

Program EVALPLOT  
(Version 2018-1)

by

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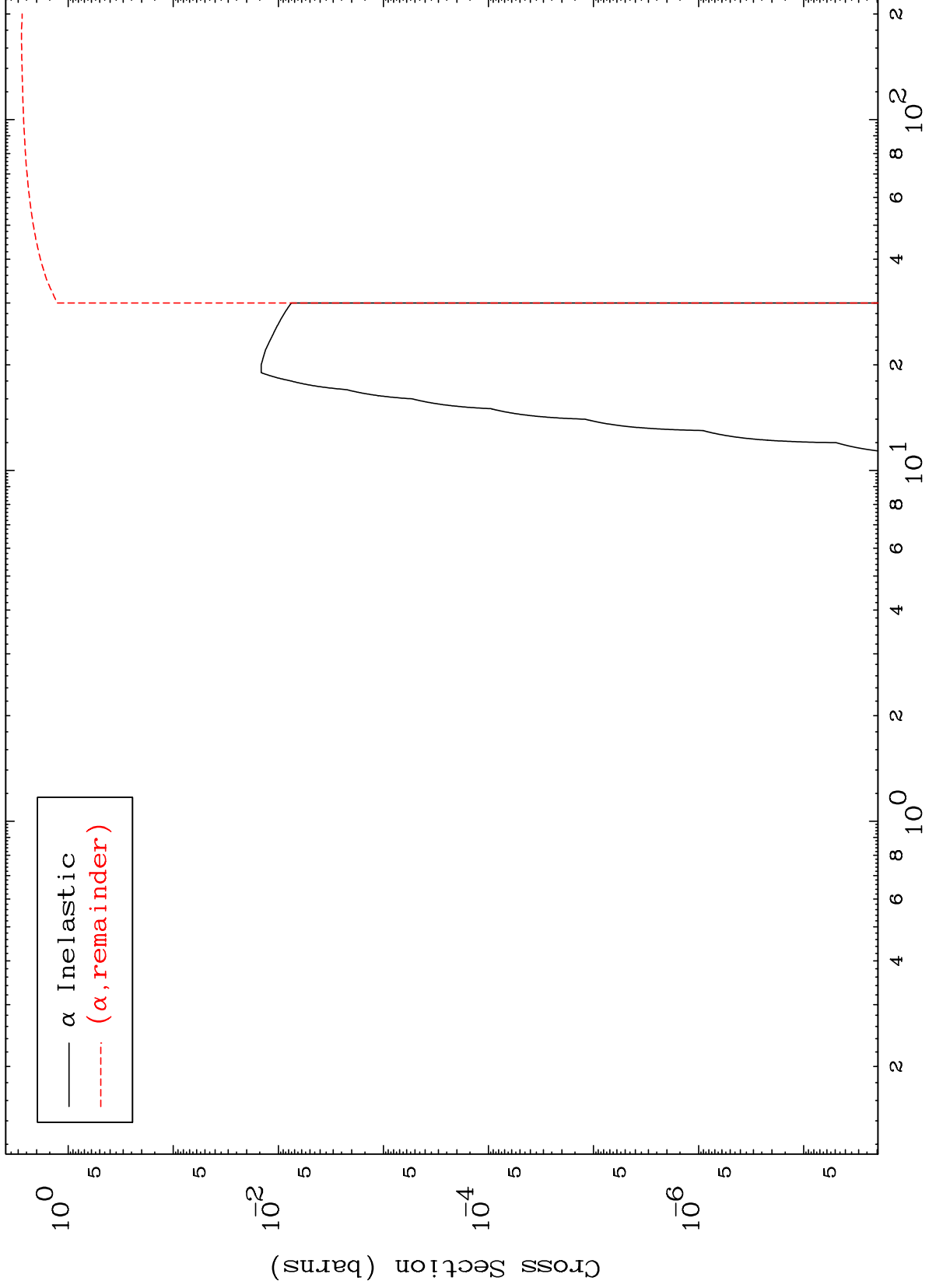
Press Mouse Button to Start

MAT 7670

$\alpha$  Major

76-Os-199

0 Kelvin Cross Sections



—  $\alpha$  Inelastic  
- - - ( $\alpha$ , remainder)

