

Program EVALPLOT  
(Version 2018-1)

by

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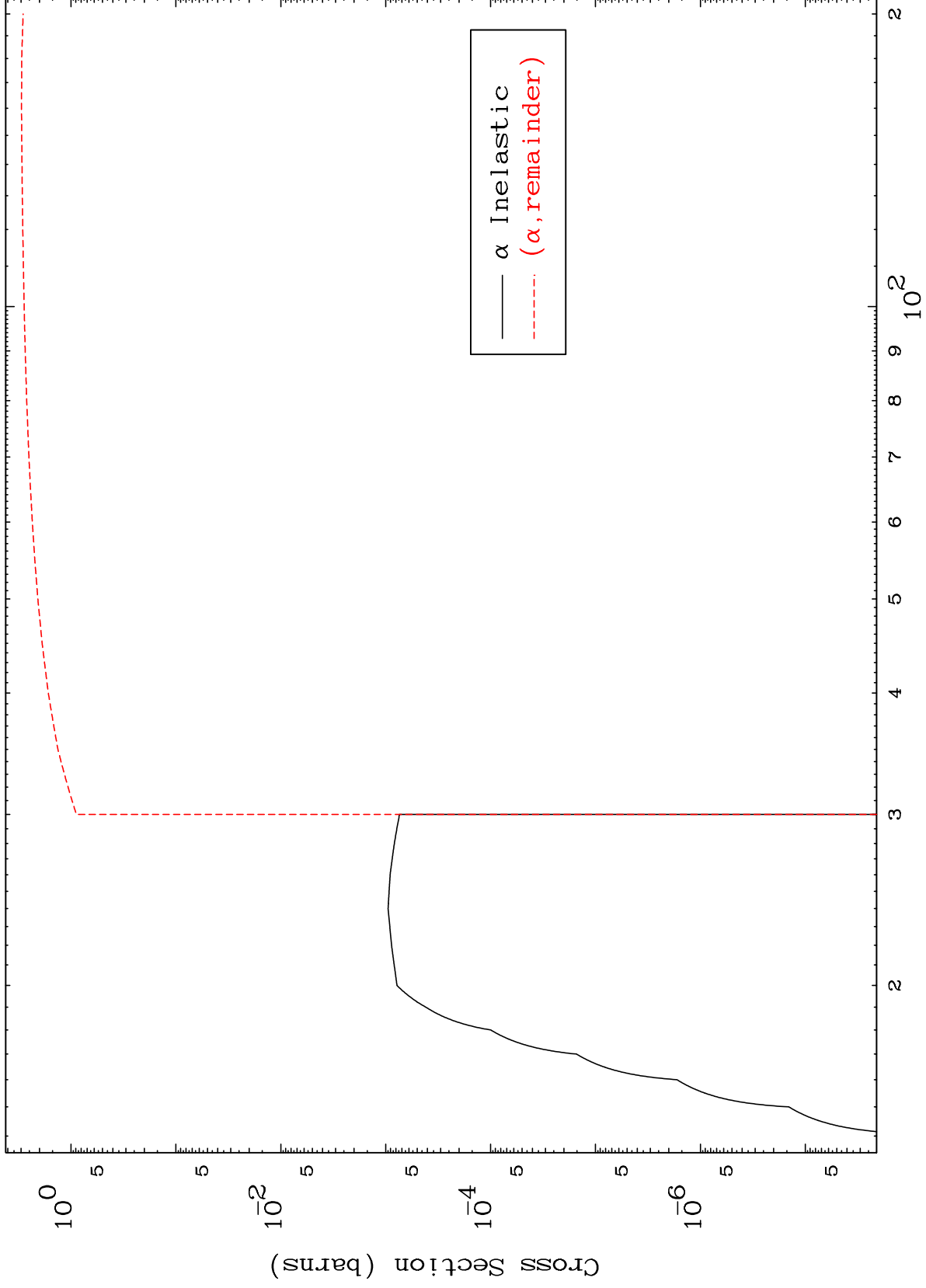
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 93334

