

Program EVALPLOT  
(Version 2018-1)

by

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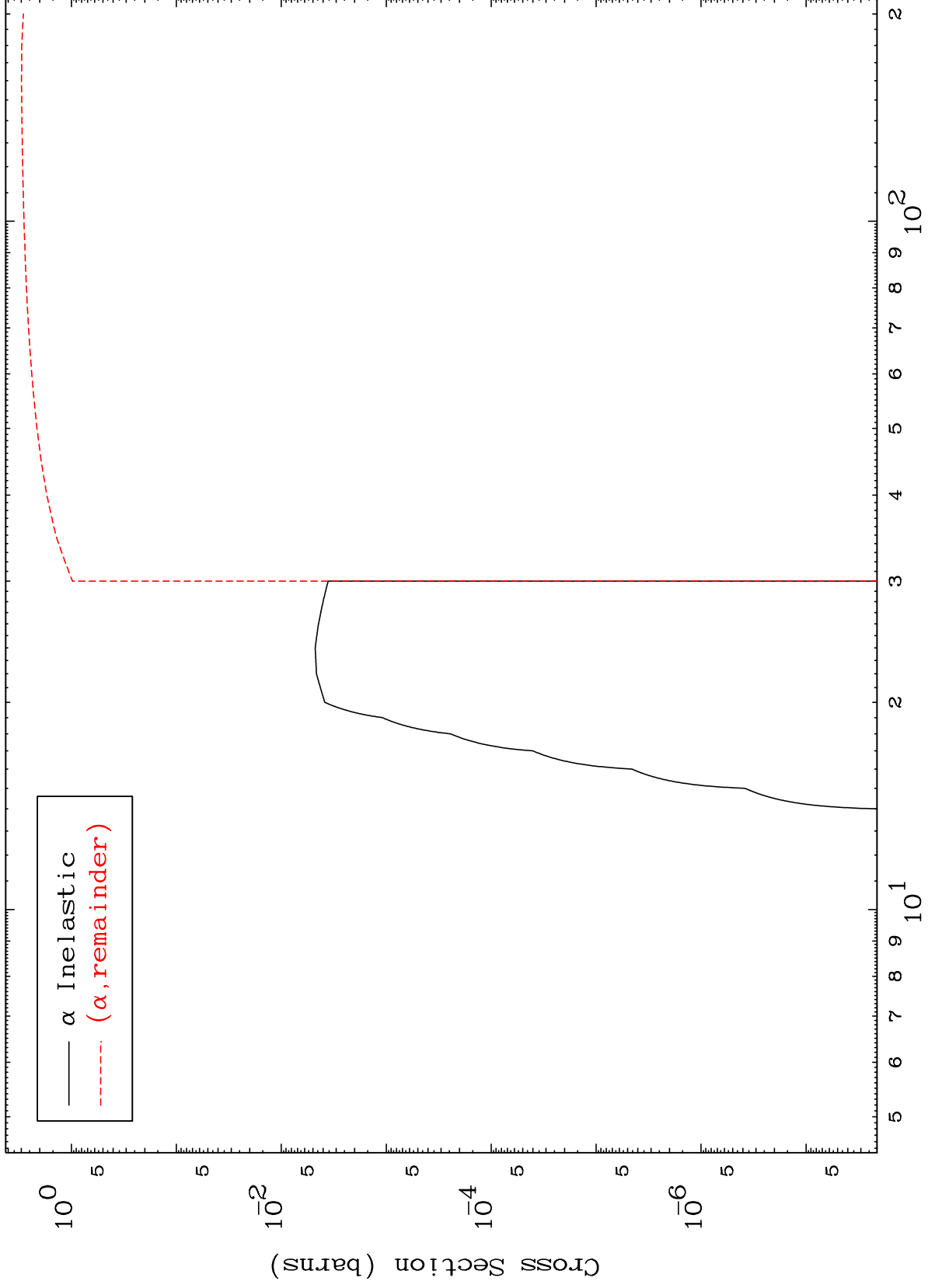
Press Mouse Button to Start