76-Os-199

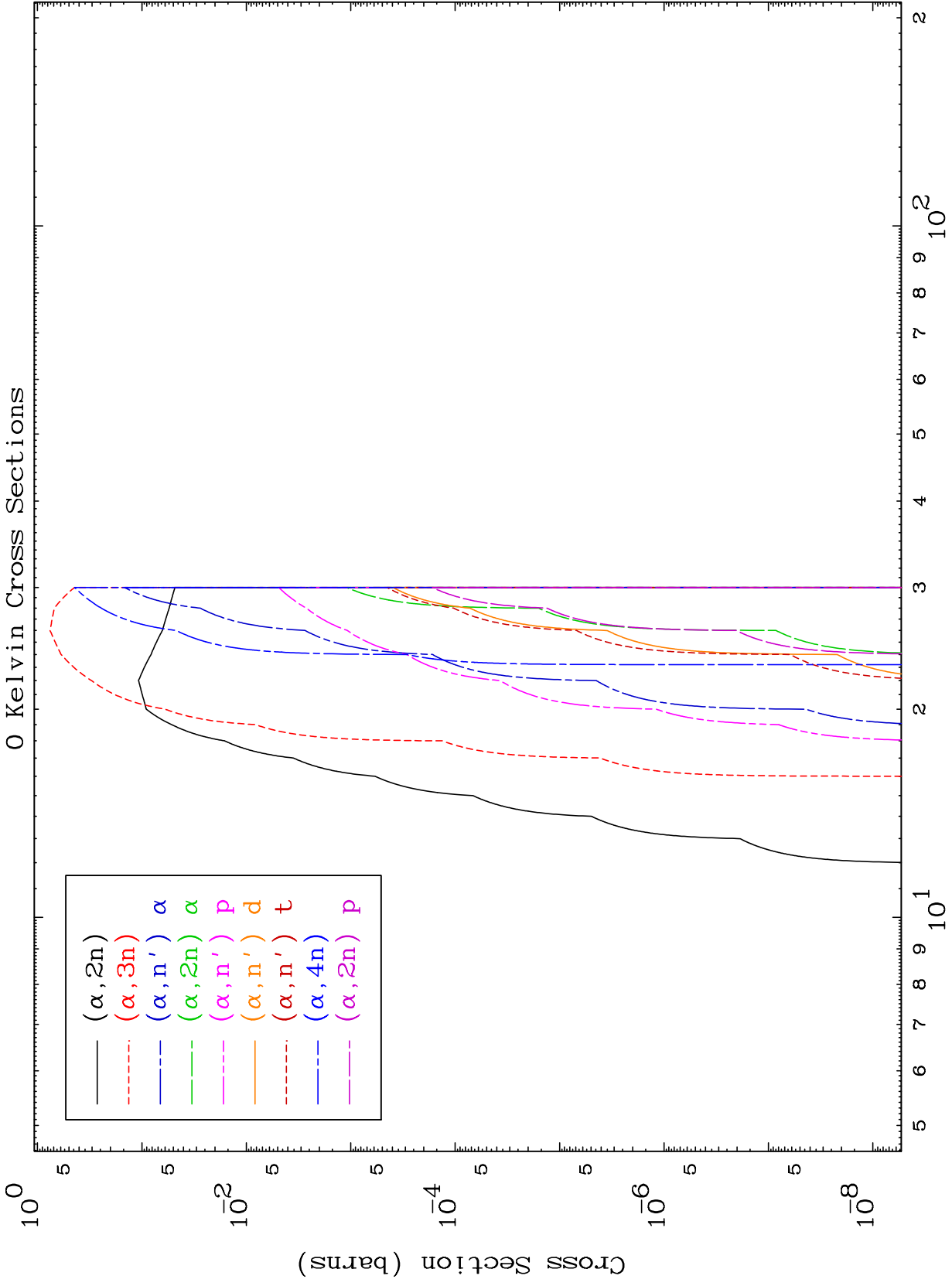
Incident Energy (MeV)

