

Program EVALPLOT  
(Version 2018-1)

by

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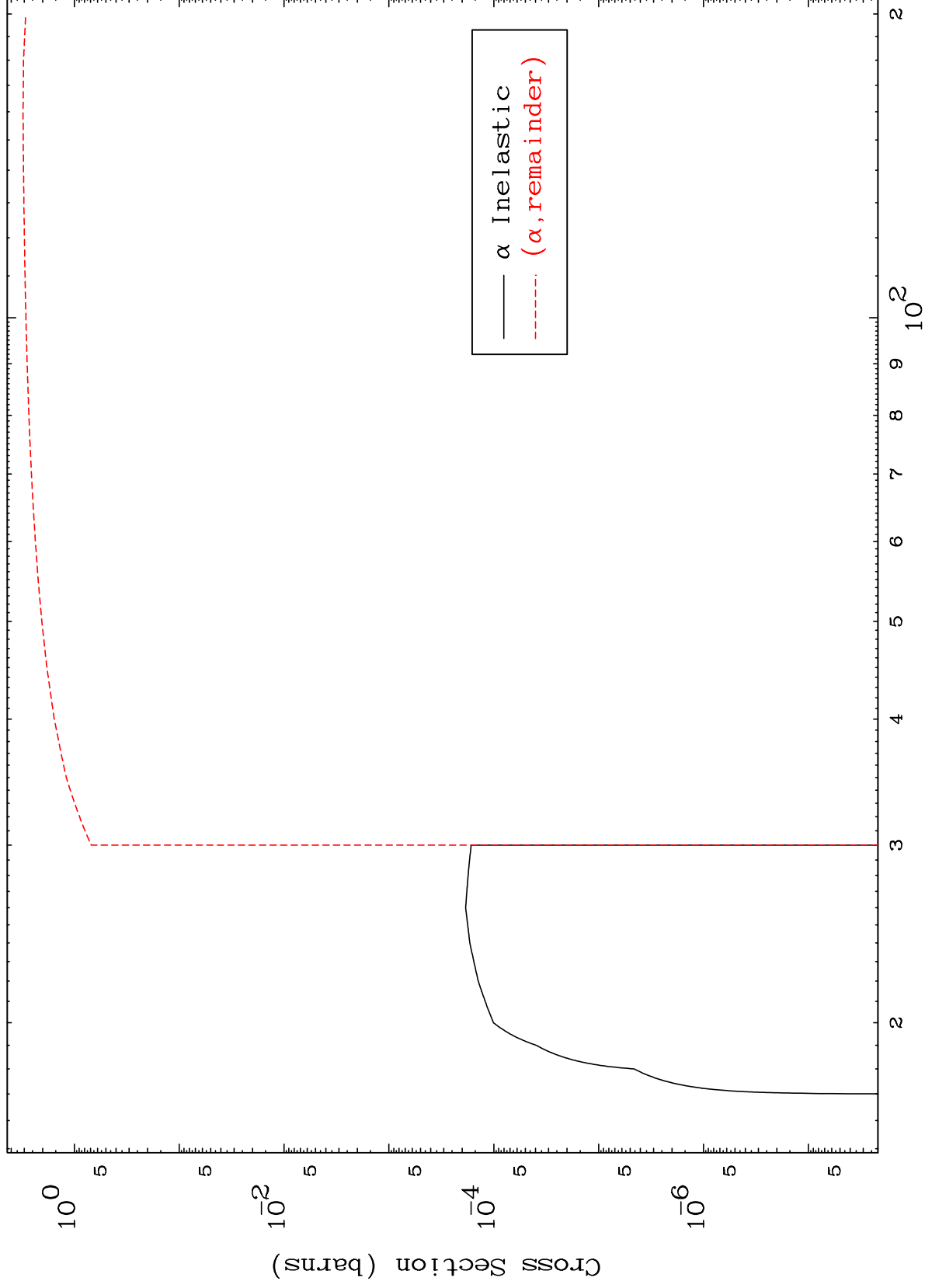
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9973

