

Program EVALPLOT  
(Version 2018-1)

by

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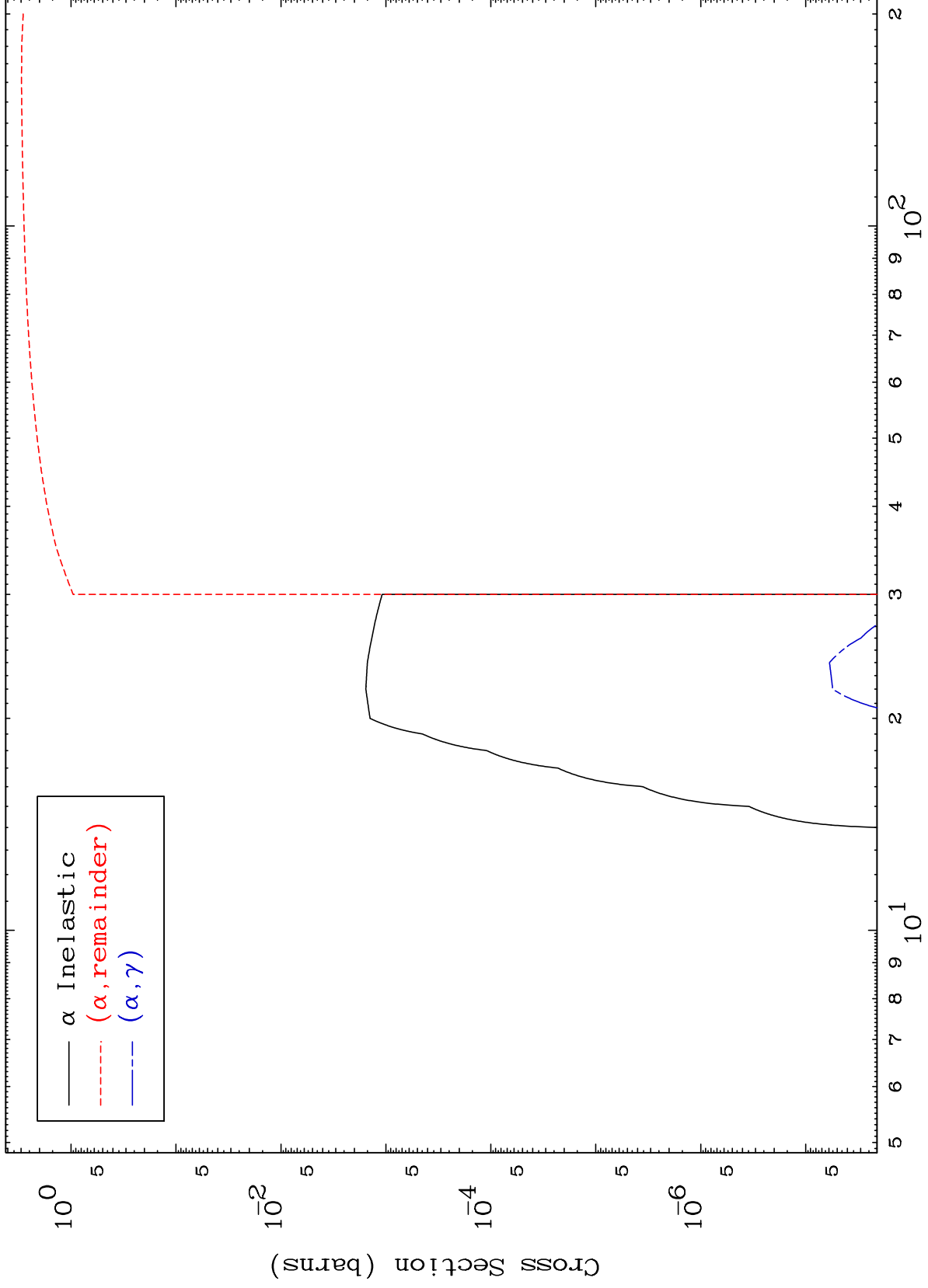
Web: [redcullen1.net/HOMEPAGE.NEW](http://redcullen1.net/HOMEPAGE.NEW)