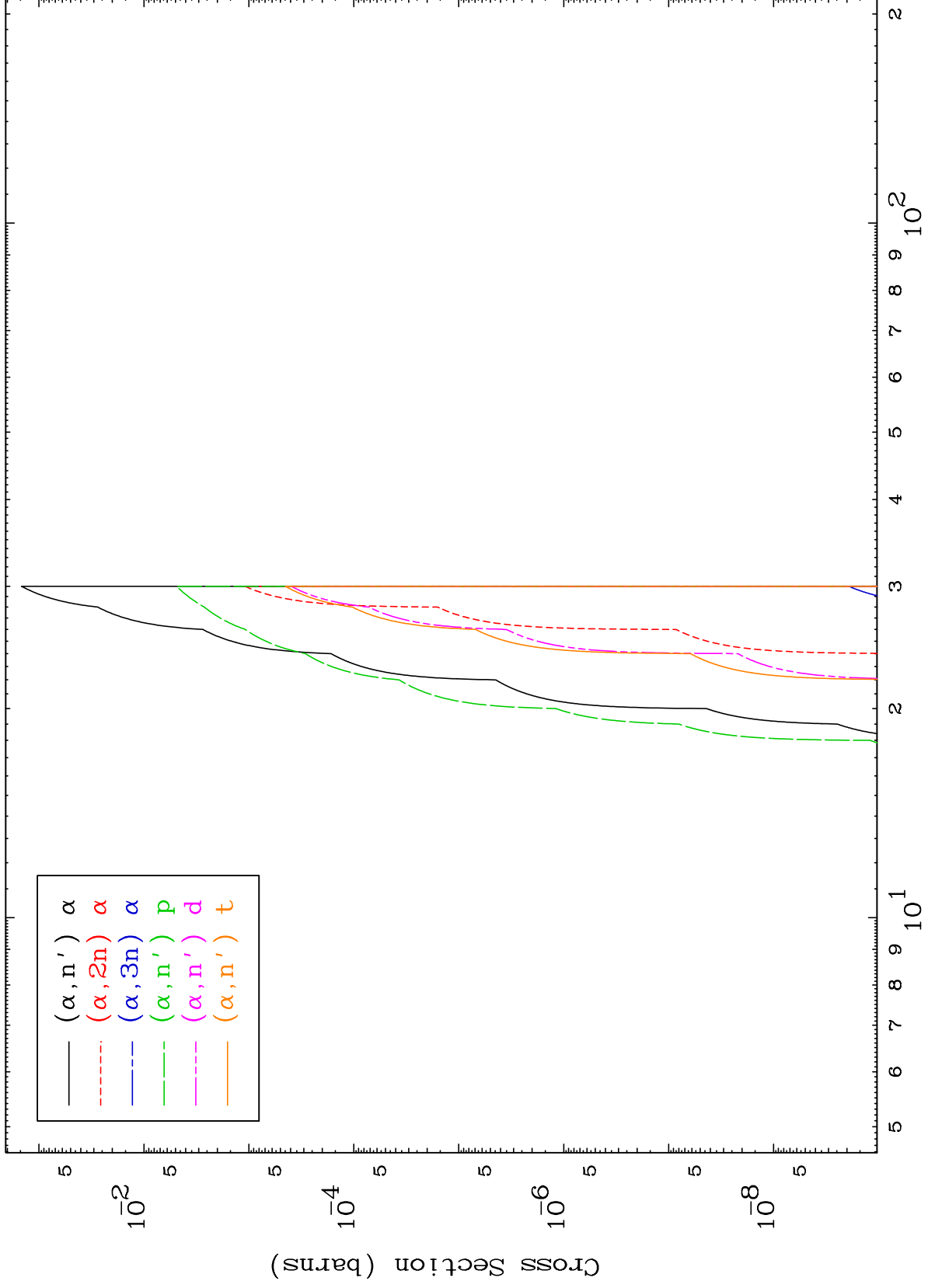
1

MAT 7670

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

76-Os-199

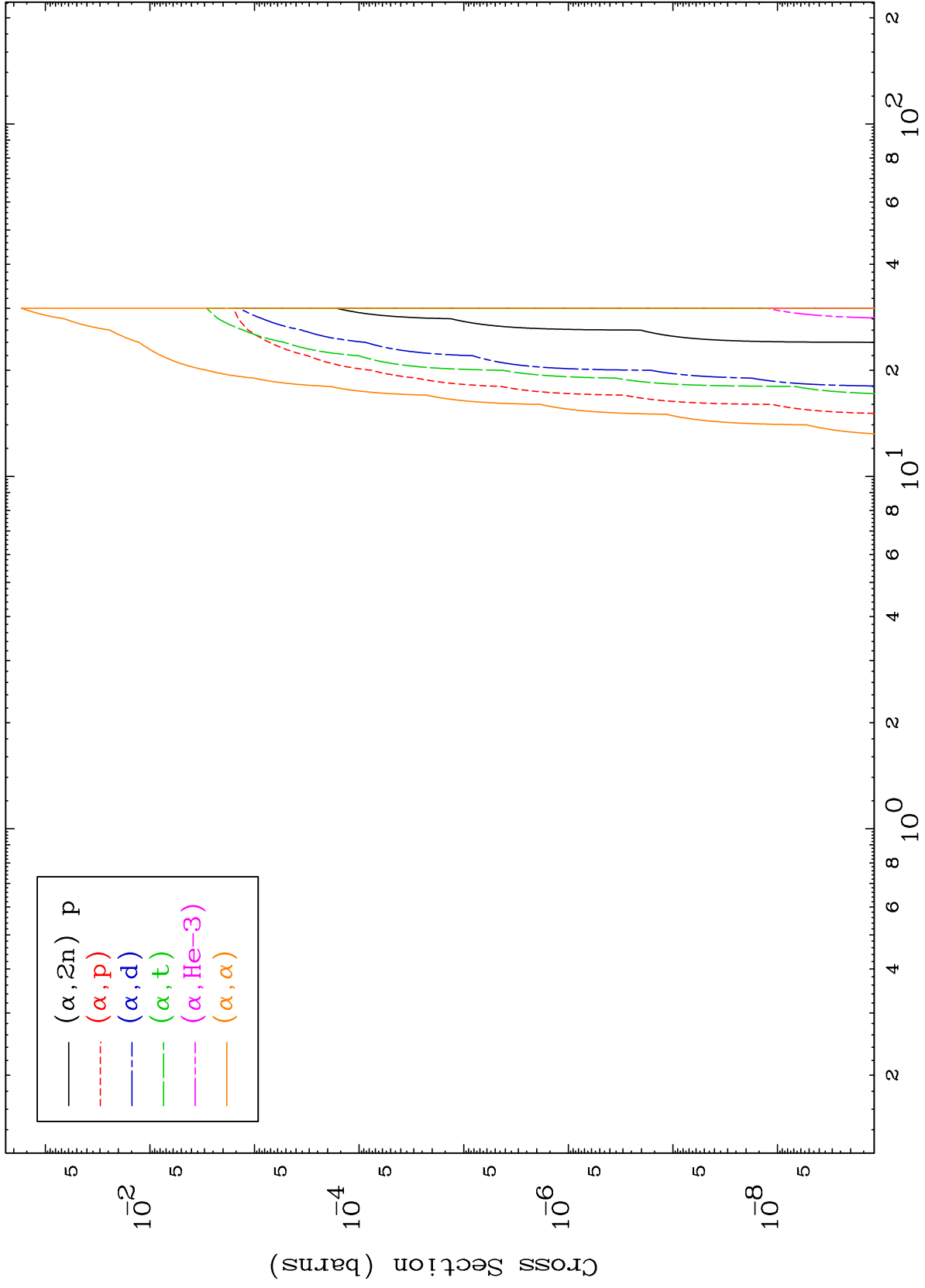




MAT 7670