MAT 9143

$\alpha$  Major

91-Pa-235

0 Kelvin Cross Sections



1

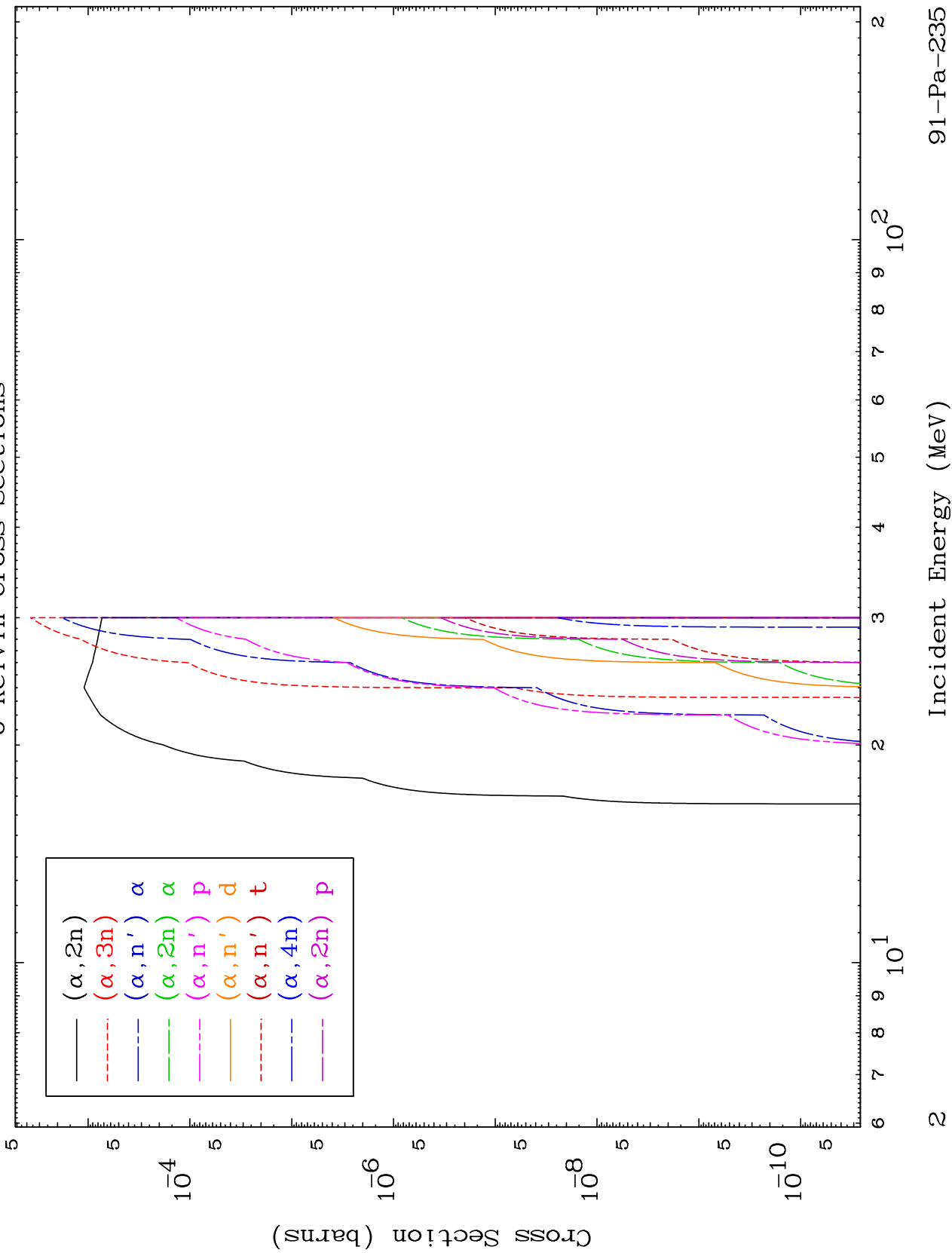
Incident Energy (MeV)

91-Pa-235

MAT 9143

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

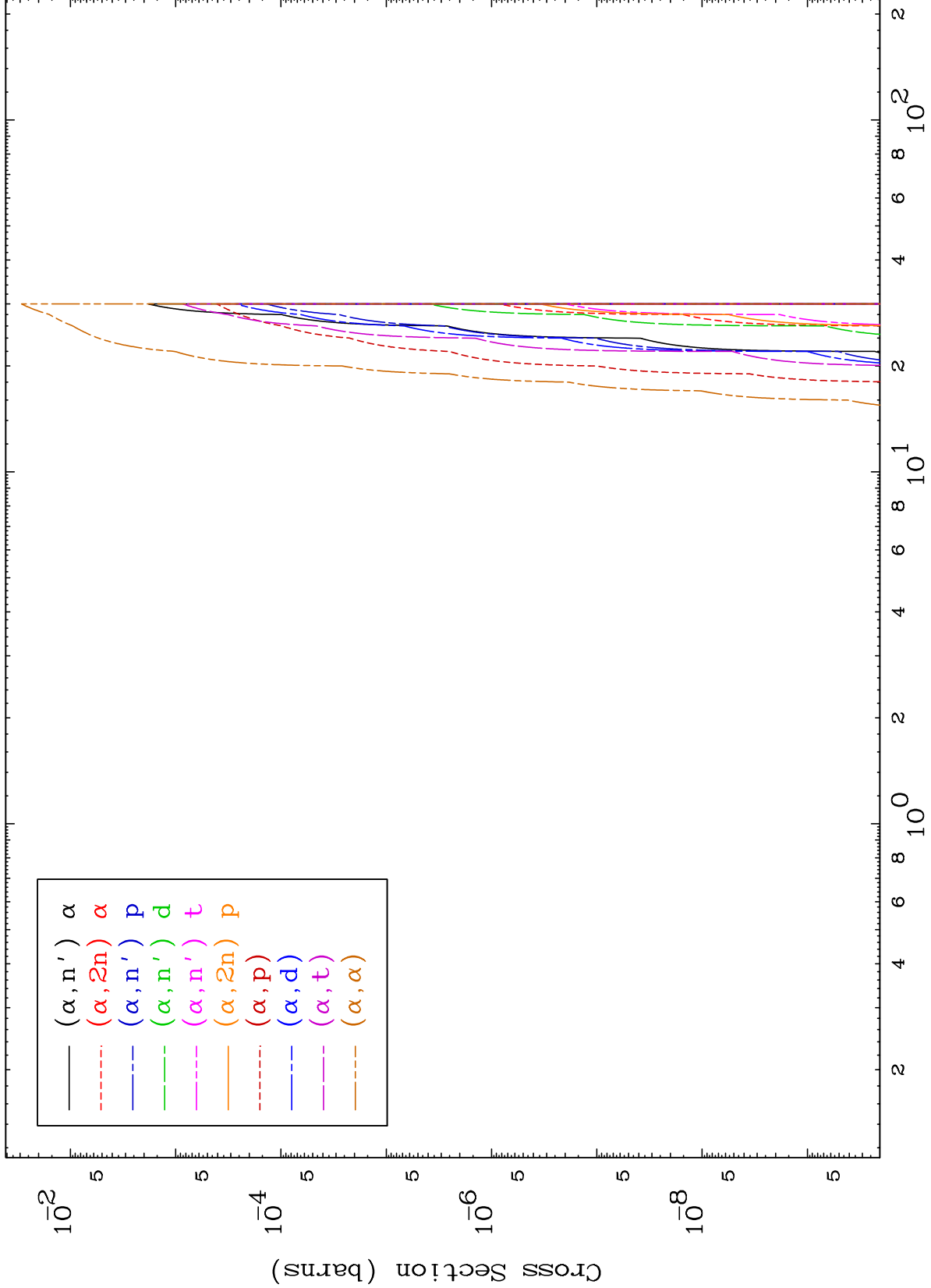
91-Pa-235



Incident Energy (MeV)

