

Program EVALPLOT  
(Version 2018-1)

by

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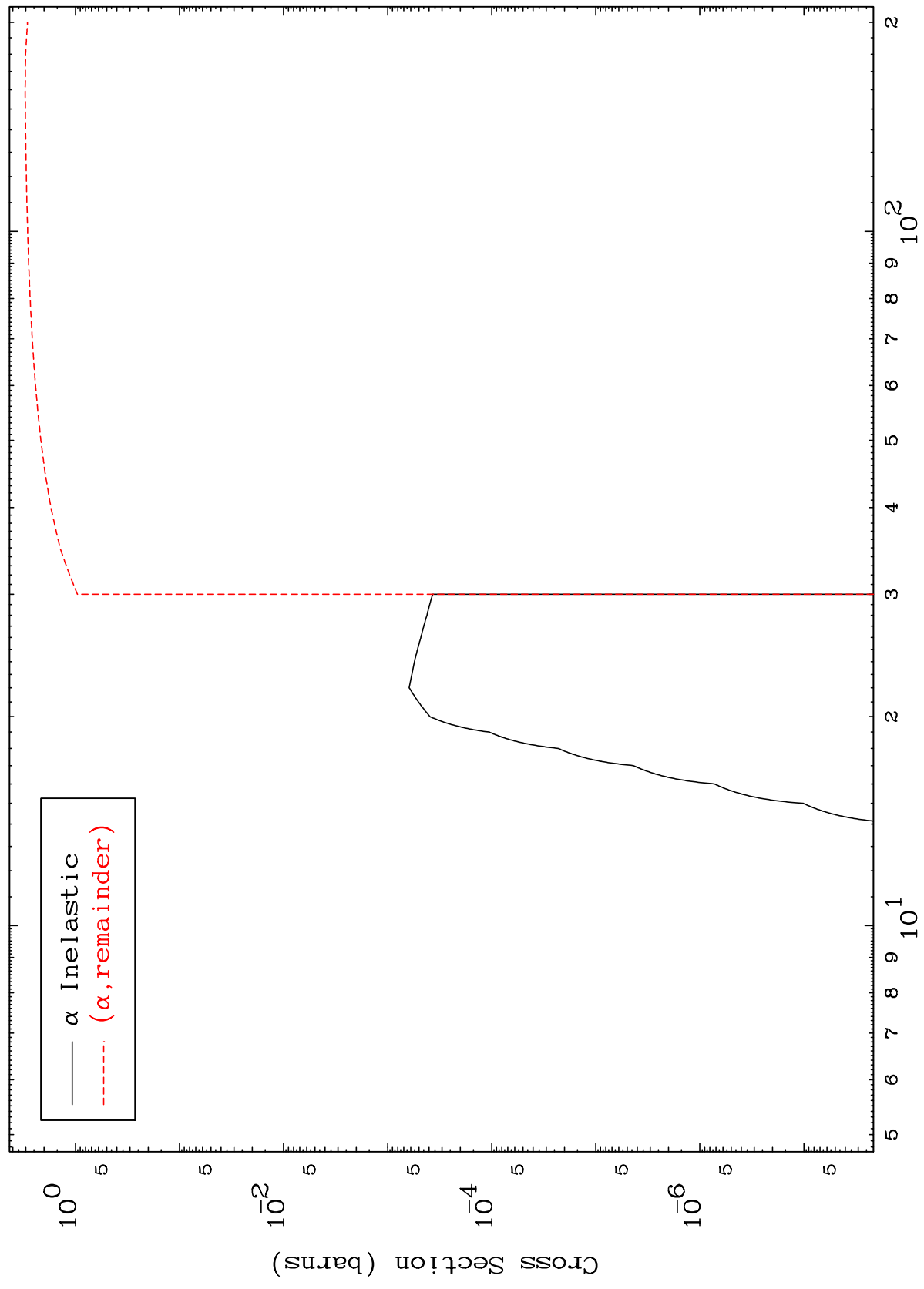
Press Mouse Button to Start

