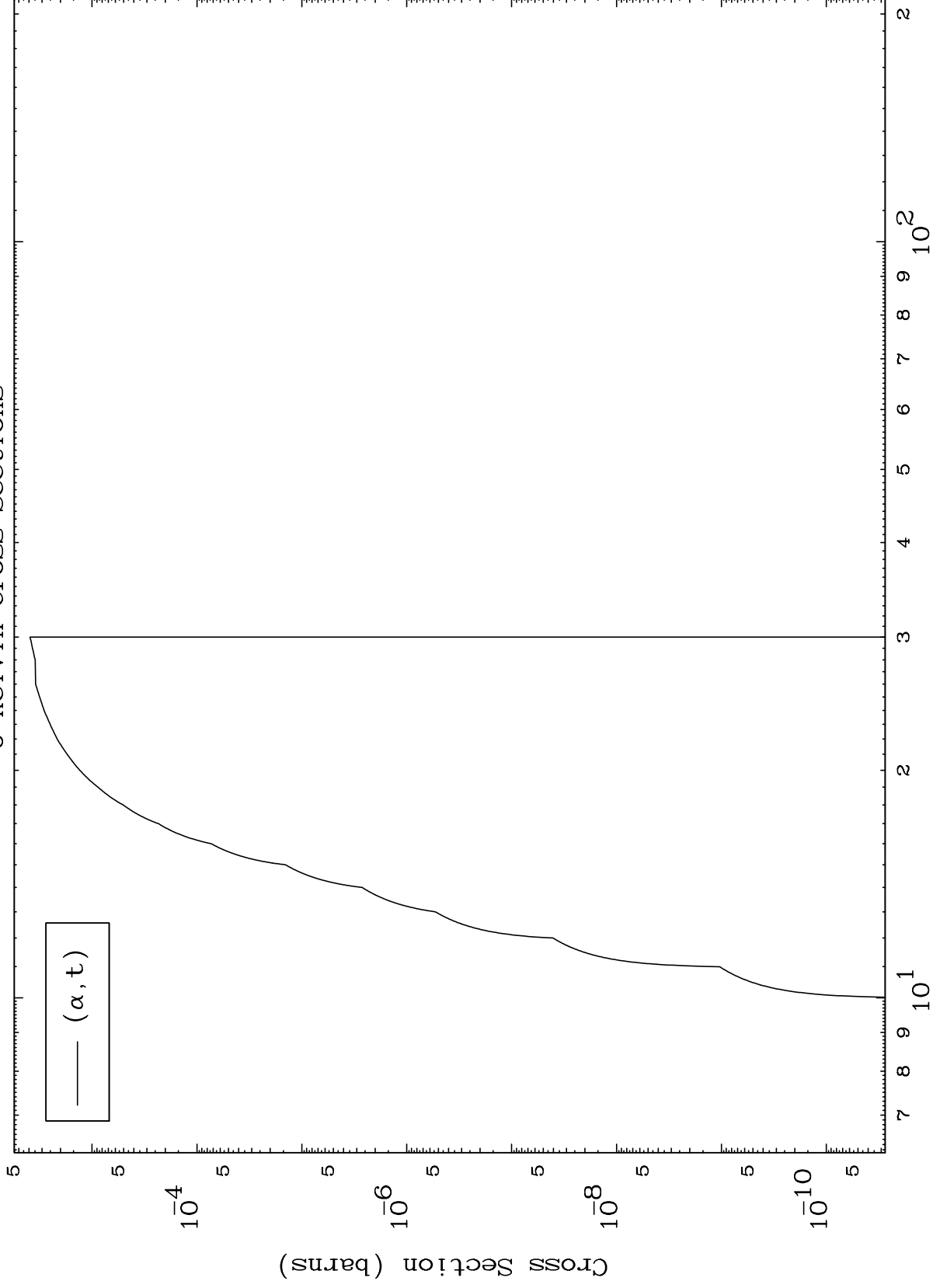
41-Nb-105

MAT 4161

( $\alpha, t$ ) Levels

41-Nb-105

0 Kelvin Cross Sections



9

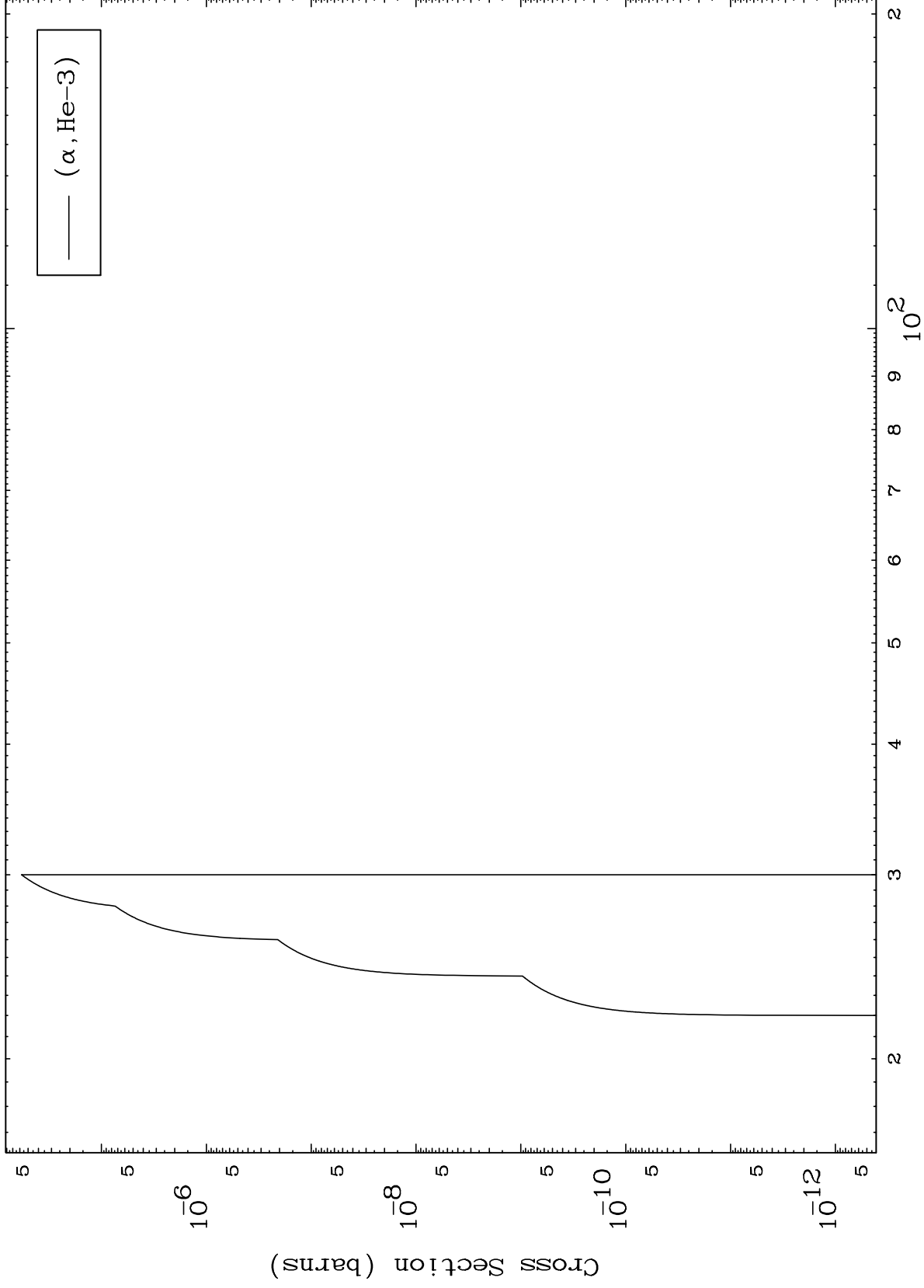
Incident Energy (MeV)

41-Nb-105

MAT 4161

( $\alpha, \text{He3}$ ) Levels  
0 Kelvin Cross Sections

41-Nb-105



10

Incident Energy (MeV)