$\alpha$  Major  
0 Kelvin Cross Sections

102-No-255



1

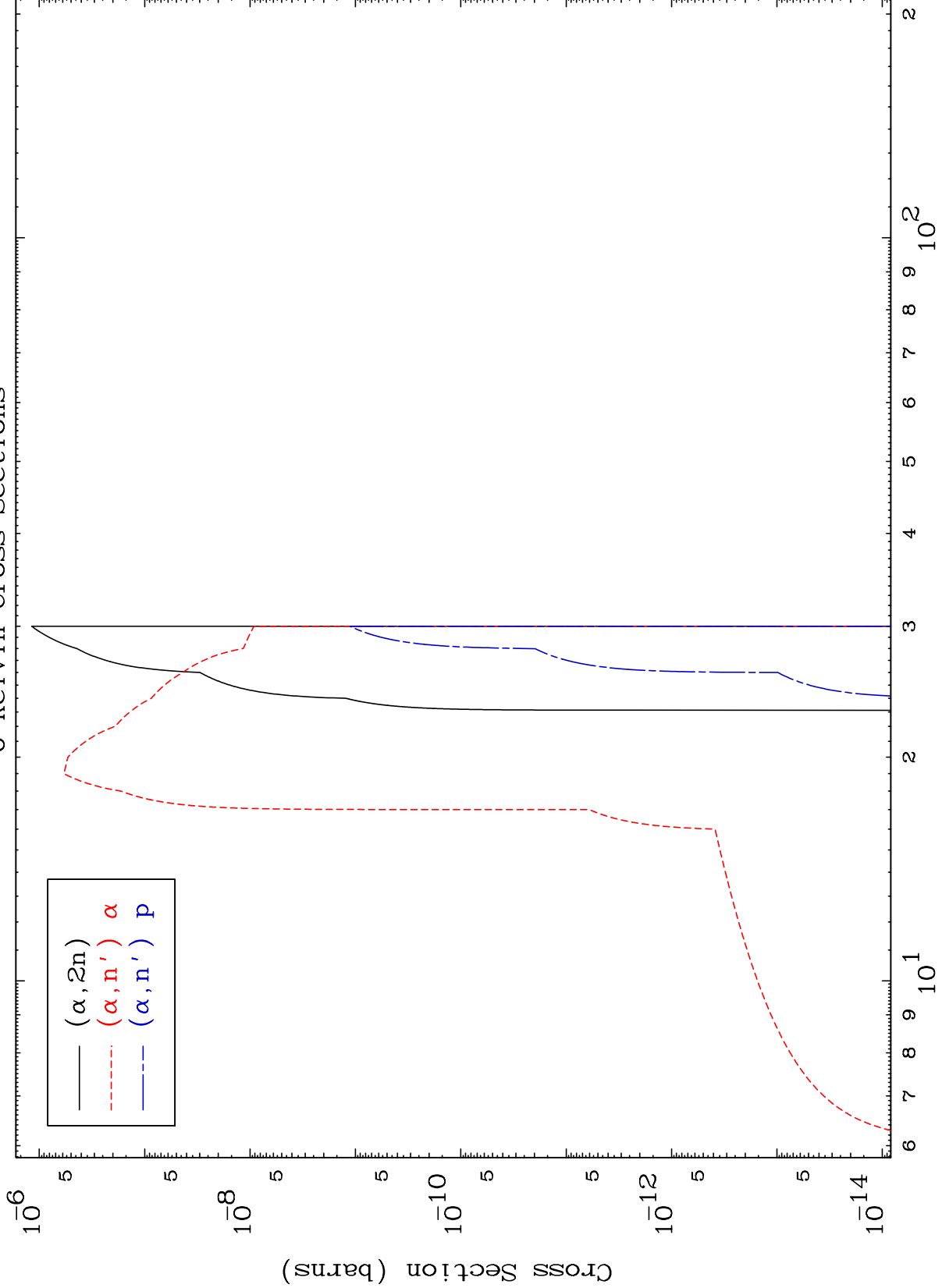
Incident Energy (MeV)

102-No-255

MAT 9973

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

102-No-255



( $\alpha, 2n$ )  
( $\alpha, n'$ )  $\alpha$   
( $\alpha, n'$ )  $p$

102-No-255

Incident Energy (MeV)

