

Program EVALPLOT  
(Version 2018-1)

by

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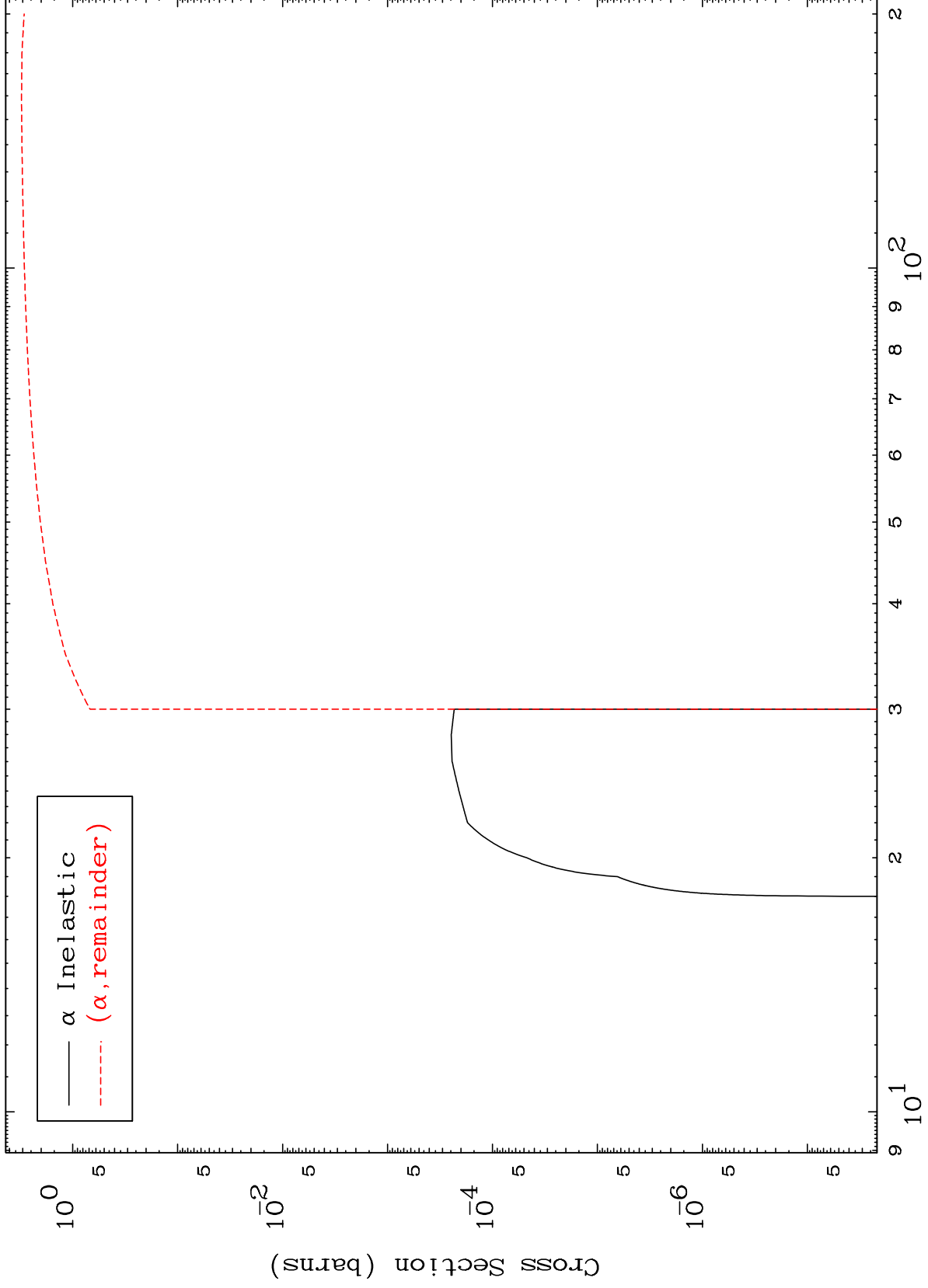
Press Mouse Button to Start