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

76-Os-199

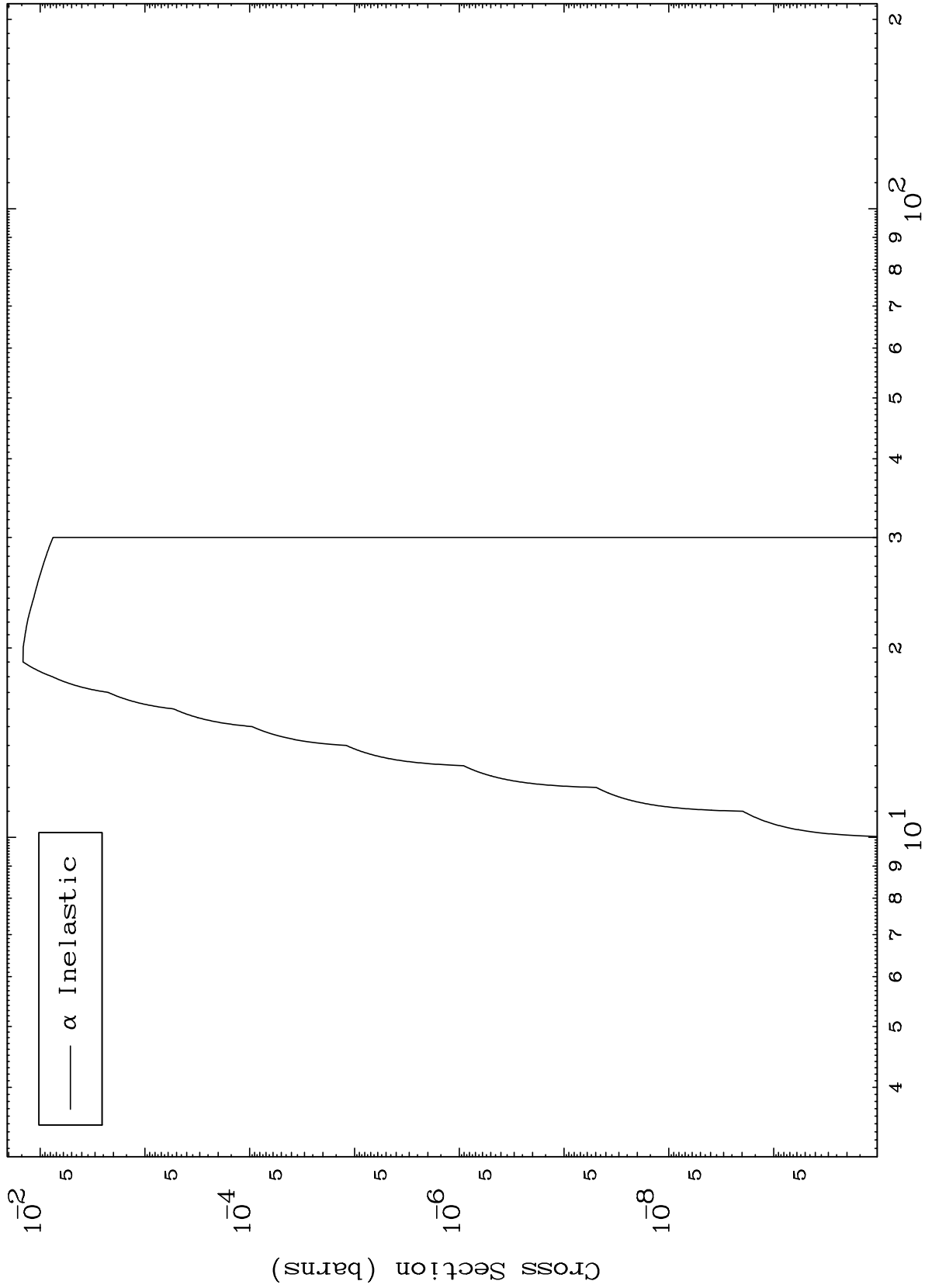


MAT 7670

( $\alpha, n'$ ) Level

76-0s-199

0 Kelvin Cross Sections