MAT 9367

0 Kelvin

$\alpha$  Major Cross Sections

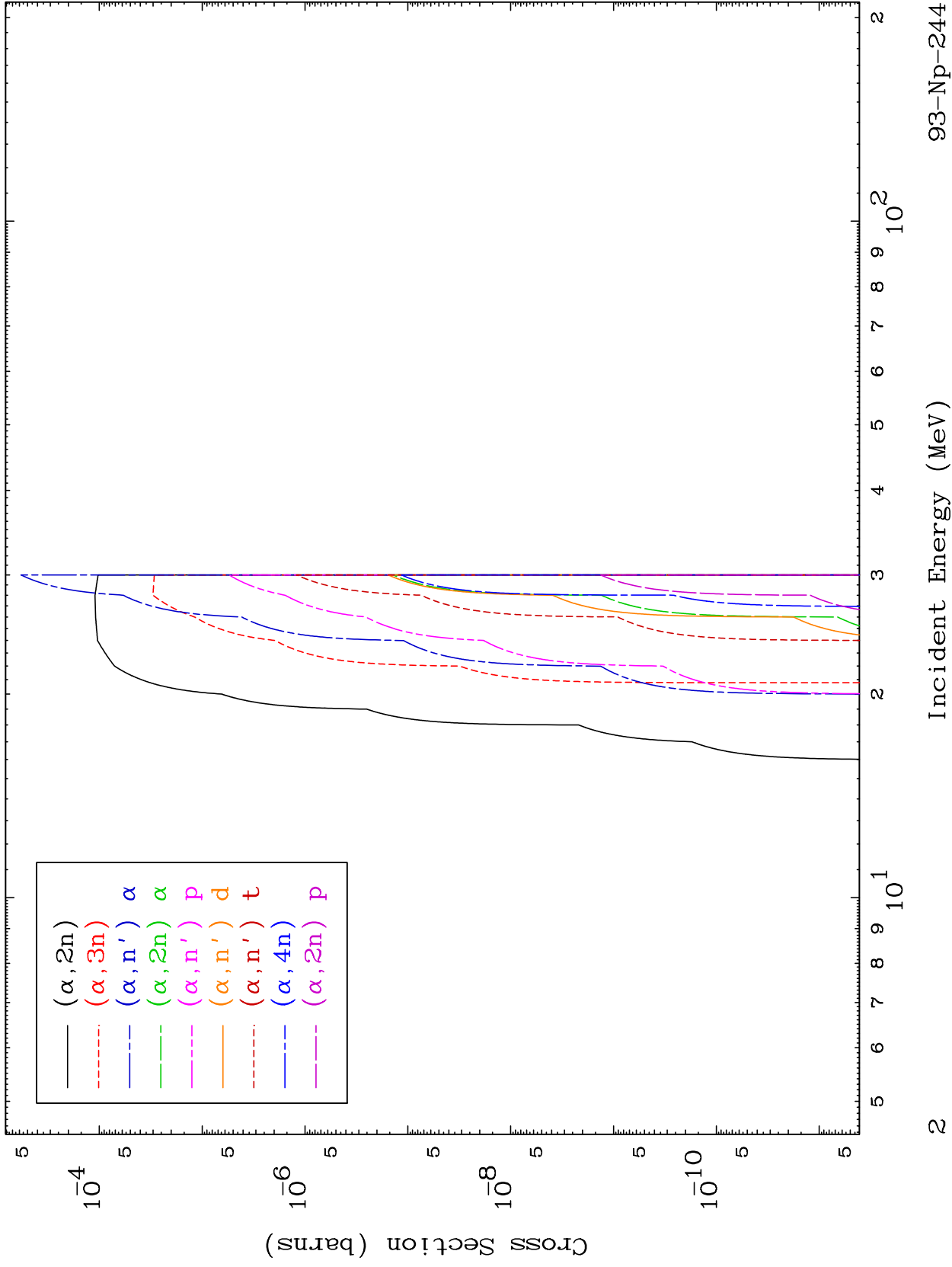
93-Np-244

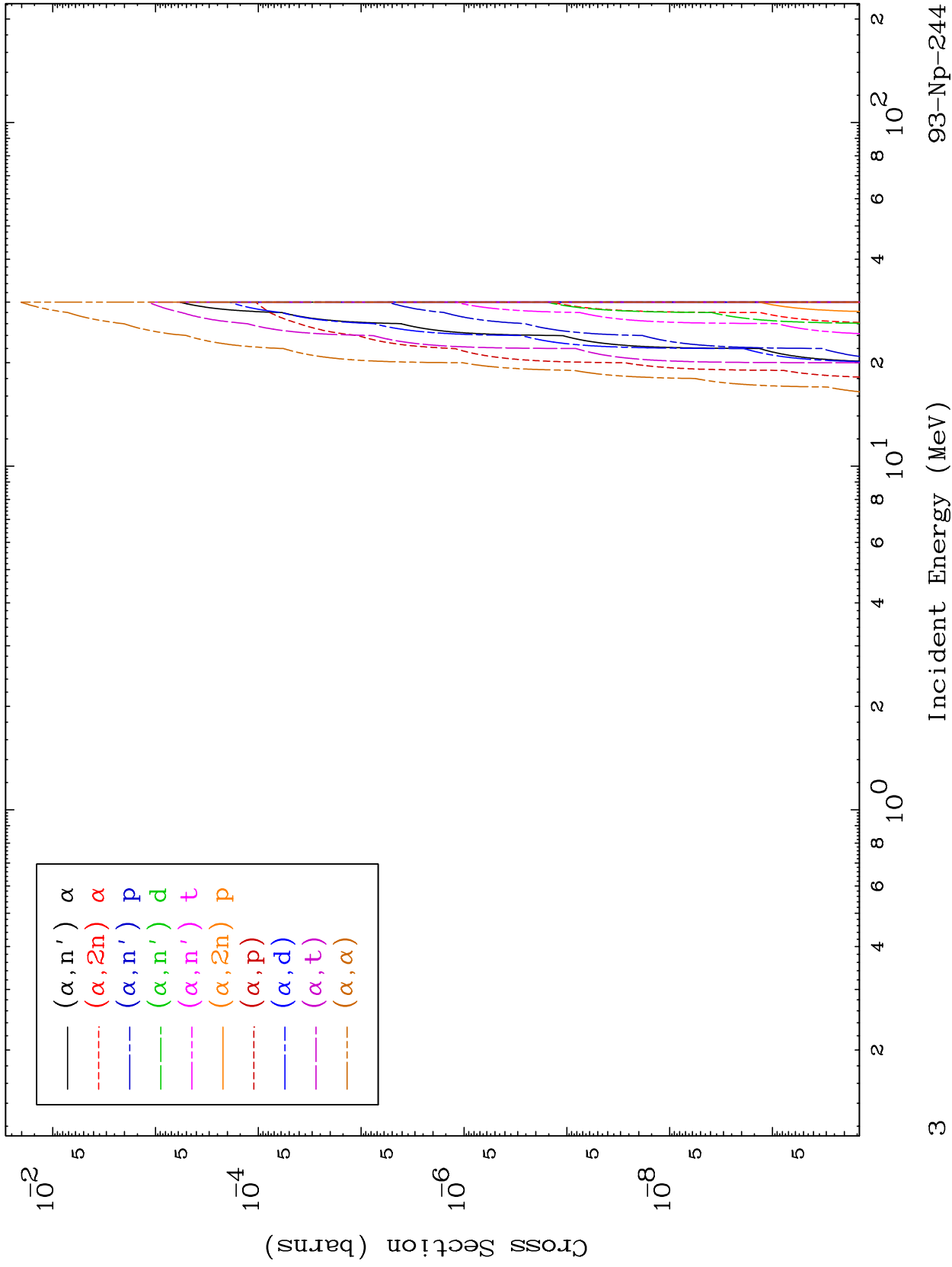


1

Incident Energy (MeV)

