

Program EVALPLOT  
(Version 2018-1)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

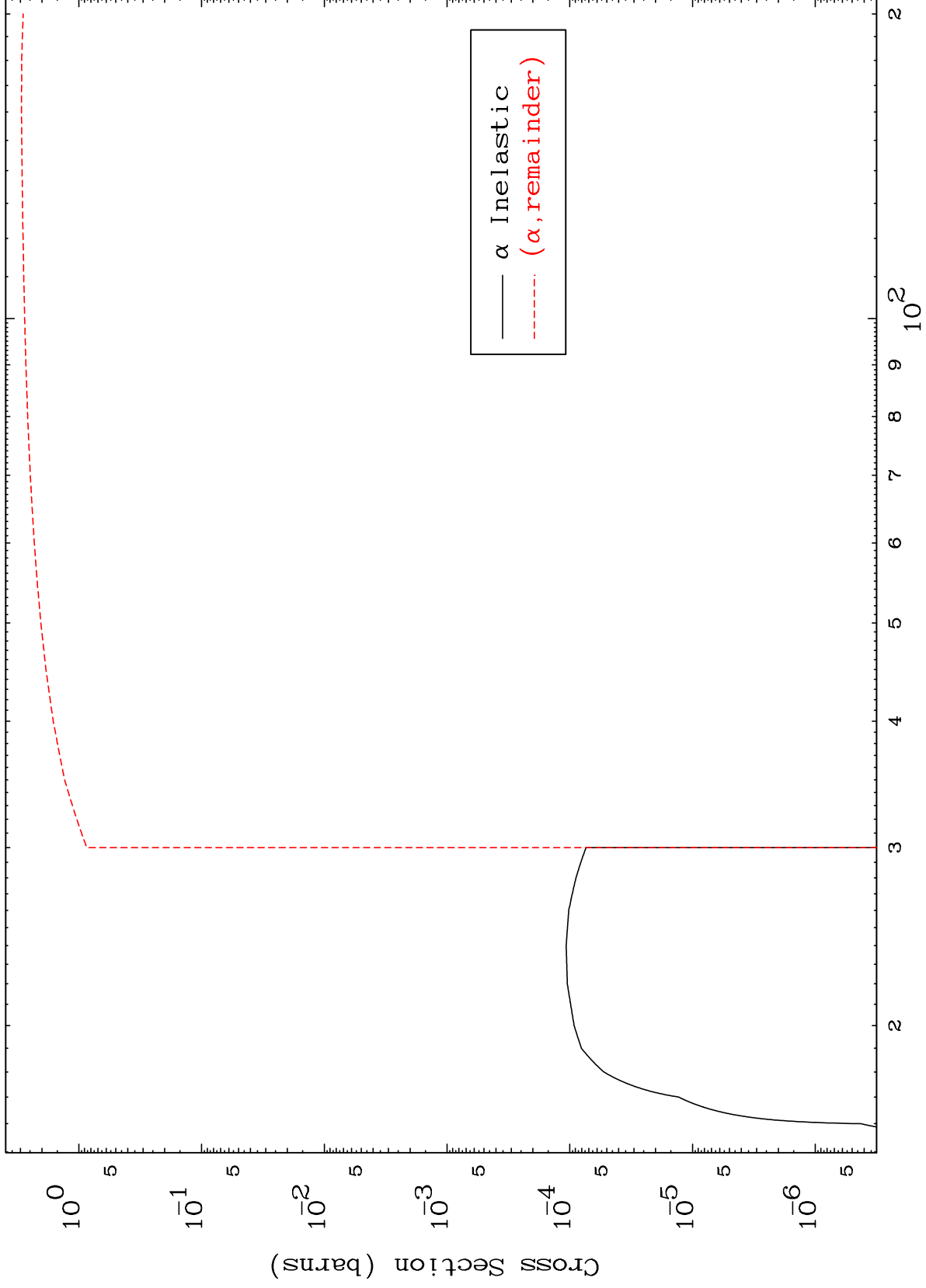
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9322