Press Mouse Button to Start

MAT 9134

0 Kelvin Major  
 $\alpha$  Major Cross Sections

91-Pa-232



1

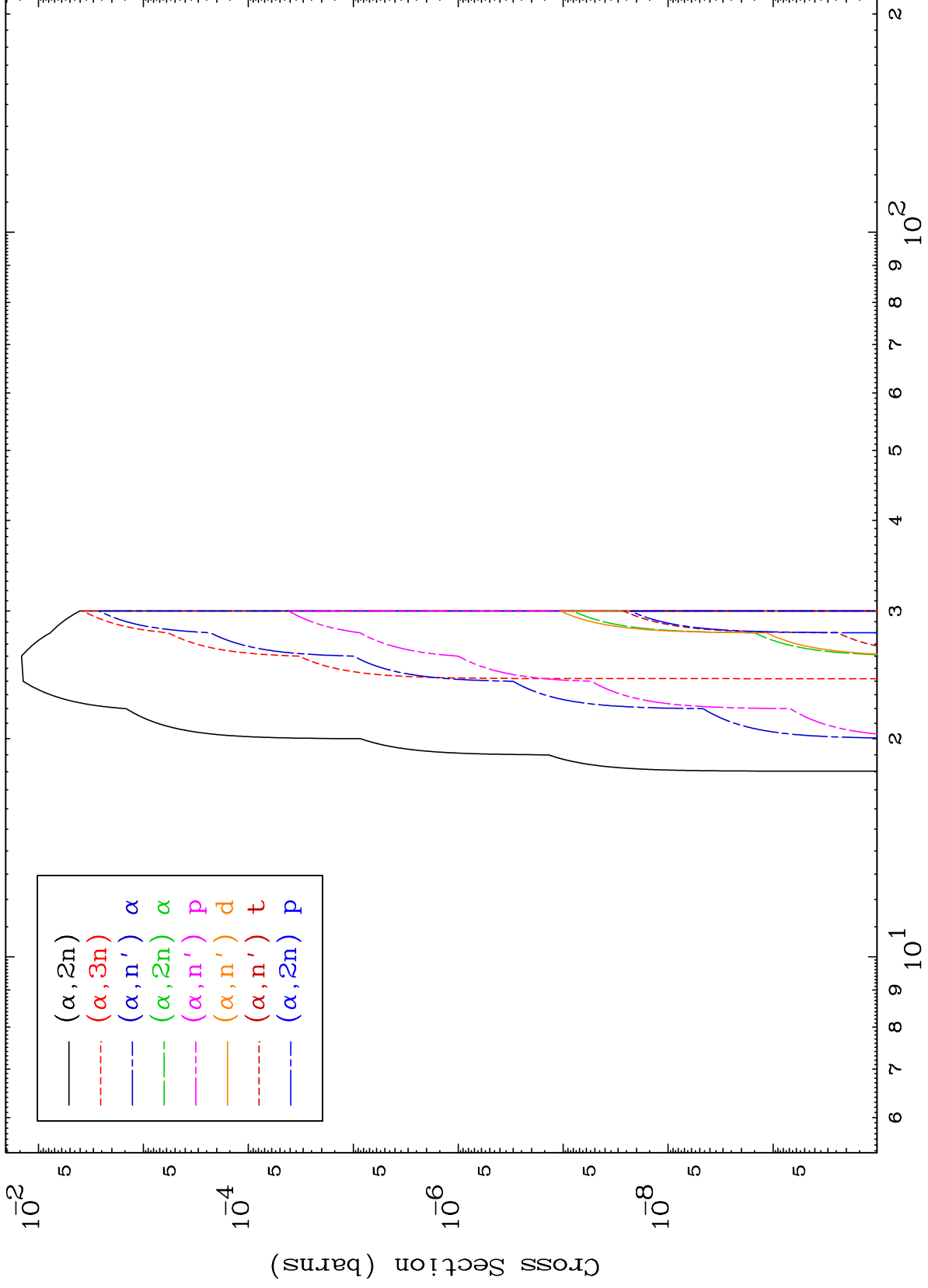
Incident Energy (MeV)

91-Pa-232

MAT 9134

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

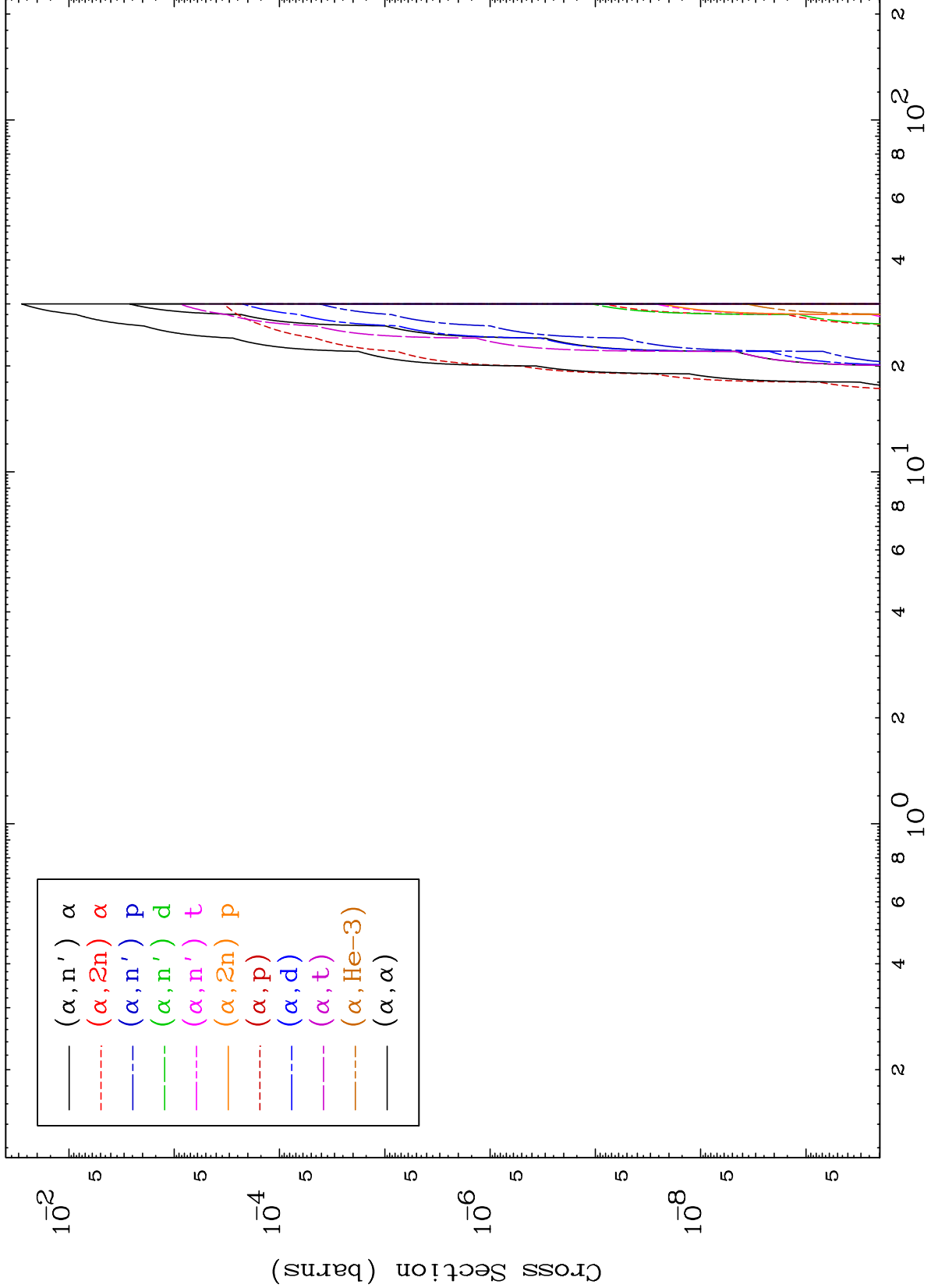
91-Pa-232



2

Incident Energy (MeV)