$\alpha$  Major  
0 Kelvin Cross Sections

93-Np-233



1

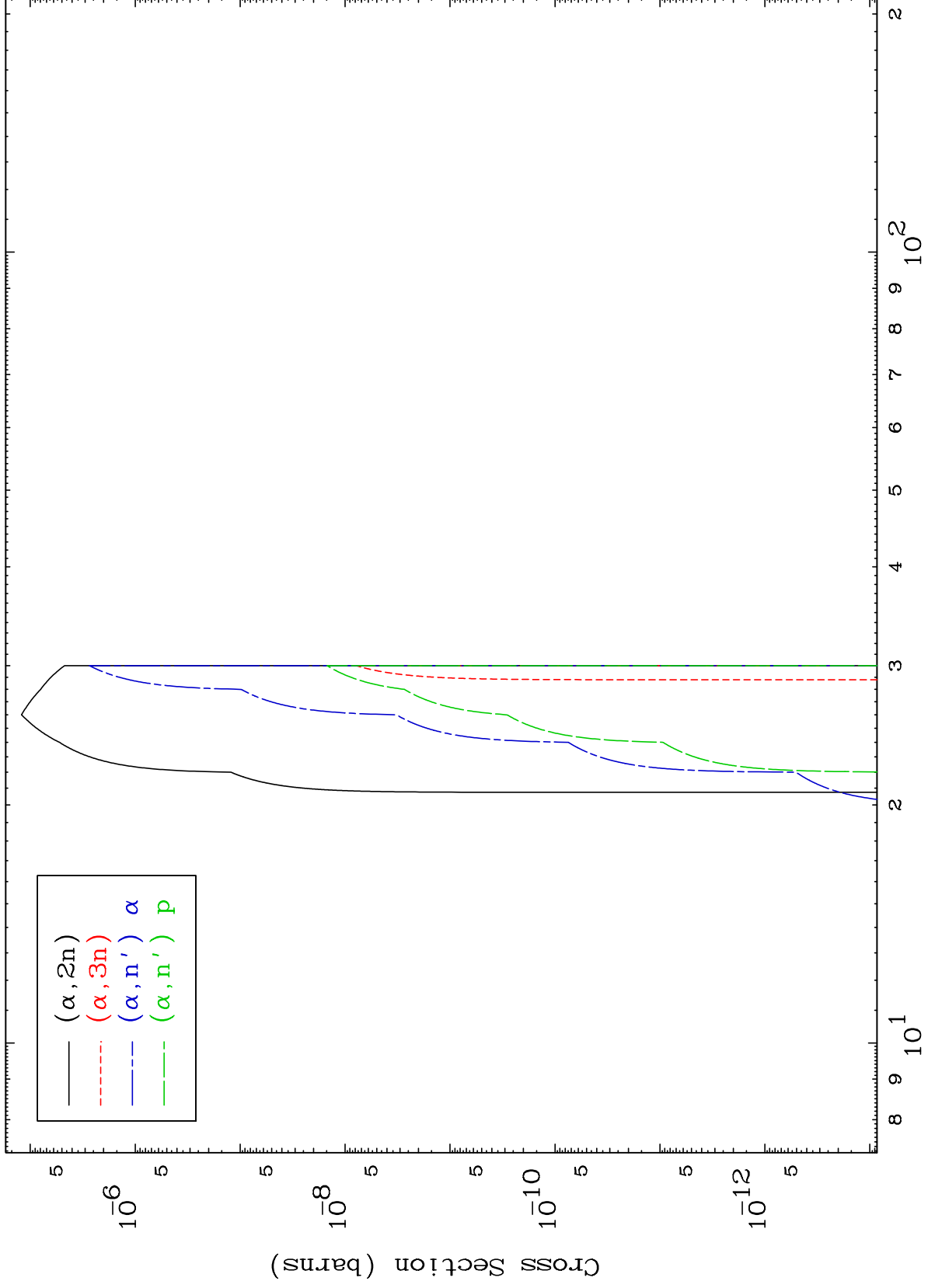
Incident Energy (MeV)