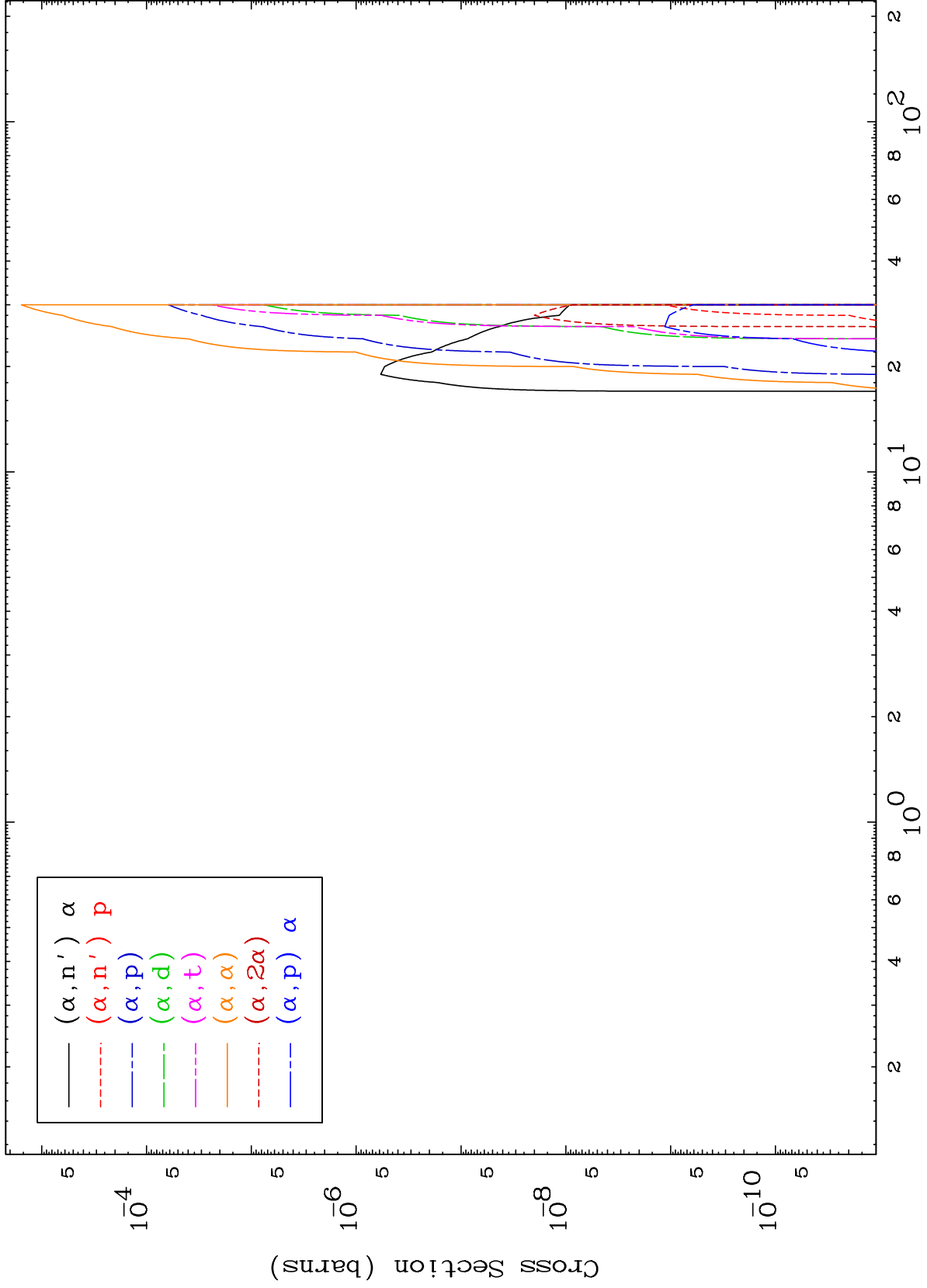
2

MAT 9973

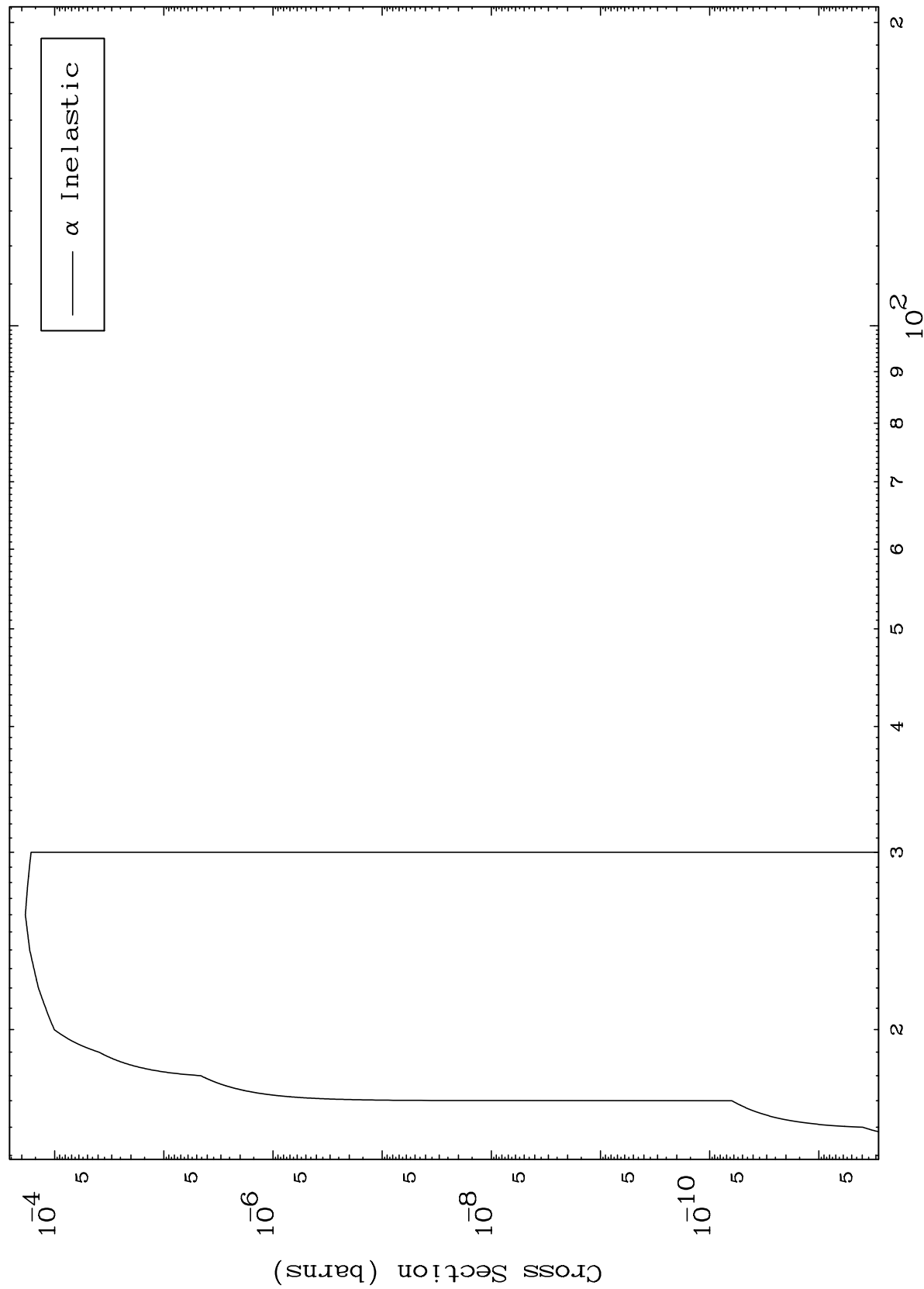
$\alpha$  Charged Particle

102-No-255

0 Kelvin Cross Sections



0 Kelvin Cross Sections