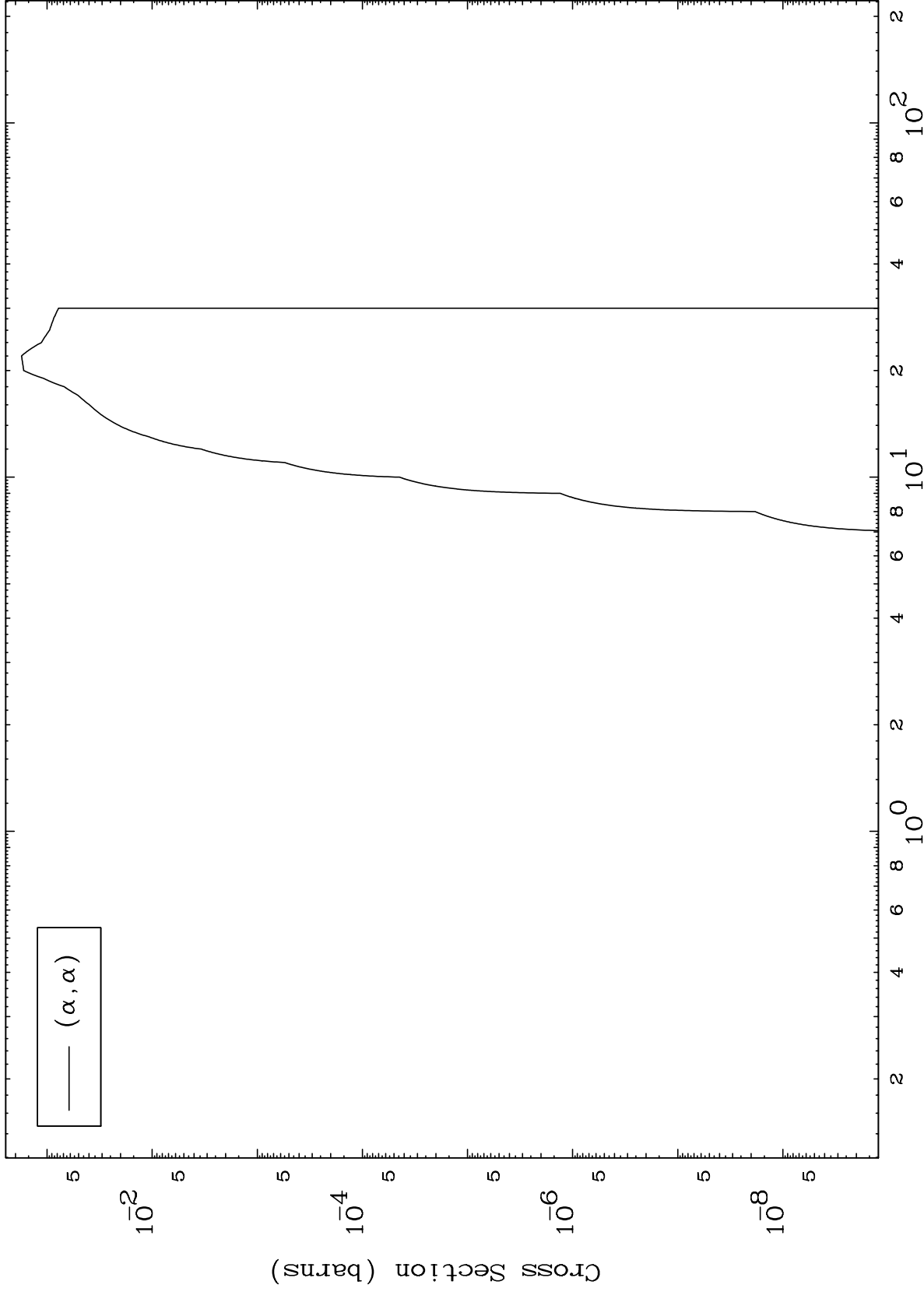
41-Nb-105

MAT 4161

( $\alpha, \alpha$ ) Levels

41-Nb-105

0 Kelvin Cross Sections



11

Incident Energy (MeV)