MAT 9972

$\alpha$  Major

102-No-254

0 Kelvin Cross Sections



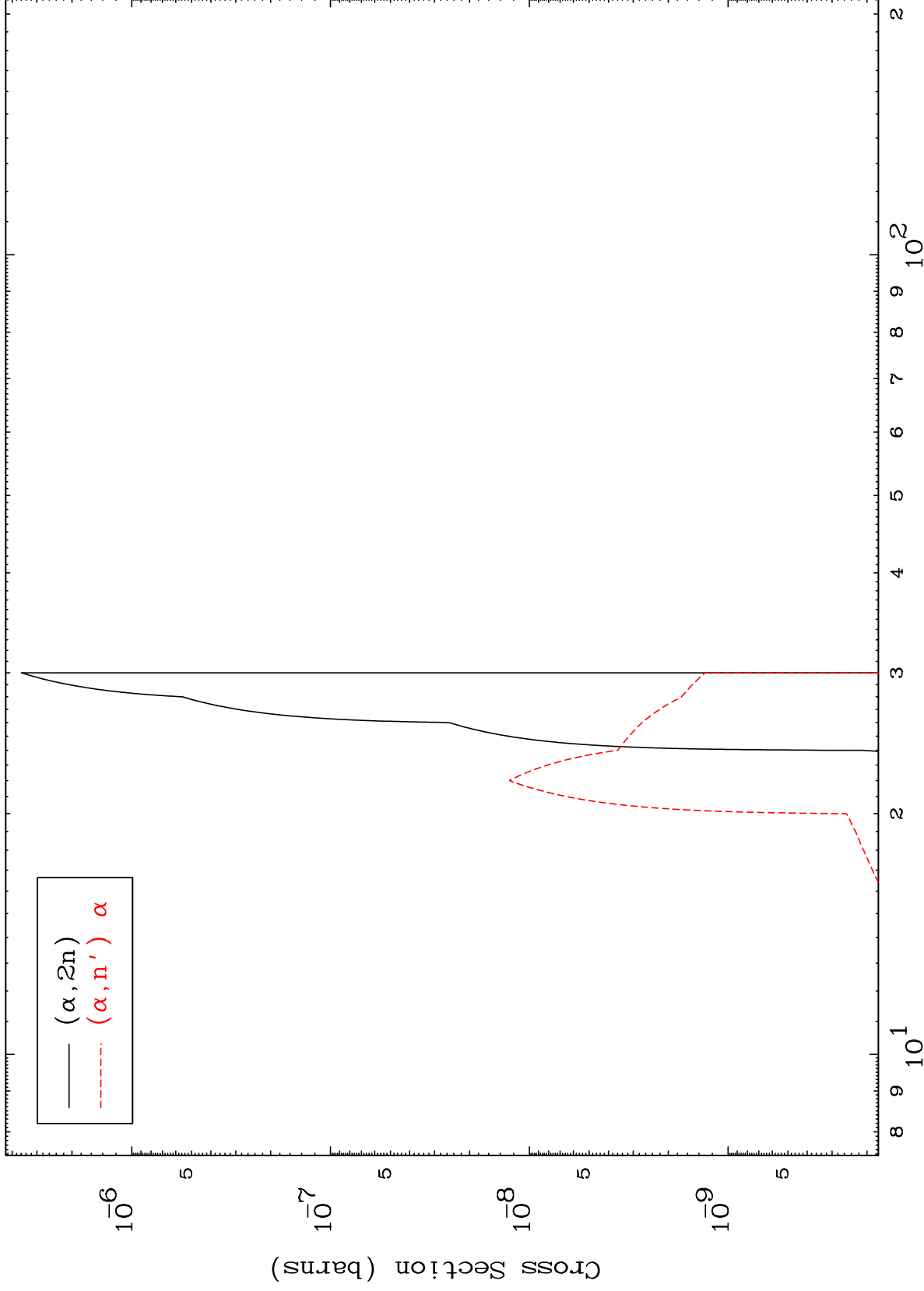
Incident Energy (MeV)