93-Np-244



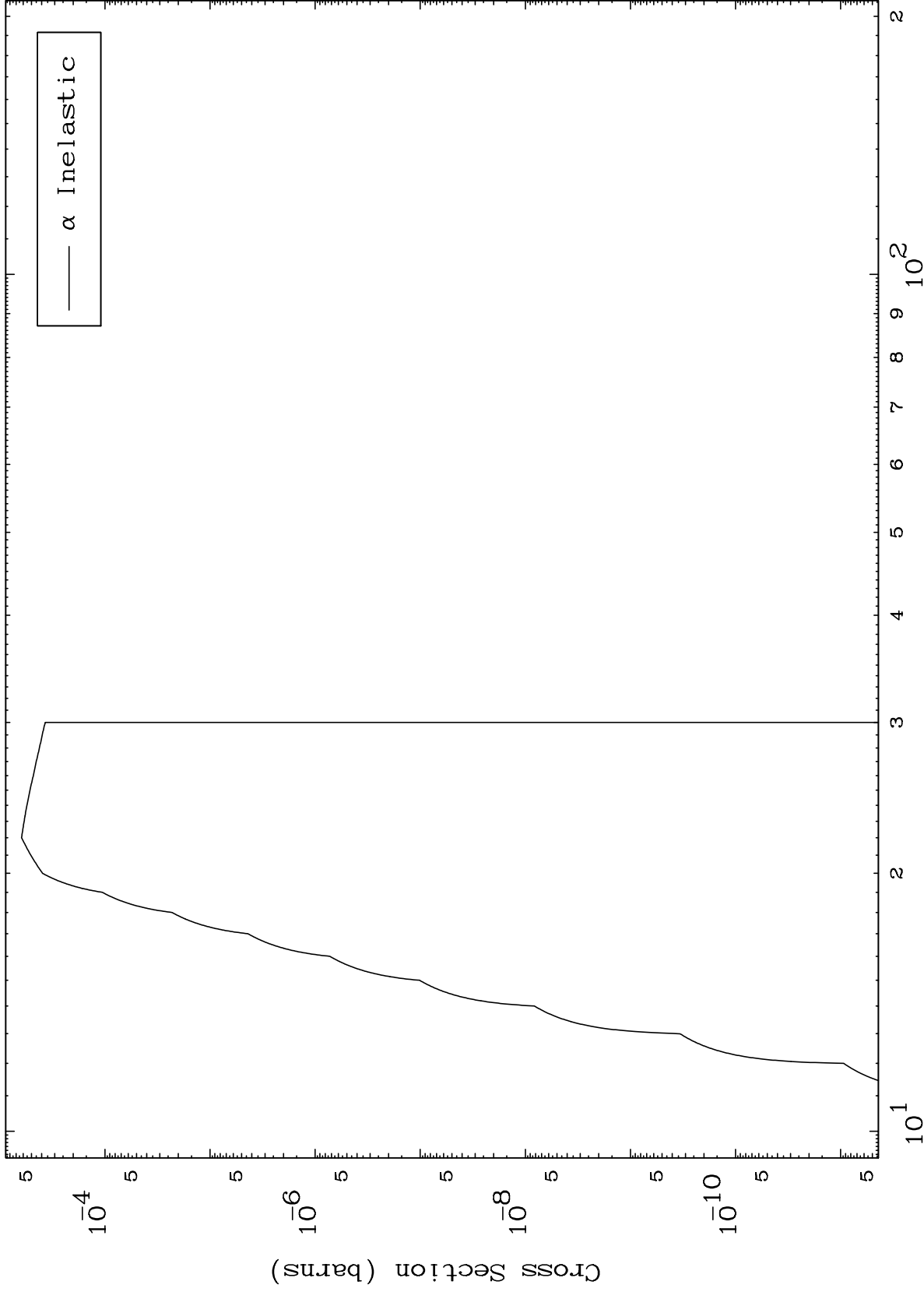


MAT 9367

( $\alpha, n'$ ) Level

93-Np-244