5

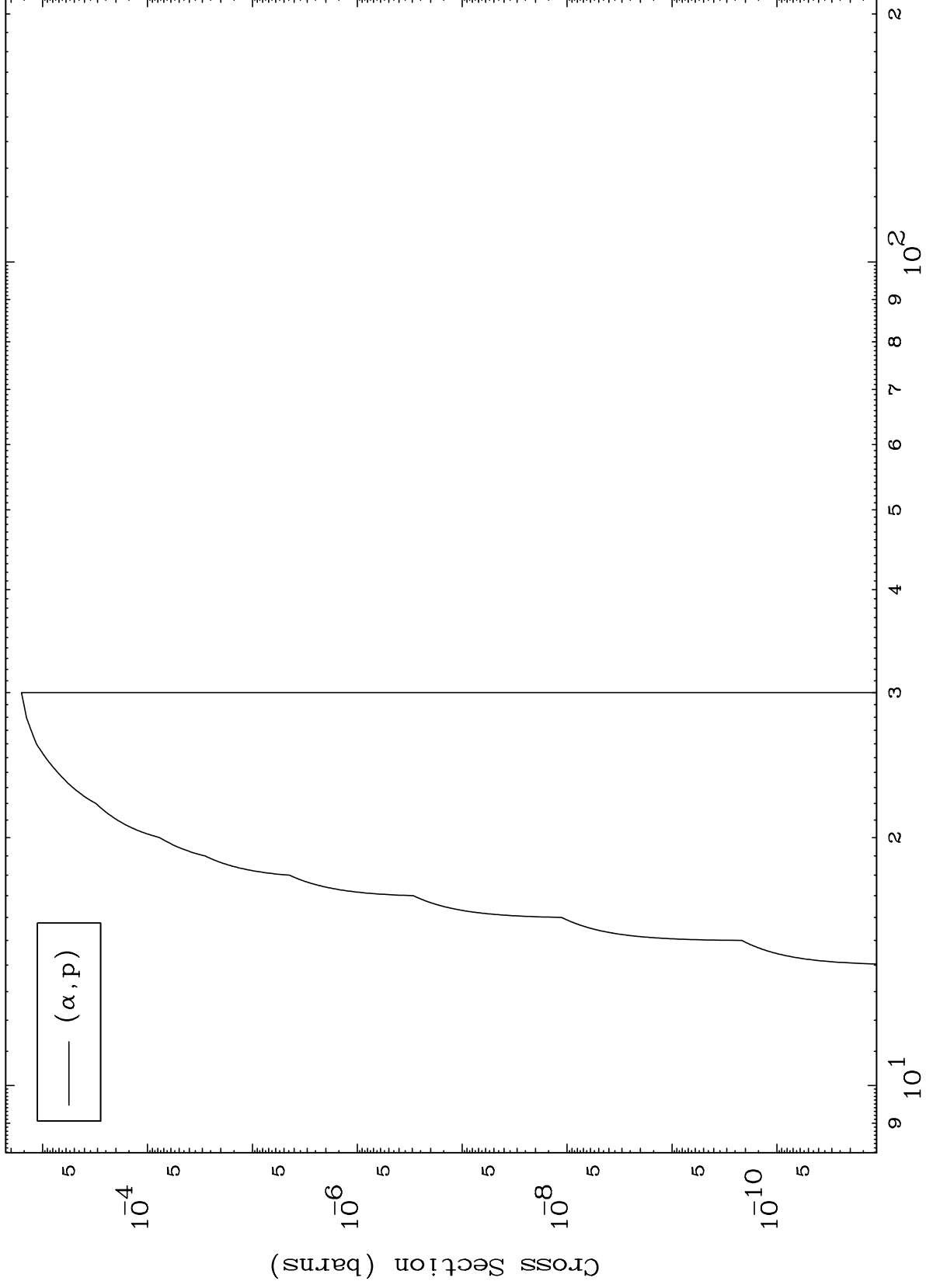
Incident Energy (MeV)

76-0s-199

MAT 7670

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

76-0s-199



Incident Energy (MeV)