91-Pa-235

2

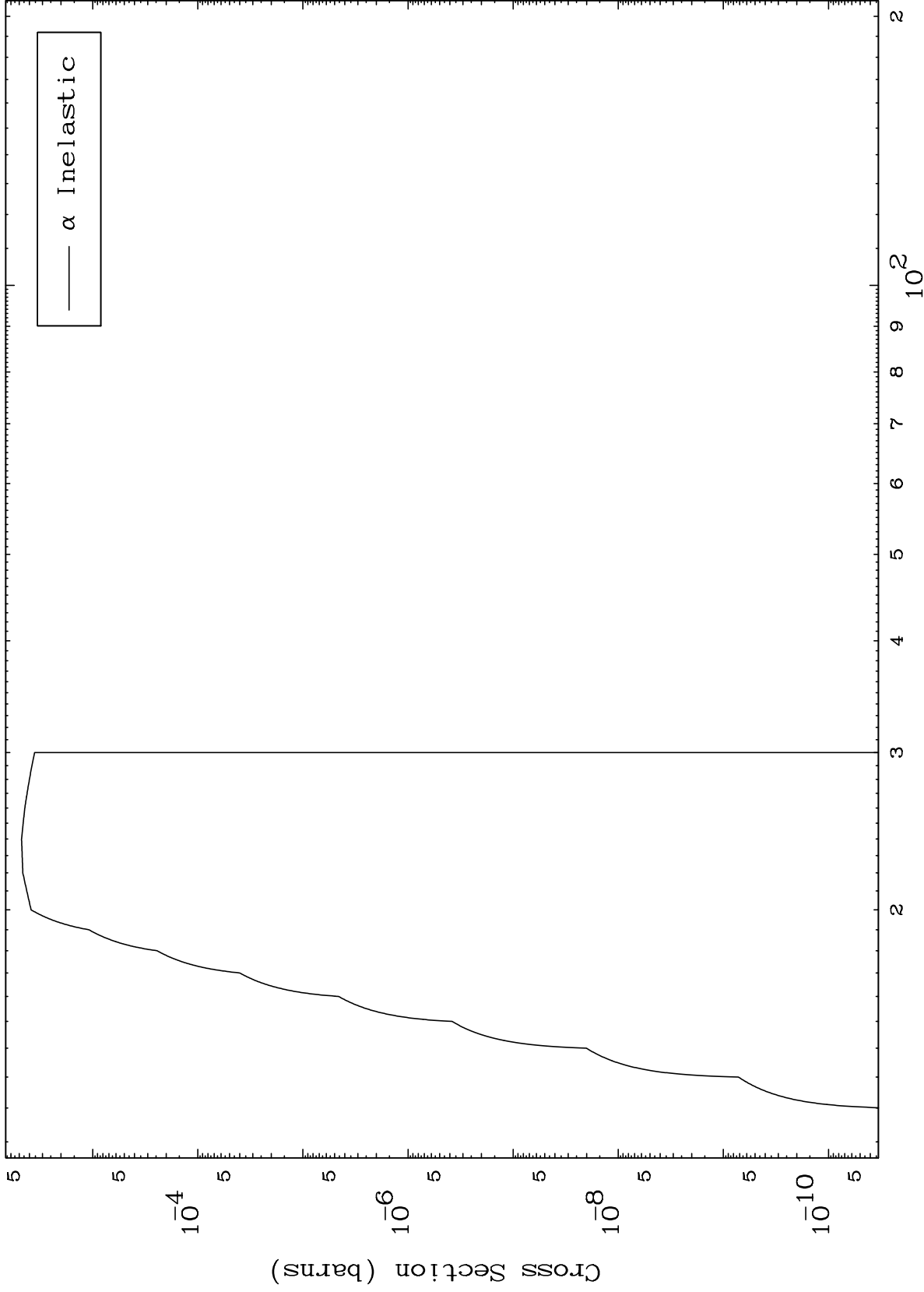


MAT 9143

( $\alpha, n'$ ) Level

91-Pa-235

0 Kelvin Cross Sections



Incident Energy (MeV)