102-No-254

MAT 9972

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

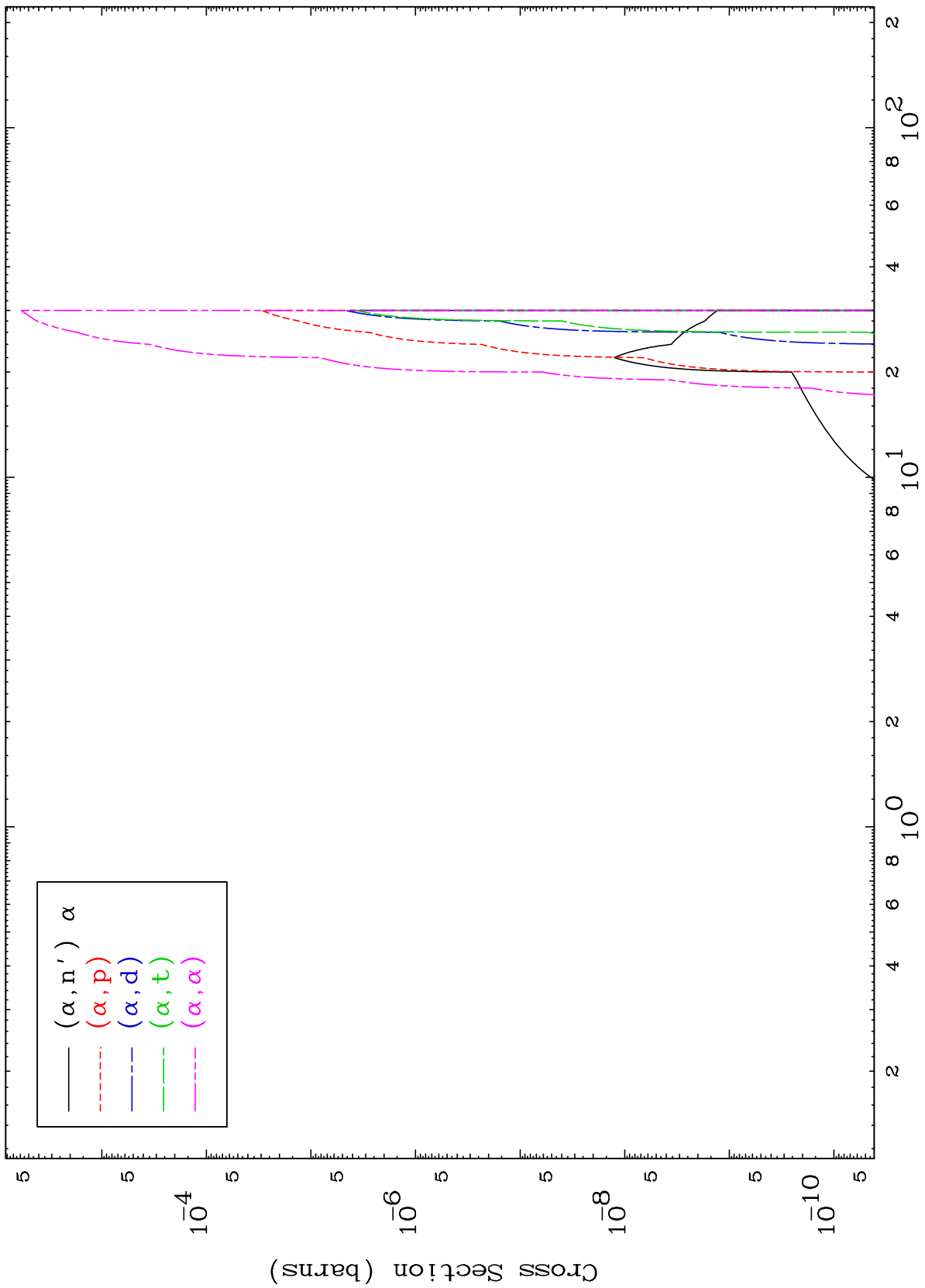