$\alpha$  Major  
0 Kelvin Cross Sections

93-Np-229



1

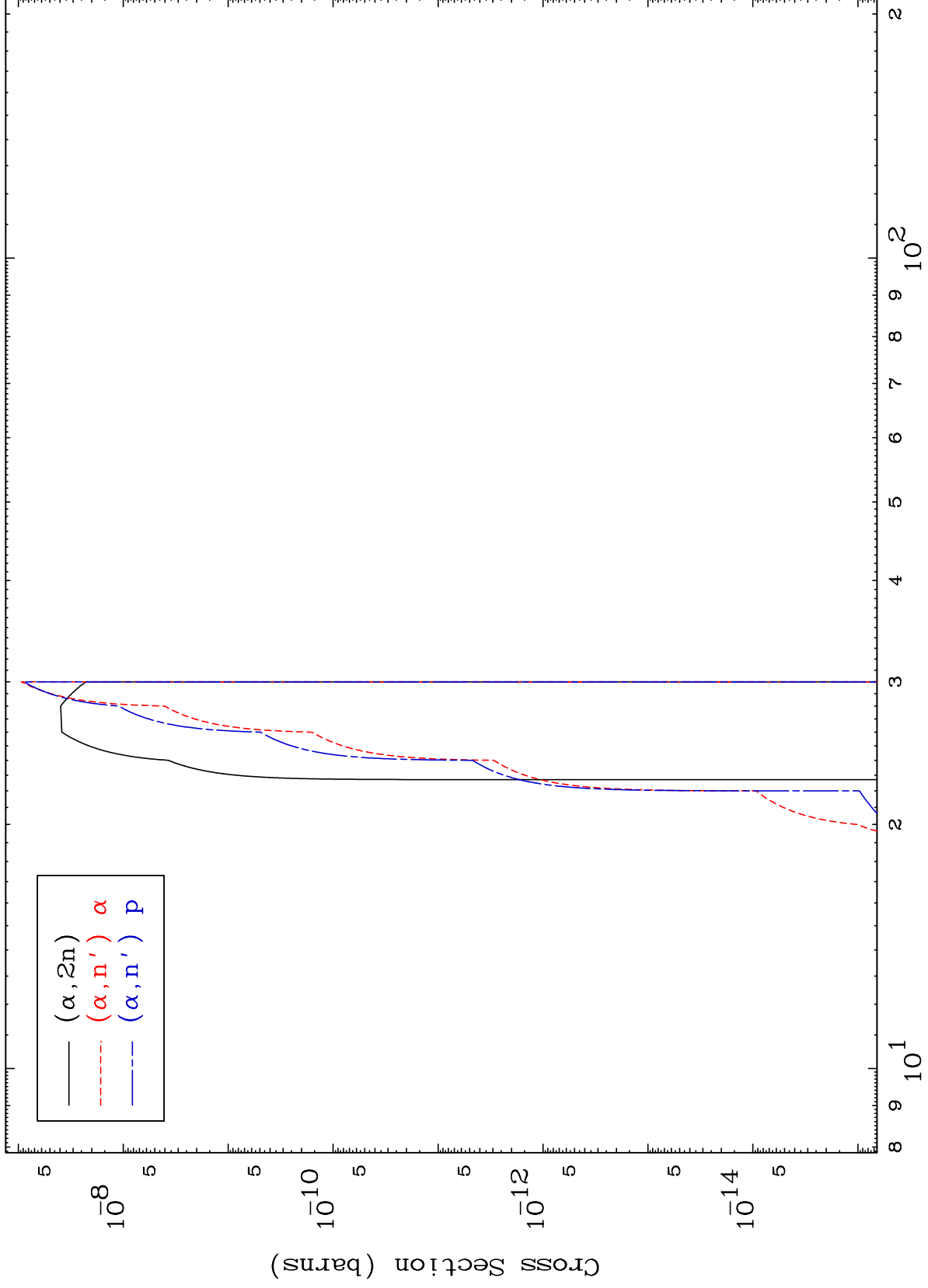
Incident Energy (MeV)