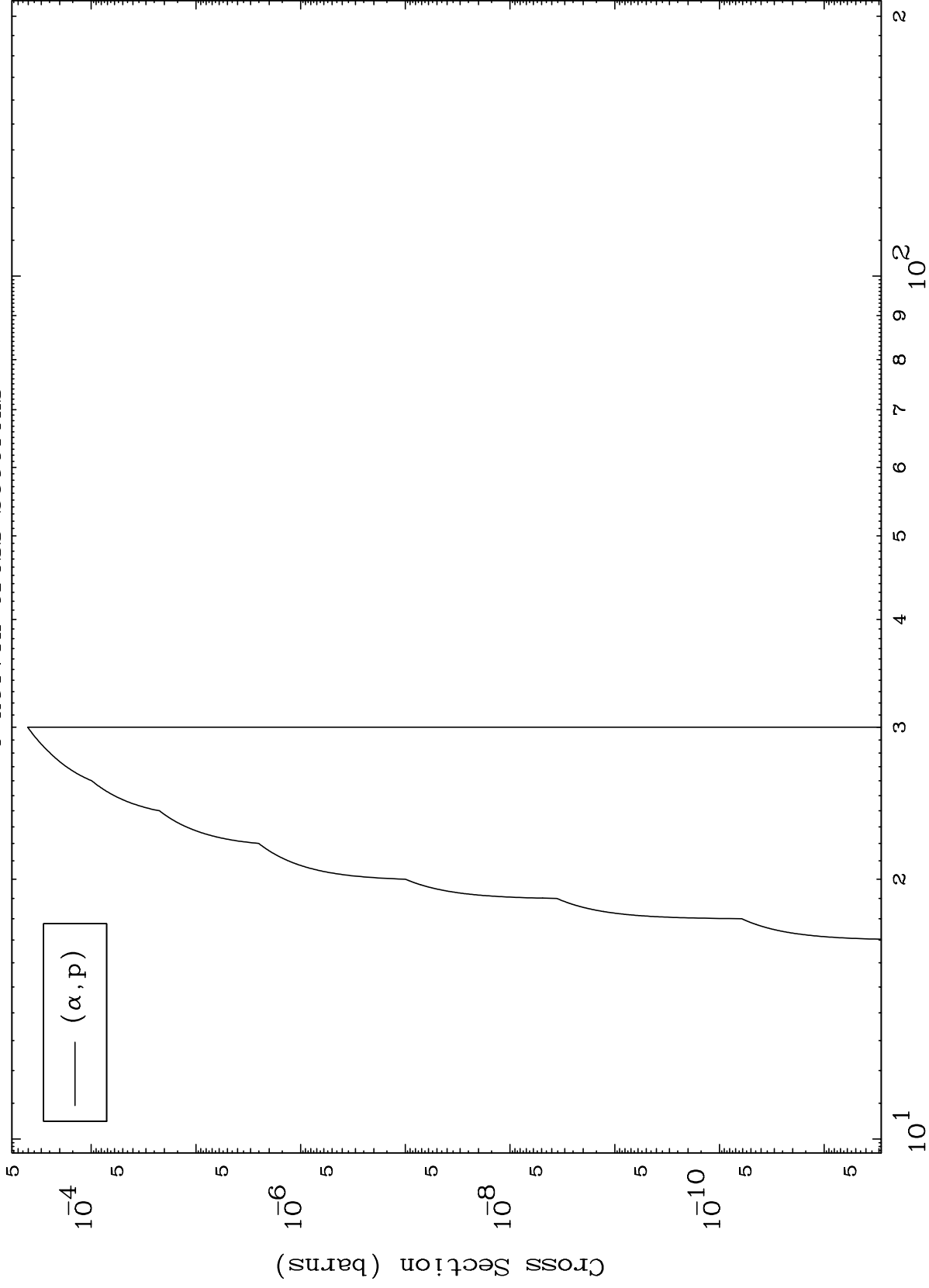
91-Pa-235

4

MAT 9143

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

91-Pa-235



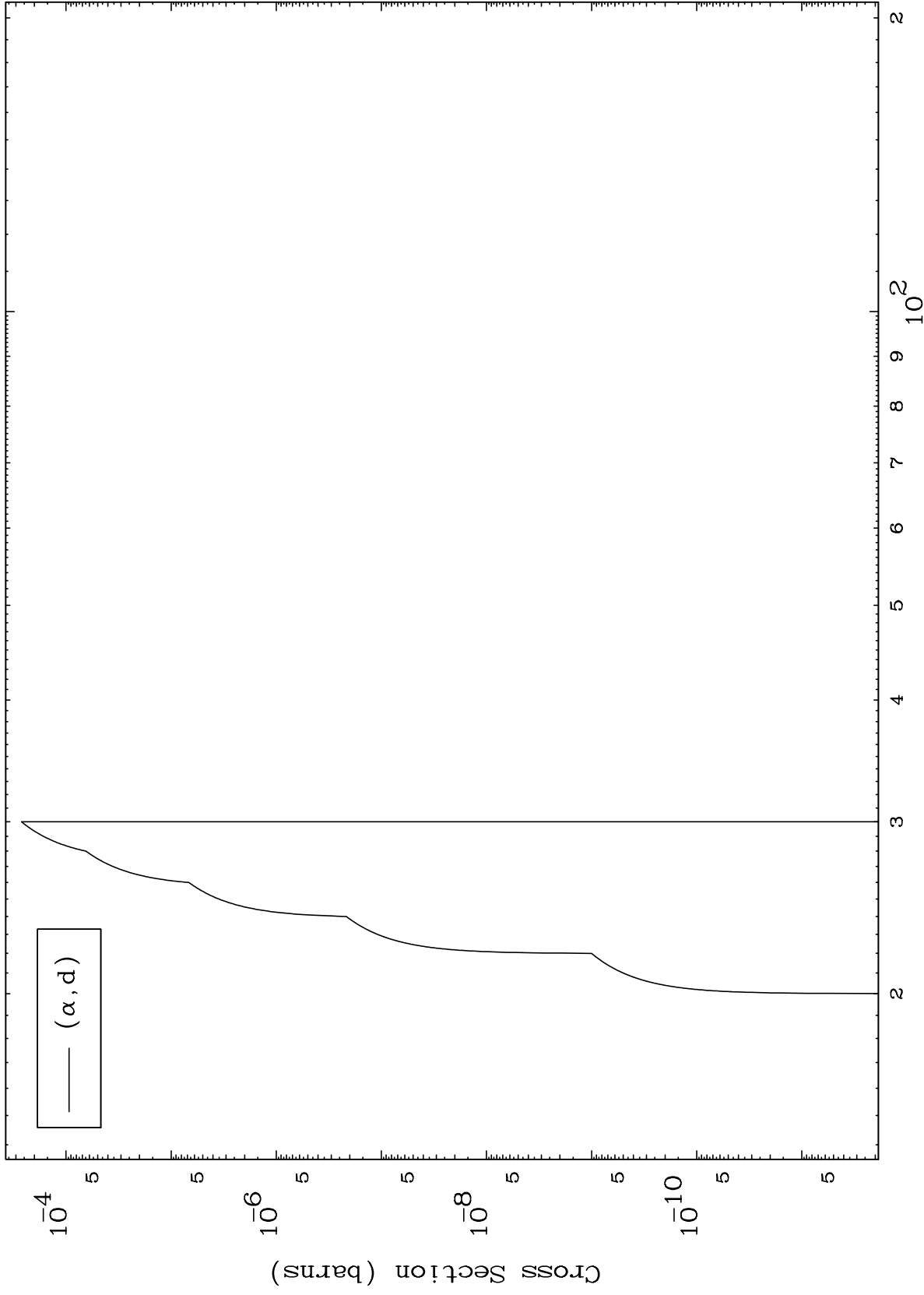
Incident Energy (MeV)