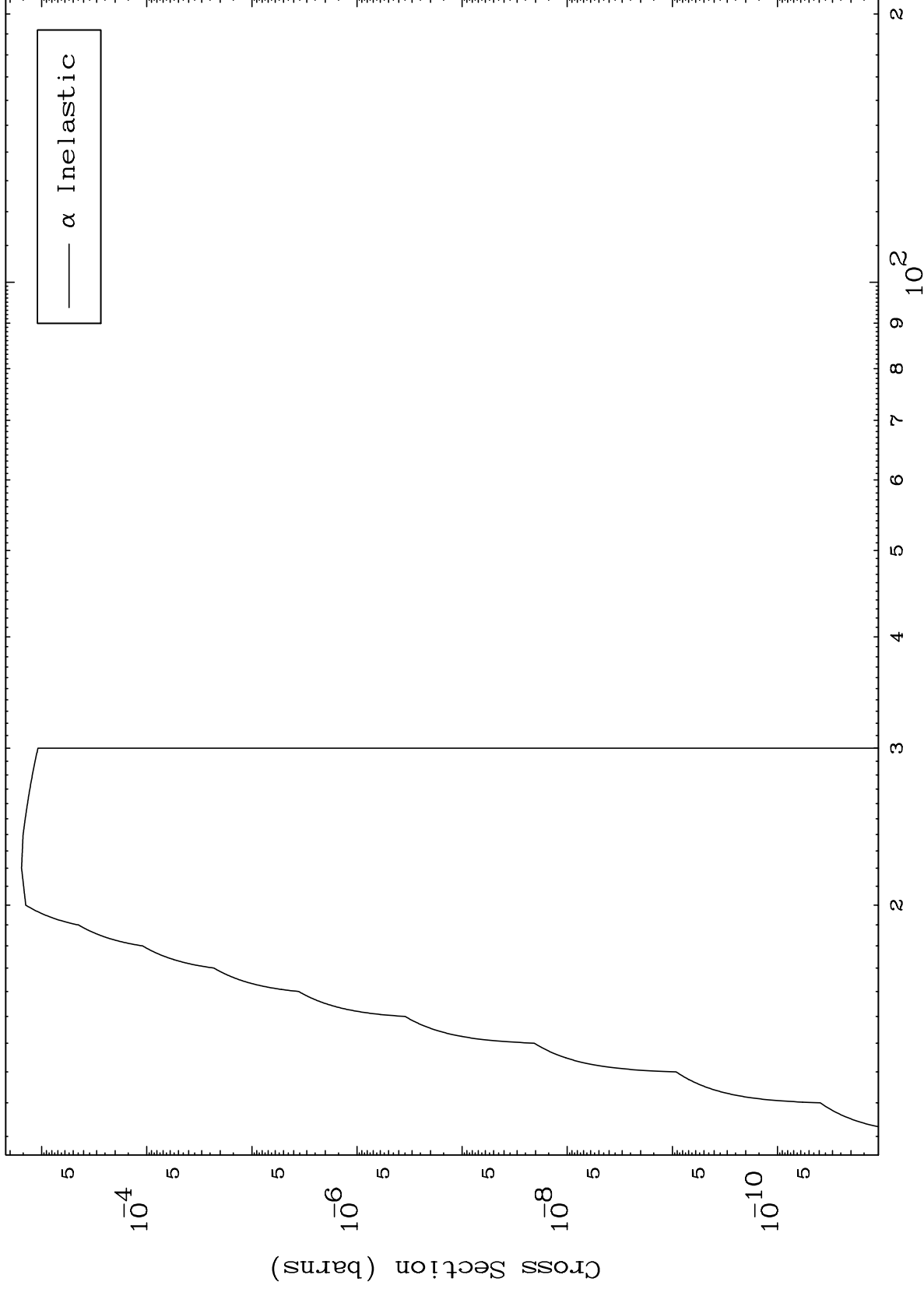
91-Pa-232



MAT 9134

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections

91-Pa-232



$\alpha$  Inelastic

Incident Energy (MeV)