93-Np-229

MAT 9322

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

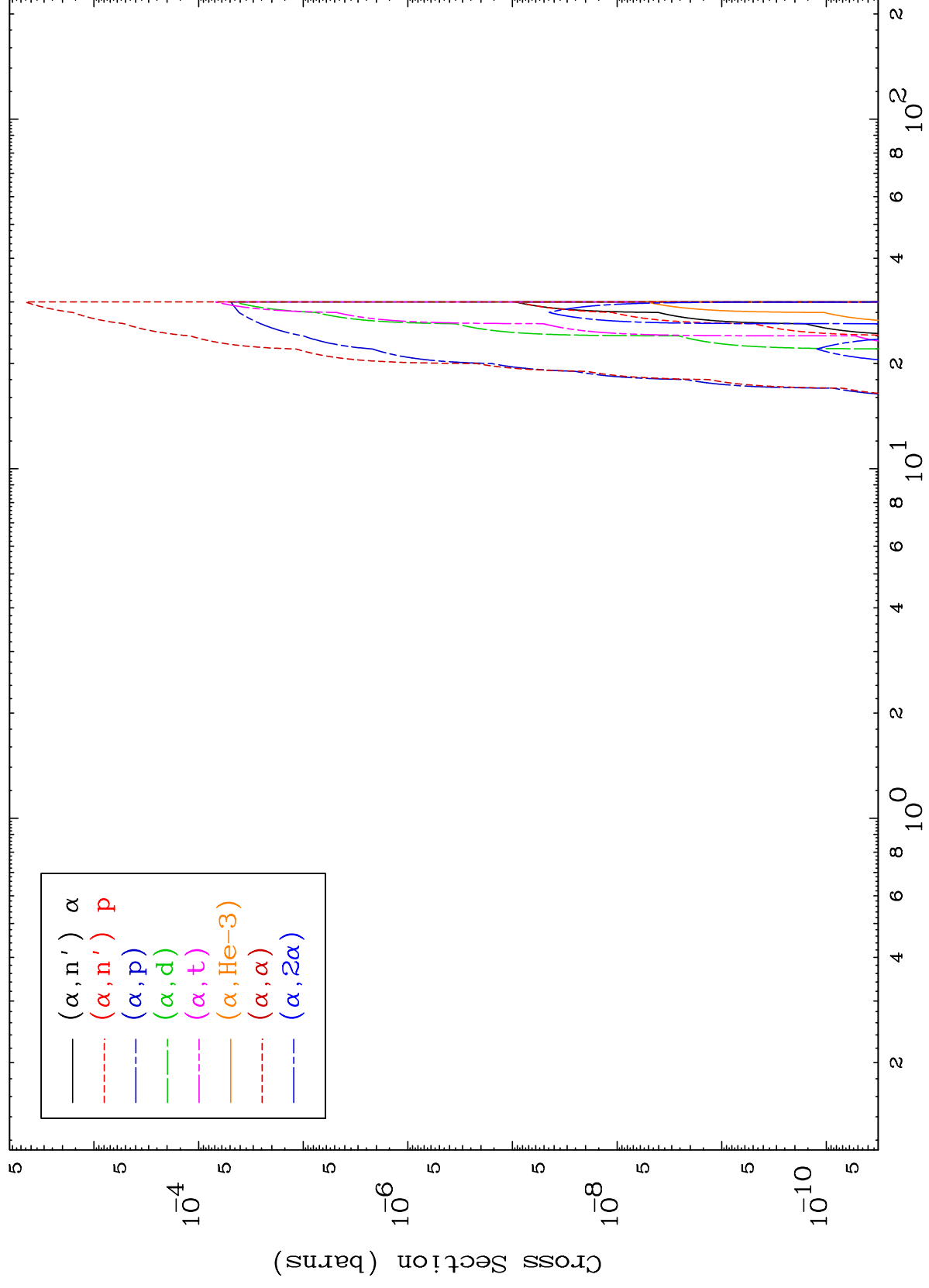
93-Np-229



2

Incident Energy (MeV)