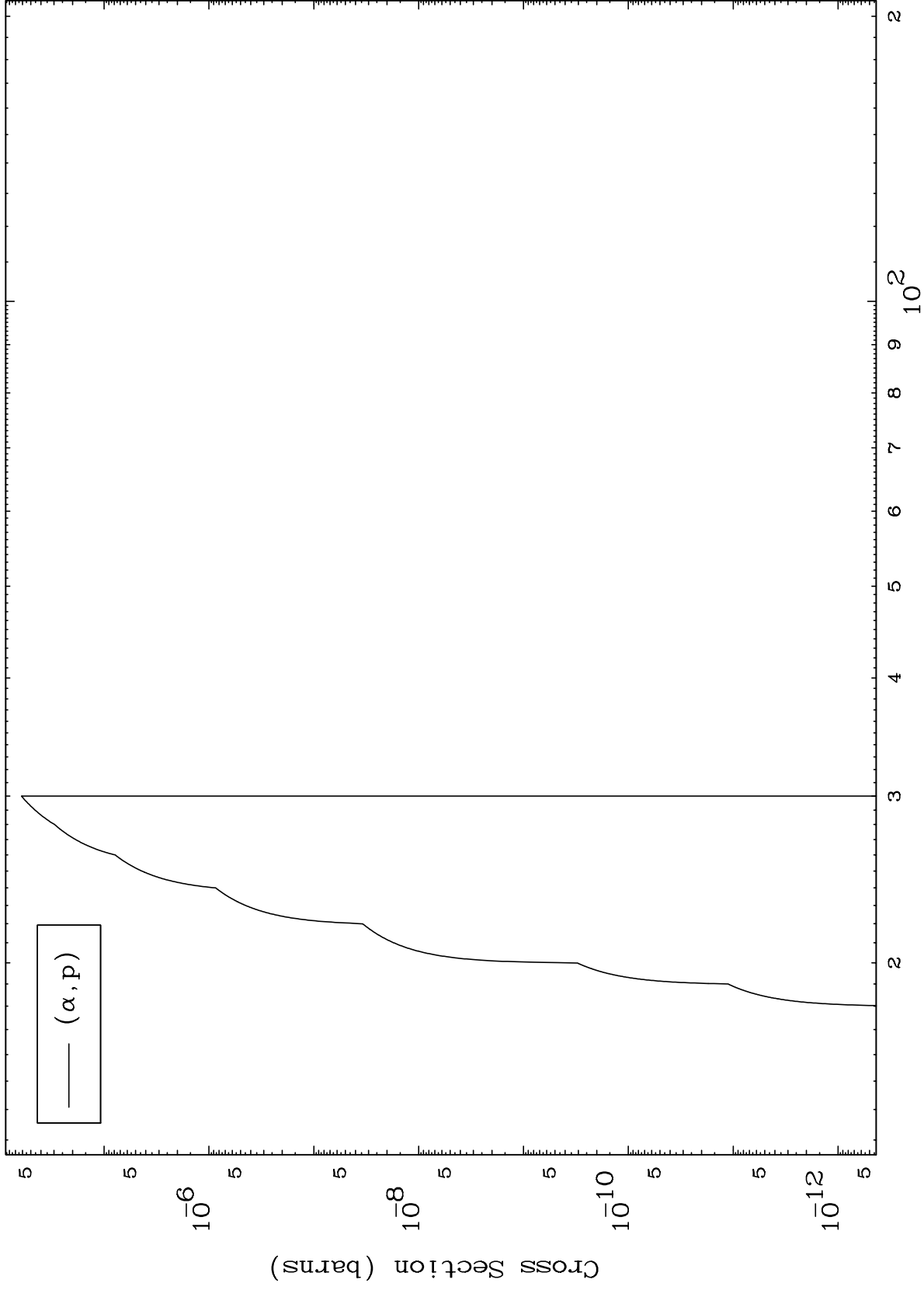


$\alpha$  Inelastic

MAT 9973

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

102-No-255



5

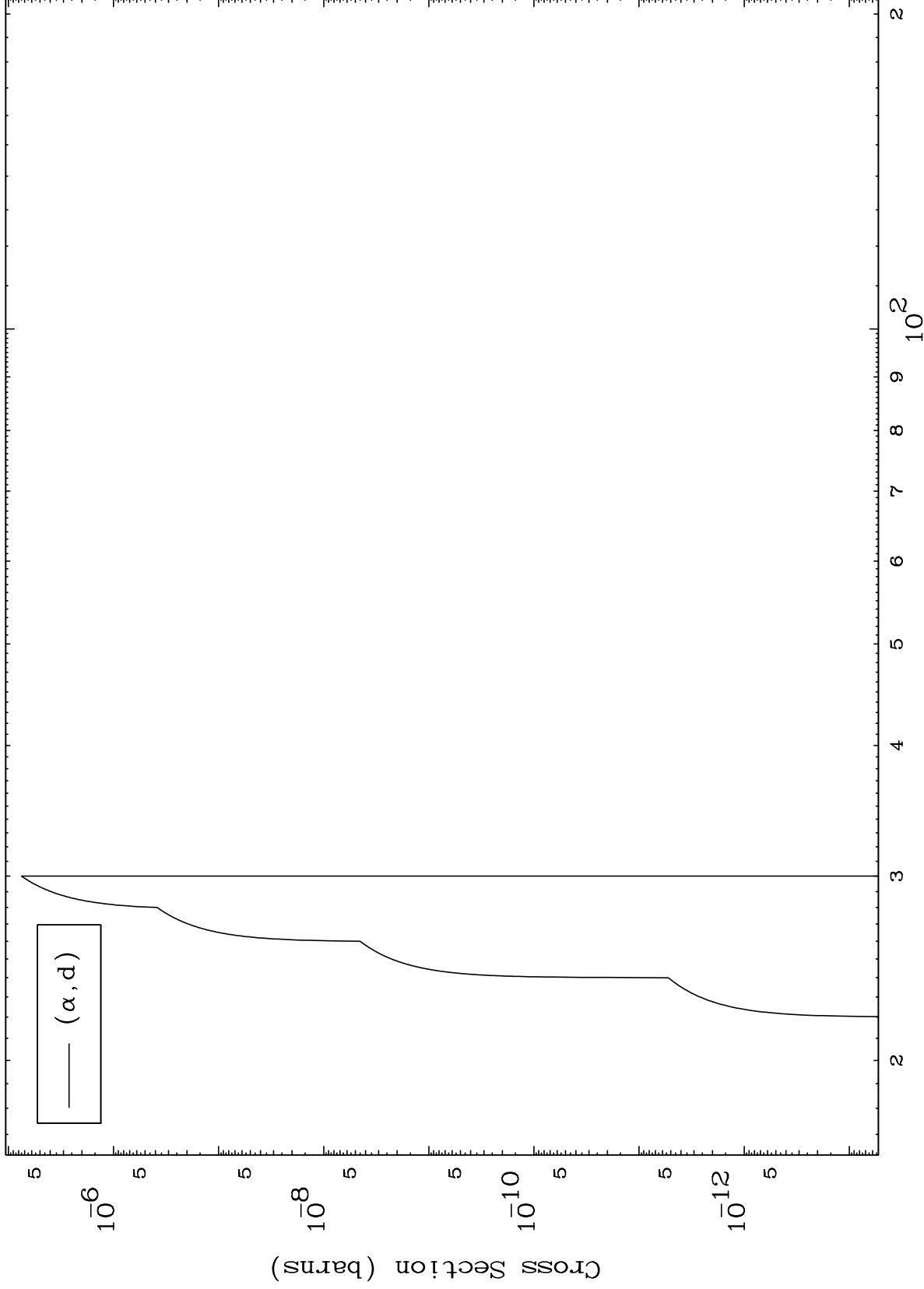