93-Np-233

MAT 9334

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

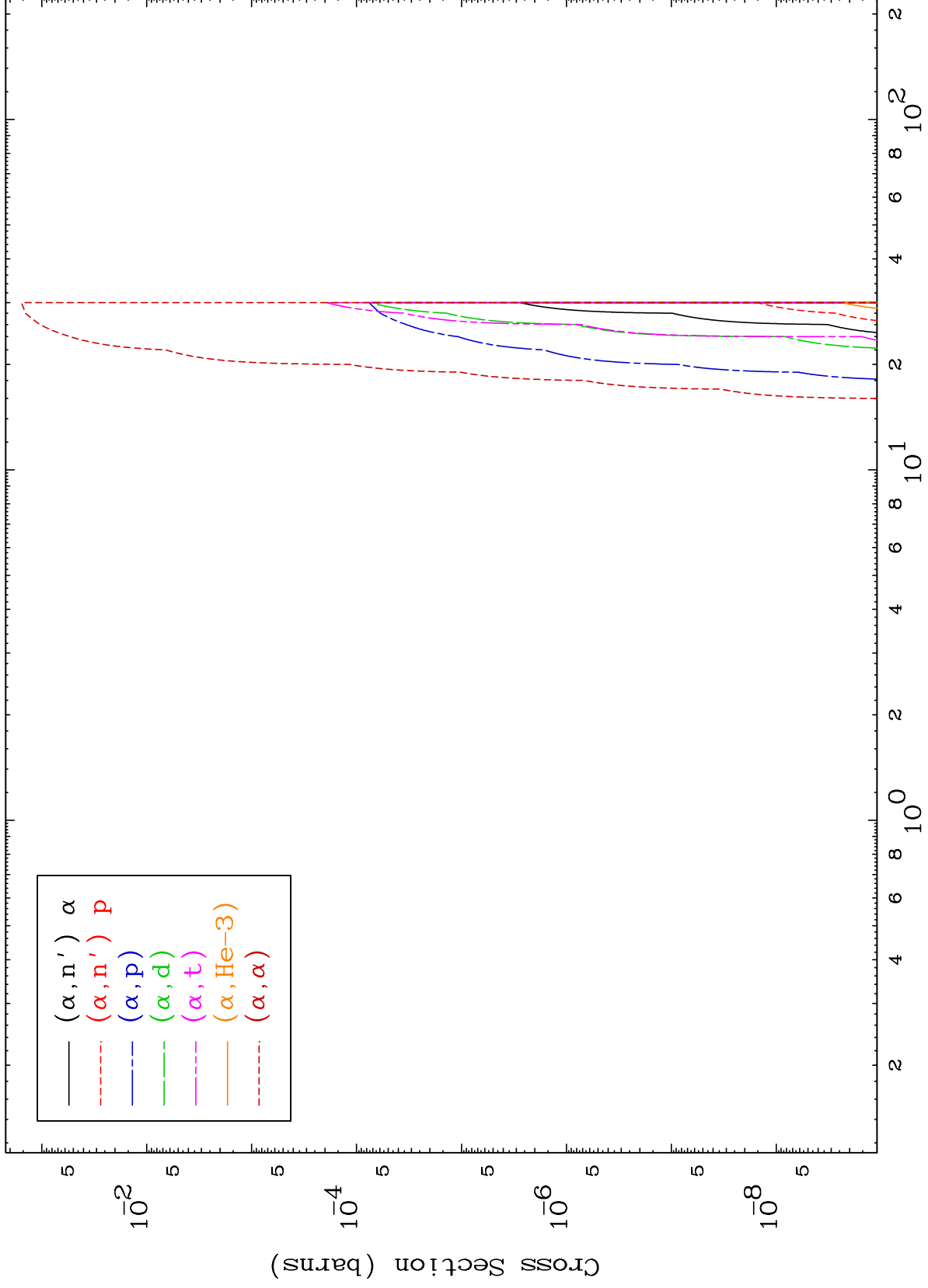
93-Np-233



2

Incident Energy (MeV)