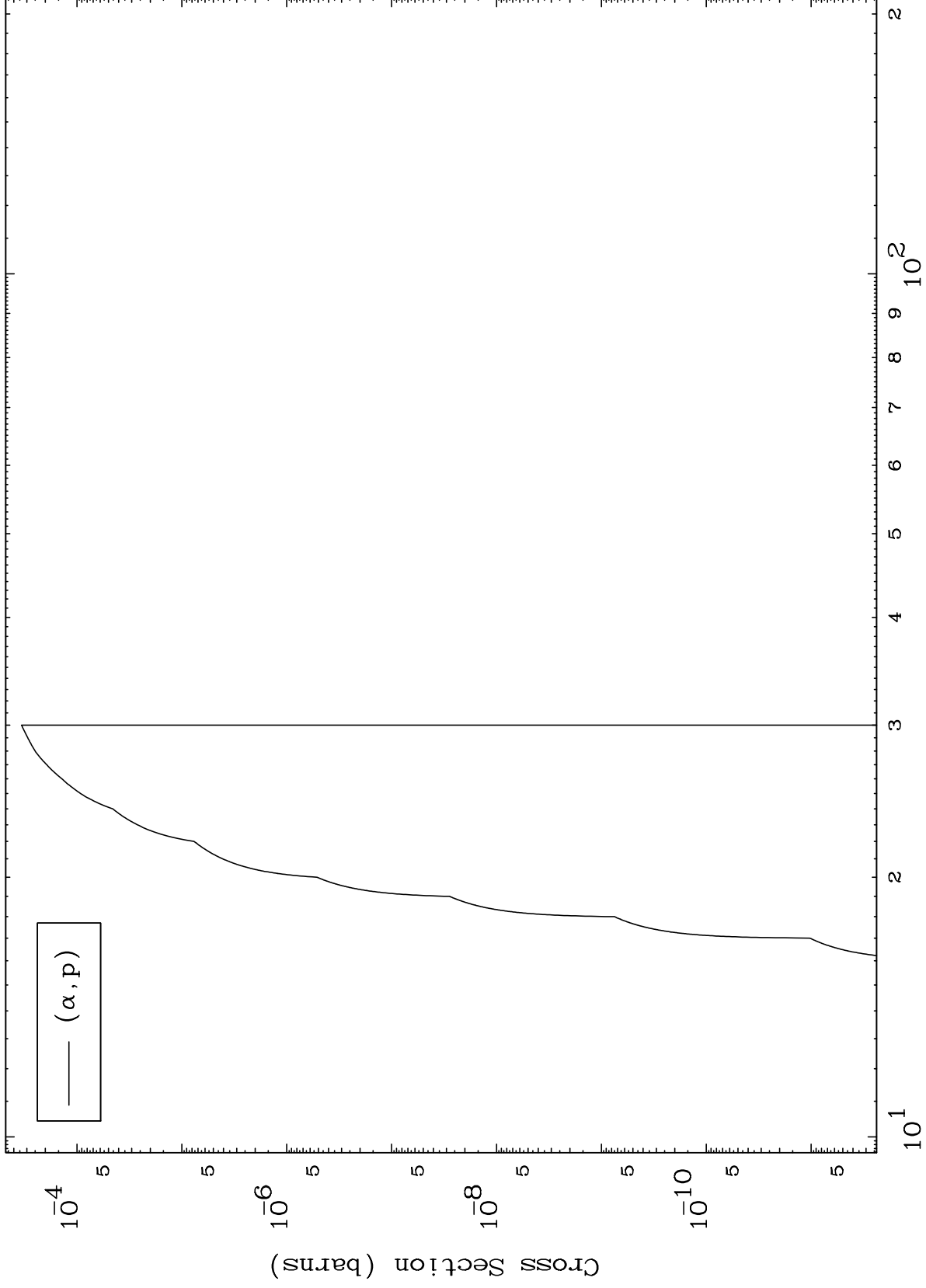
91-Pa-232

4

MAT 9134

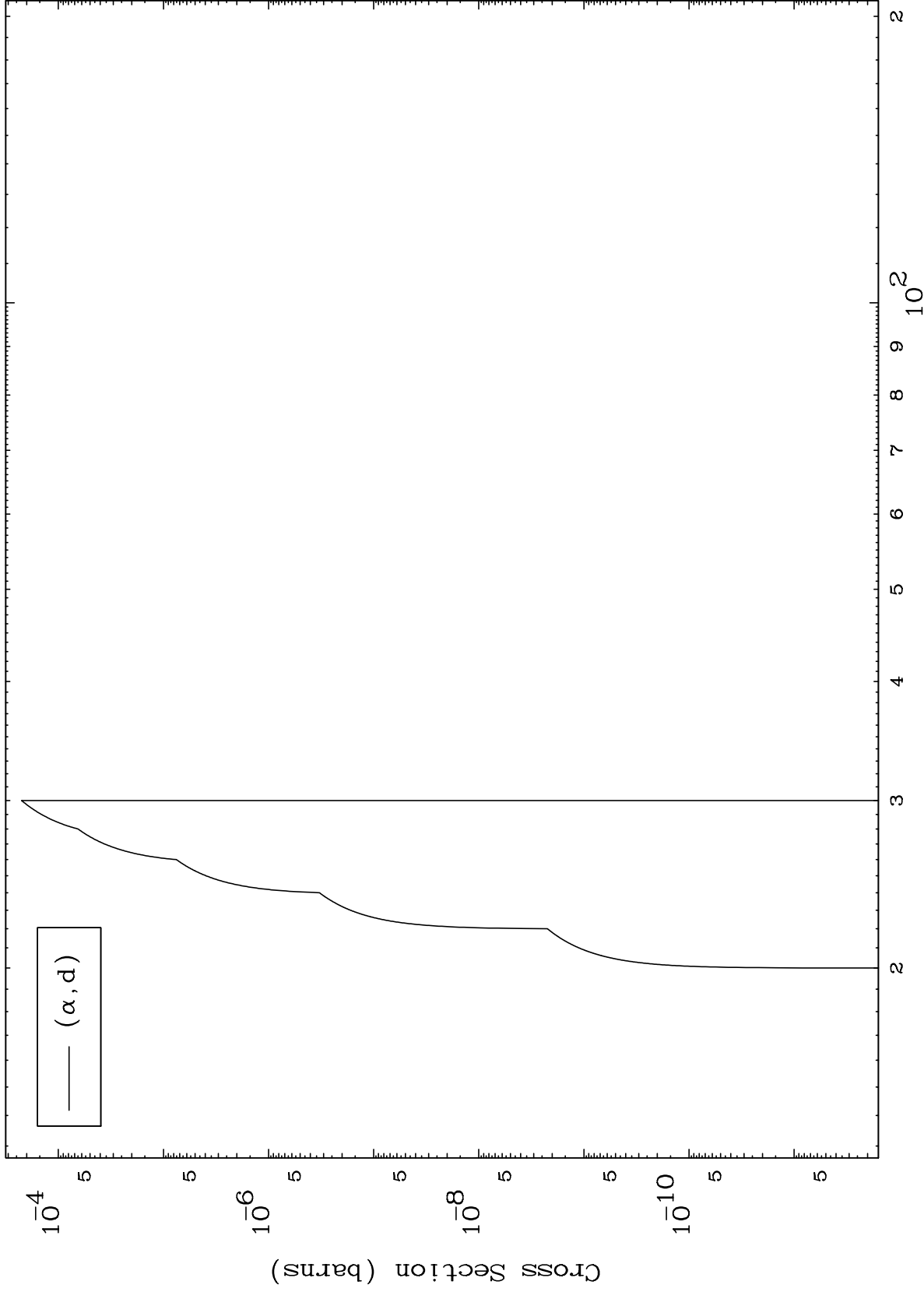
( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