91-Pa-235

MAT 9143

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

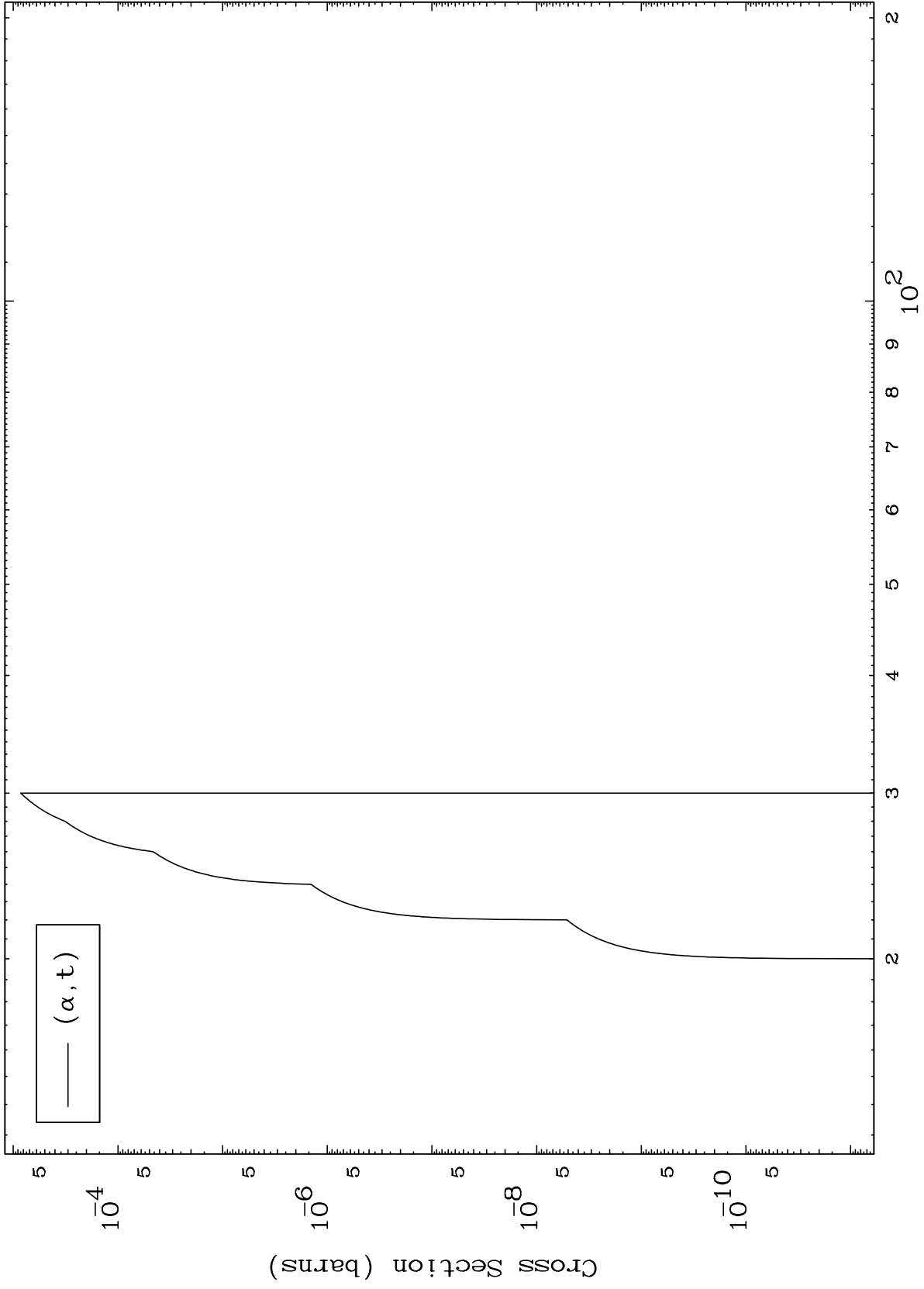
91-Pa-235



6

Incident Energy (MeV)