102-No-254



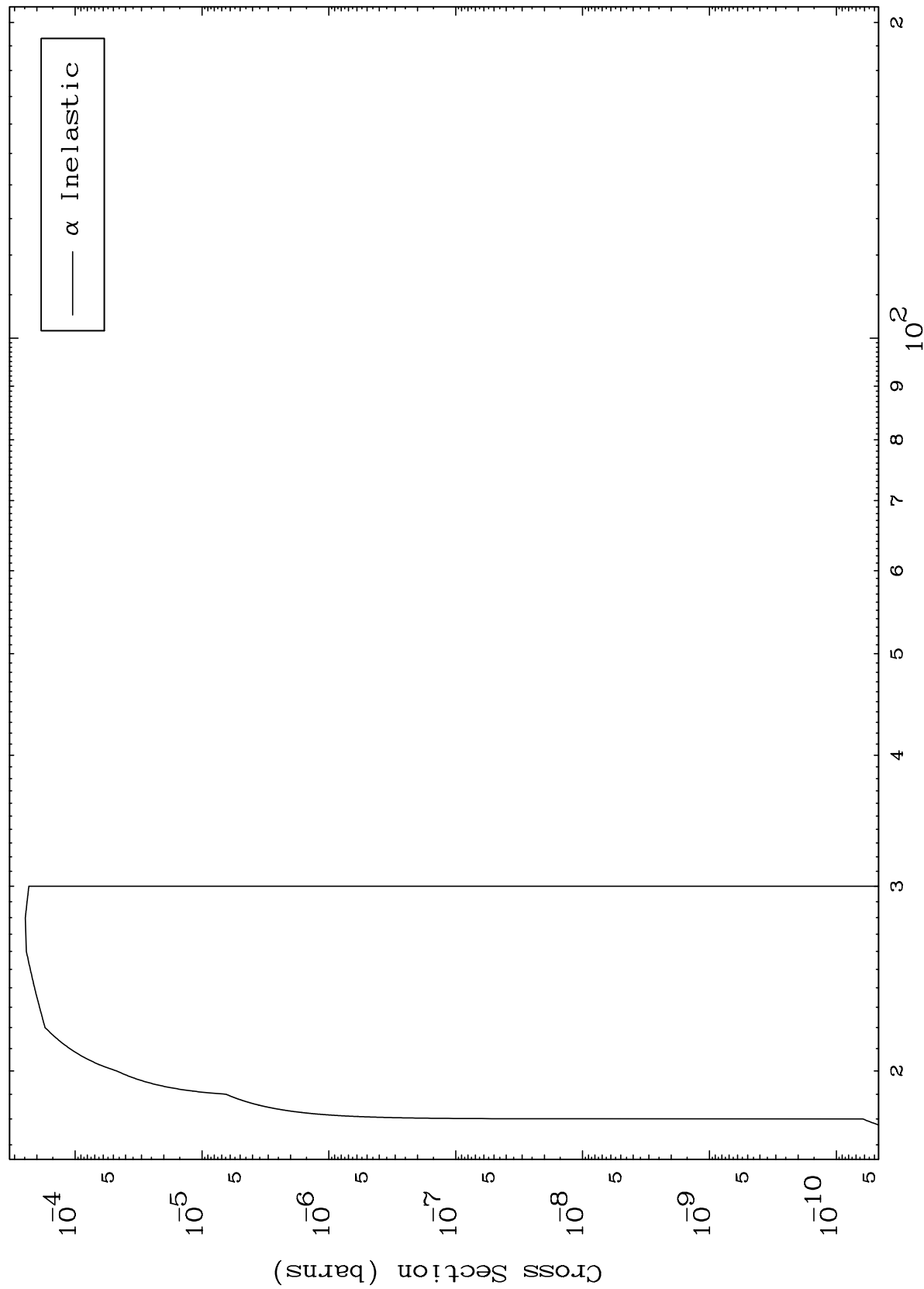
2

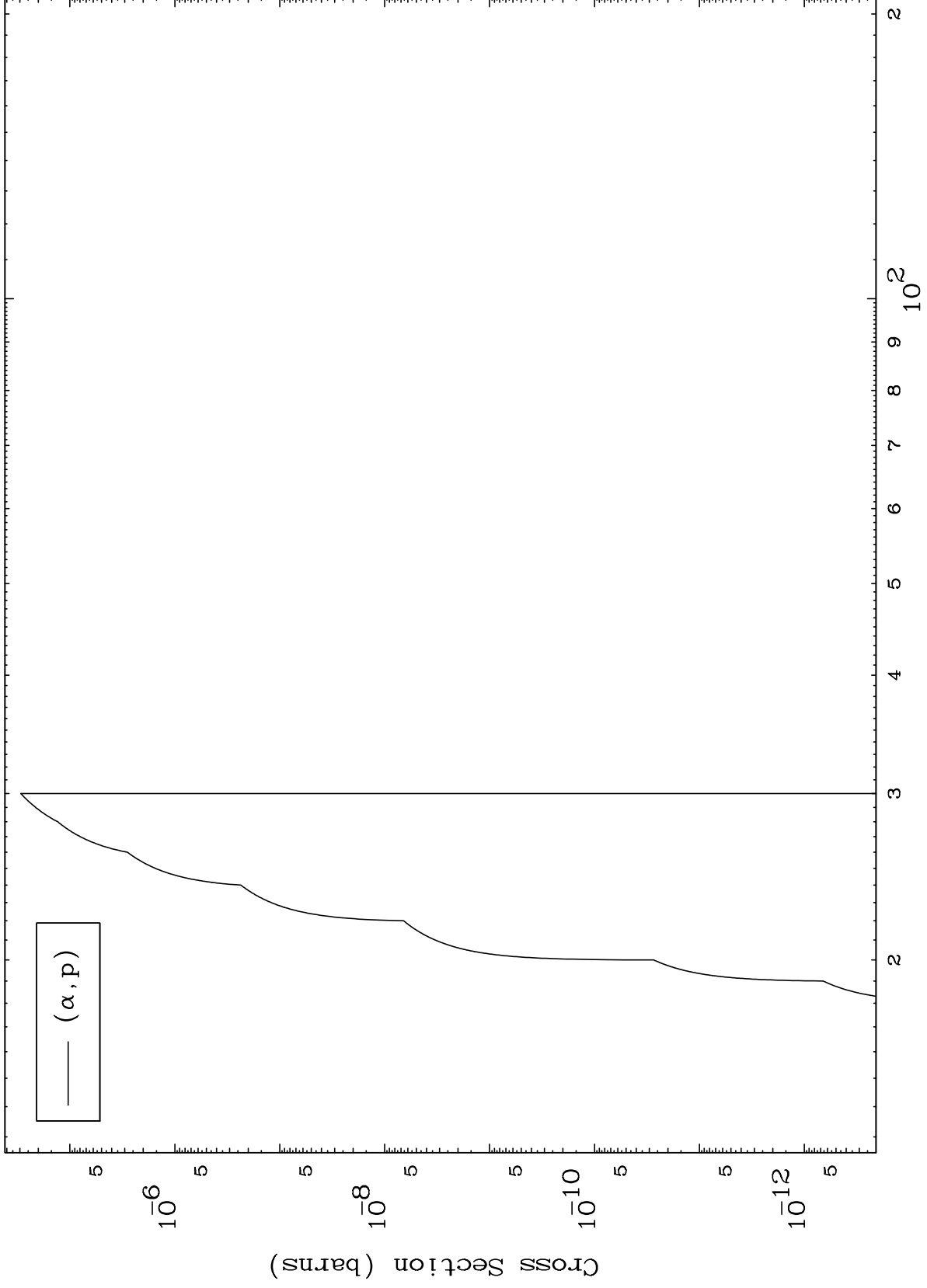
Incident Energy (MeV)

