

Program EVALPLOT  
(Version 2018-1)

by

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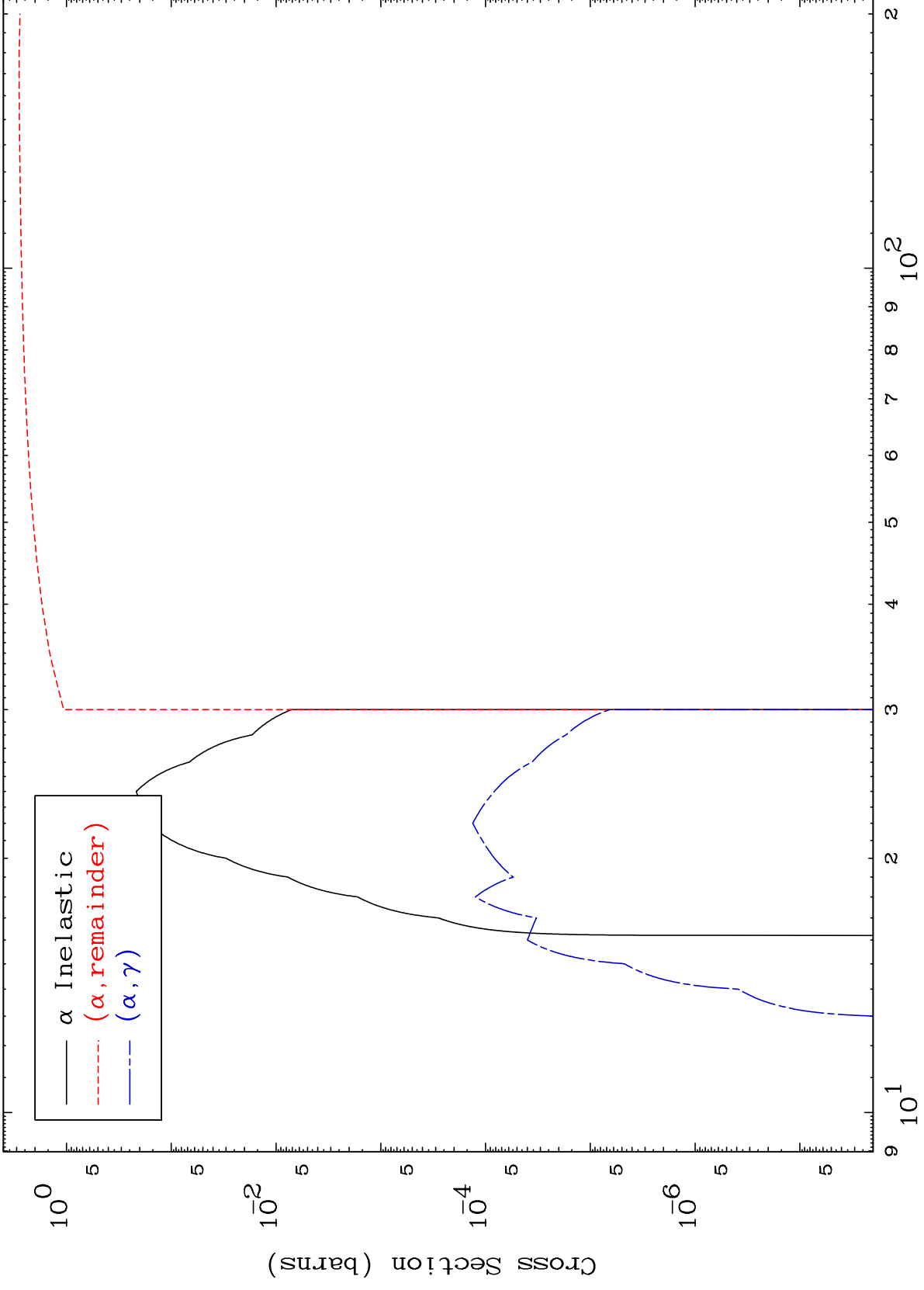
Press Mouse Button to Start