91-Pa-235

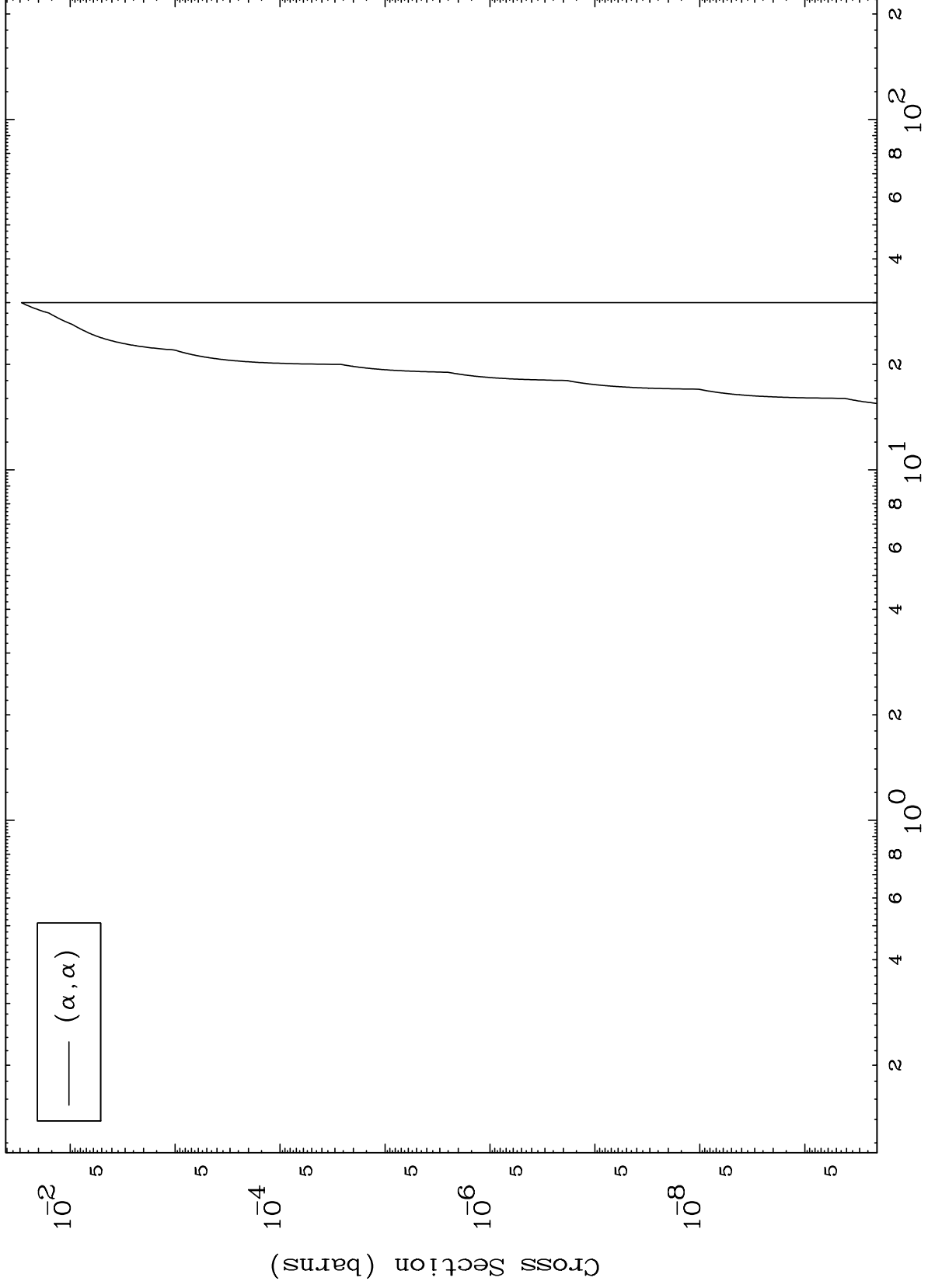




MAT 9143

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

91-Pa-235

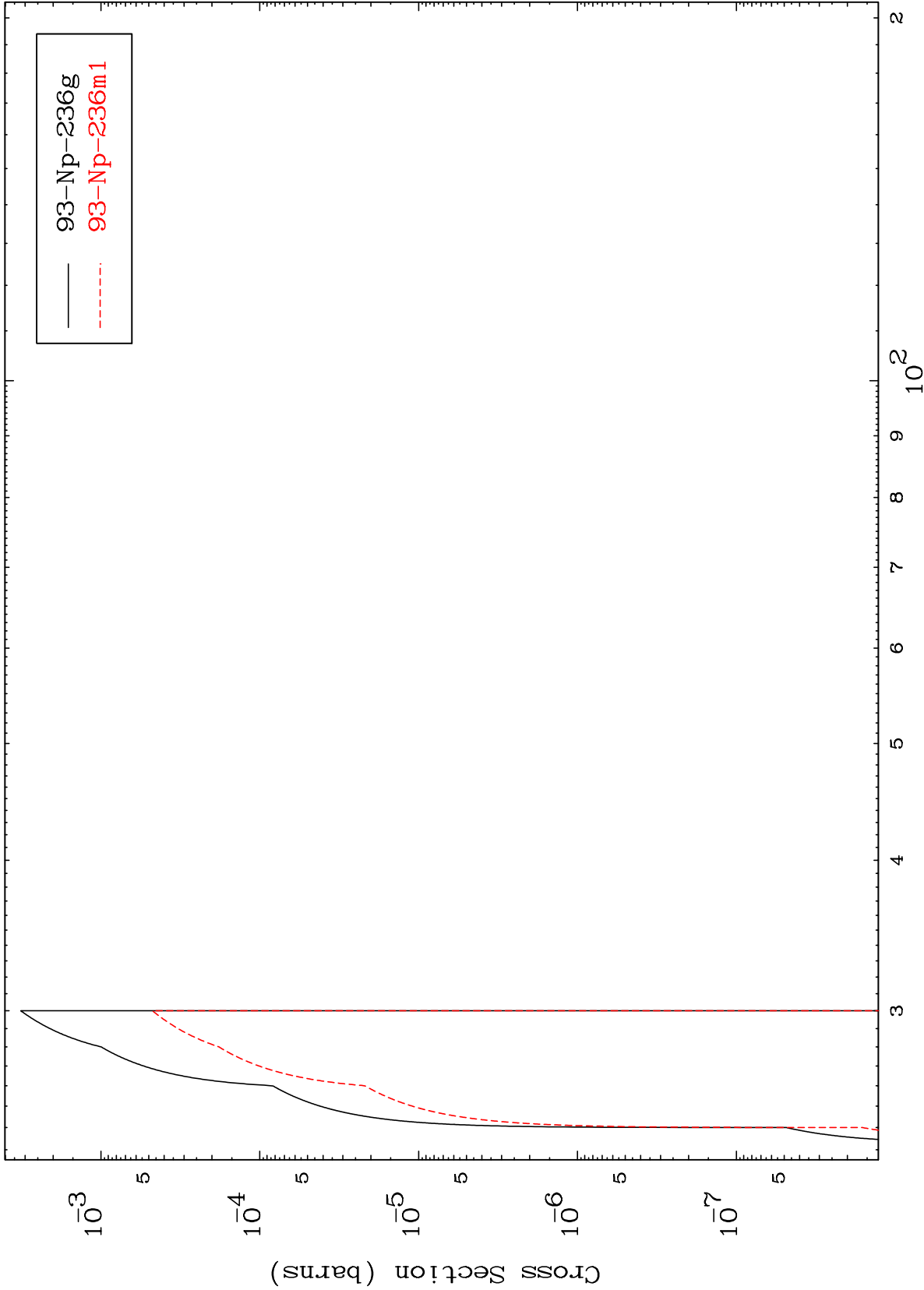


MAT 9143

( $\alpha, 3n$ )

91-Pa-235

Radionuclide Production Cross Section



9

Incident Energy (MeV)