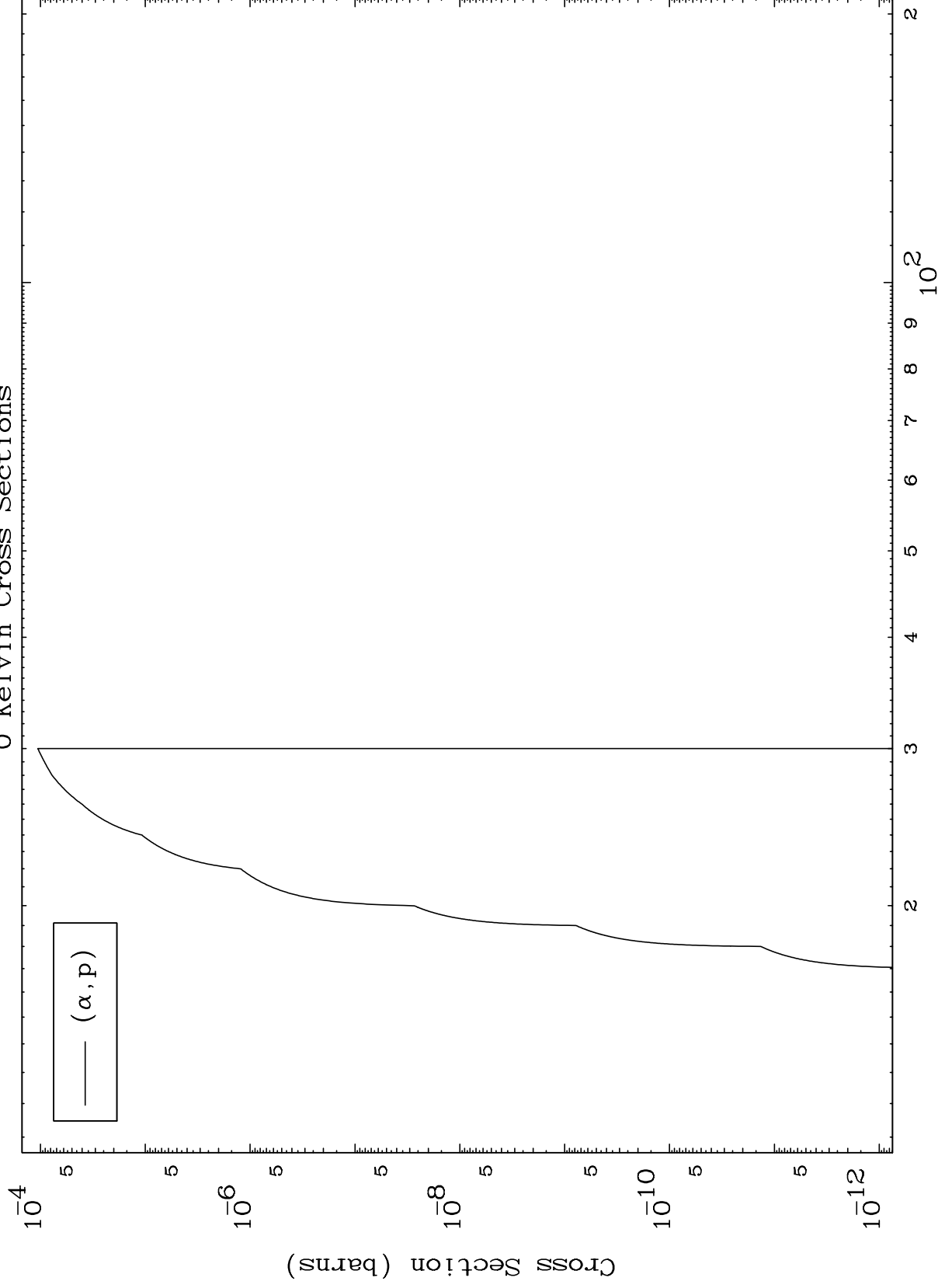
0 Kelvin Cross Sections

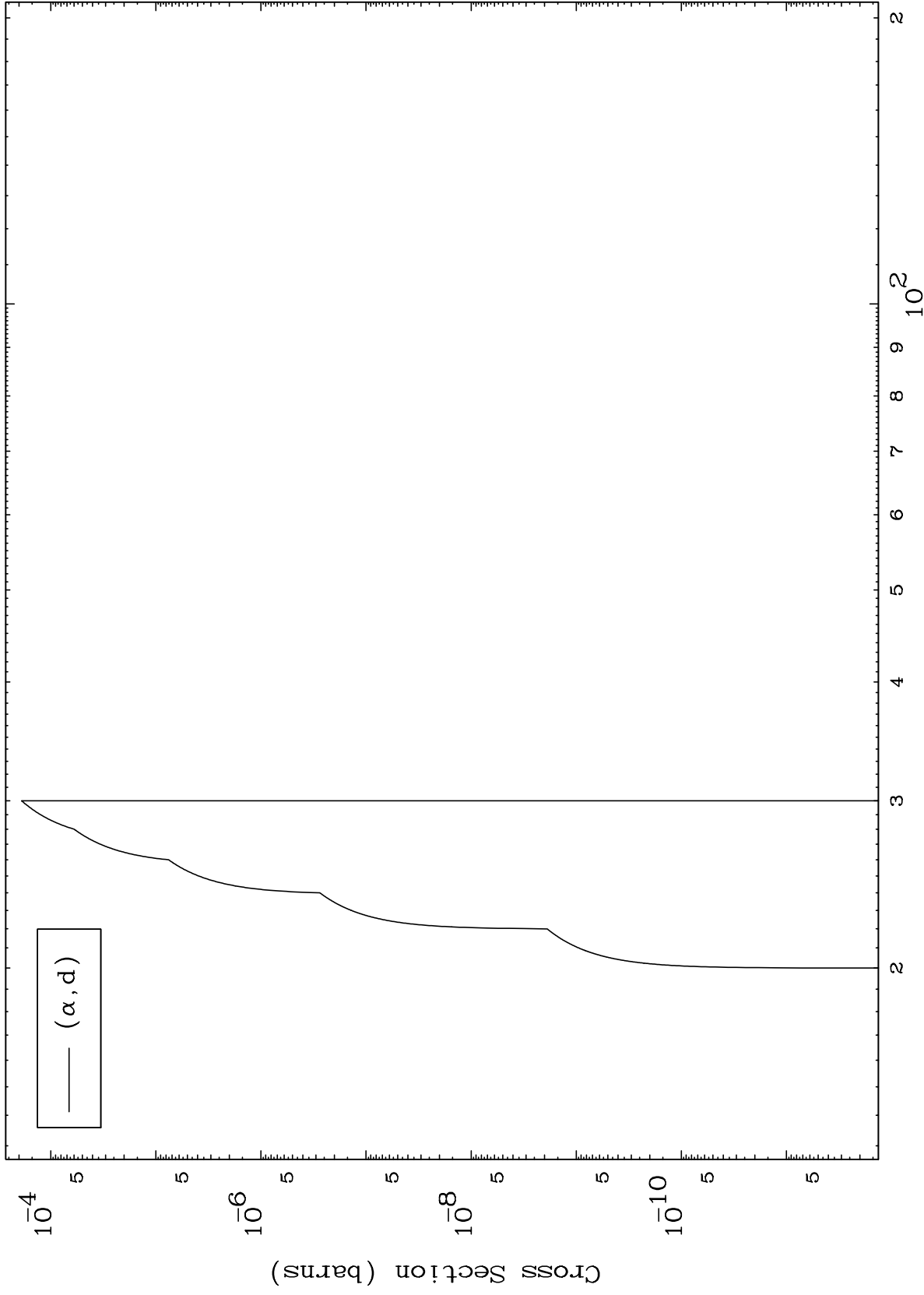


