

Program EVALPLOT  
(Version 2018-1)

by

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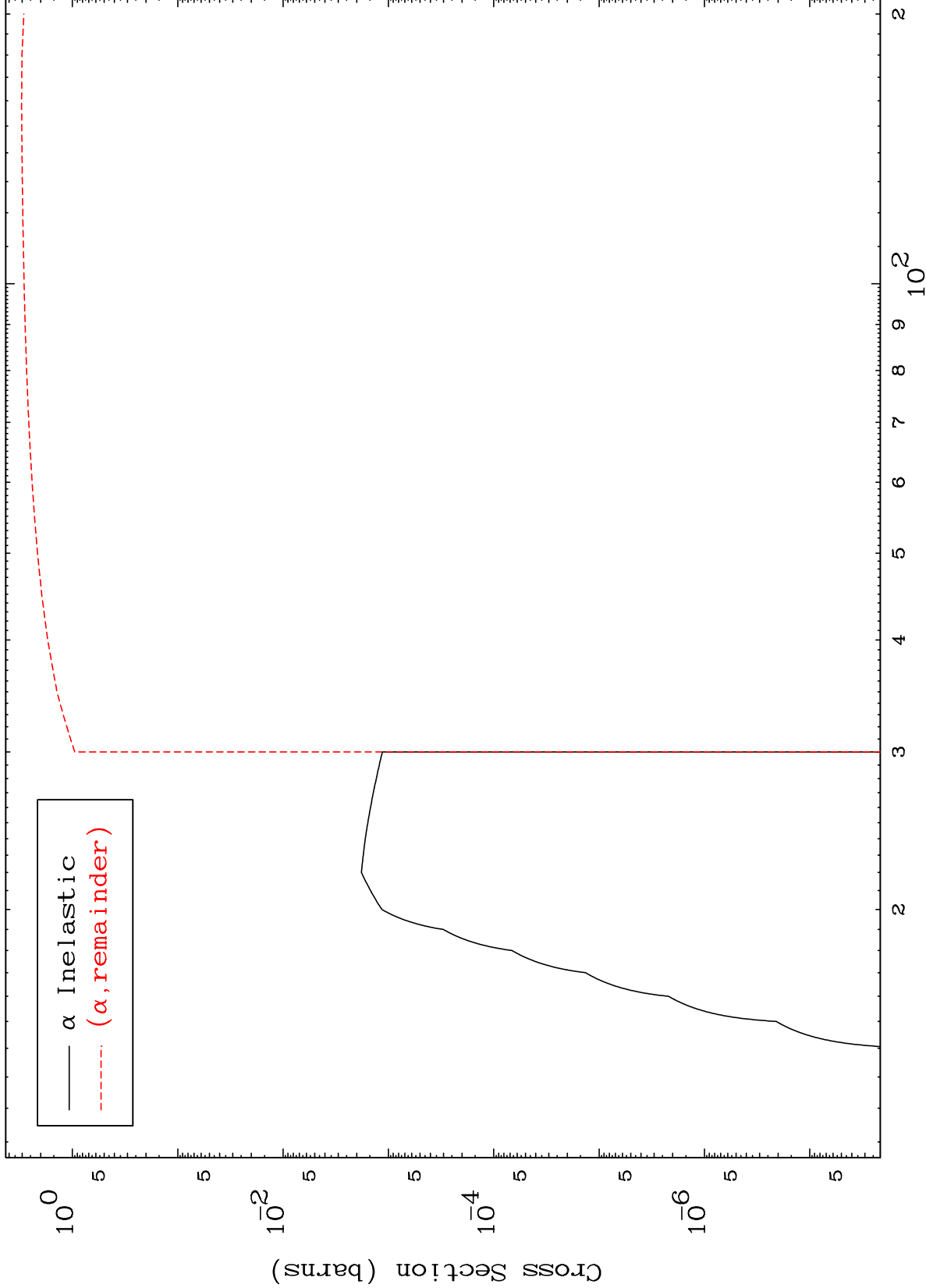
Press Mouse Button to Start