93-Np-229

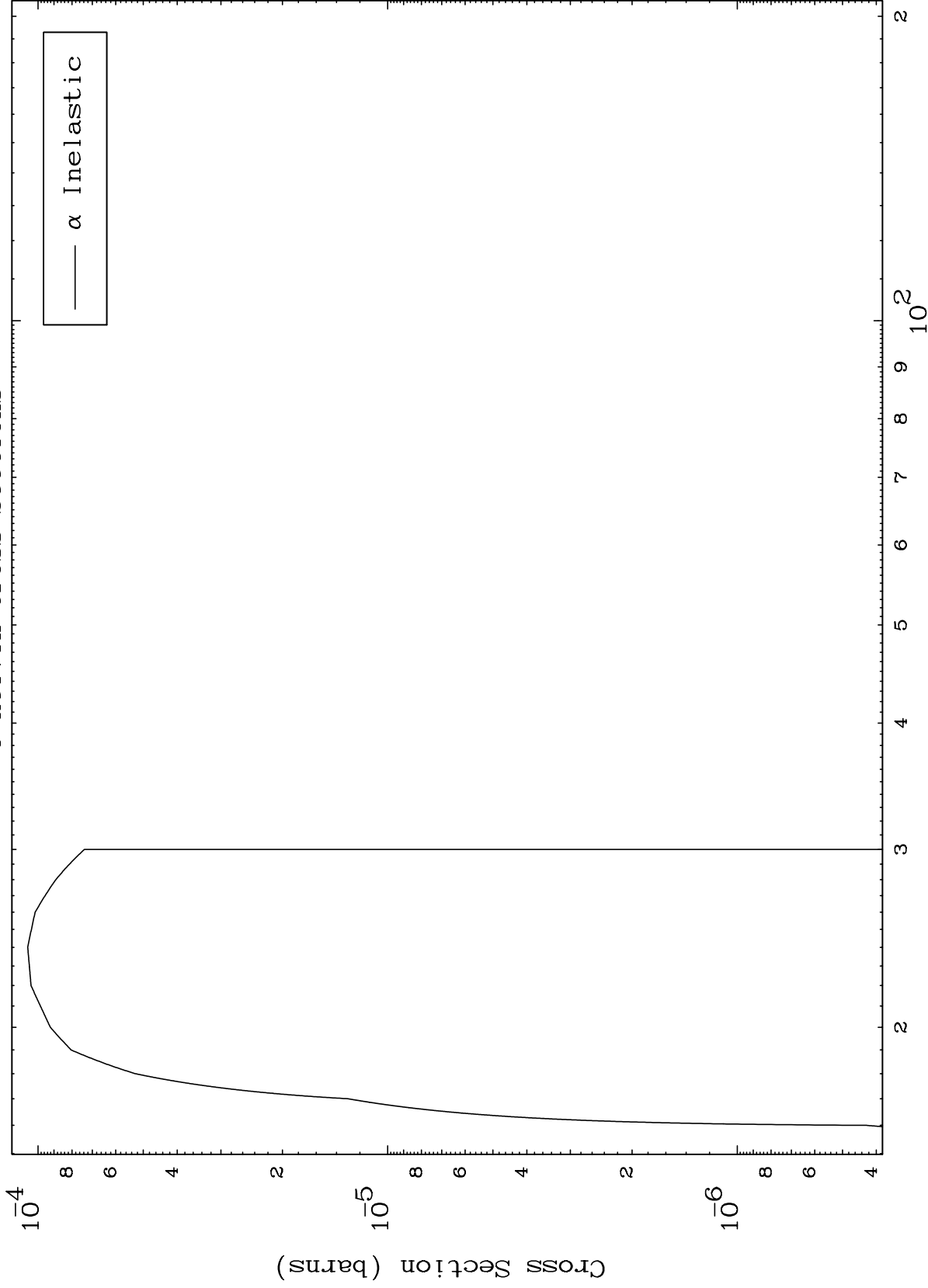


MAT 9322

( $\alpha, n'$ ) Level

93-Np-229

0 Kelvin Cross Sections



4

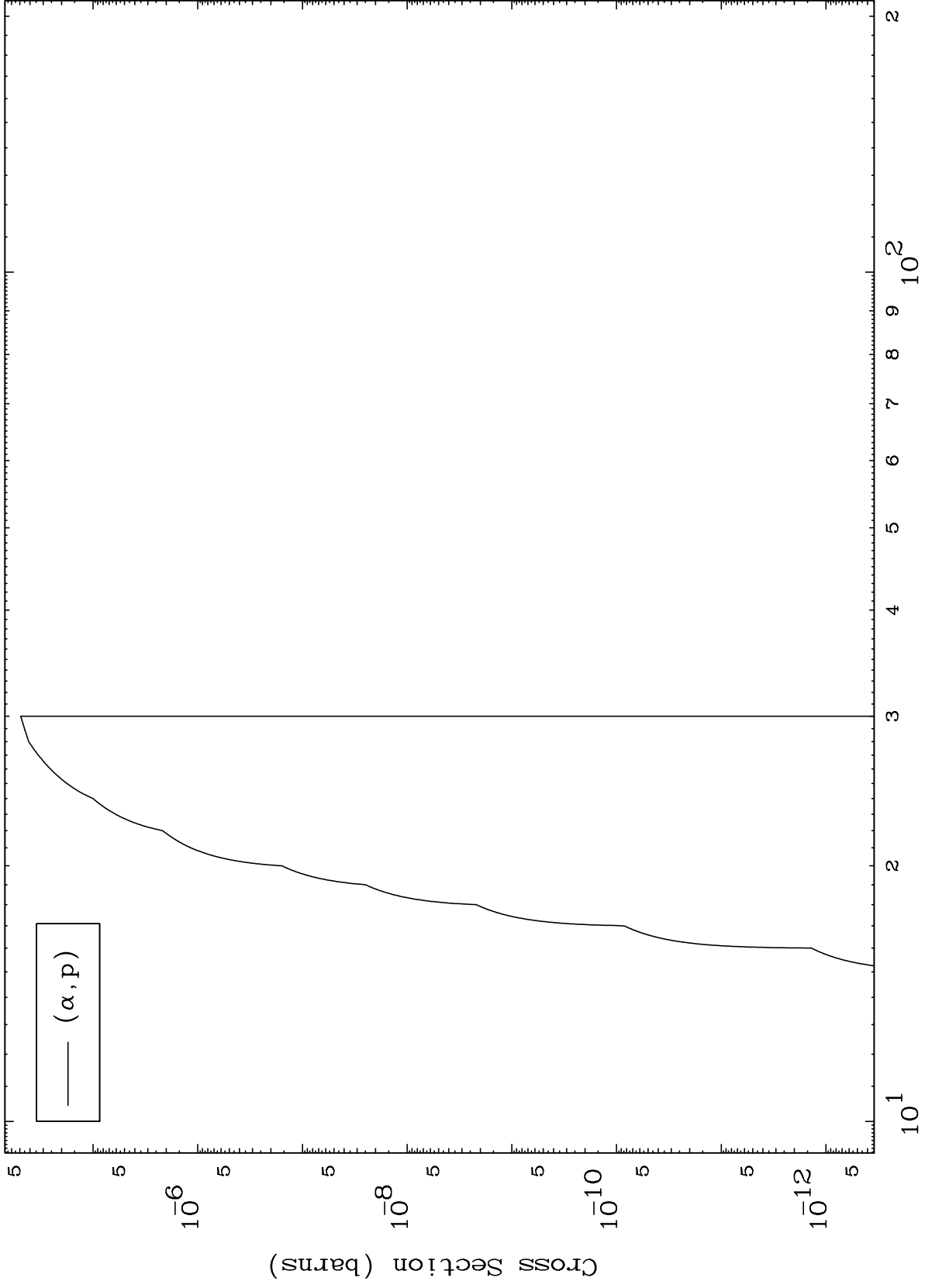
Incident Energy (MeV)