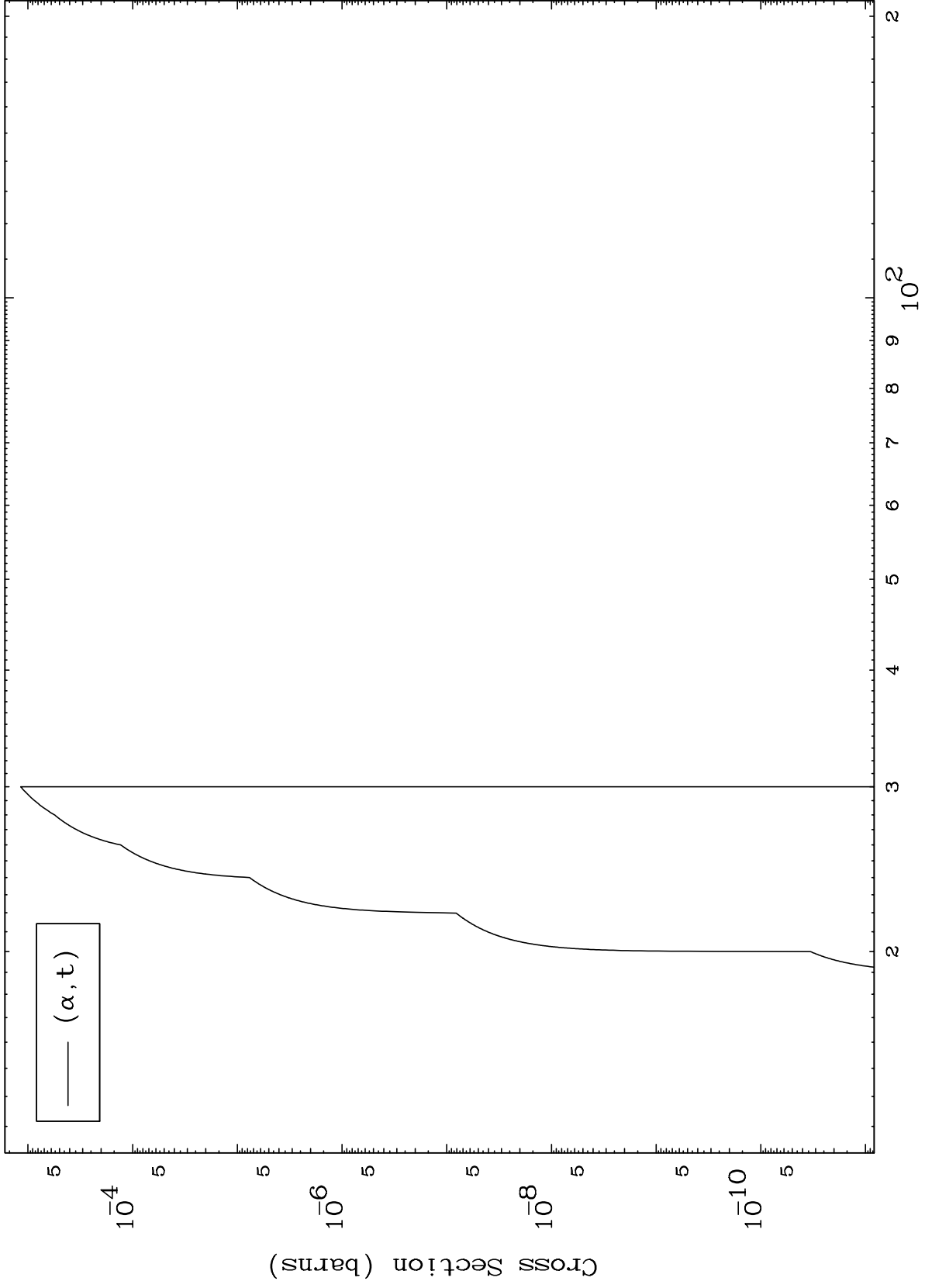
$\alpha$  Inelastic

Incident Energy (MeV)