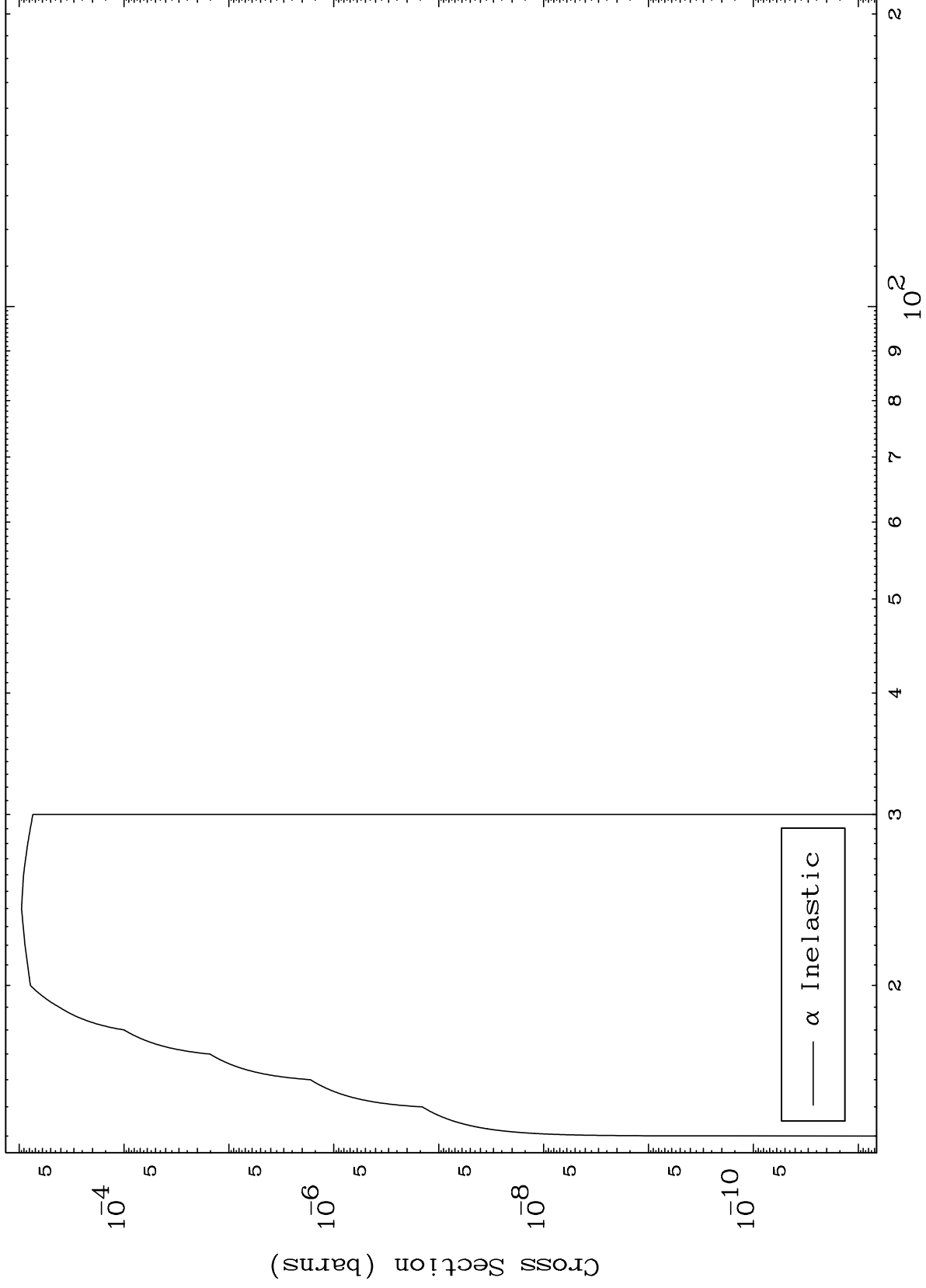
93-Np-233



MAT 93334

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