93-Np-229

MAT 9322

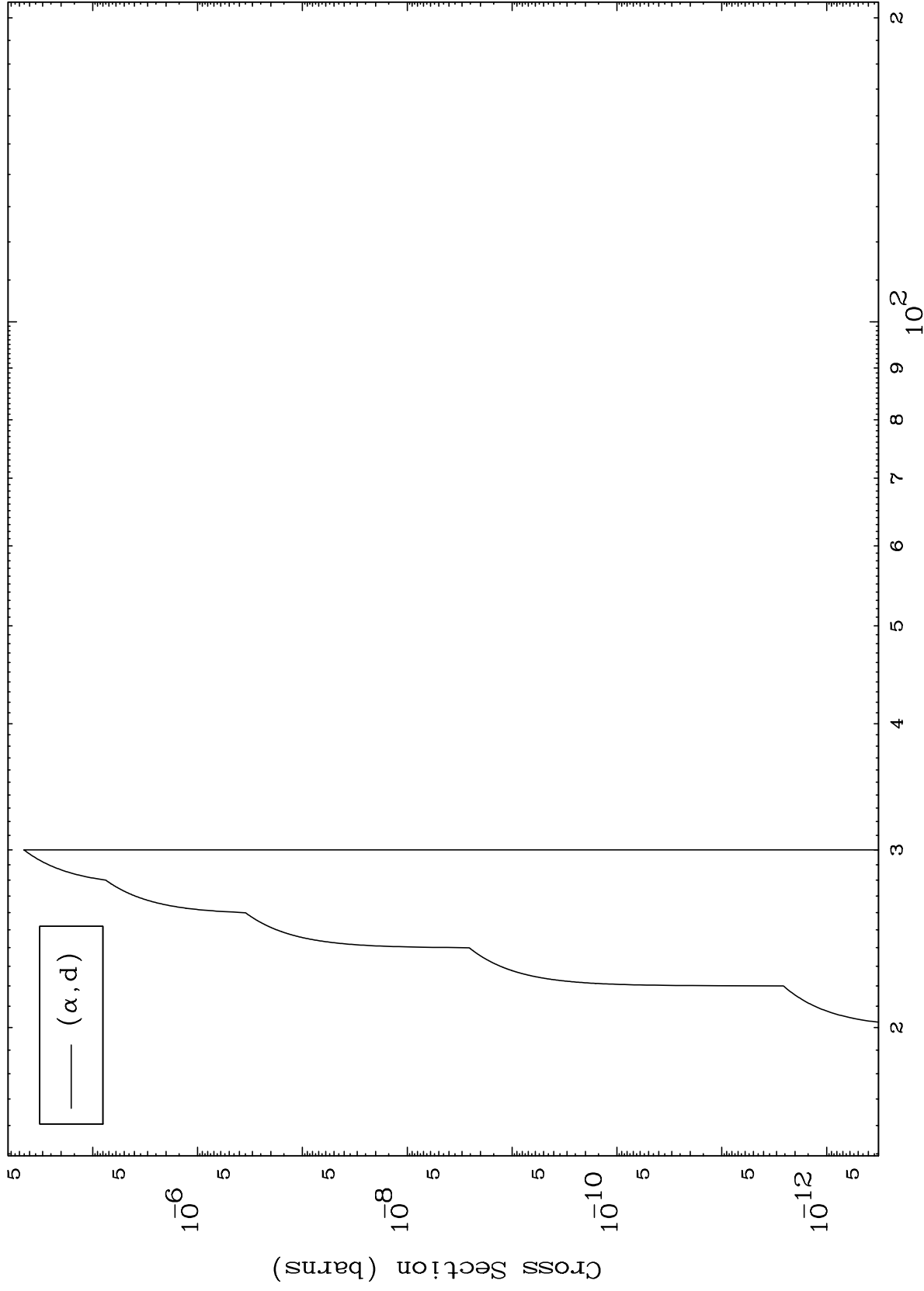
( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