Incident Energy (MeV)

102-No-255

MAT 9973

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

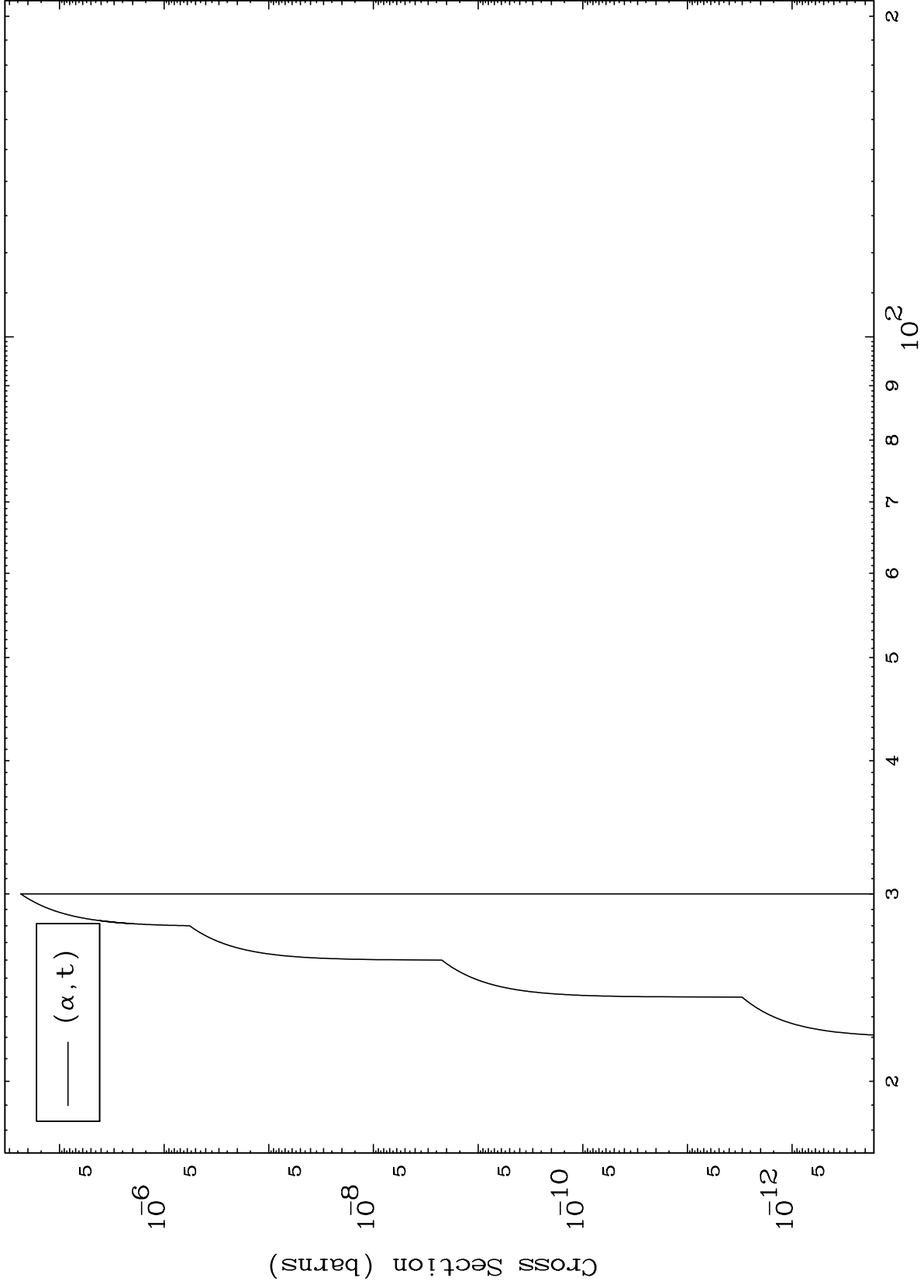
102-No-255



6

Incident Energy (MeV)