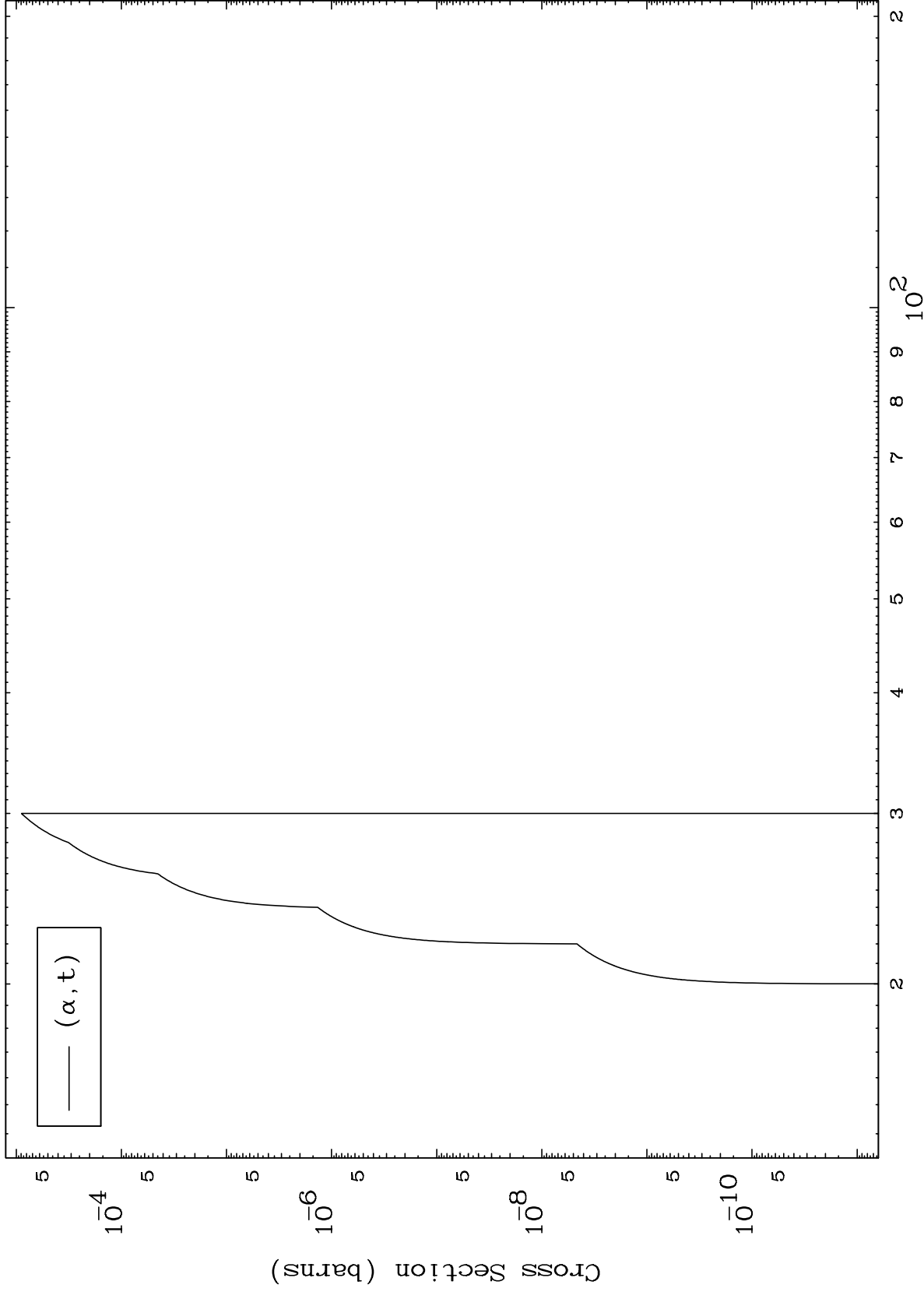
91-Pa-232



Incident Energy (MeV)