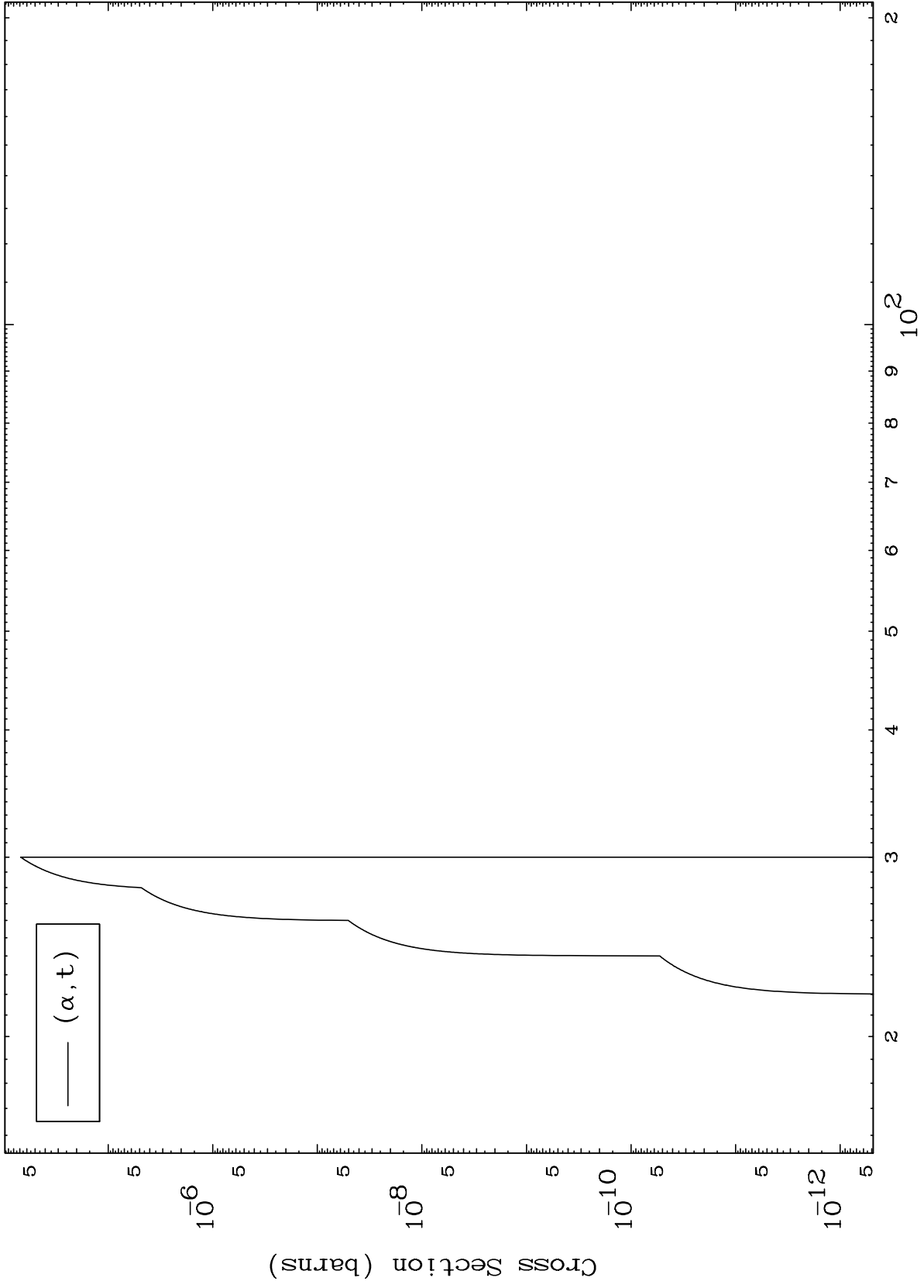
93-Np-229



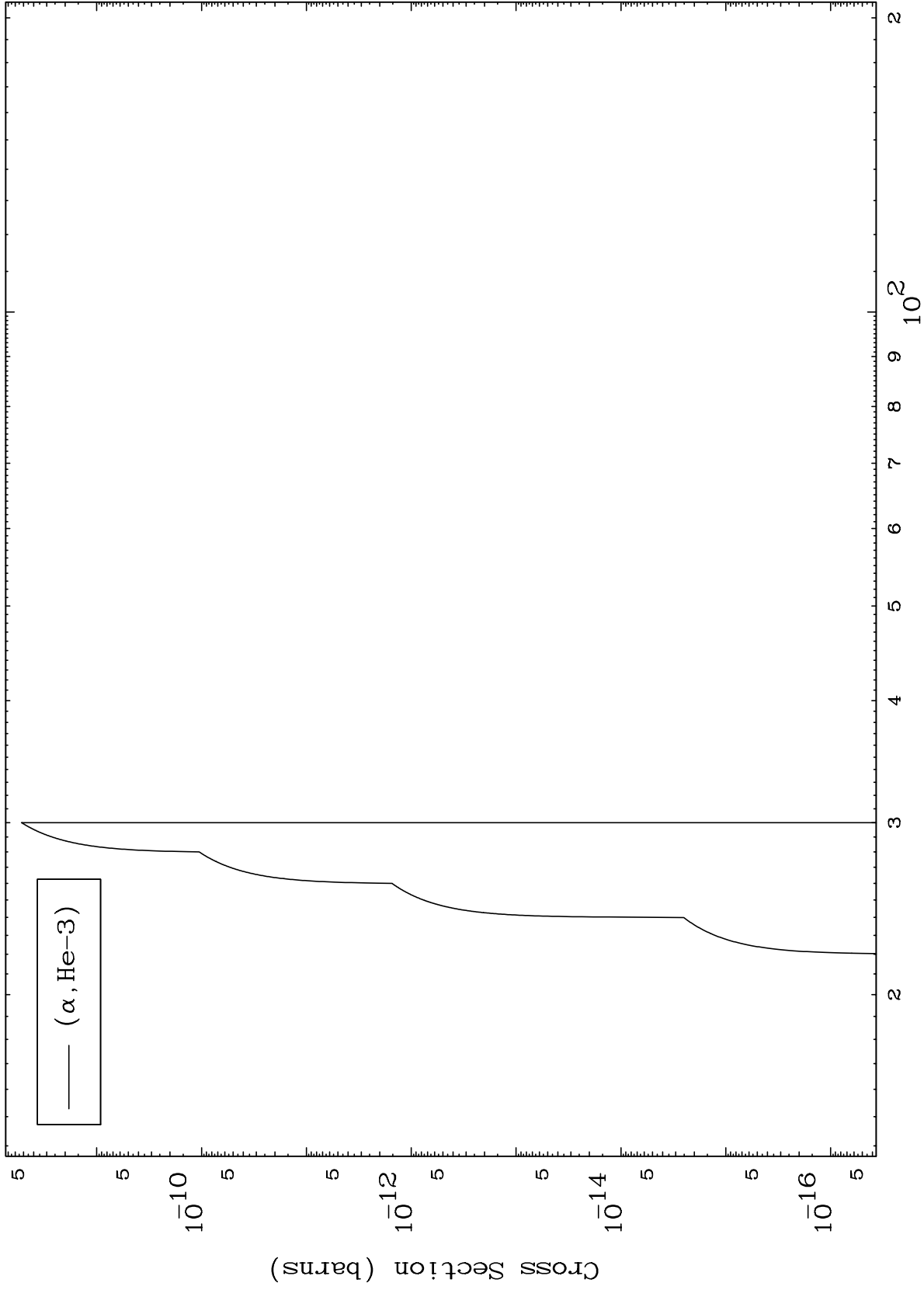
Incident Energy (MeV)