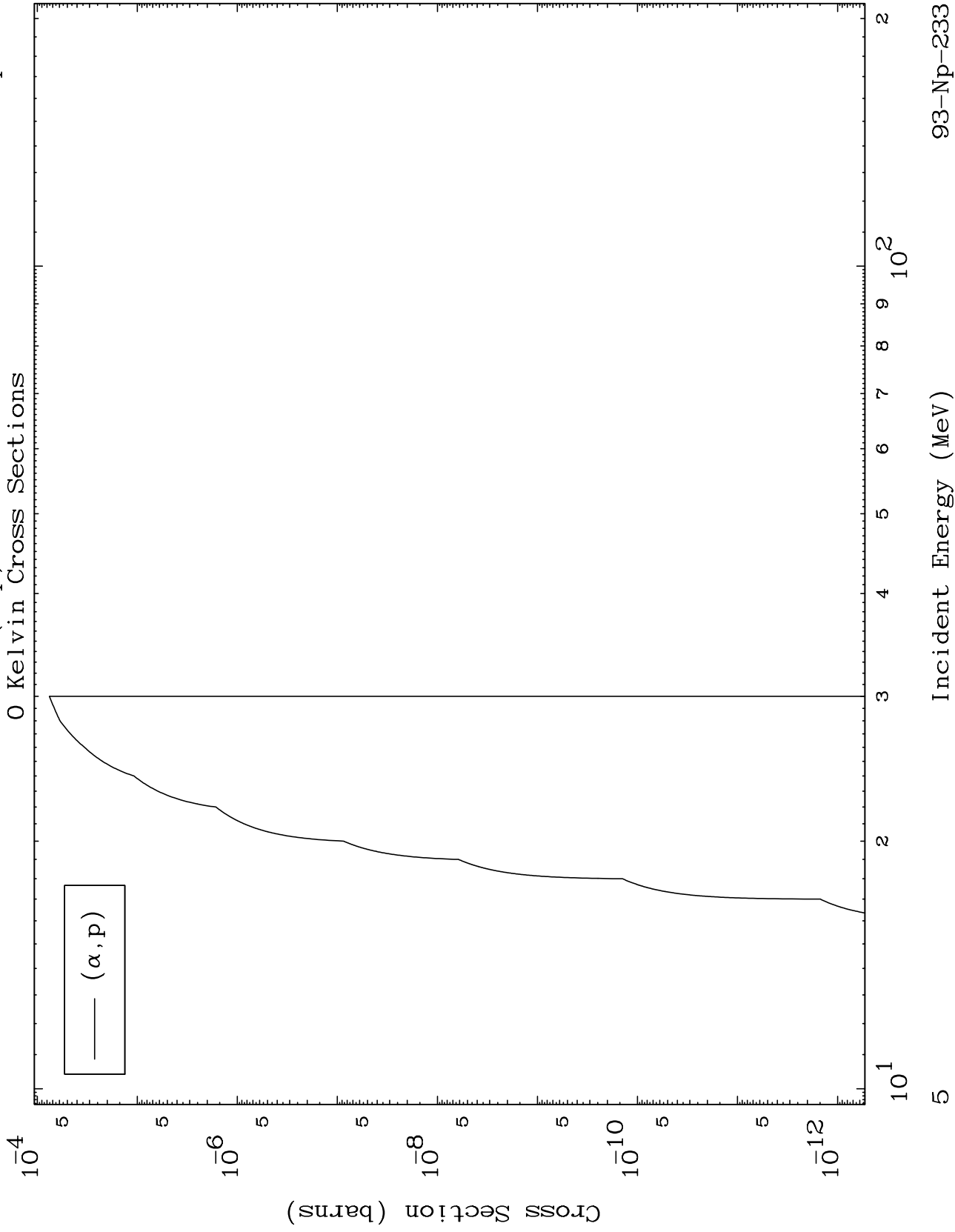
93-Np-233



MAT 93334

( $\alpha, p$ ) Levels

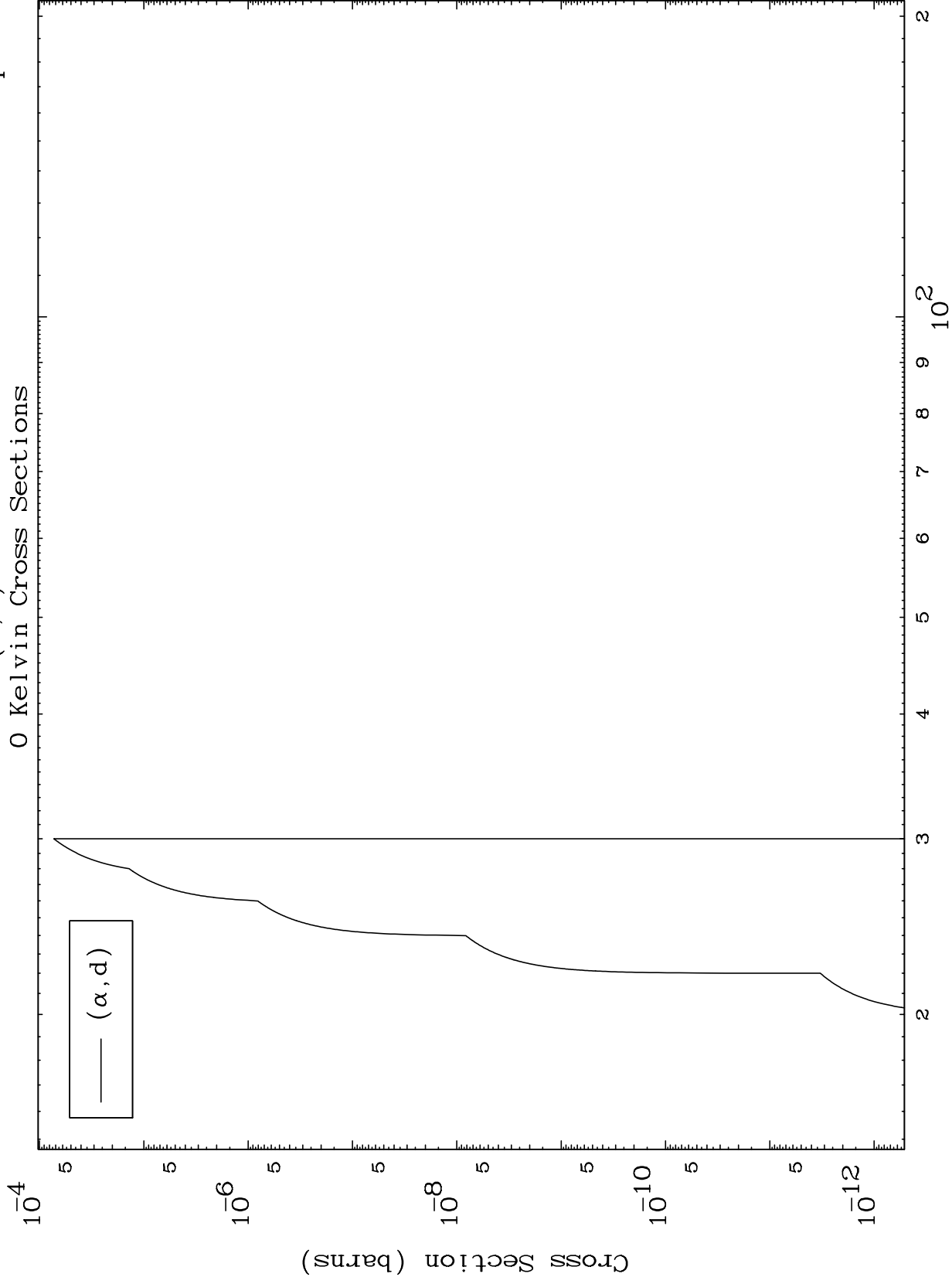