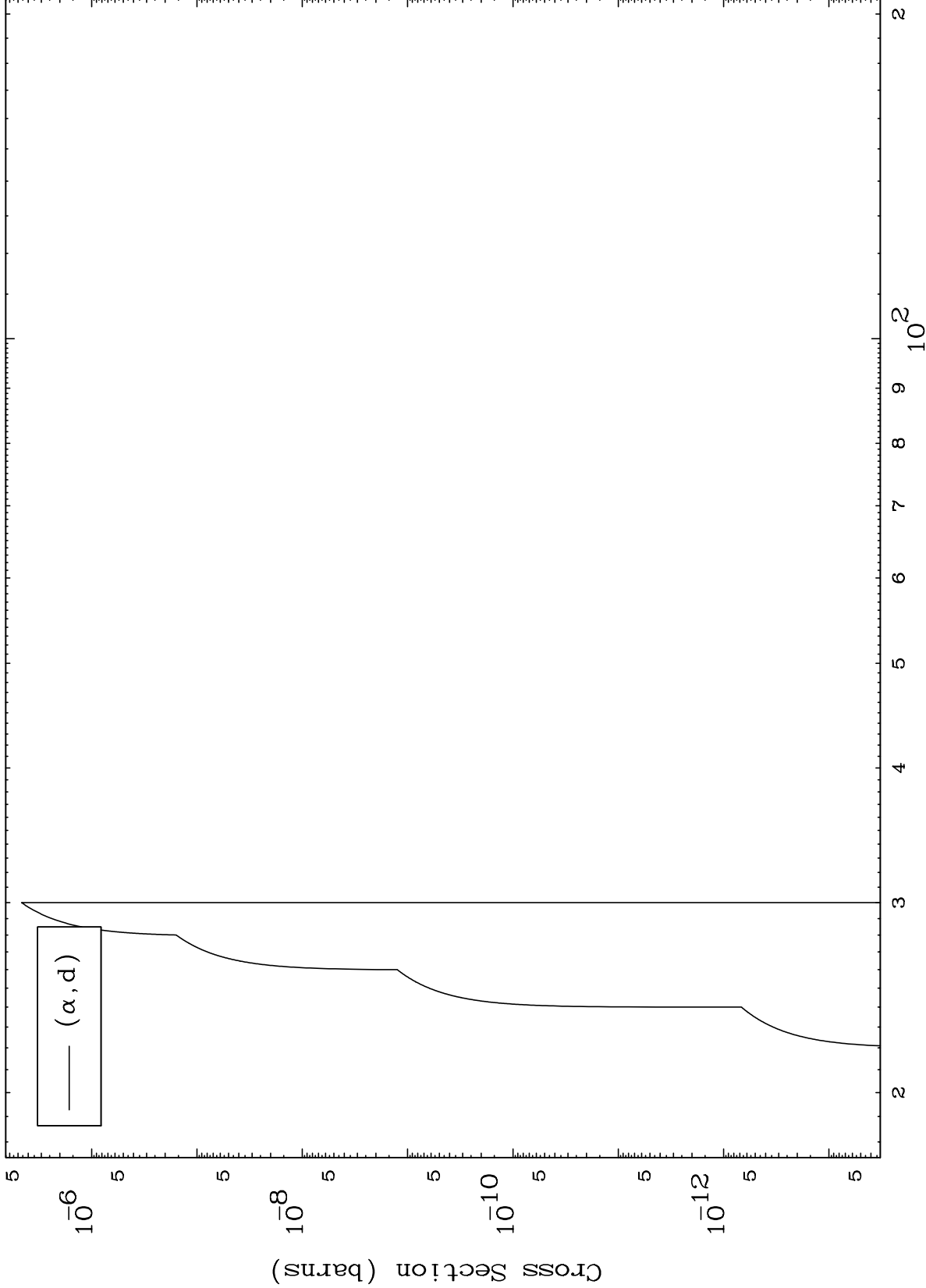
102-No-254

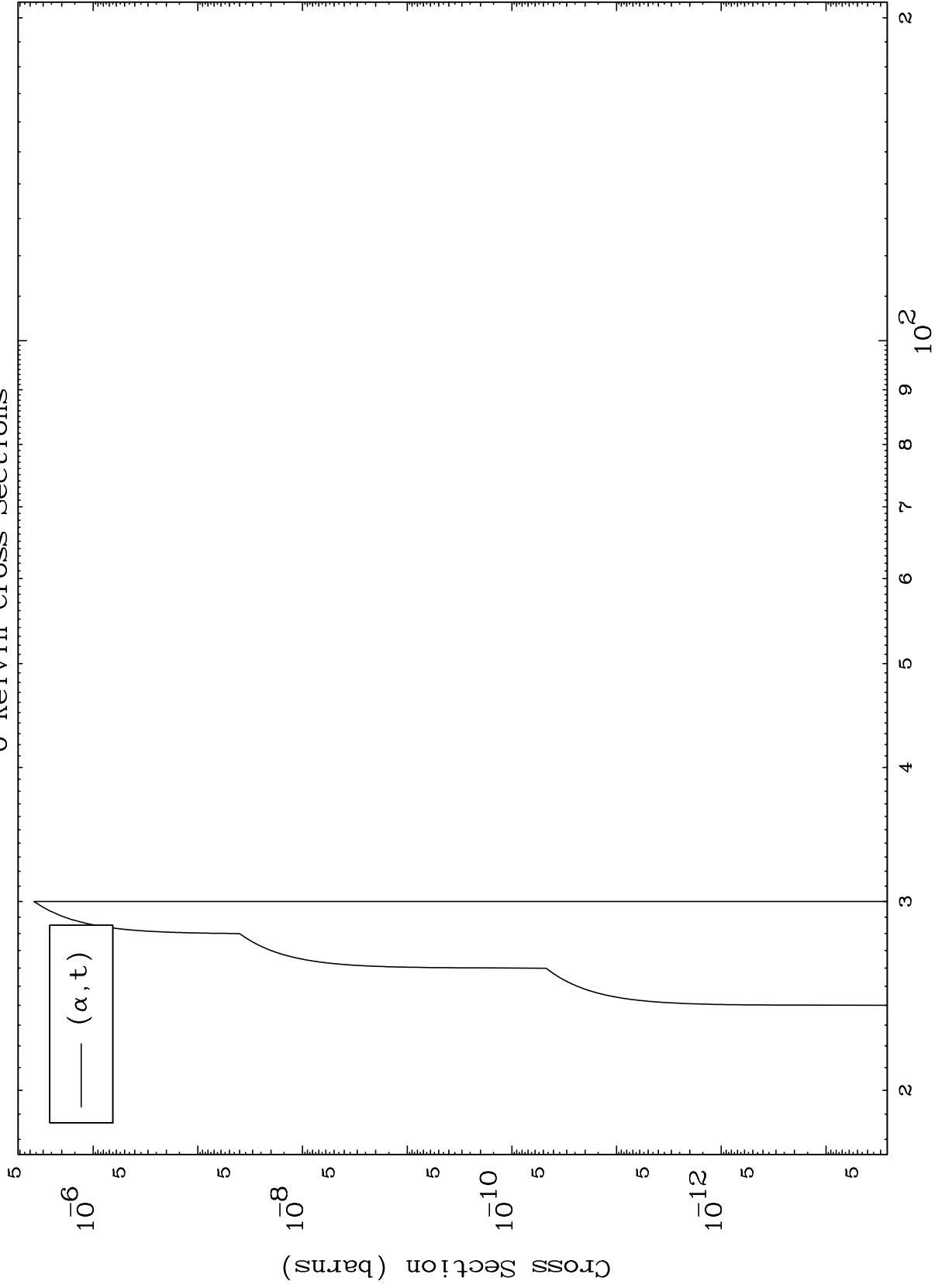


0 Kelvin Cross Sections