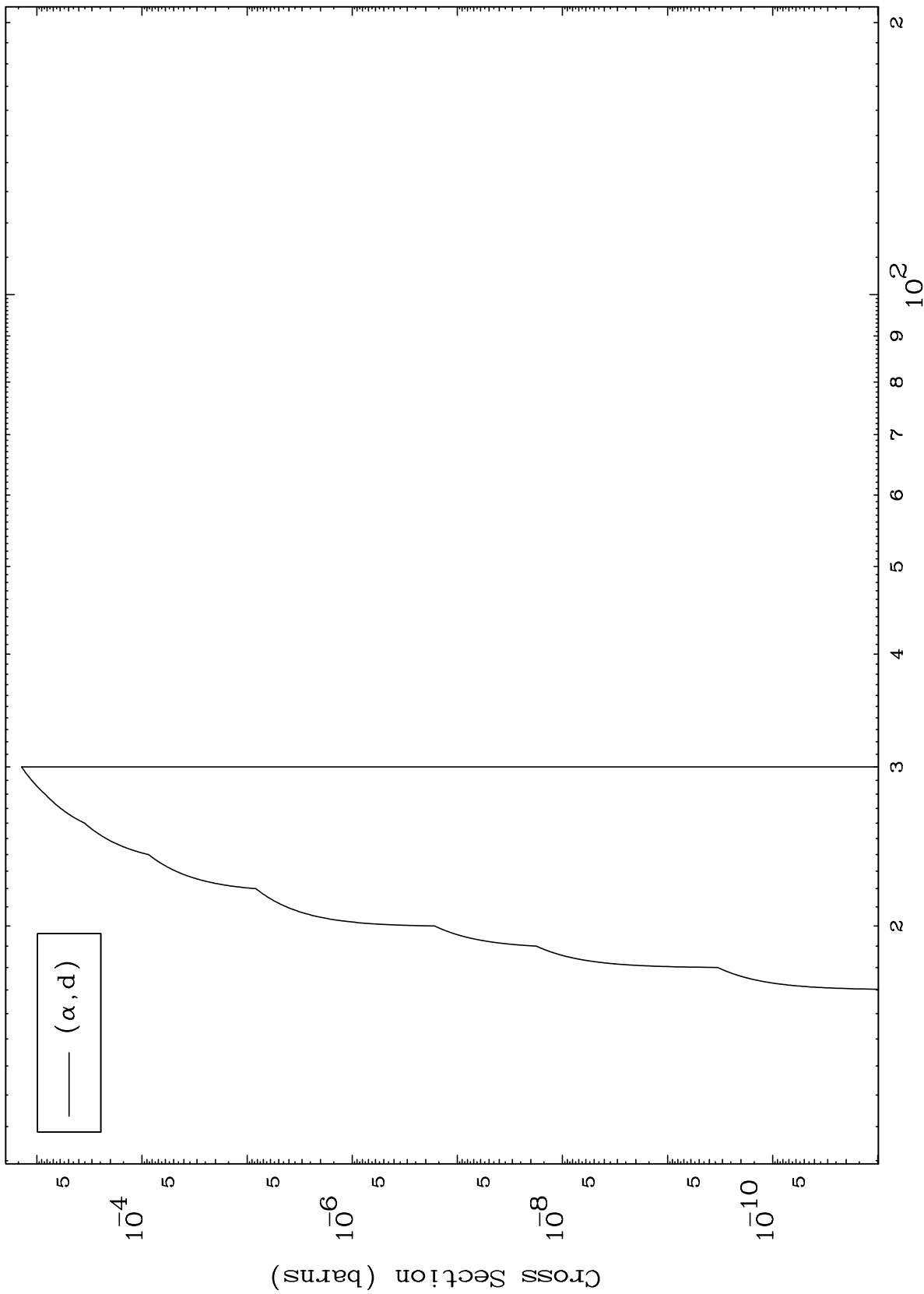
76-0s-199

6

MAT 7670

76-0s-199

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