MAT 8437

$\alpha$  Major

84-Po-210

0 Kelvin Cross Sections



$\alpha$  Inelastic  
 $(\alpha, \text{remainder})$   
 $(\alpha, \gamma)$

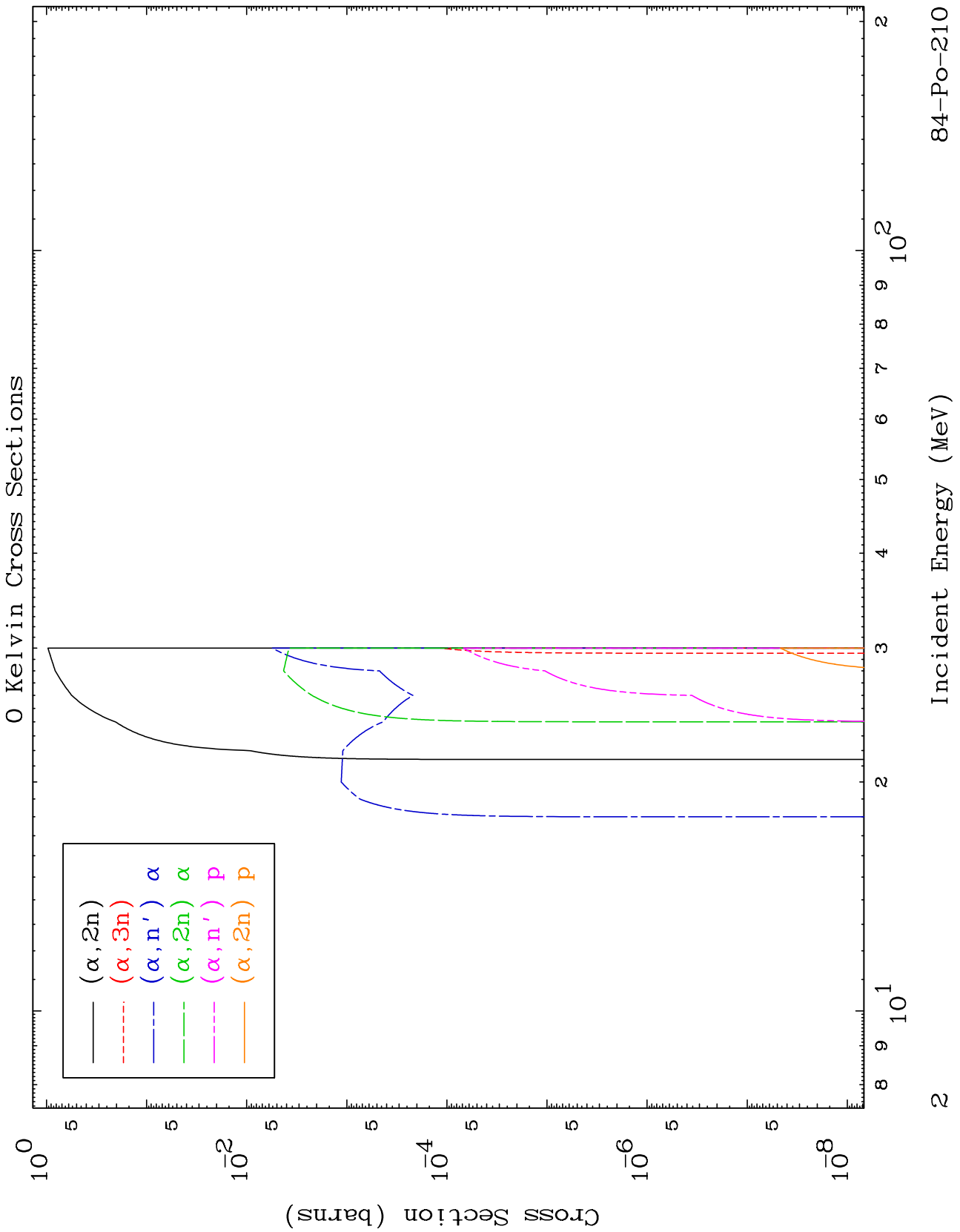
Incident Energy (MeV)