102-No-255



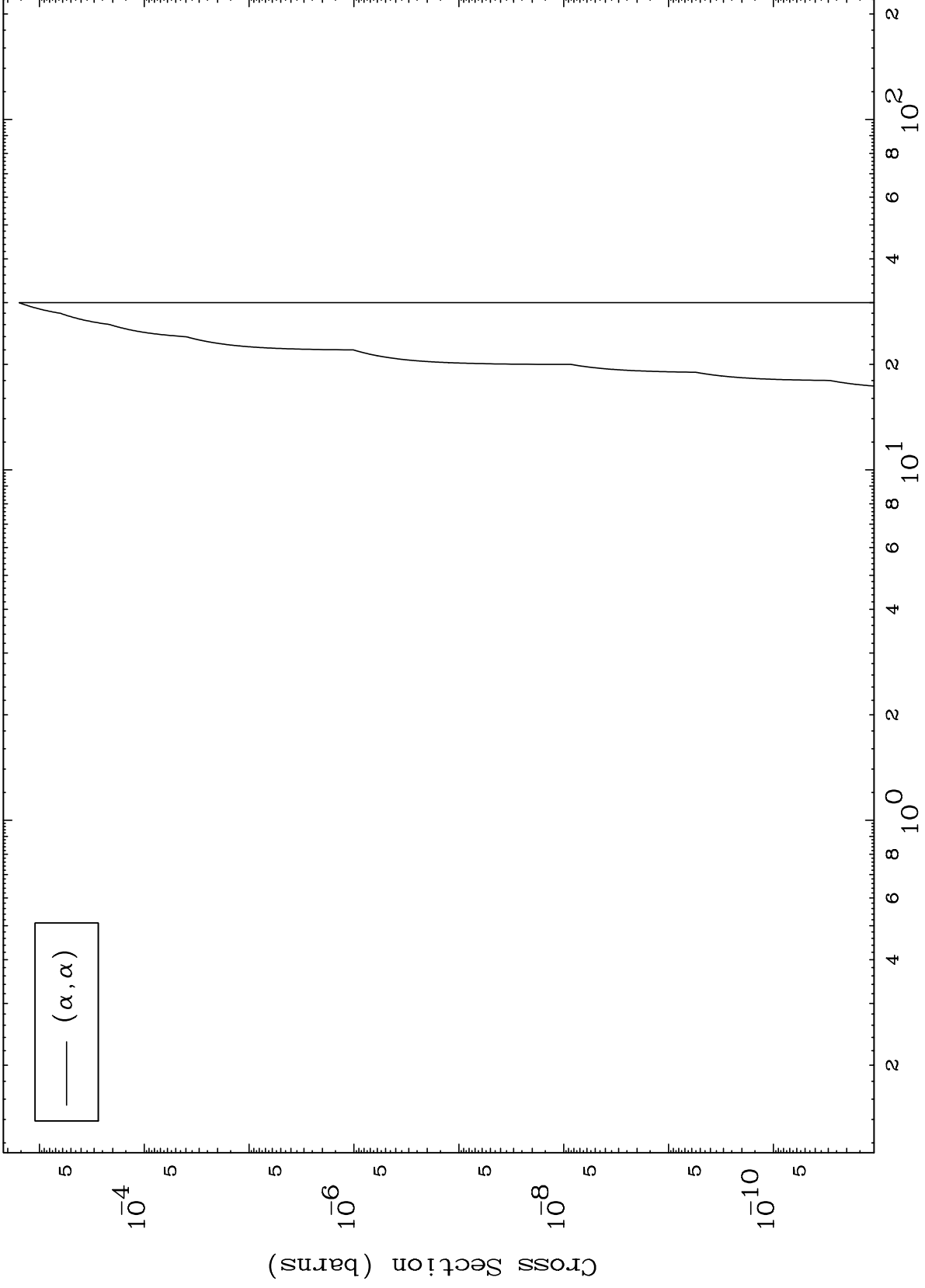


MAT 9973

( $\alpha, \alpha$ ) Levels

102-No-255

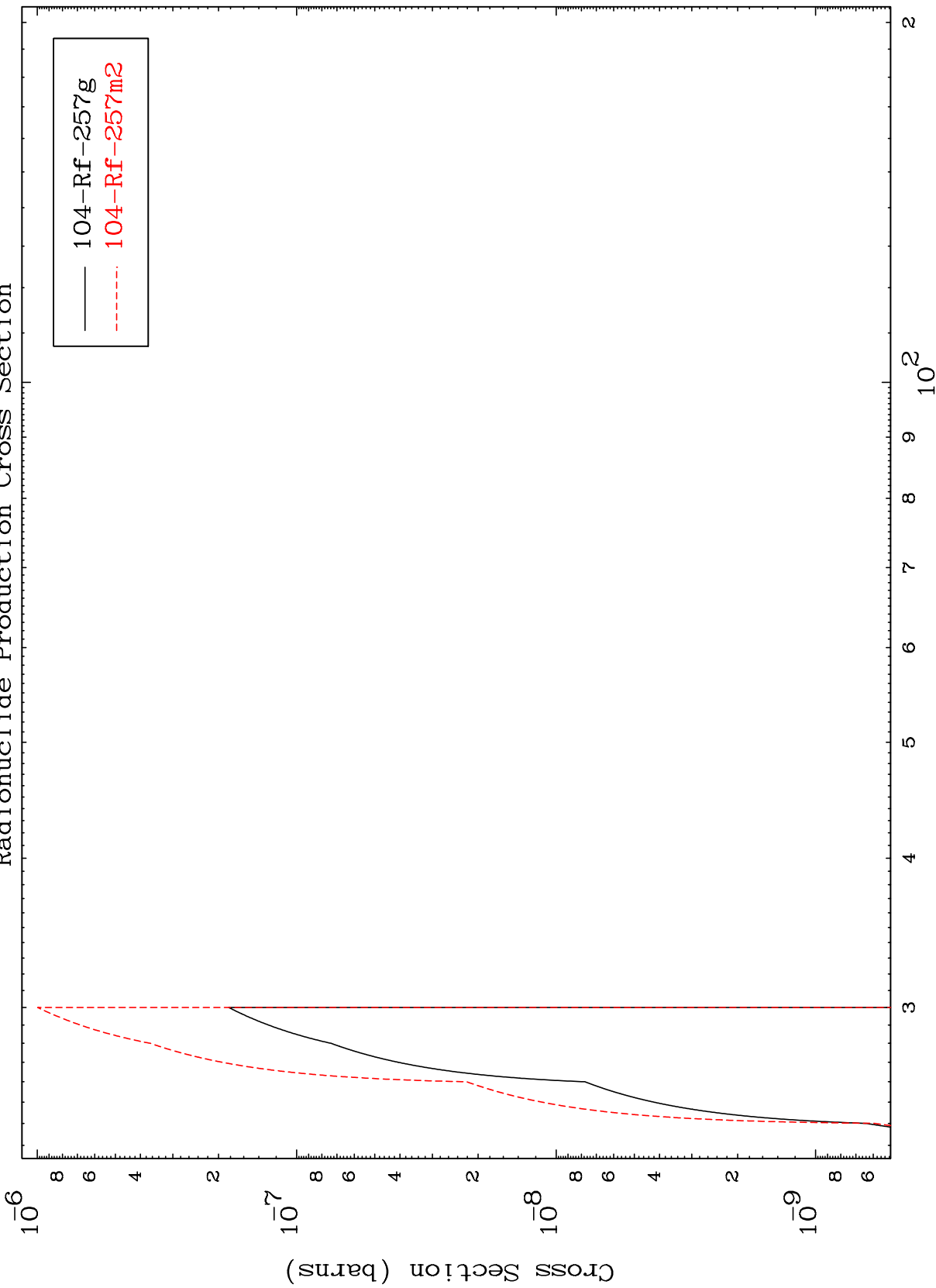