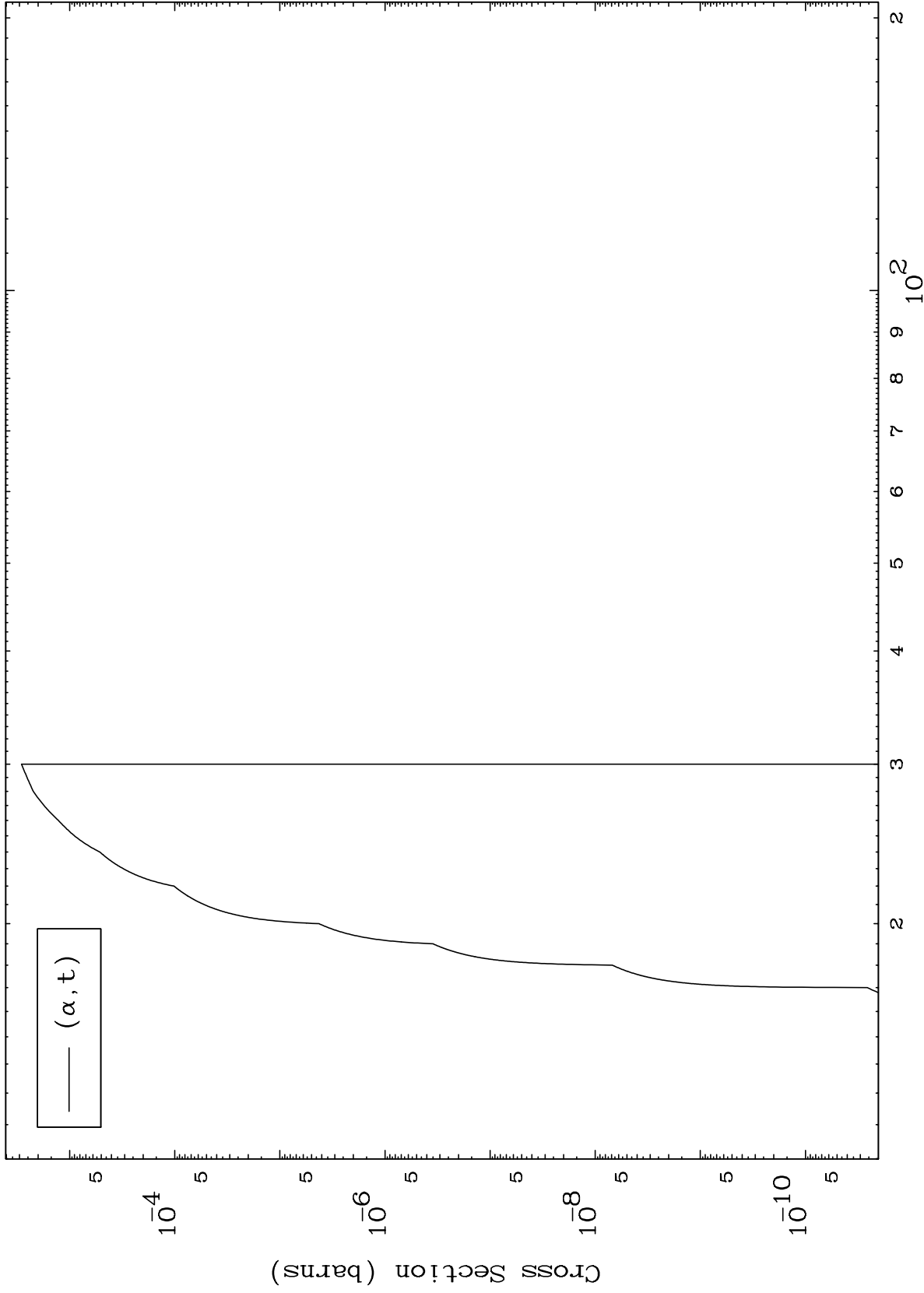


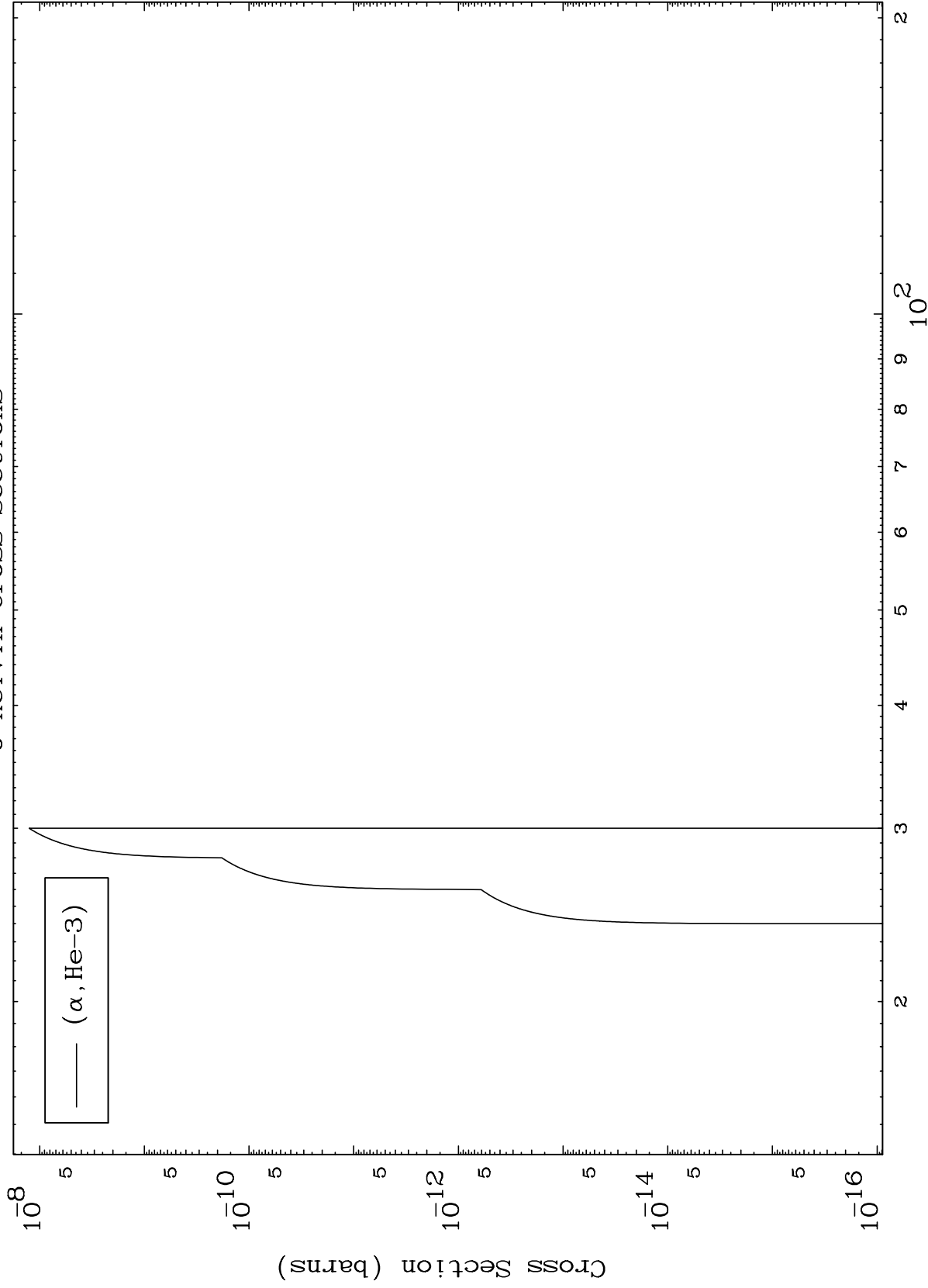
76-0s-199

Incident Energy (MeV)