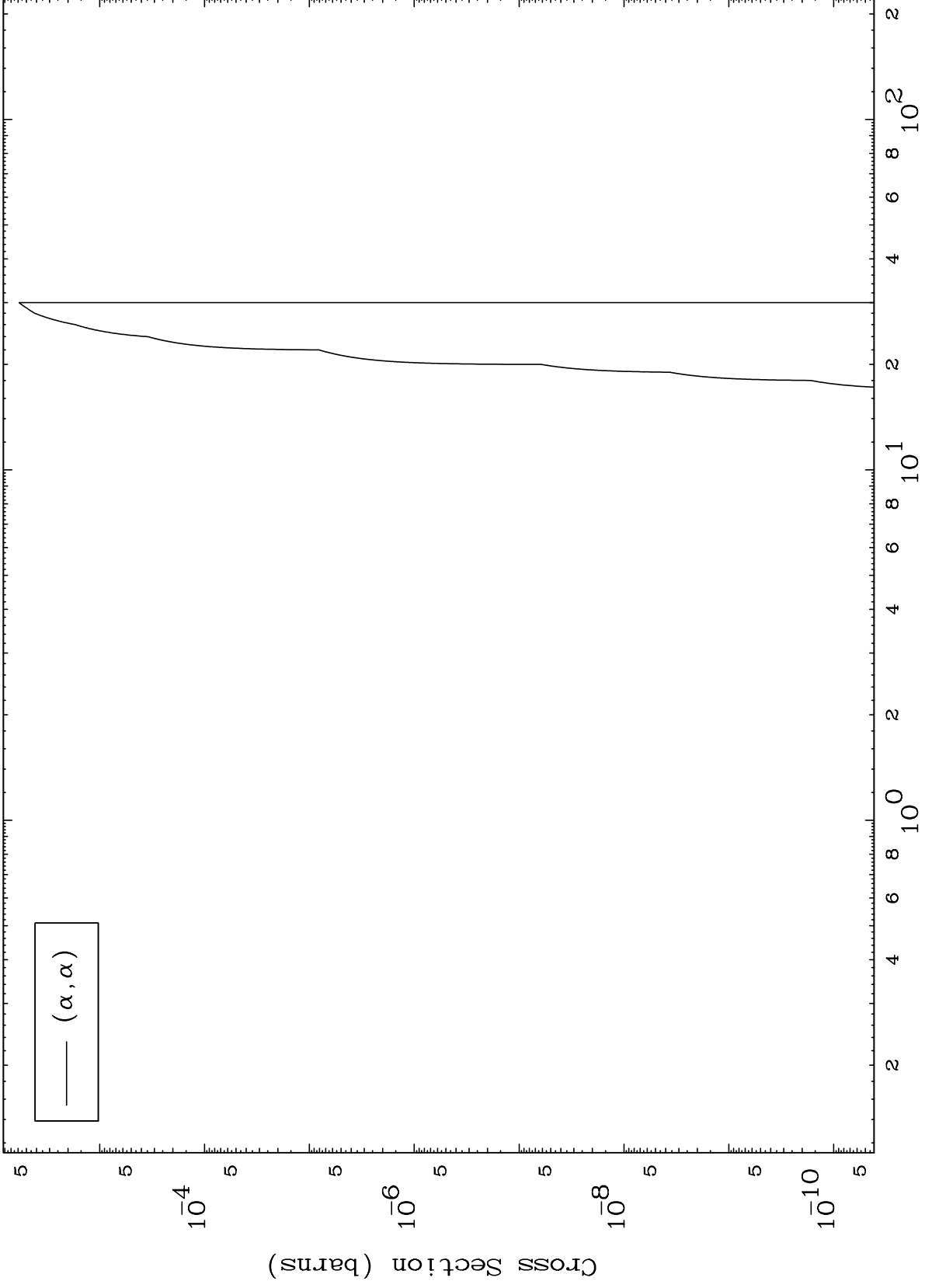


MAT 9972

( $\alpha, \alpha$ ) Levels

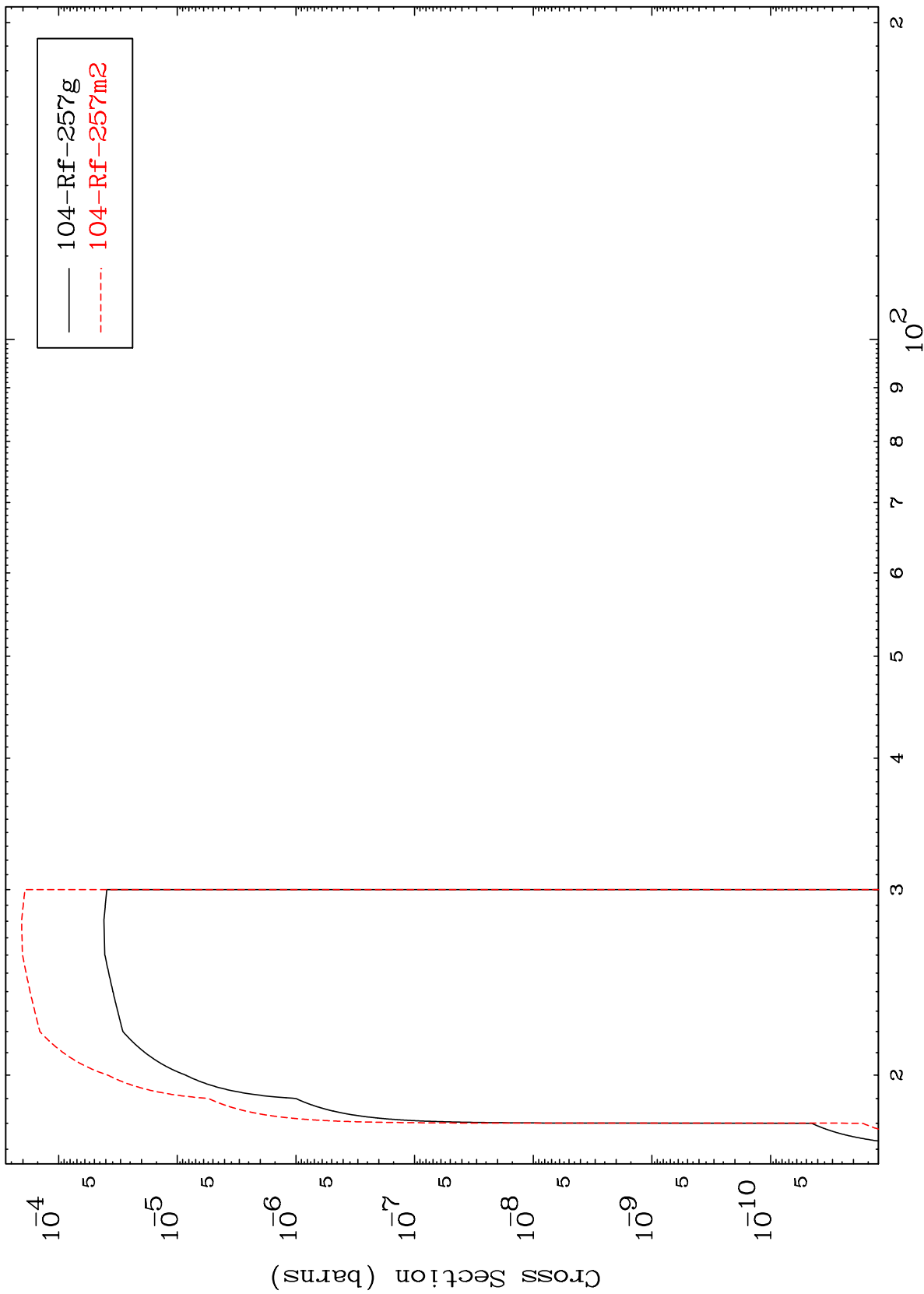
102-No-254

0 Kelvin Cross Sections