93-Np-244





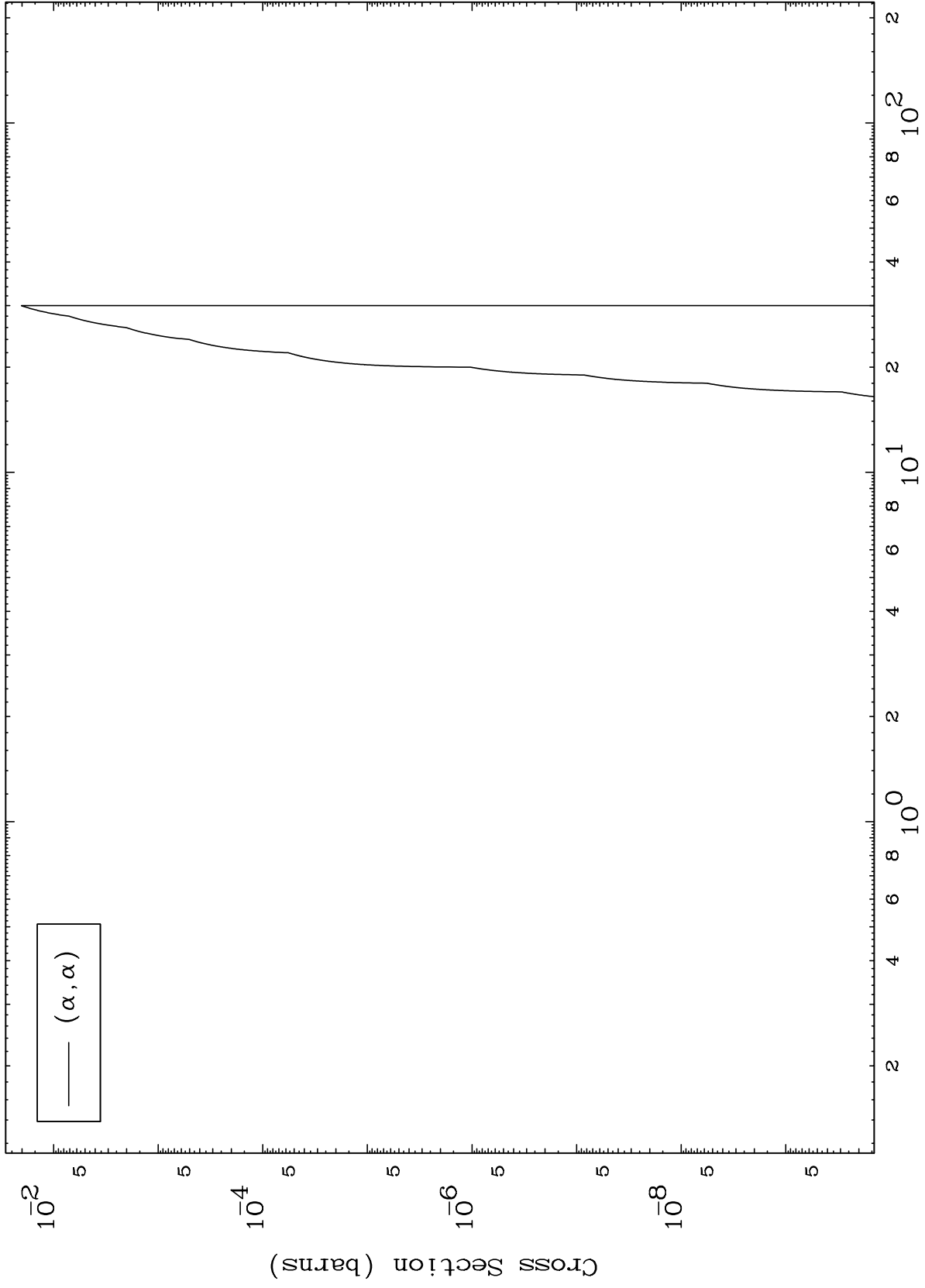




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( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