93-Np-233



MAT 9334

93-Np-233

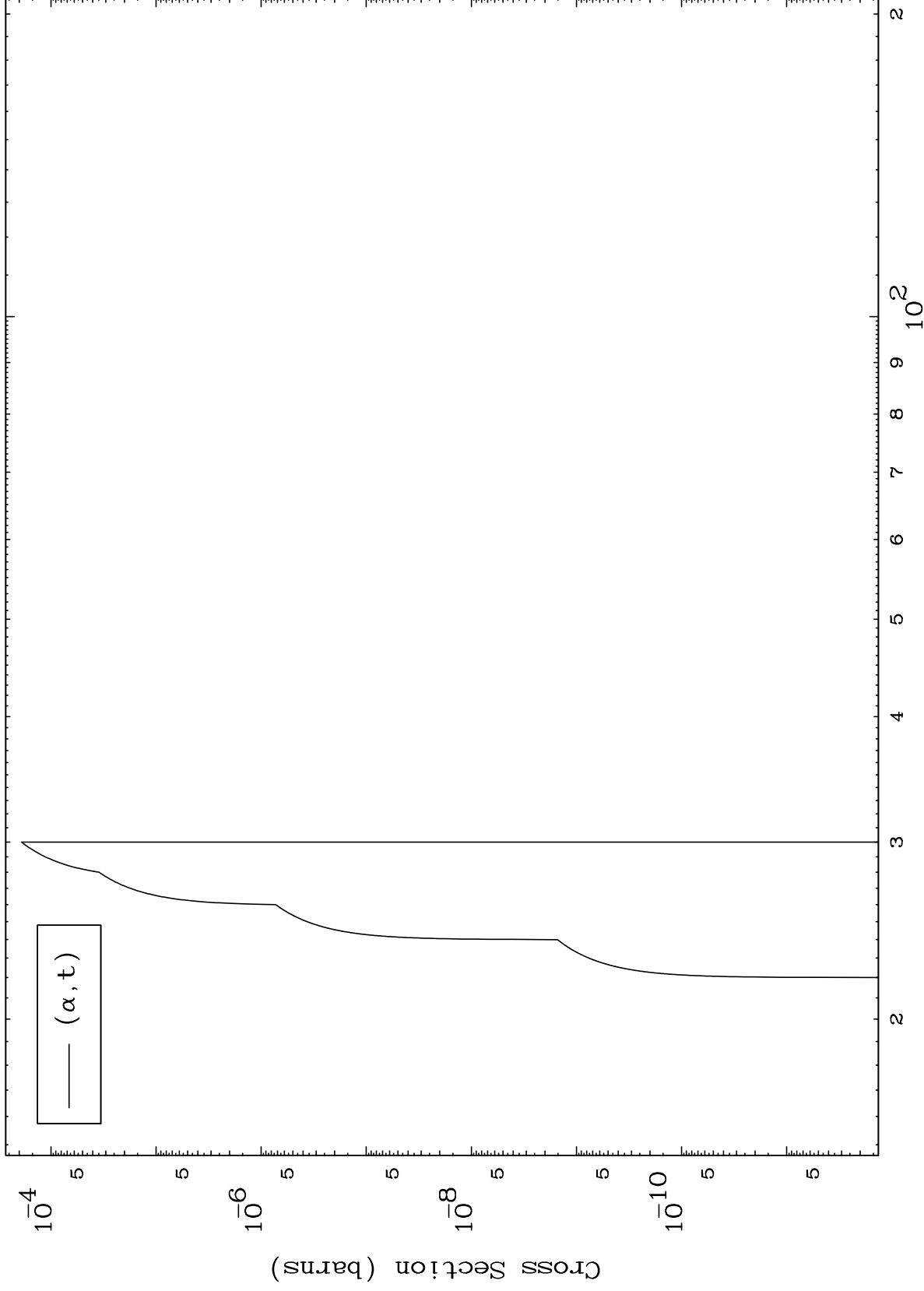
( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections



93-Np-233

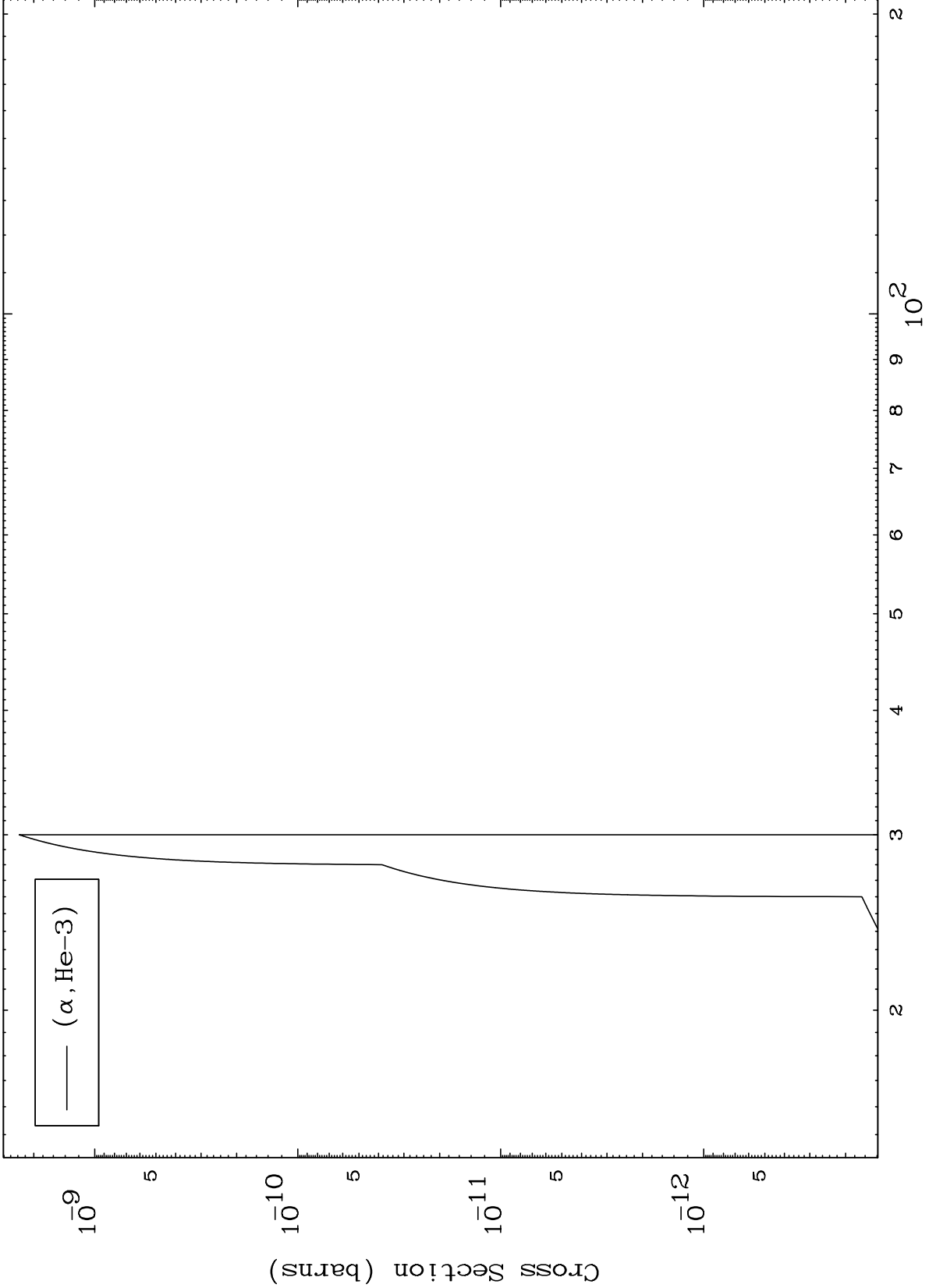
Incident Energy (MeV)