MAT 9364

$\alpha$  Major

93-Np-243

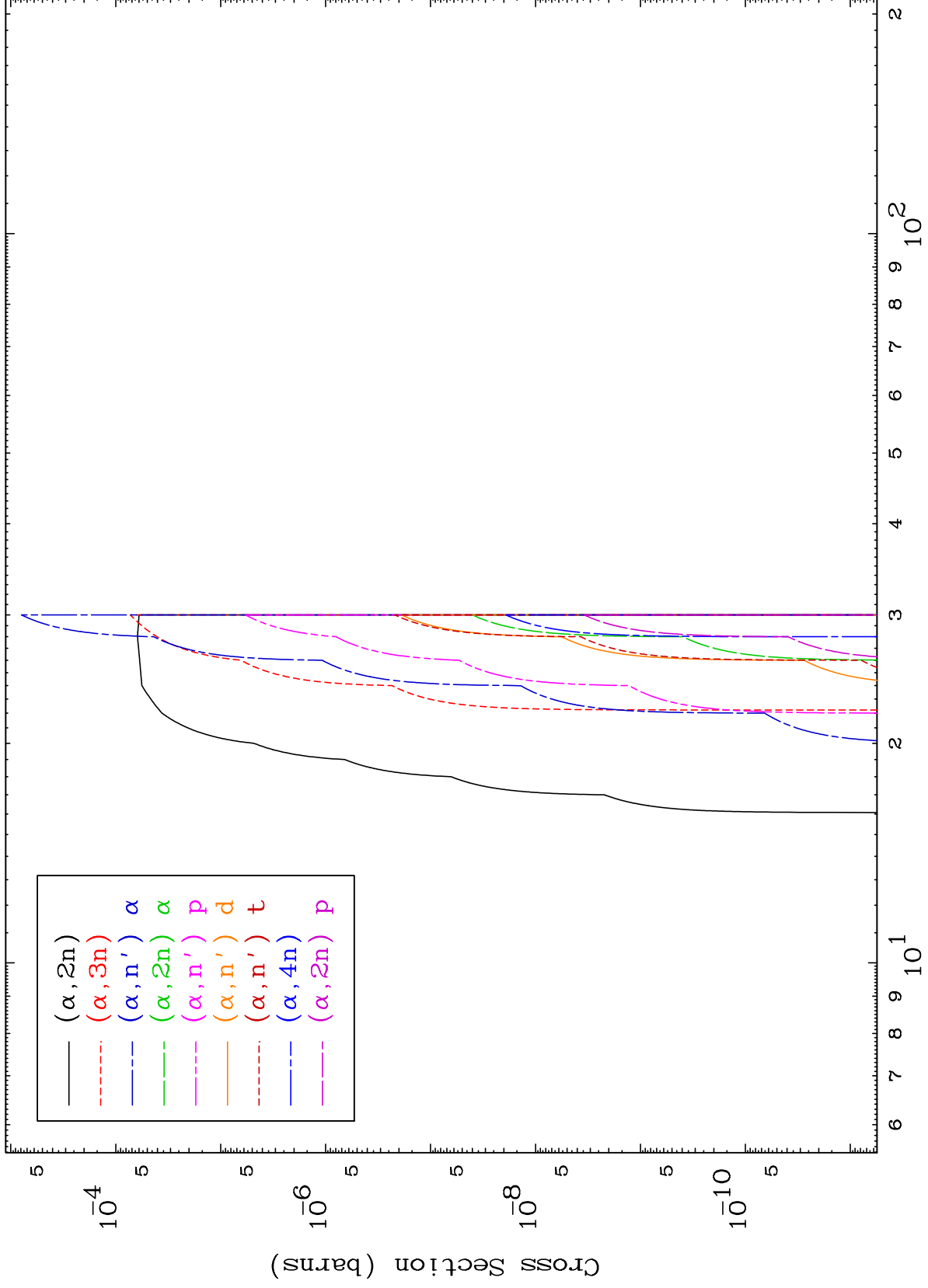
0 Kelvin Cross Sections