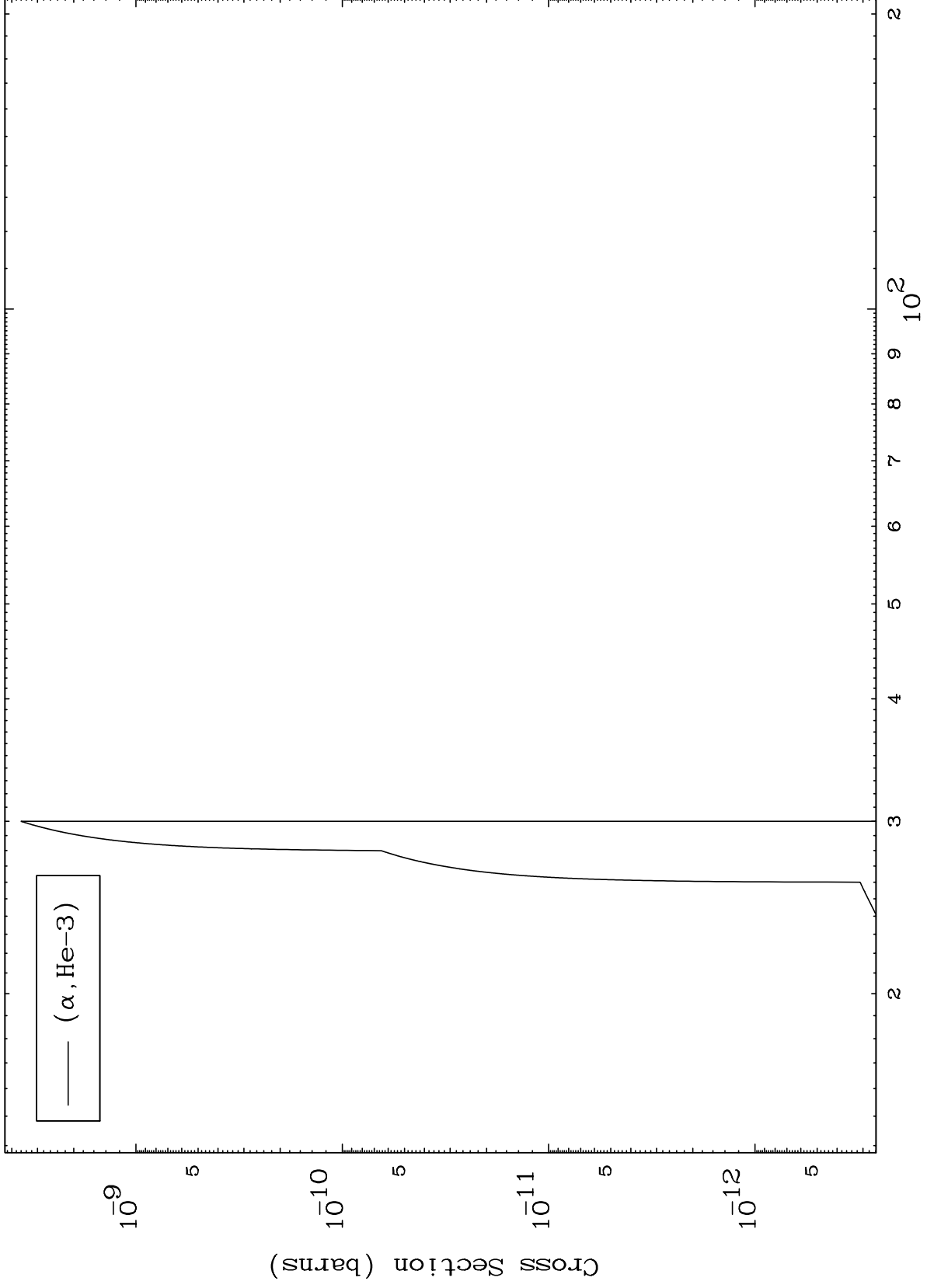
91-Pa-232







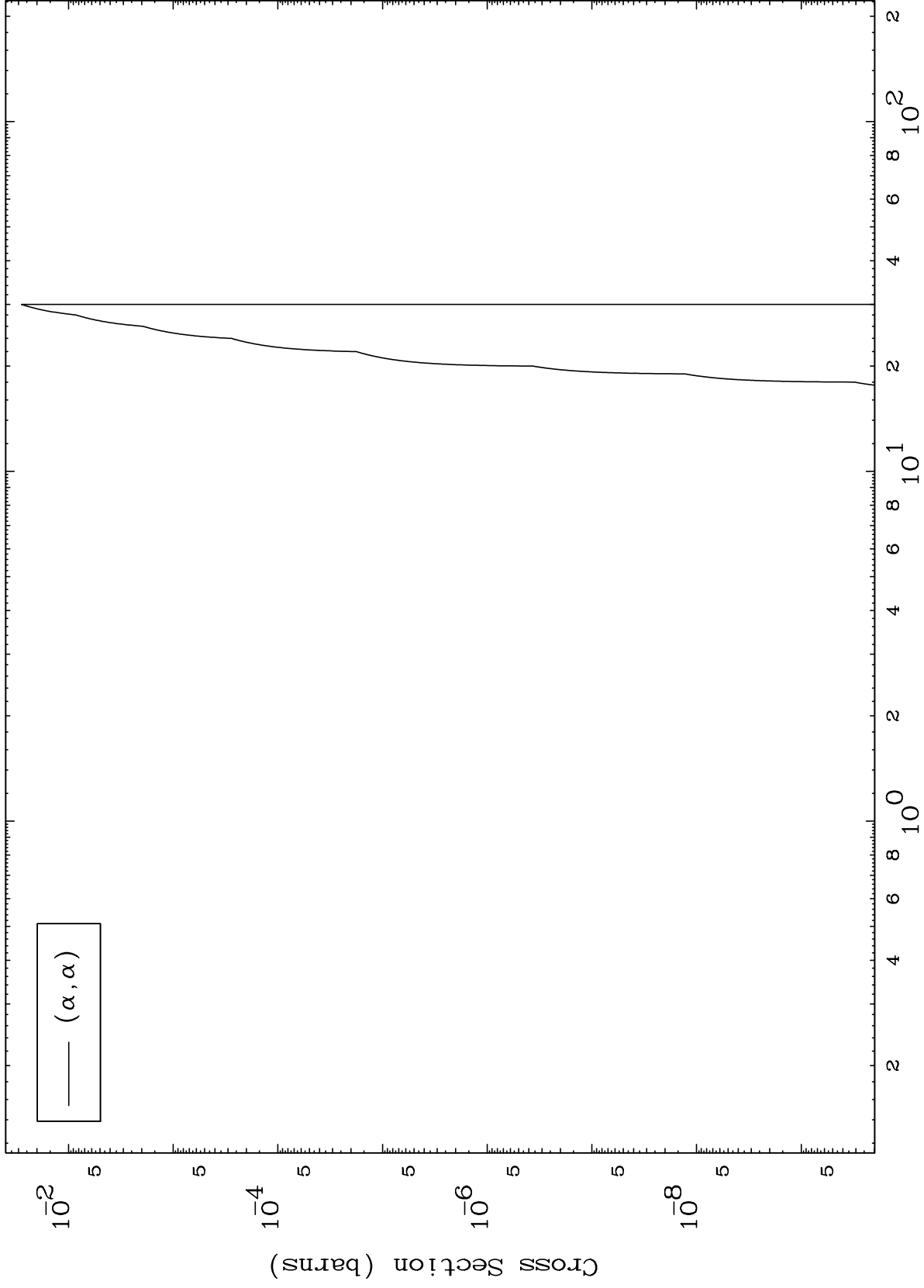
( $\alpha, \text{He}3$ ) Levels  
0 Kelvin Cross Sections