7





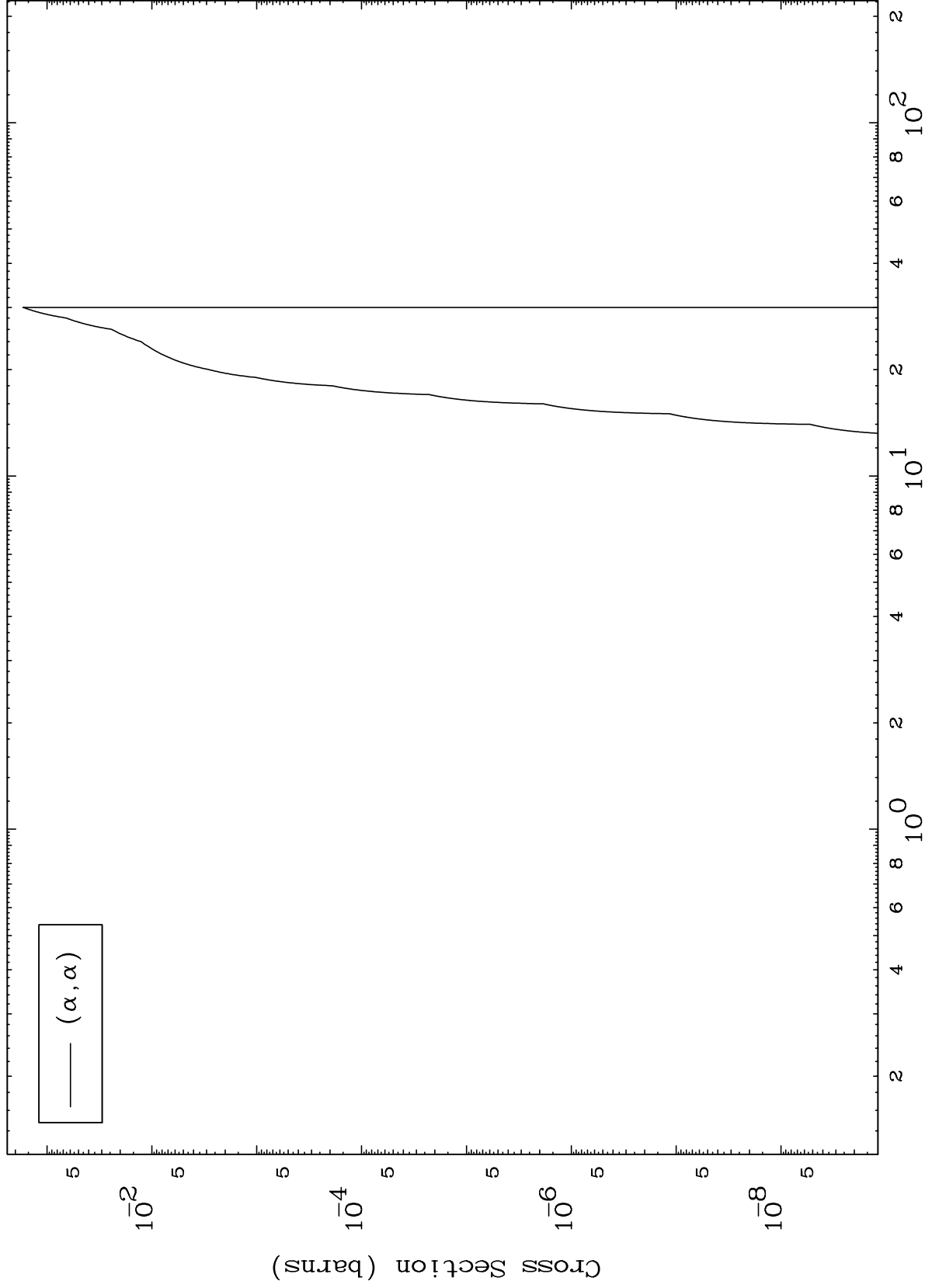


MAT 7670

( $\alpha, \alpha$ ) Levels

76-0s-199

0 Kelvin Cross Sections



10

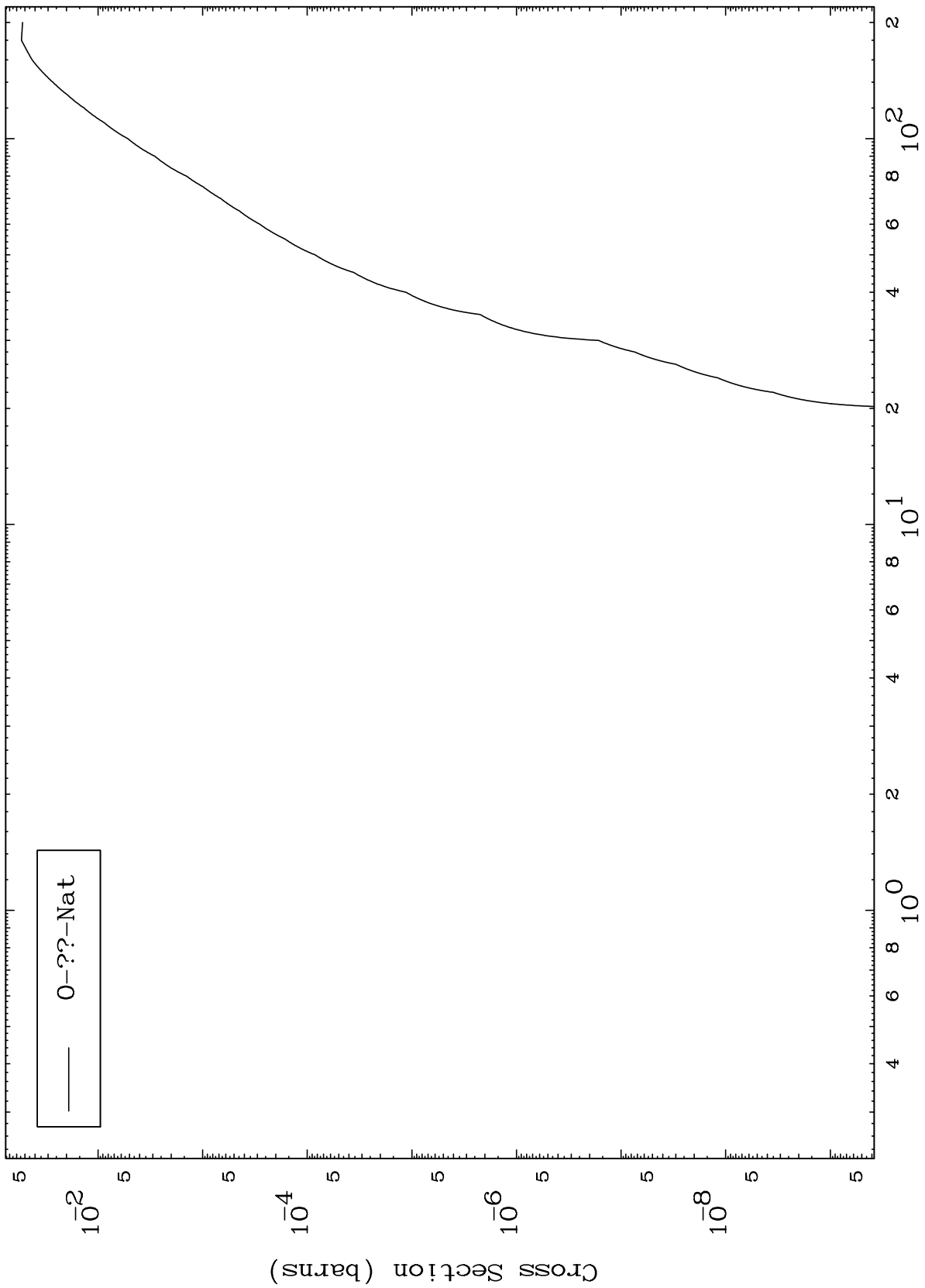
Incident Energy (MeV)

76-0s-199

MAT 7670

76-0s-199

$\alpha$  Fission  
Radionuclide Production Cross Section



76-0s-199

Incident Energy (MeV)