0 Kelvin Cross Sections



MAT 9973

102-No-255

$(\alpha, 2n)$   
Radionuclide Production Cross Section



9

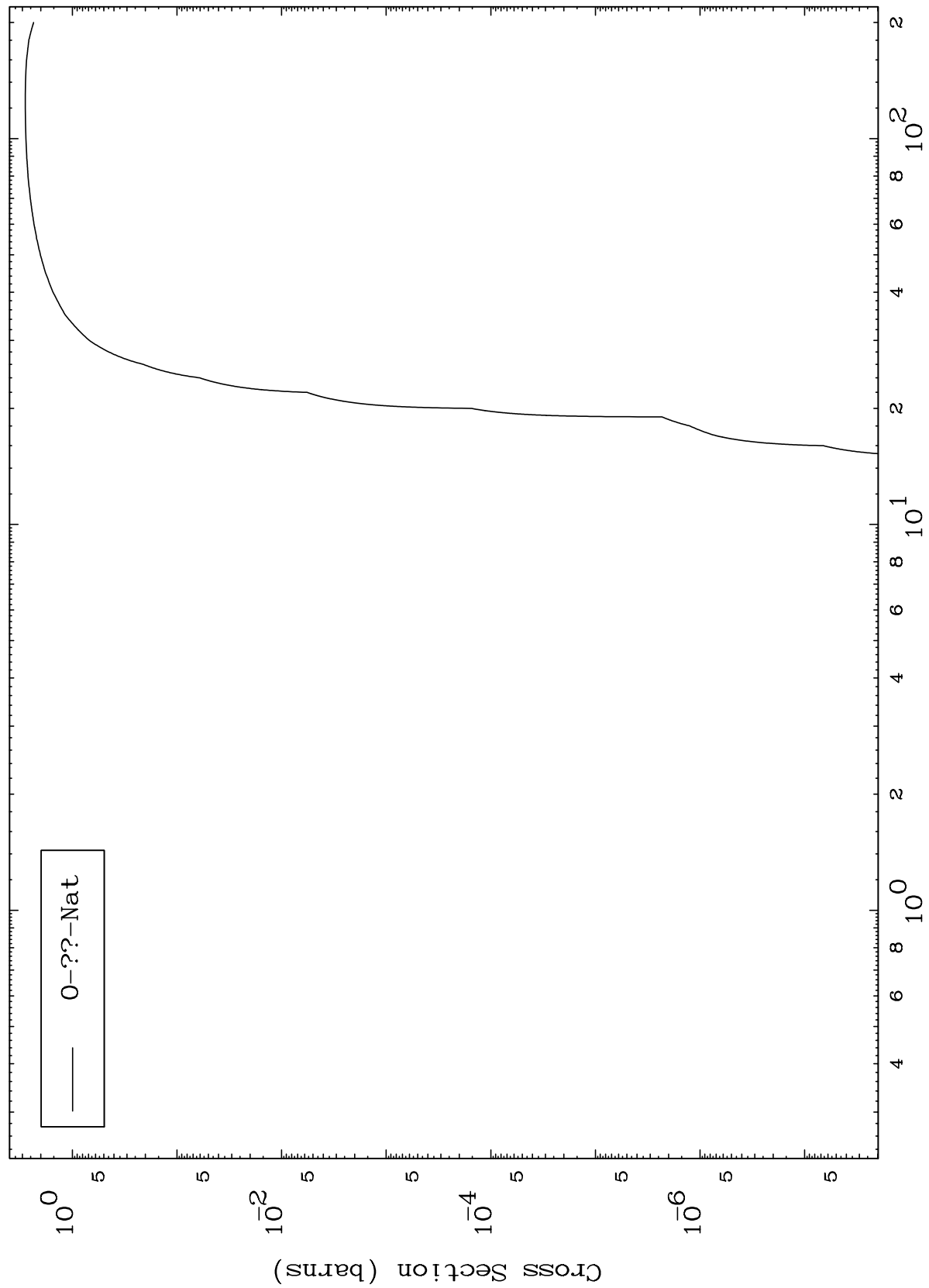
Incident Energy (MeV)