93-Np-229





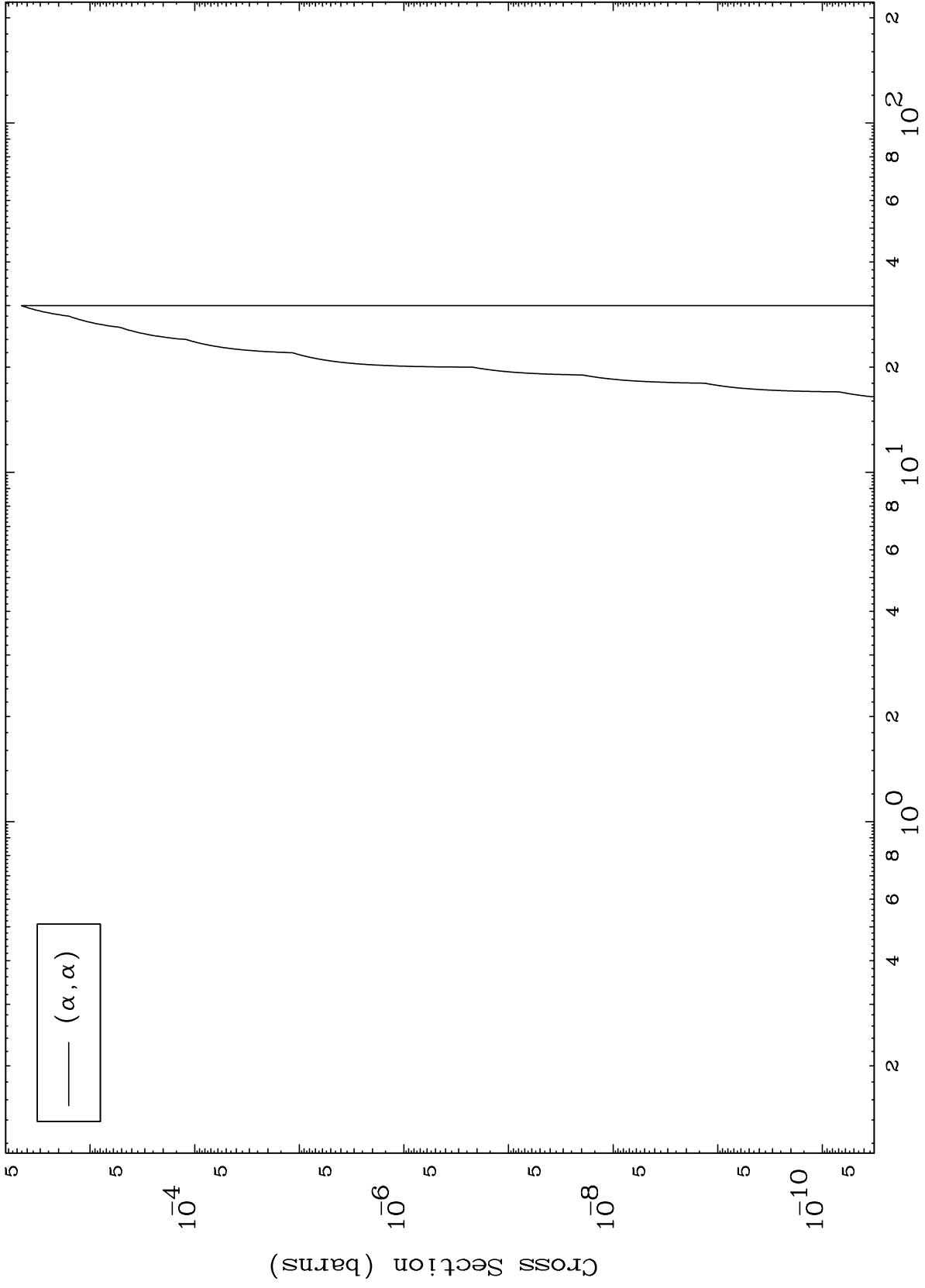




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( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