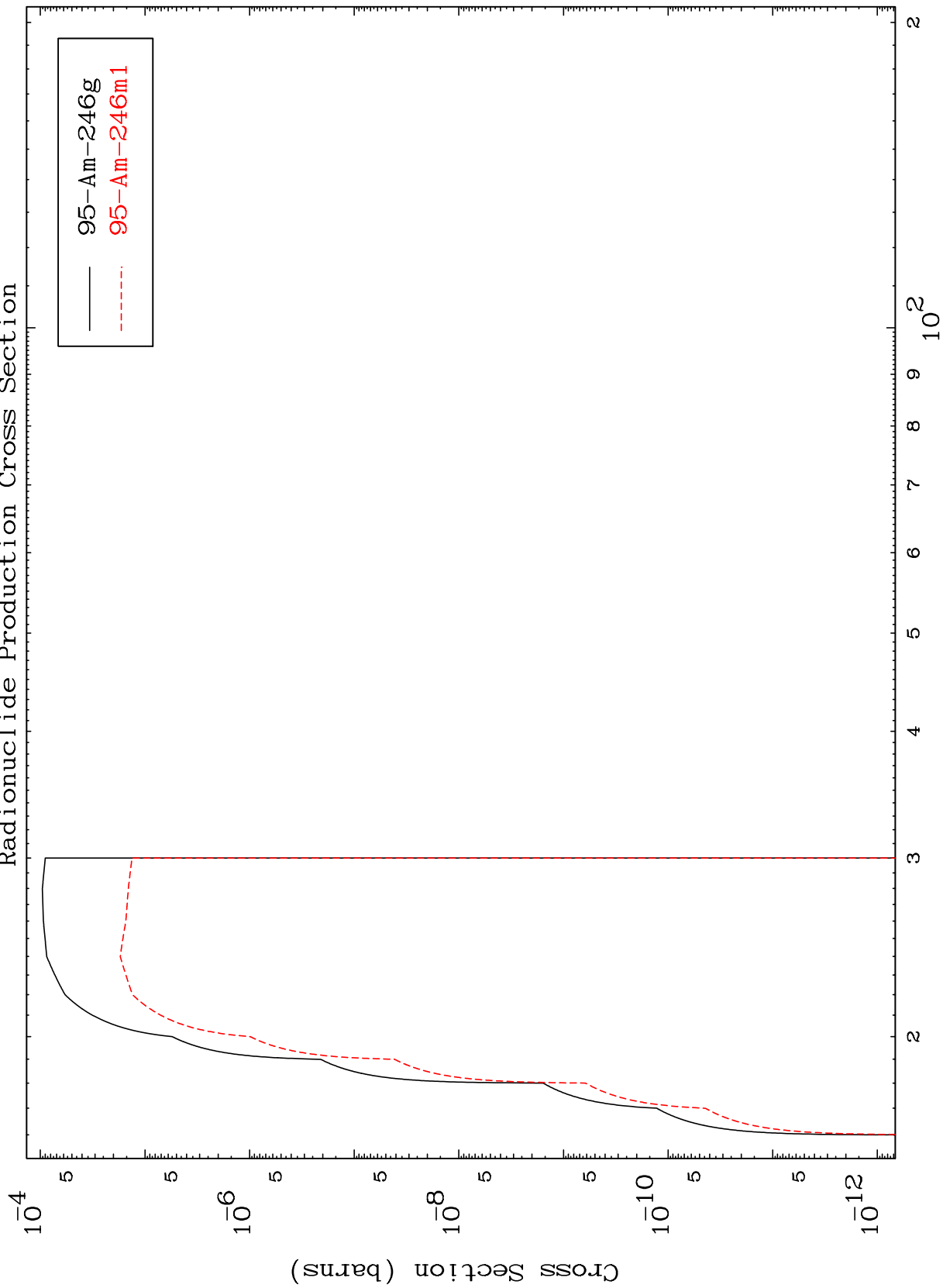
93-Np-244



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93-Np-244

( $\alpha, 2n$ )  
Radionuclide Production Cross Section



9

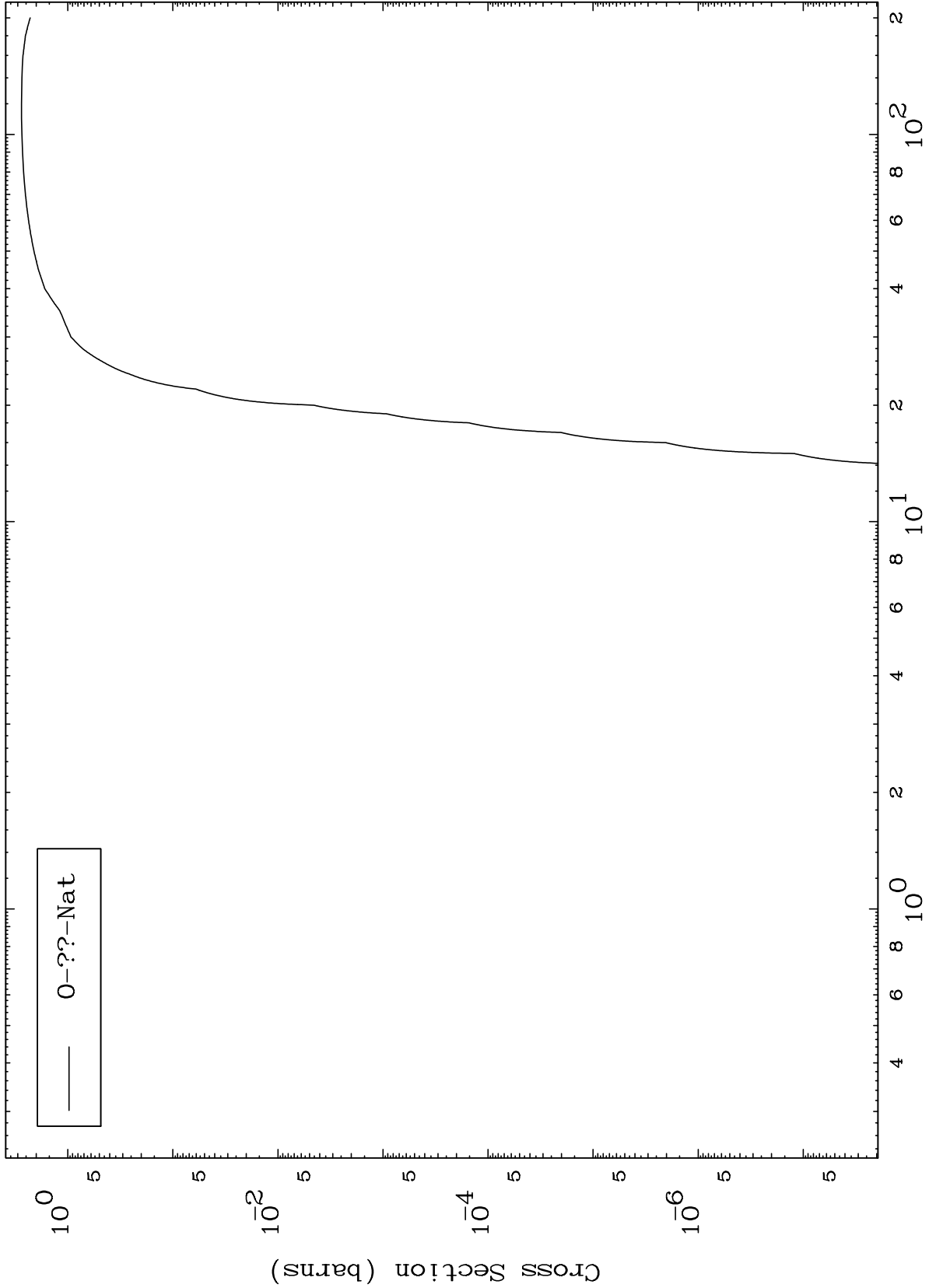
Incident Energy (MeV)

93-Np-244

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93-Np-244

$\alpha$  Fission  
Radionuclide Production Cross Section



10

Incident Energy (MeV)