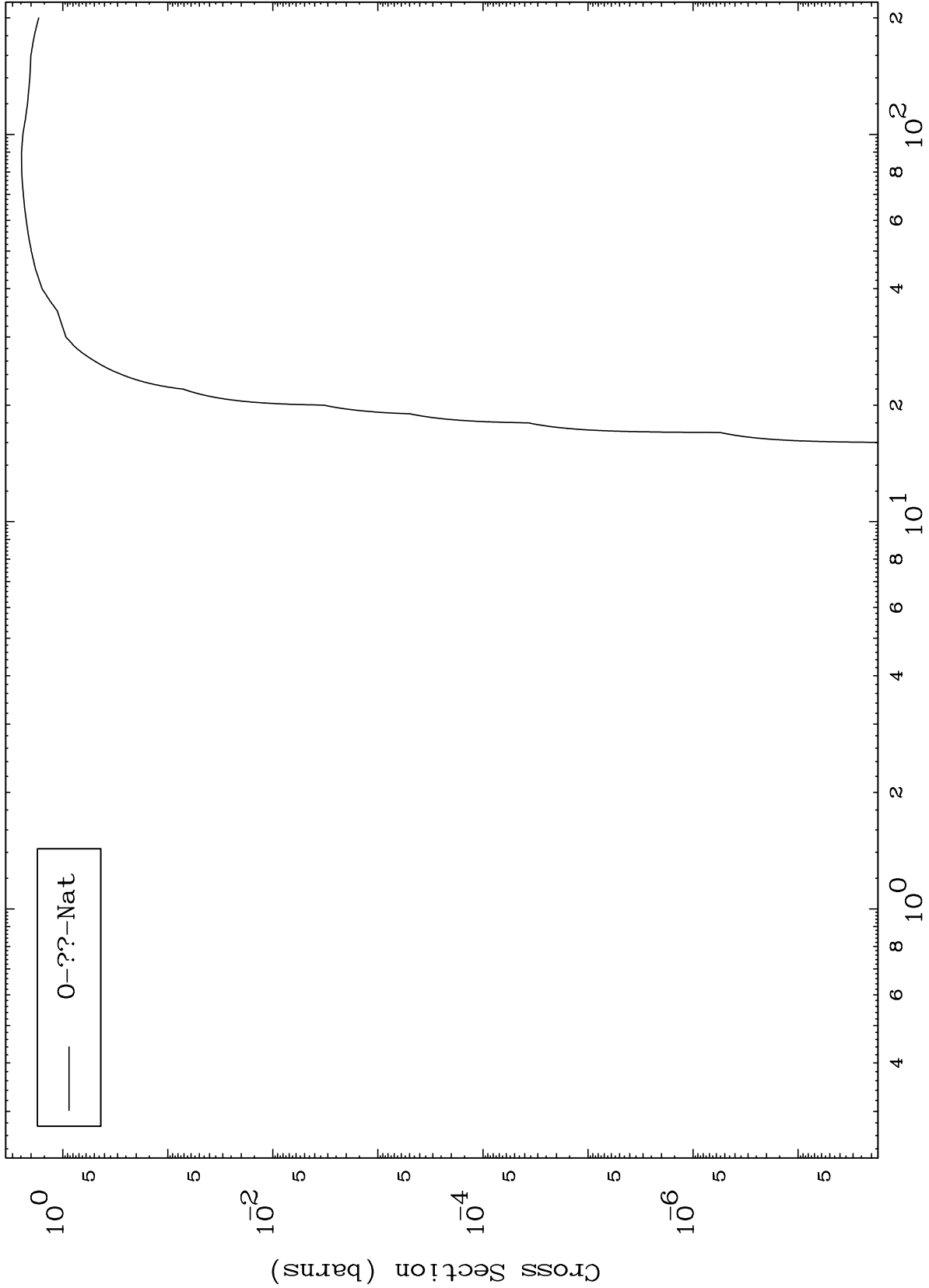
91-Pa-235

MAT 9143

91-Pa-235

$\alpha$  Fission

Radionuclide Production Cross Section



10

Incident Energy (MeV)