6





( $\alpha, \text{He}3$ ) Levels  
0 Kelvin Cross Sections

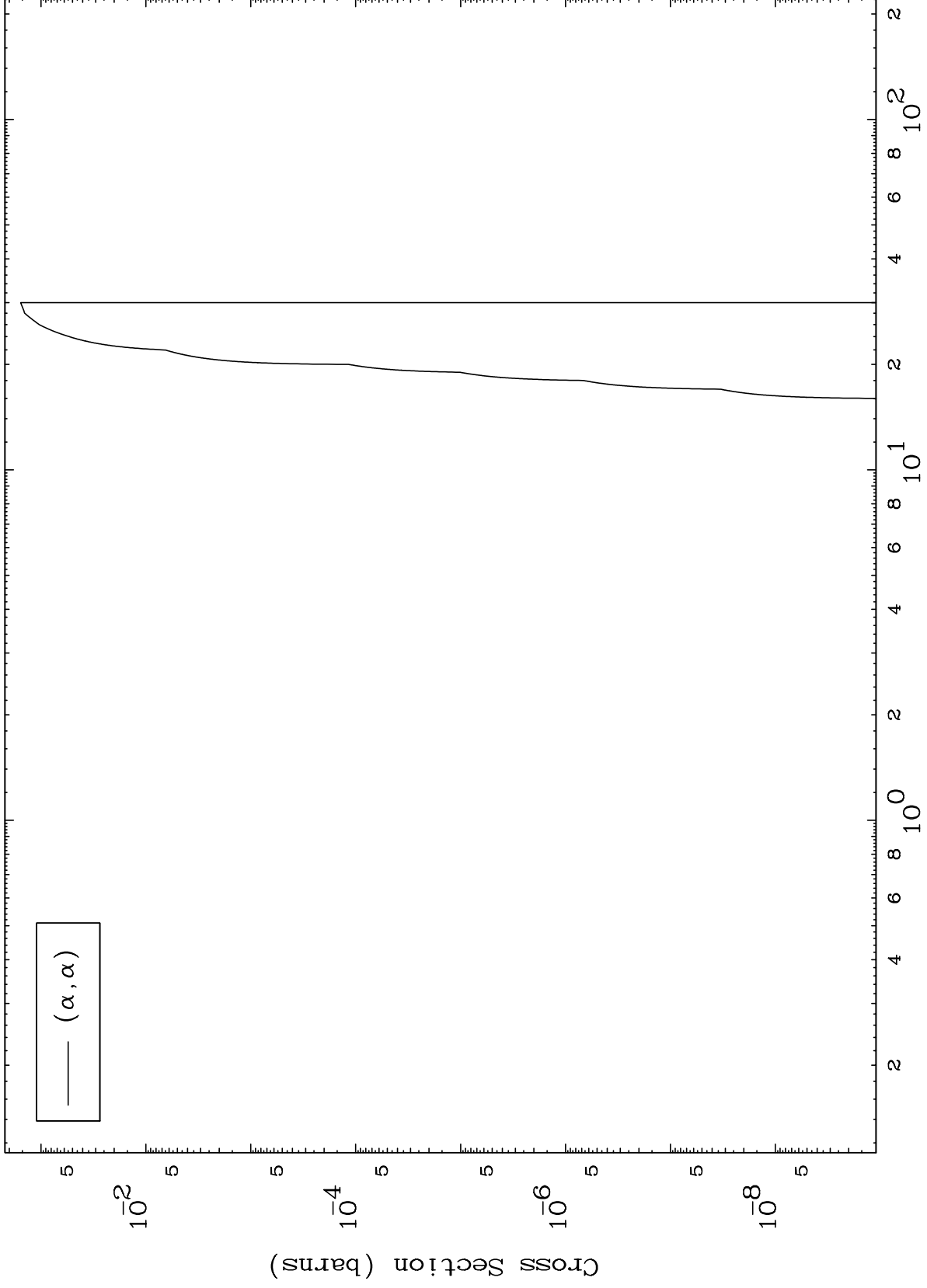


MAT 9334

( $\alpha, \alpha$ ) Levels

93-Np-233

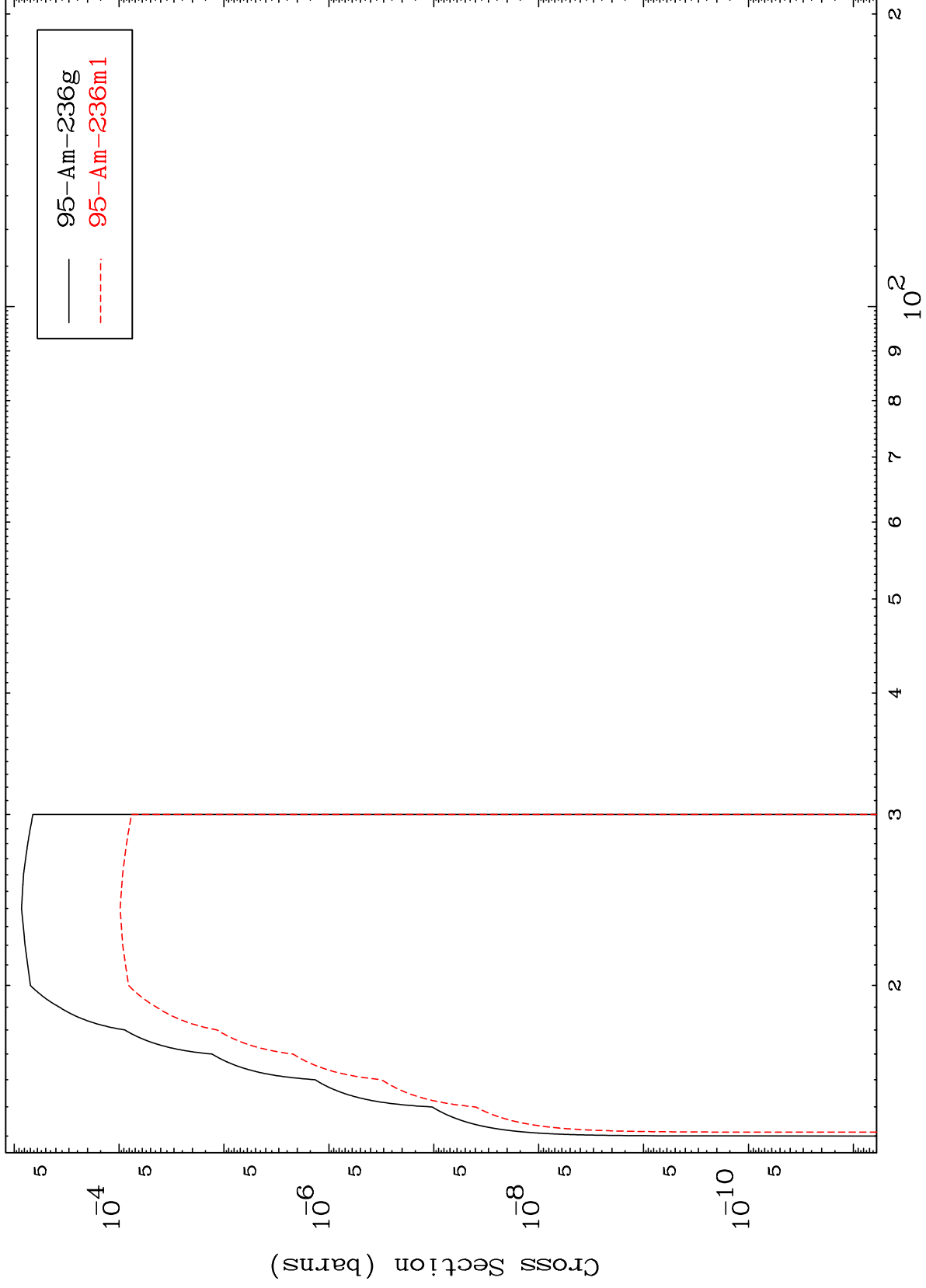
0 Kelvin Cross Sections



MAT 9334

93-Np-233

$\alpha$  Inelastic  
Radionuclide Production Cross Section



10

Incident Energy (MeV)

93-Np-233

MAT 9334

93-Np-233

$\alpha$  Fission  
Radionuclide Production Cross Section