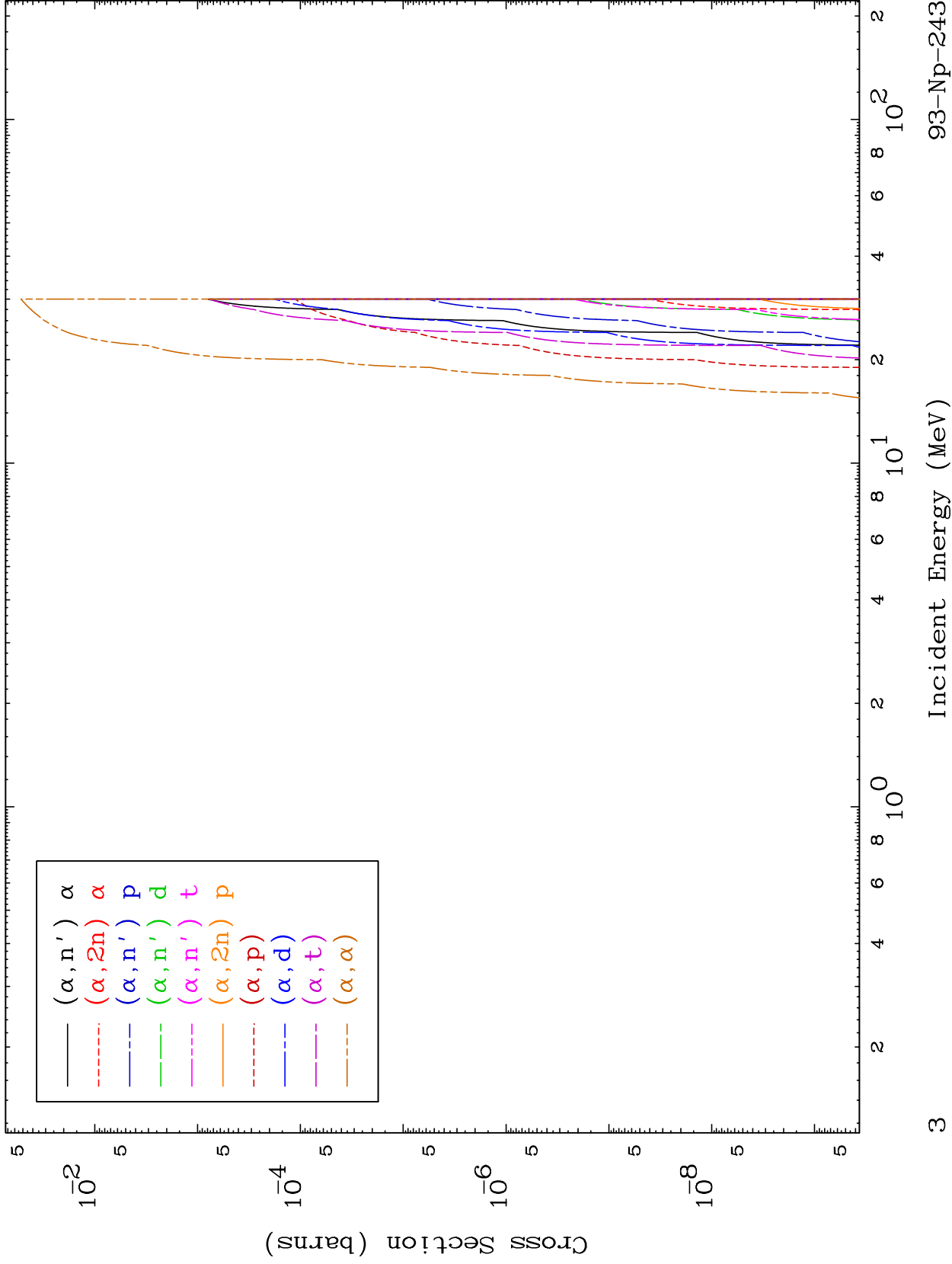


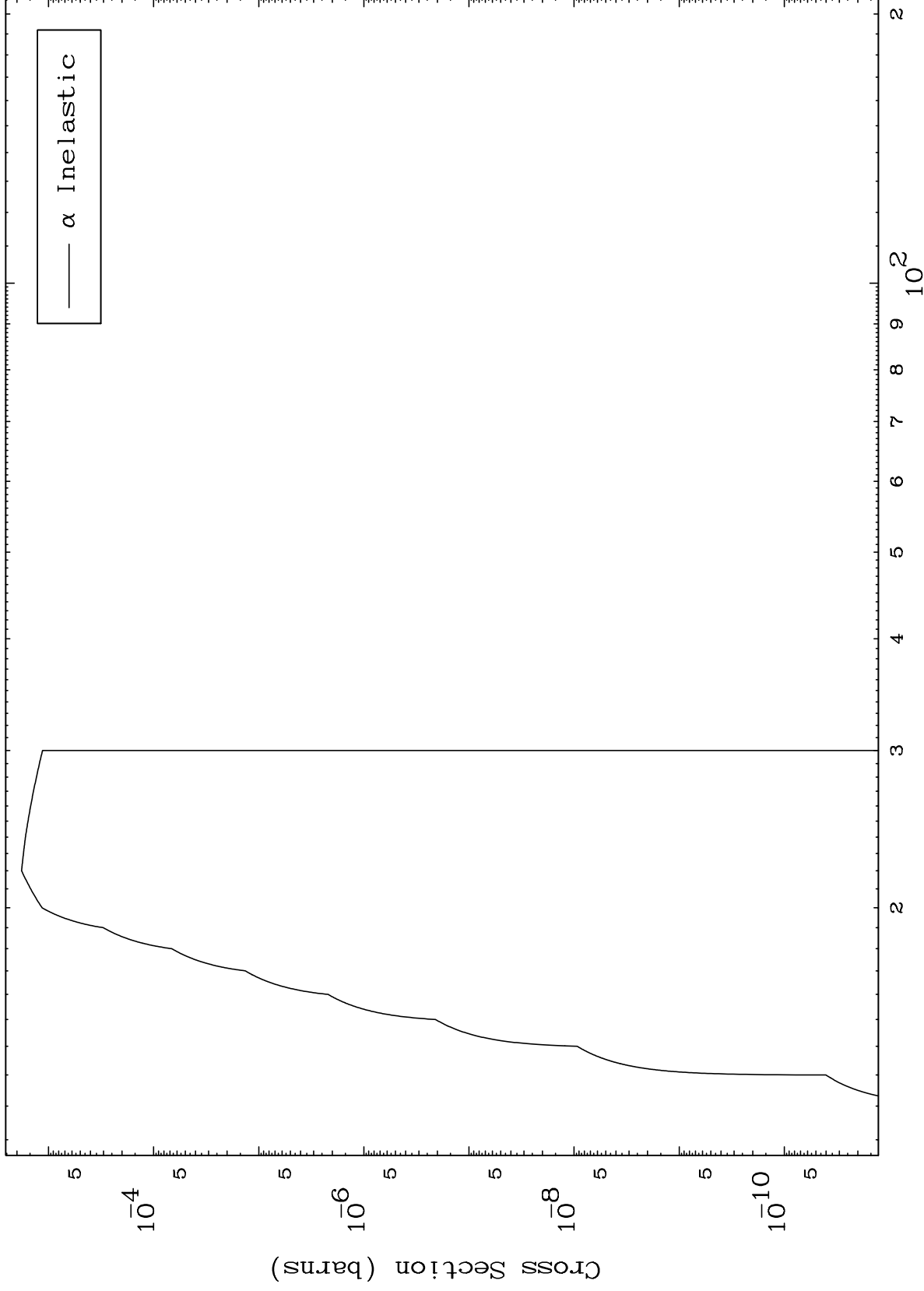
1

Incident Energy (MeV)