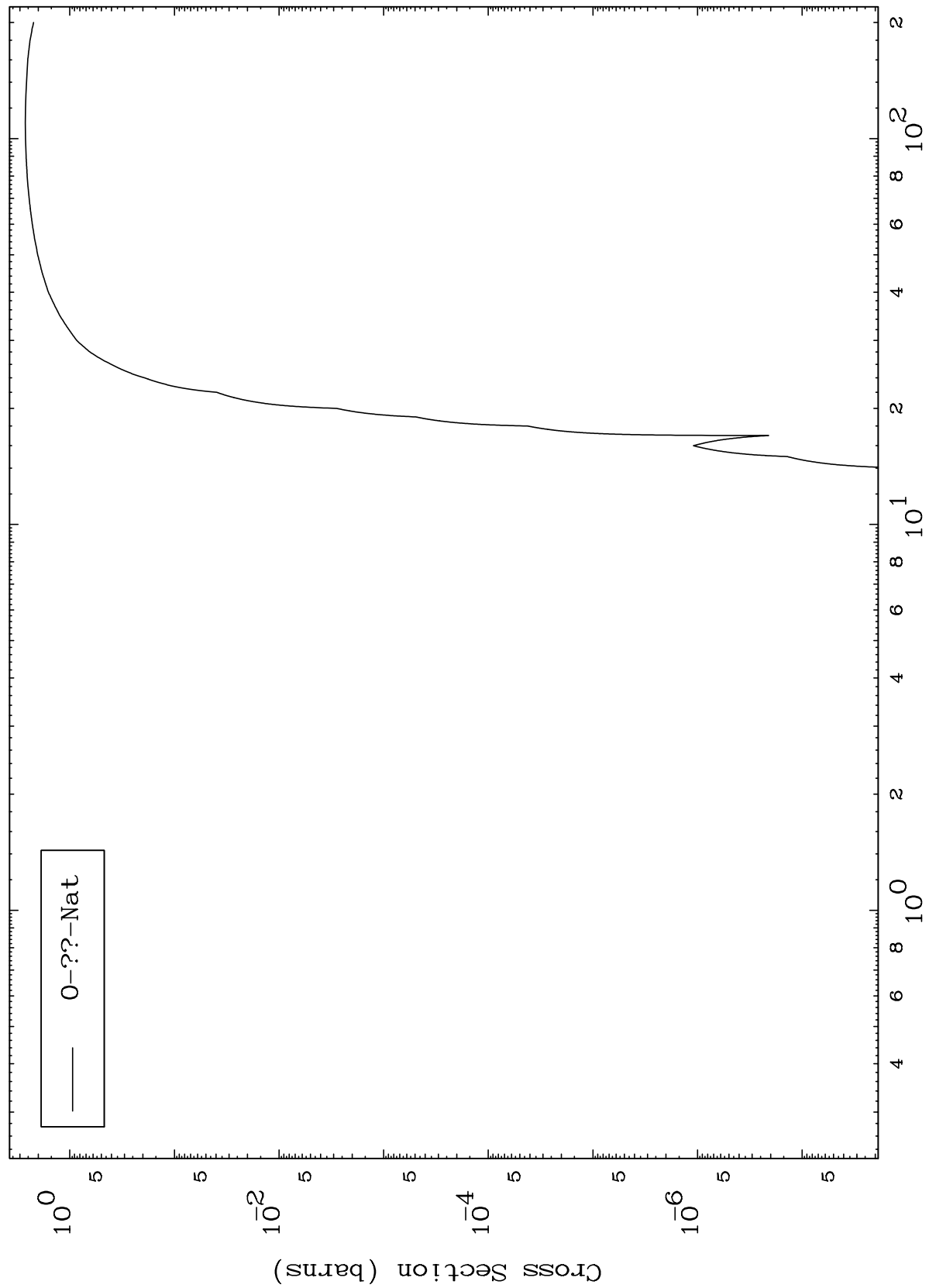
93-Np-229



MAT 9322

93-Np-229

$\alpha$  Fission  
Radionuclide Production Cross Section



93-Np-229

Incident Energy (MeV)

10