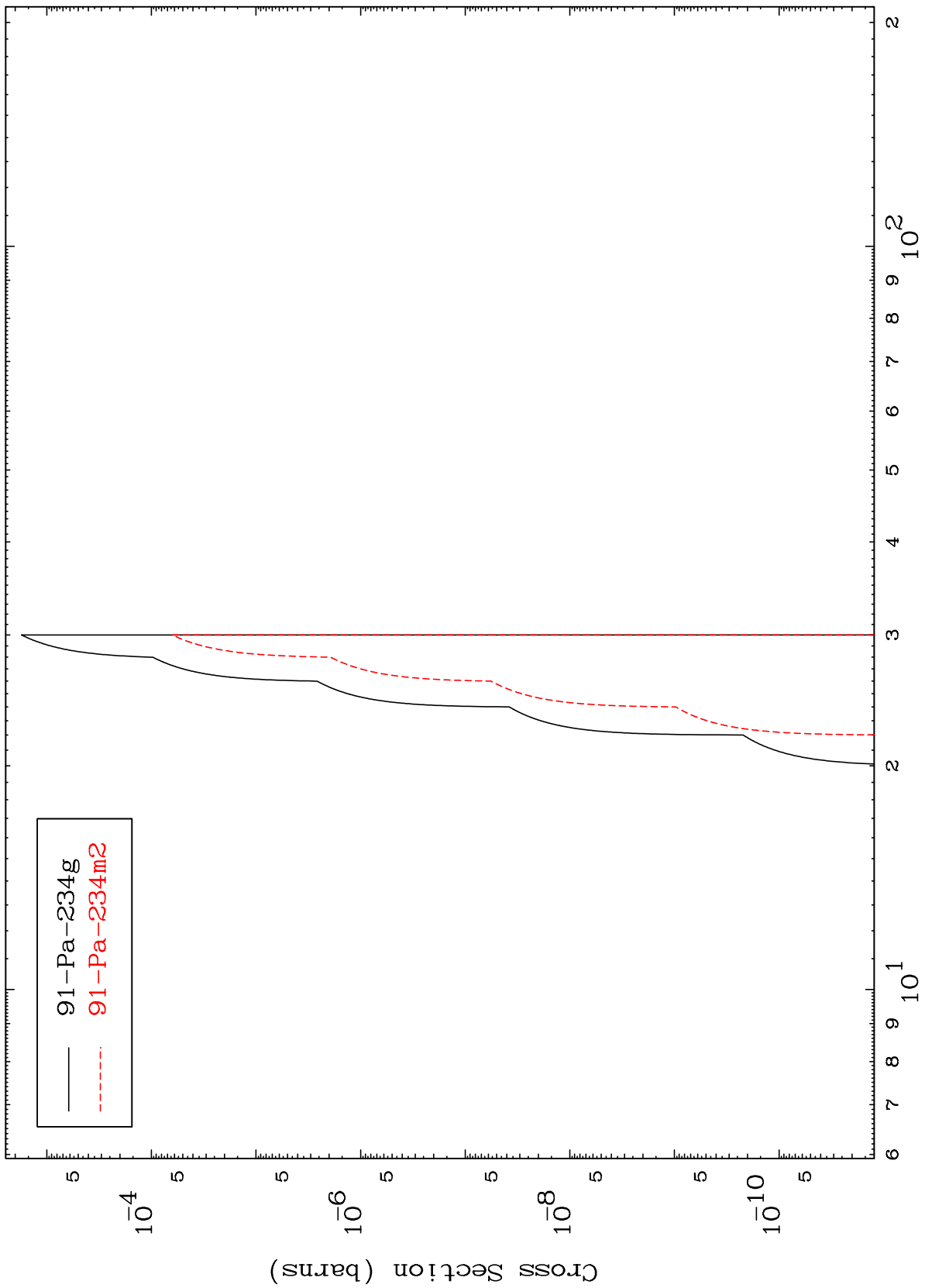
91-Pa-235

MAT 9143

$(\alpha, n')$   $\alpha$

91-Pa-235

Radionuclide Production Cross Section



11

Incident Energy (MeV)

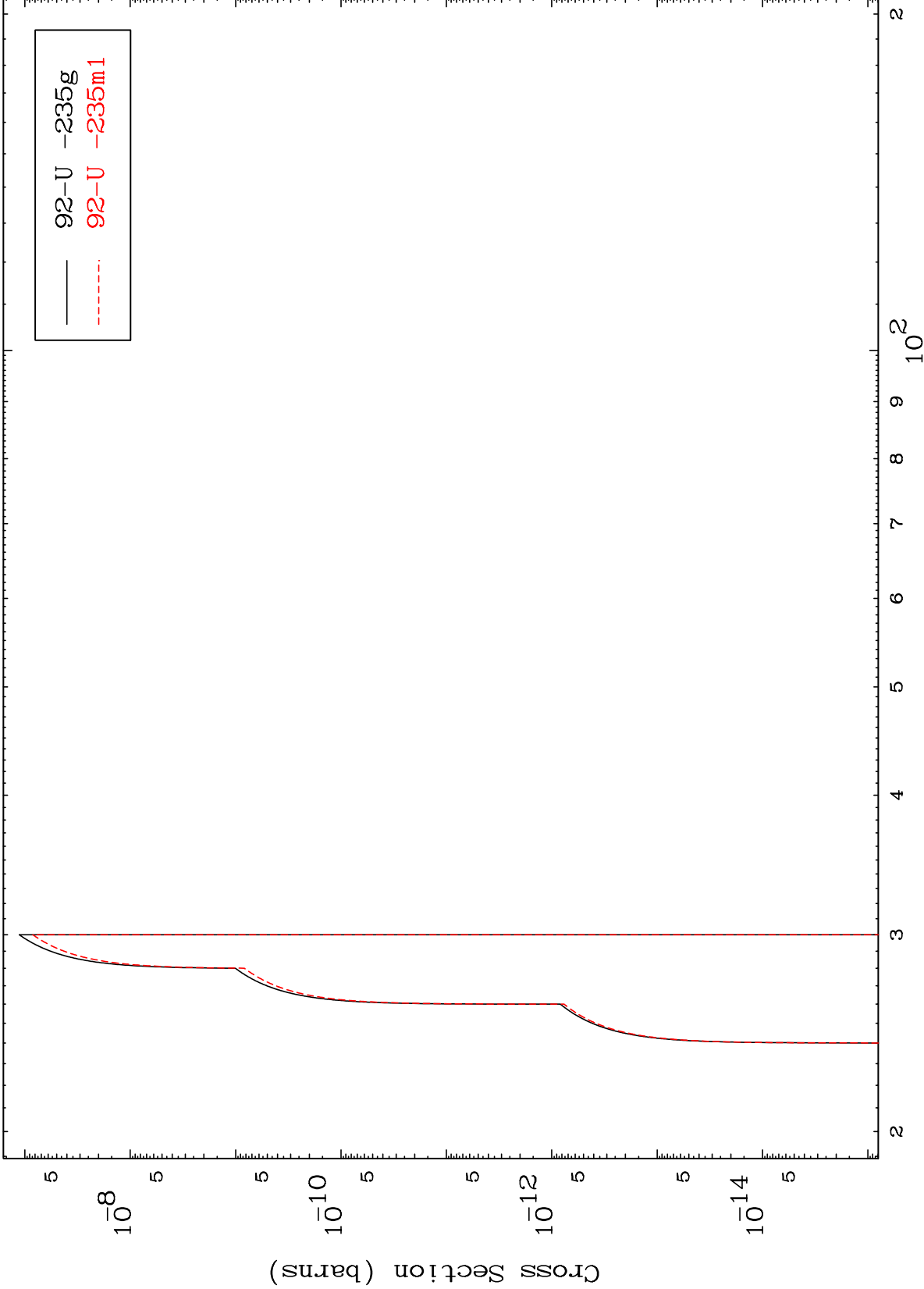
91-Pa-235

MAT 9143

( $\alpha, n'$ ) t

91-Pa-235

Radionuclide Production Cross Section



12

Incident Energy (MeV)

91-Pa-235