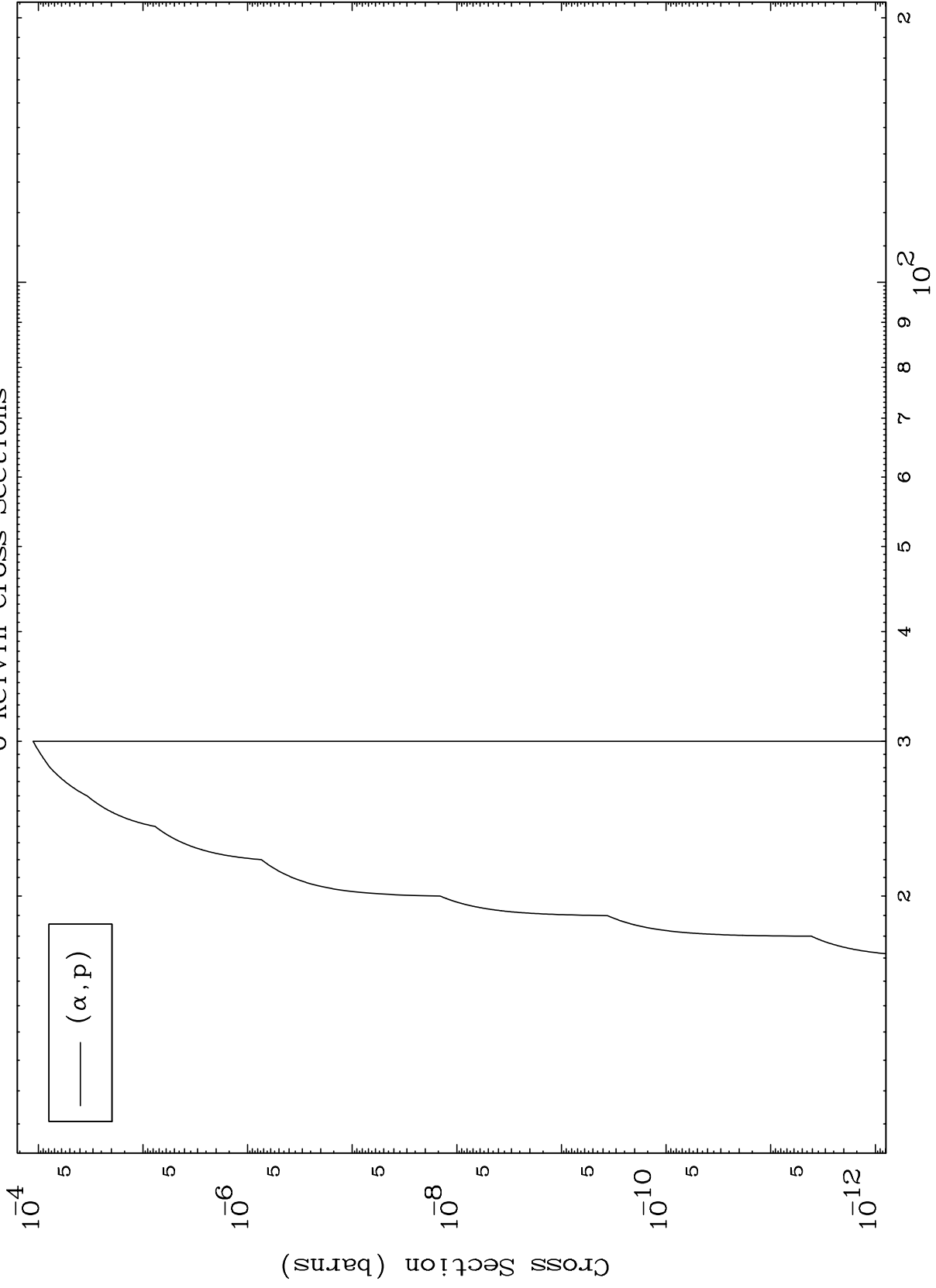
93-Np-243

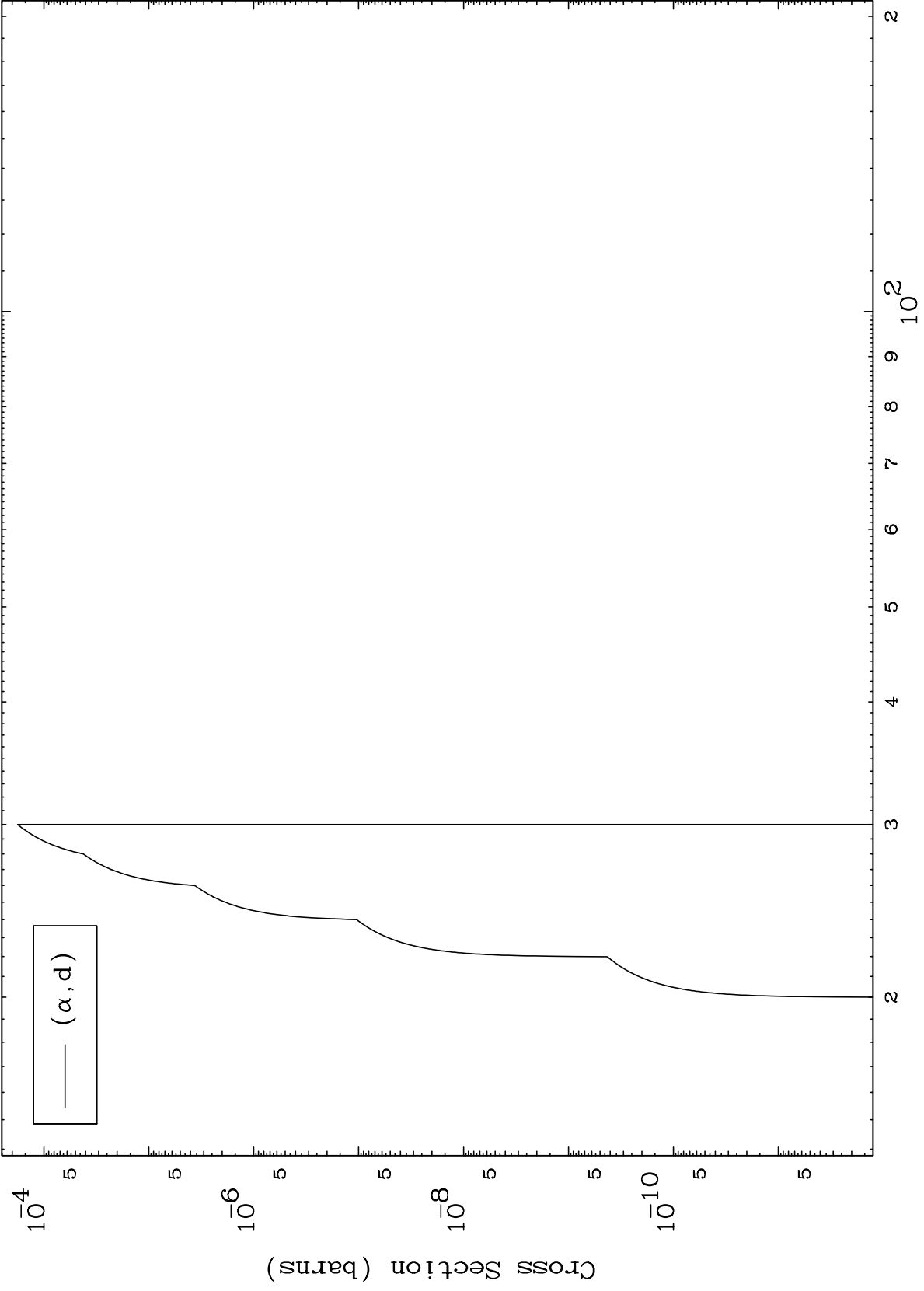


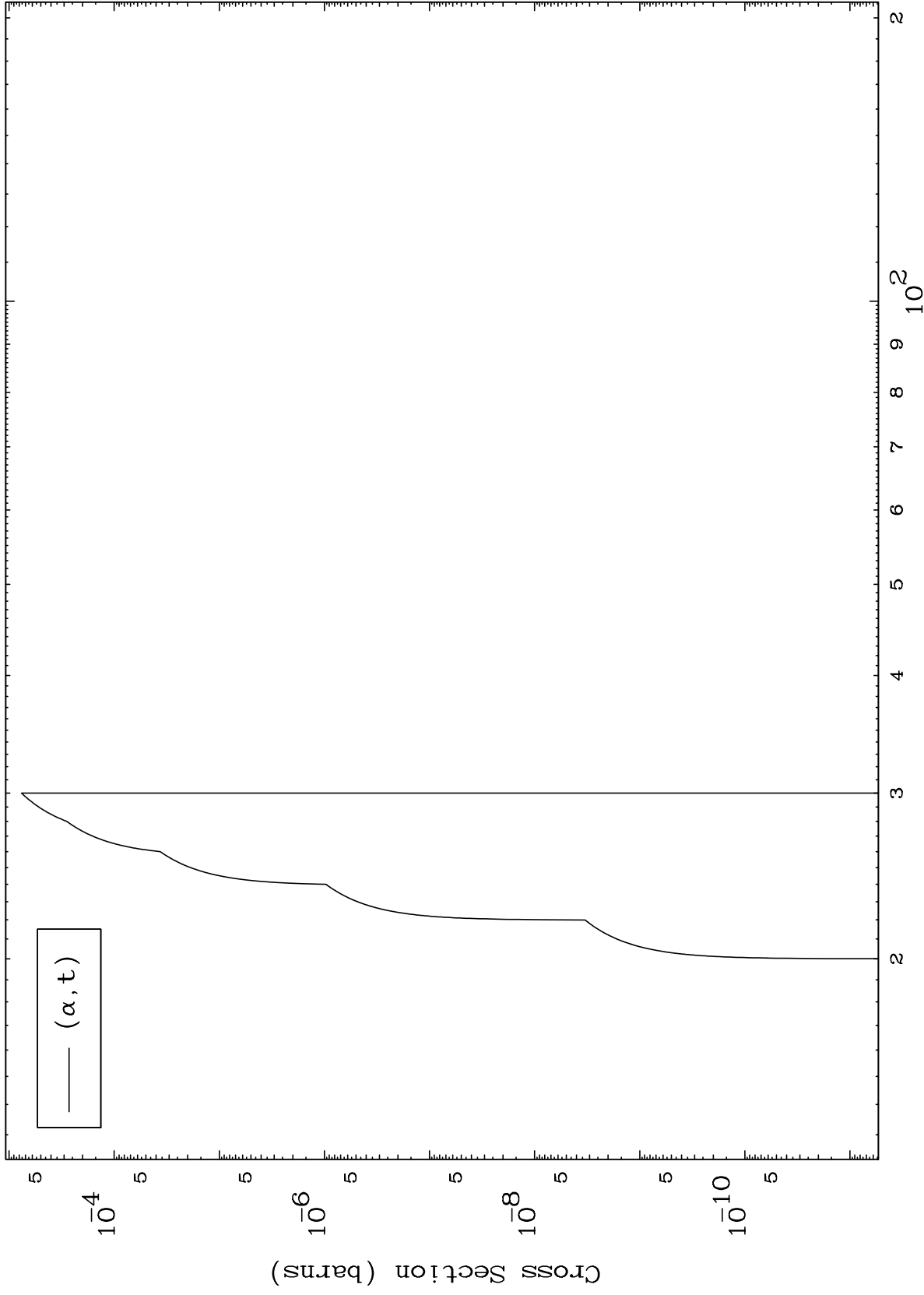




( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections







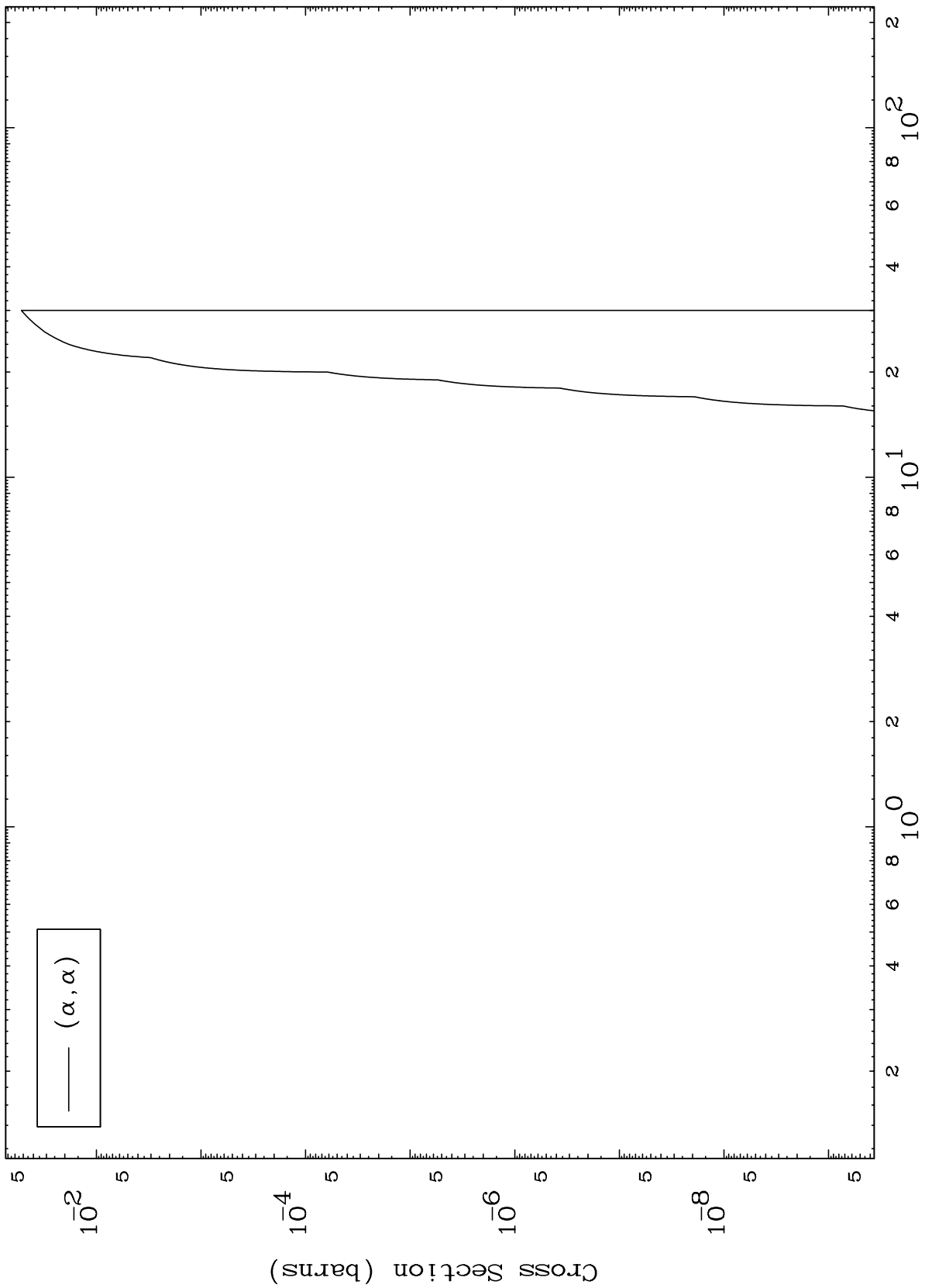


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( $\alpha, \alpha$ ) Levels

93-Np-243

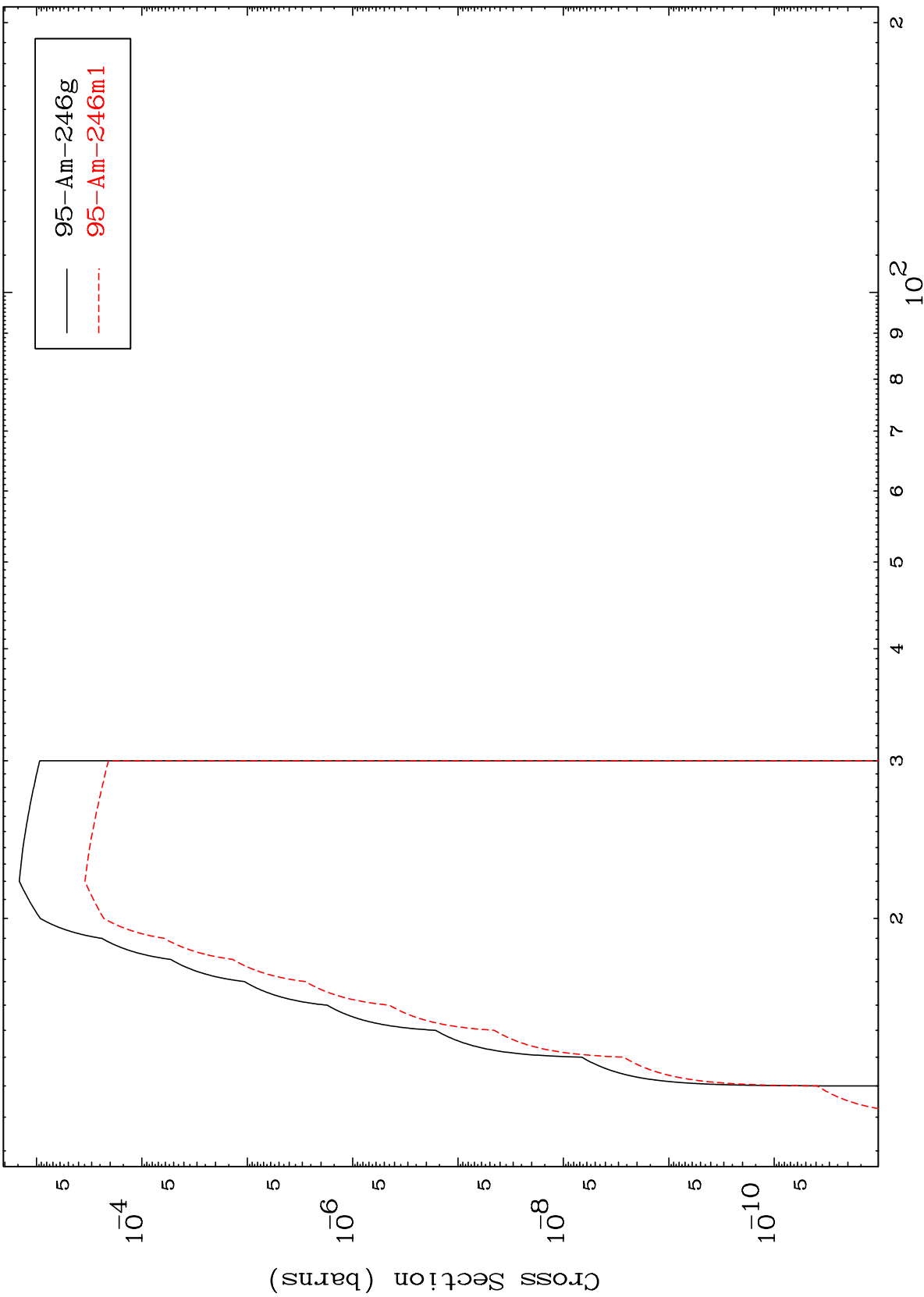
0 Kelvin Cross Sections



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Radionuclide Production Cross Section  
 $\alpha$  Inelastic



9

Incident Energy (MeV)