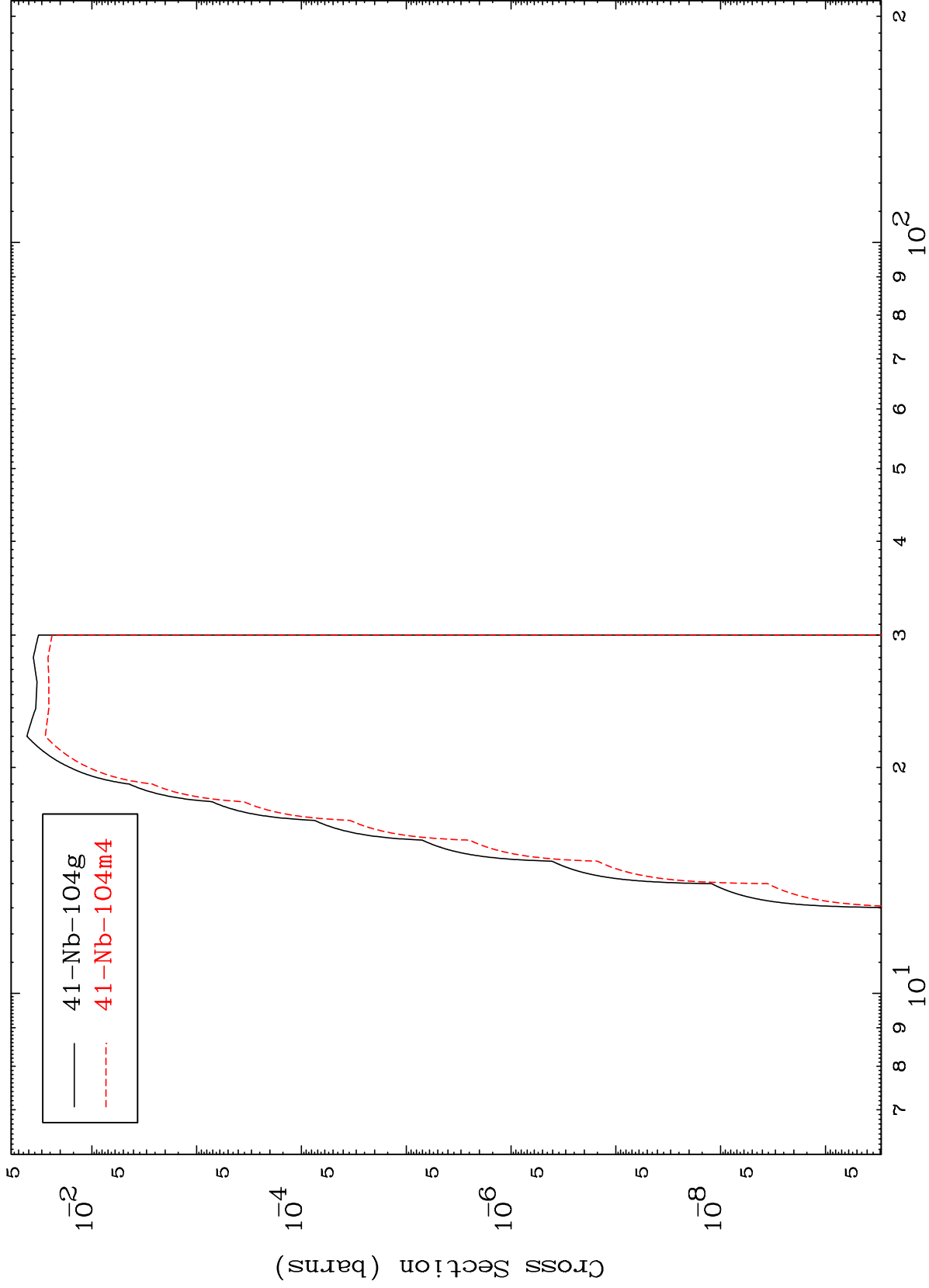
41-Nb-105

MAT 4161

41-Nb-105

( $\alpha, n'$ )  $\alpha$

Radionuclide Production Cross Section



12

Incident Energy (MeV)