Radionuclide Production Cross Section

$\alpha$  Inelastic

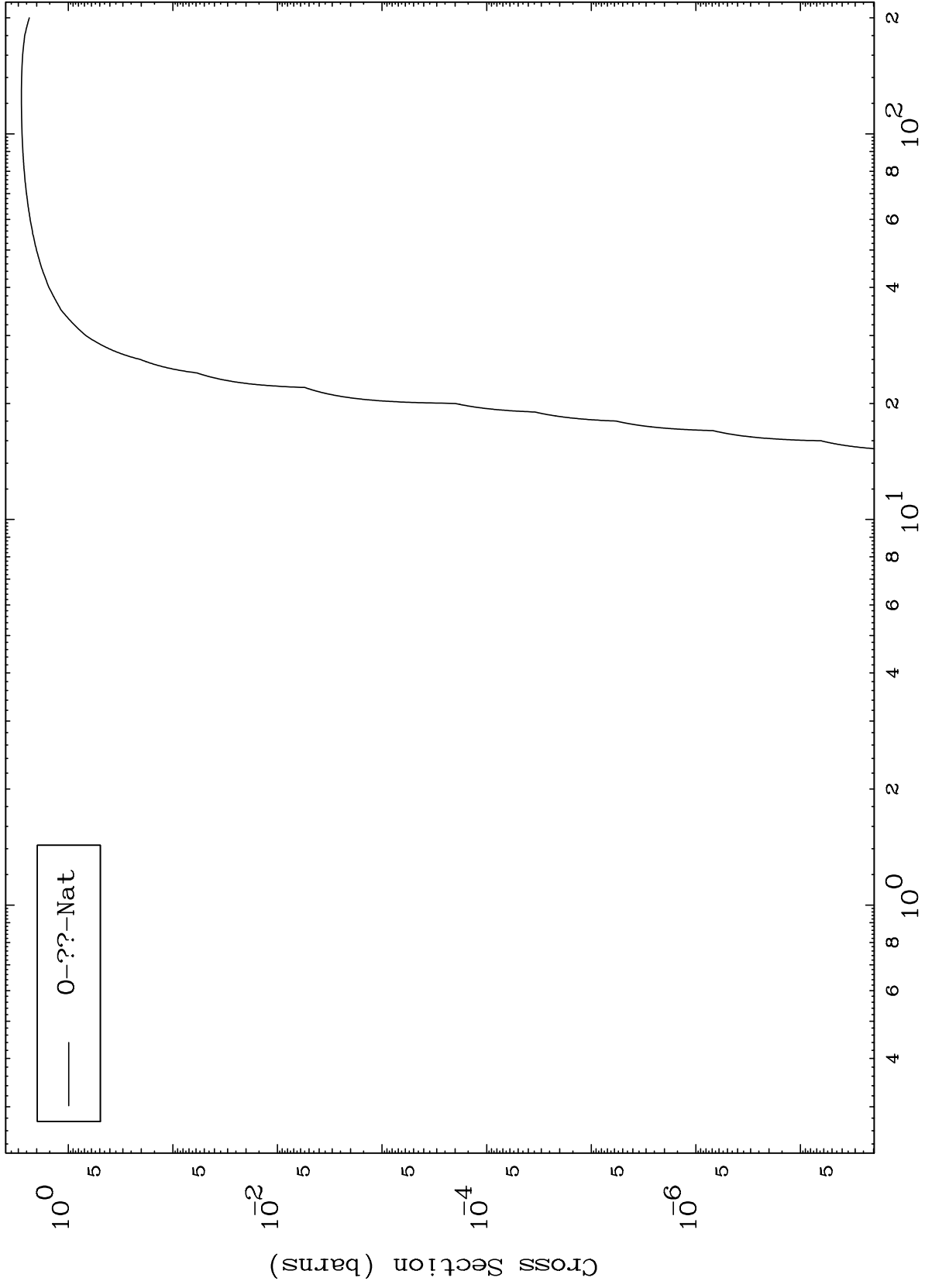


MAT 9972

102-No-254

$\alpha$  Fission

Radionuclide Production Cross Section



10

Incident Energy (MeV)

102-No-254

( $\alpha, t$ )  
Radionuclide Production Cross Section