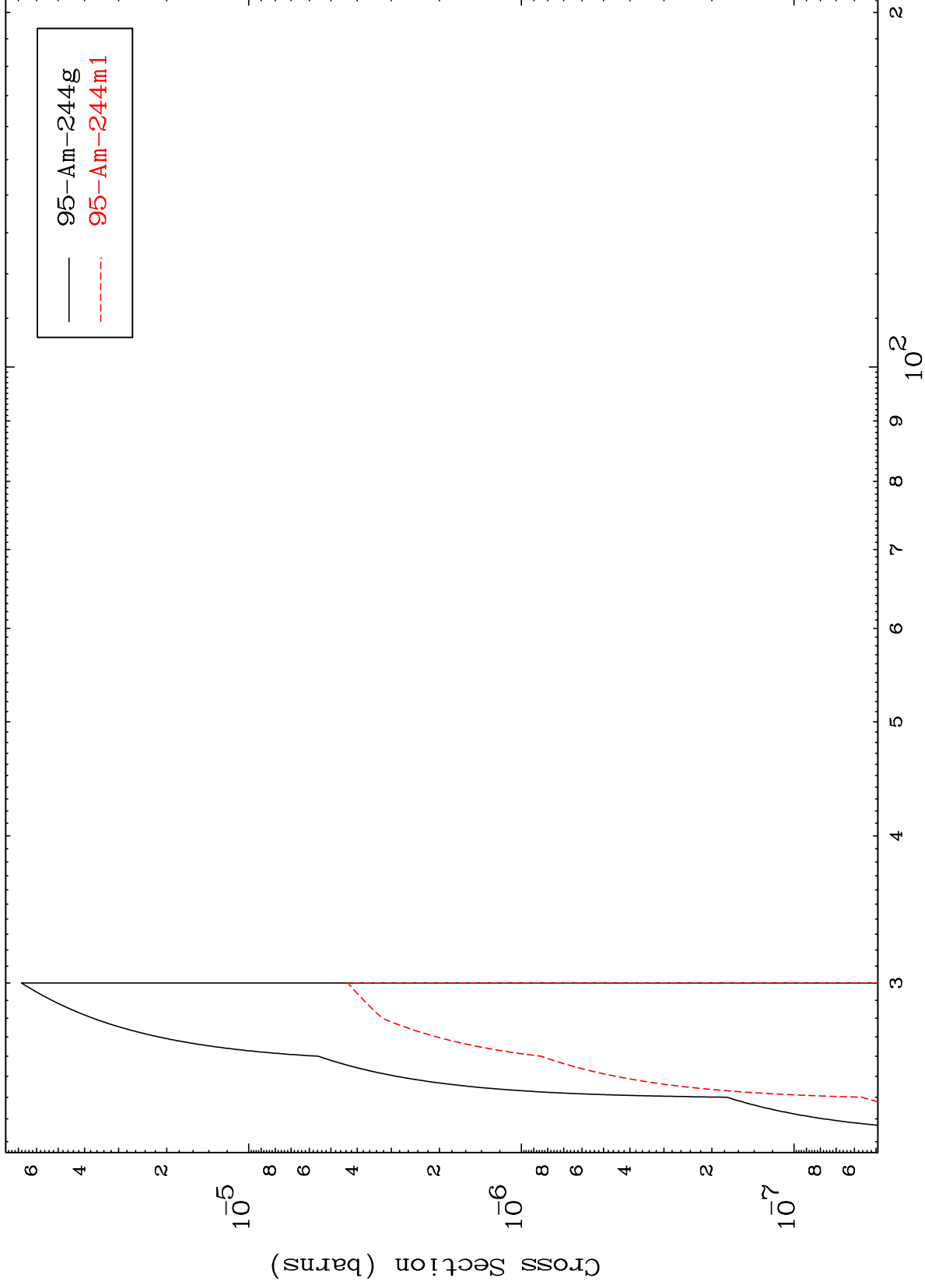
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( $\alpha, 3n$ )

93-Np-243

Radionuclide Production Cross Section



10

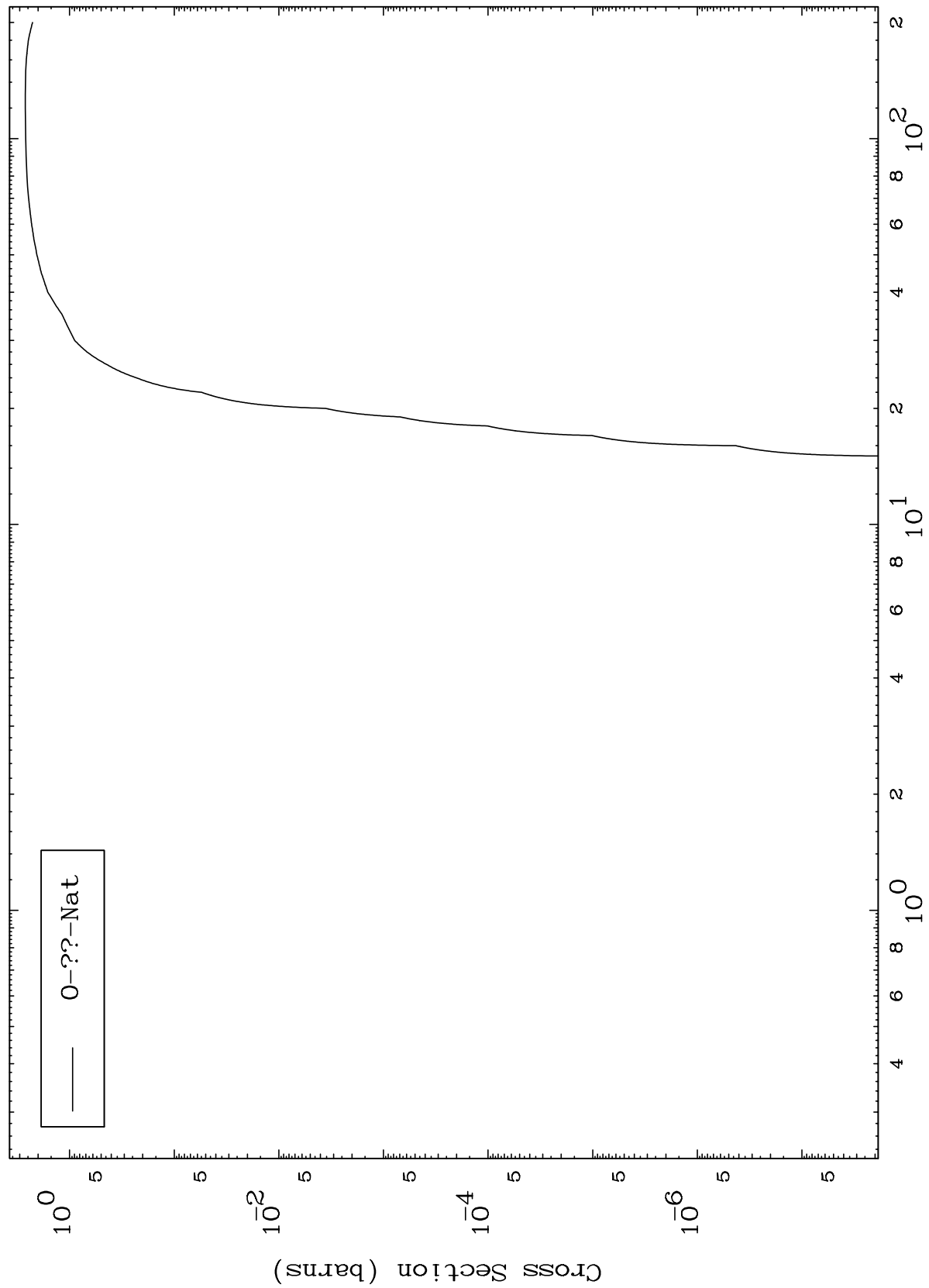
Incident Energy (MeV)