MAT 9134

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

91-Pa-232

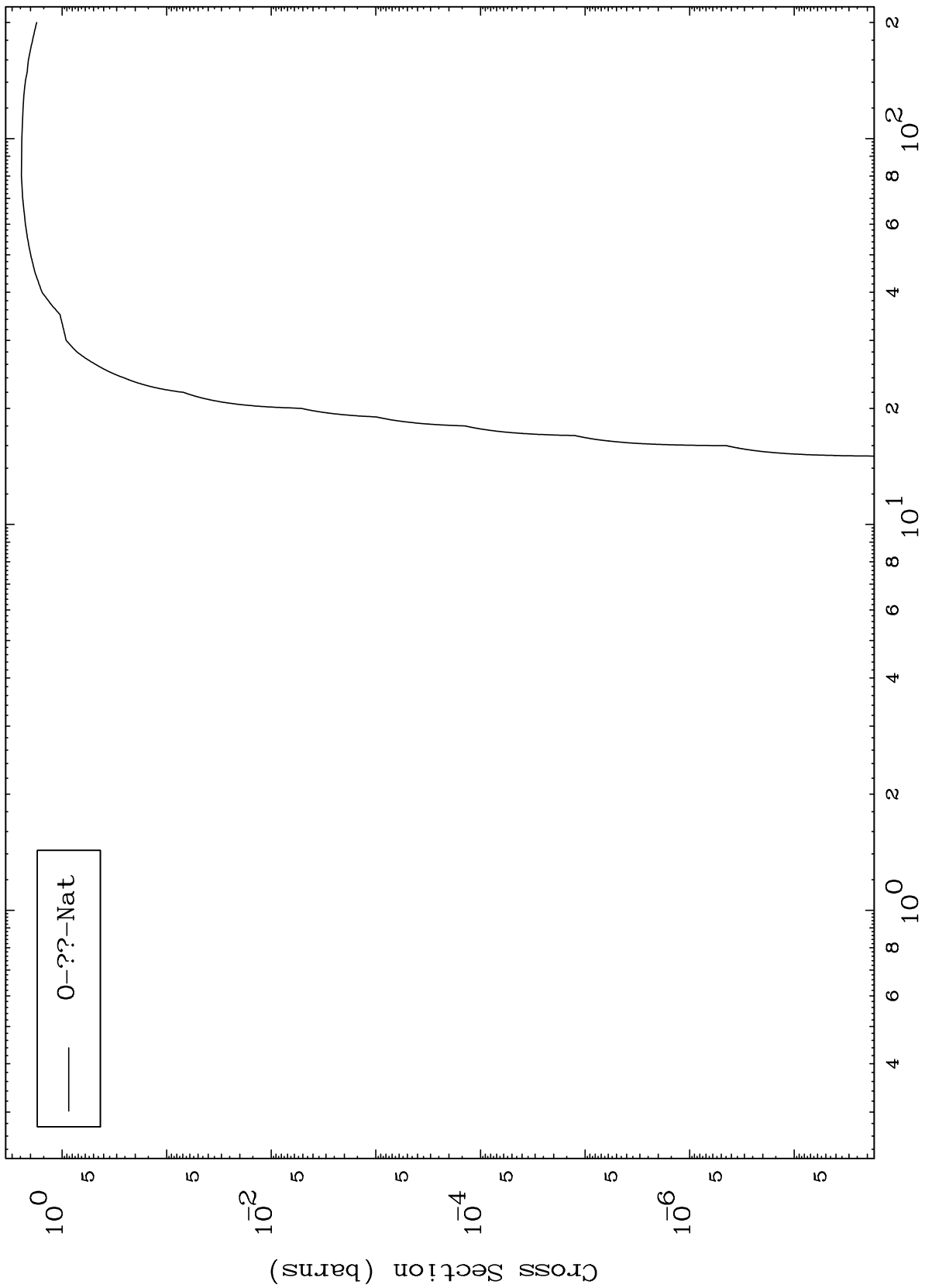


MAT 9134

91-Pa-232

$\alpha$  Fission

Radionuclide Production Cross Section



10

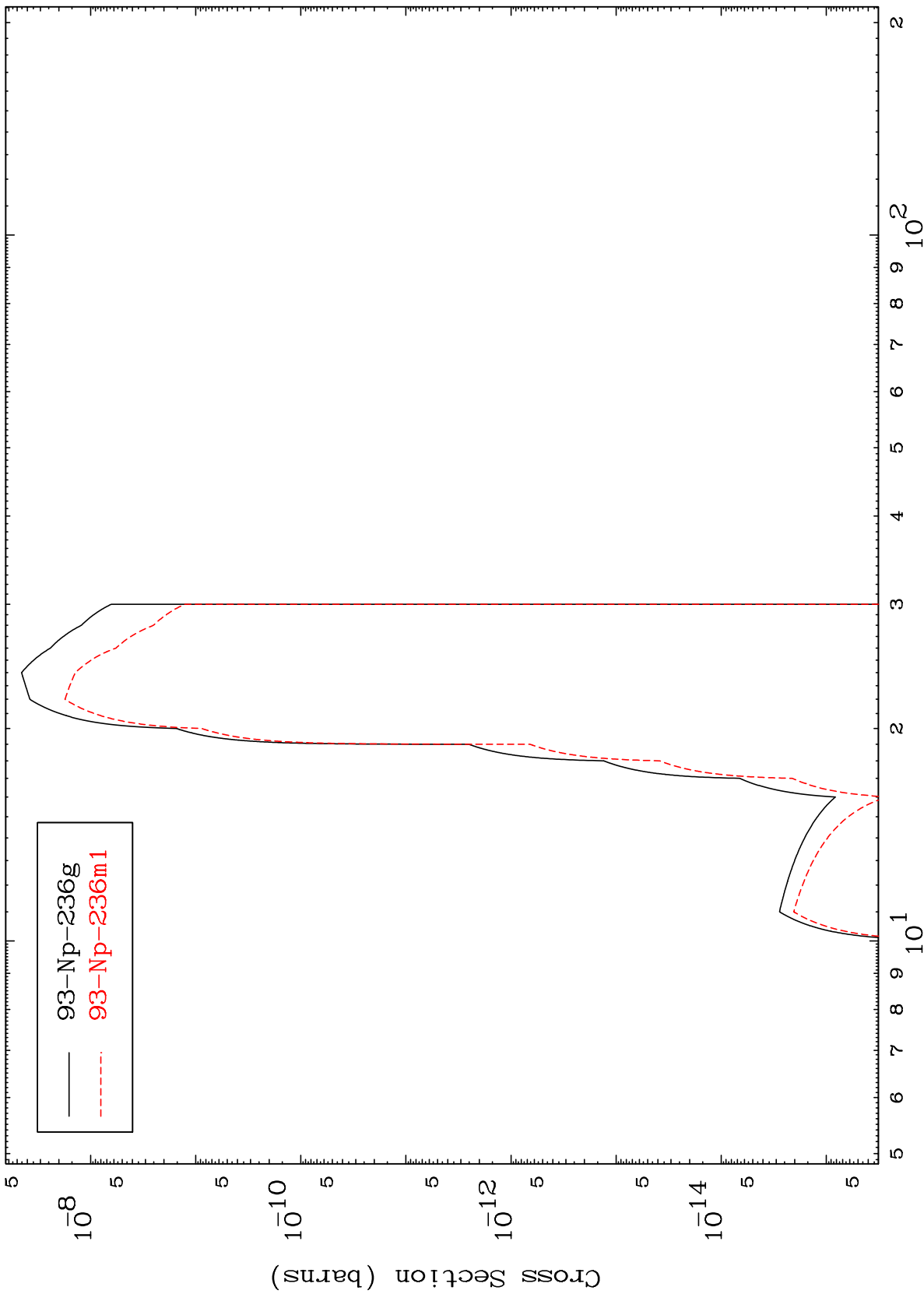
Incident Energy (MeV)