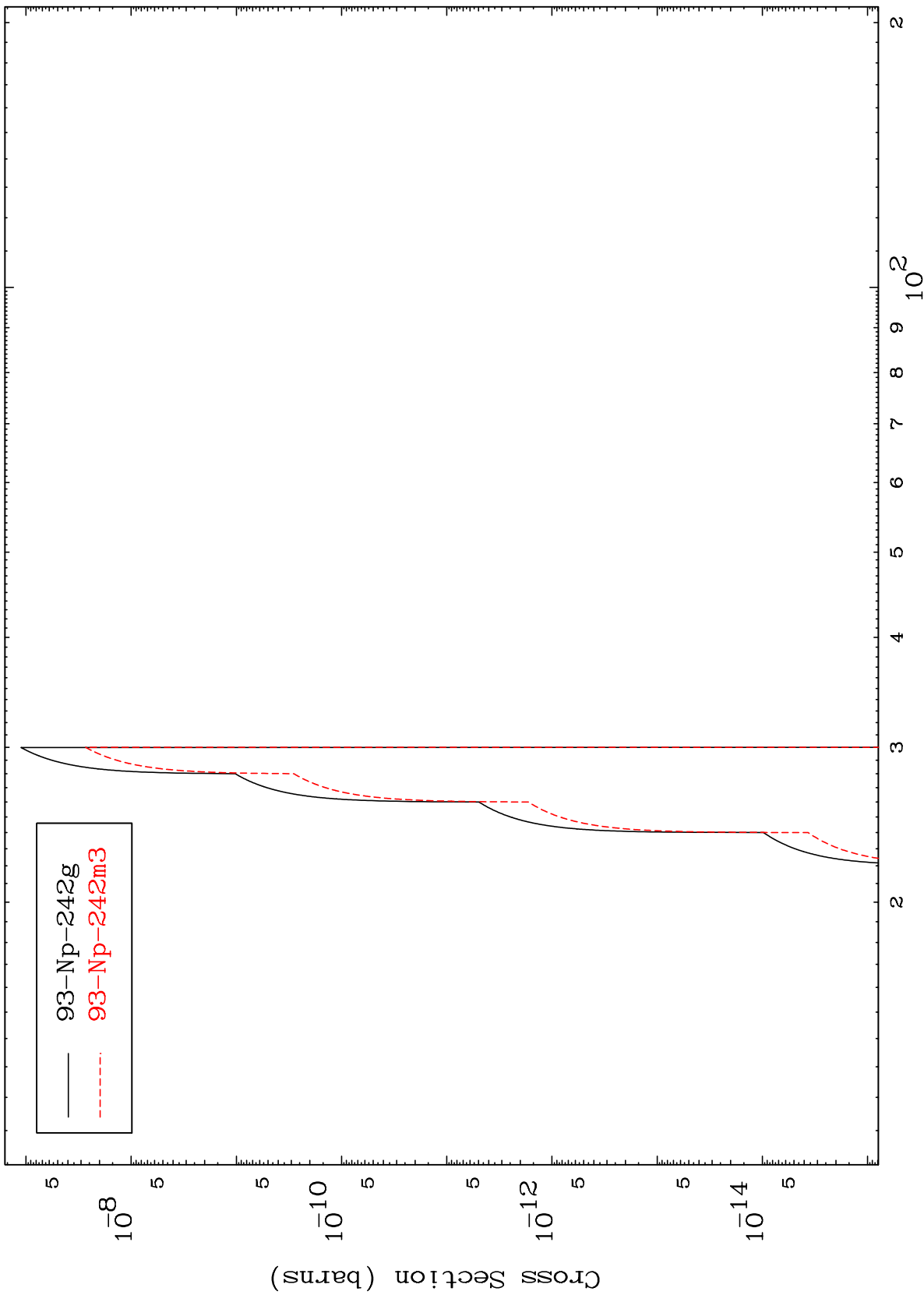
93-Np-244

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( $\alpha, 2n$ )  $\alpha$

93-Np-244

Radionuclide Production Cross Section

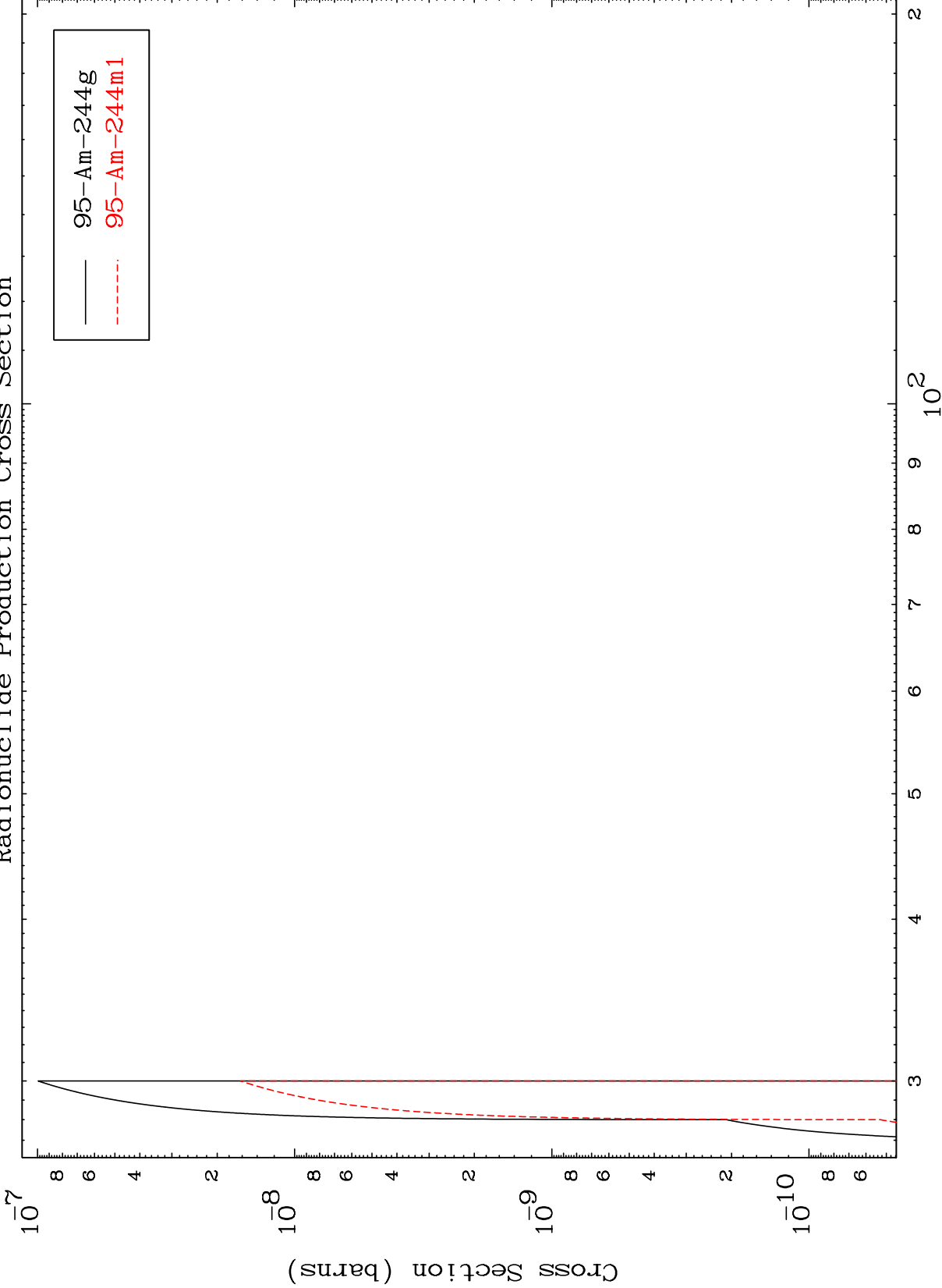


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( $\alpha, 4n$ )

93-Np-244

Radionuclide Production Cross Section



12

Incident Energy (MeV)

93-Np-244