102-No-255

MAT 9973

102-No-255

$\alpha$  Fission  
Radionuclide Production Cross Section



10

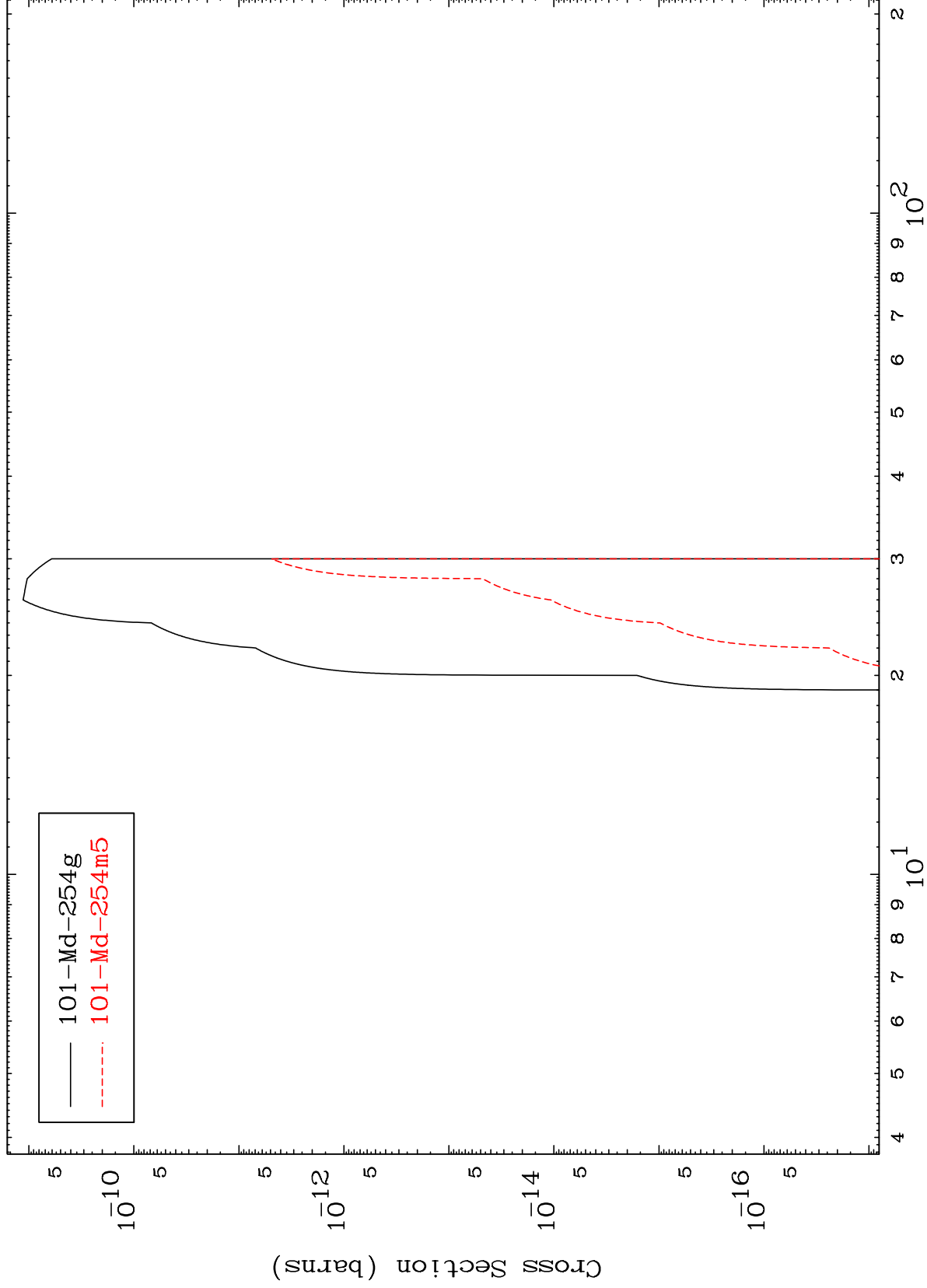
102-No-255

MAT 9973

( $\alpha, p$ )  $\alpha$

102-No-255

Radionuclide Production Cross Section



11

Incident Energy (MeV)

102-No-255