91-Pa-232

MAT 9134

91-Pa-232

( $\alpha, \gamma$ )  
Radionuclide Production Cross Section



91-Pa-232

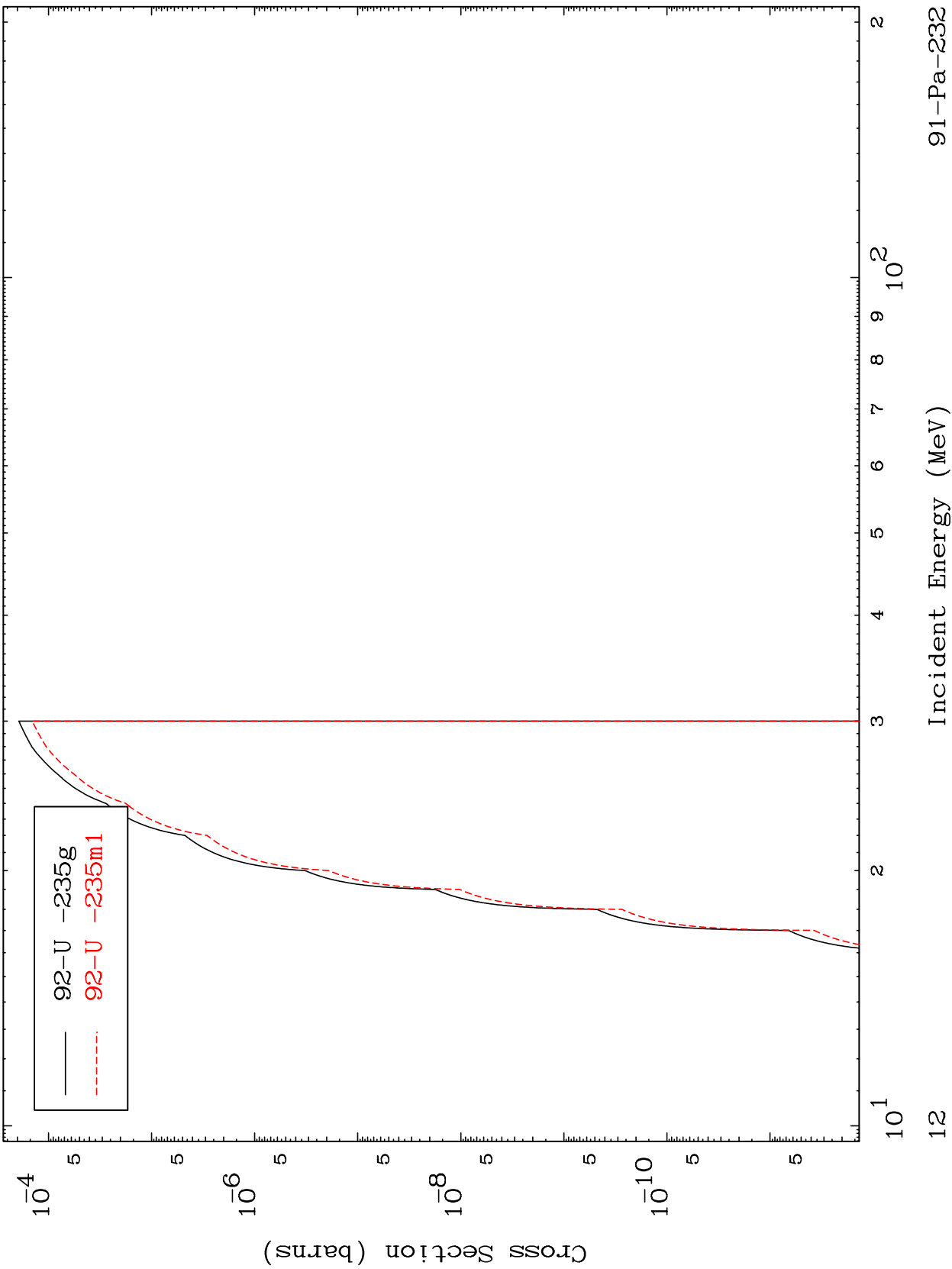
Incident Energy (MeV)

11

MAT 9134

91-Pa-232

( $\alpha, p$ )  
Radionuclide Production Cross Section



92-U -235g  
92-U -235m1

91-Pa-232

Incident Energy (MeV)

12