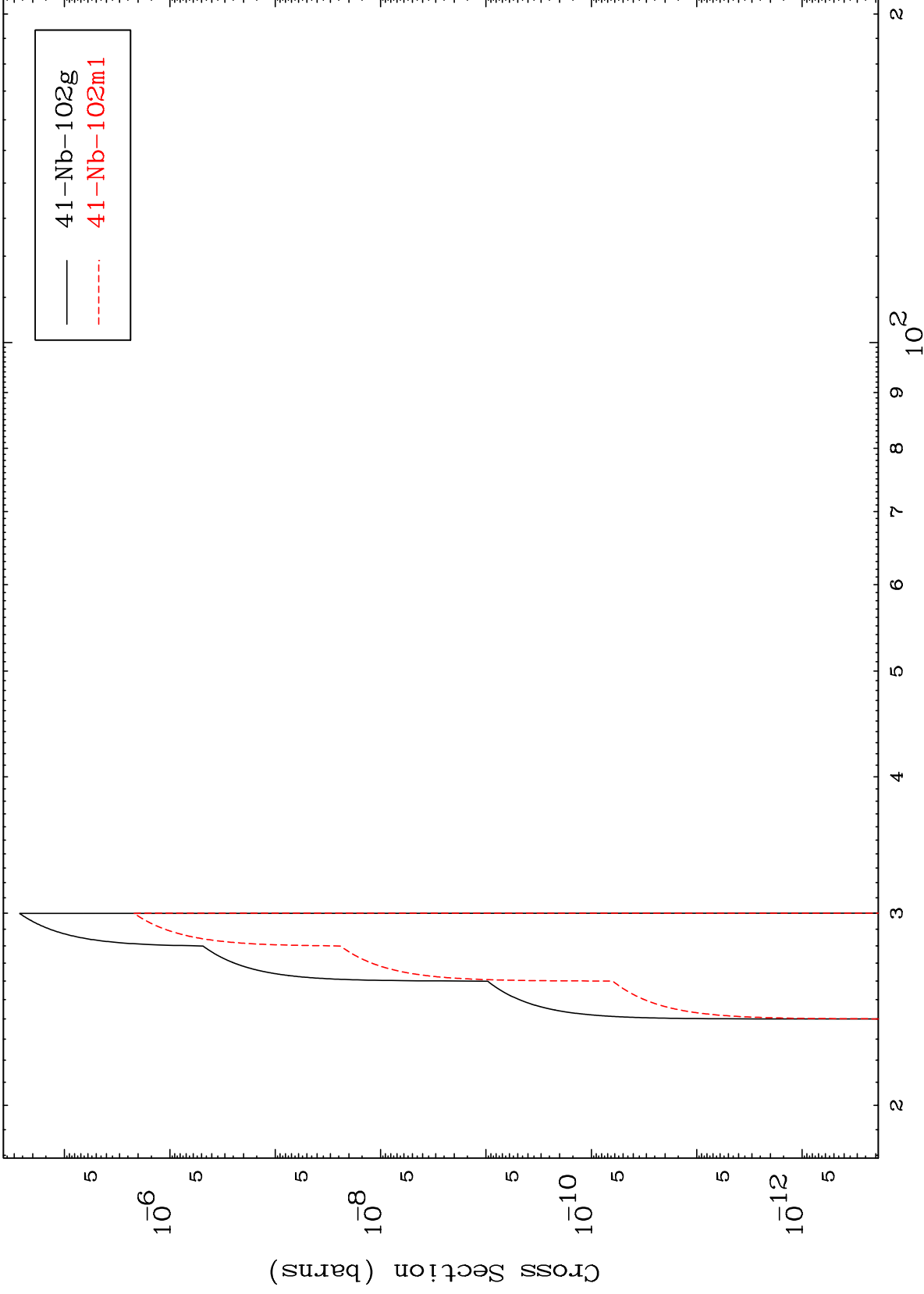
41-Nb-105

MAT 4161

$(\alpha, 3n) \alpha$

41-Nb-105

Radionuclide Production Cross Section



13

Incident Energy (MeV)

41-Nb-105