93-Np-243

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93-Np-243

$\alpha$  Fission  
Radionuclide Production Cross Section

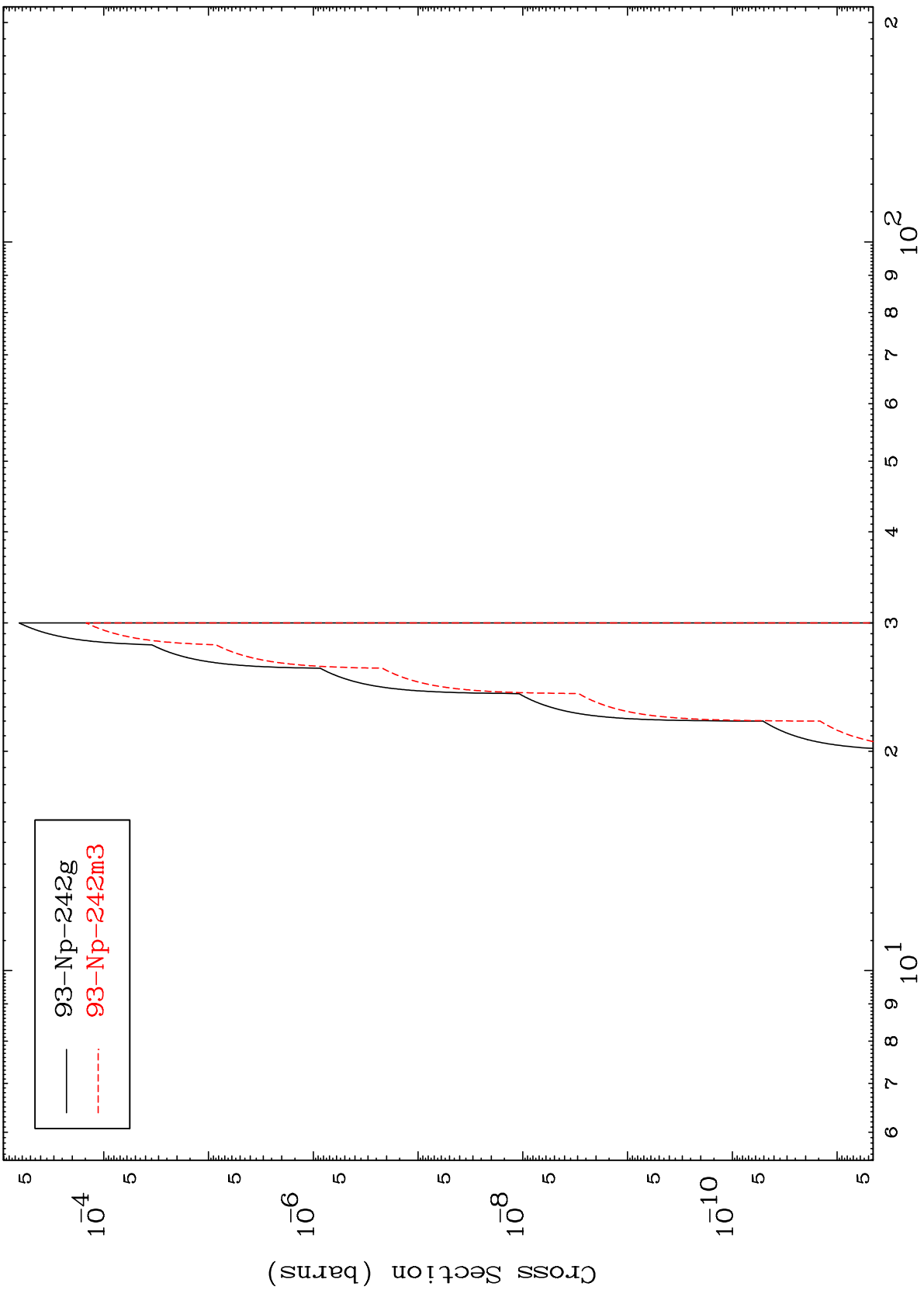


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93-Np-243

$(\alpha, n')$   $\alpha$

Radionuclide Production Cross Section



12

Incident Energy (MeV)

93-Np-243