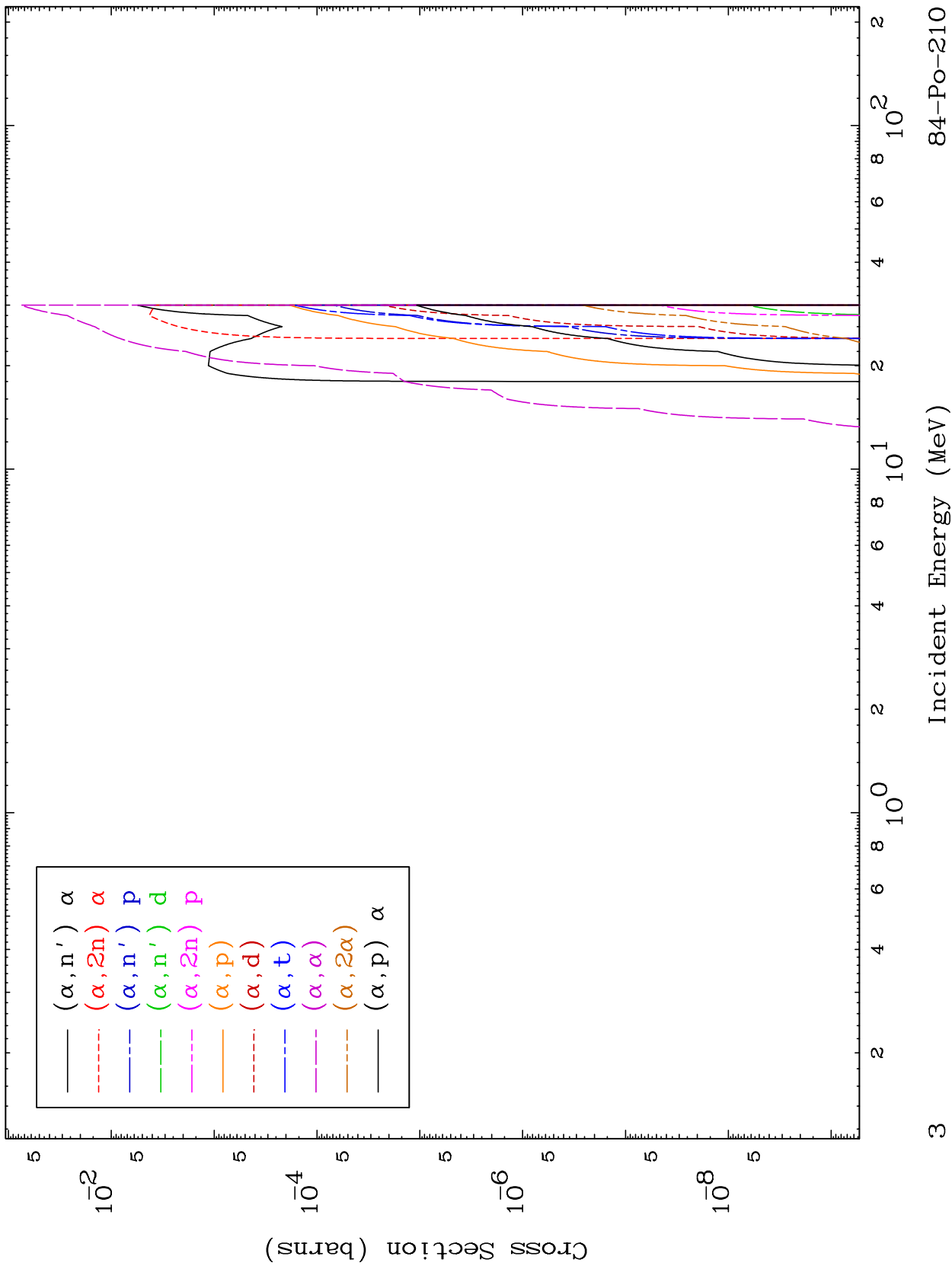
84-Po-210

MAT 8437

$\alpha$  Neutron Production

84-Po-210



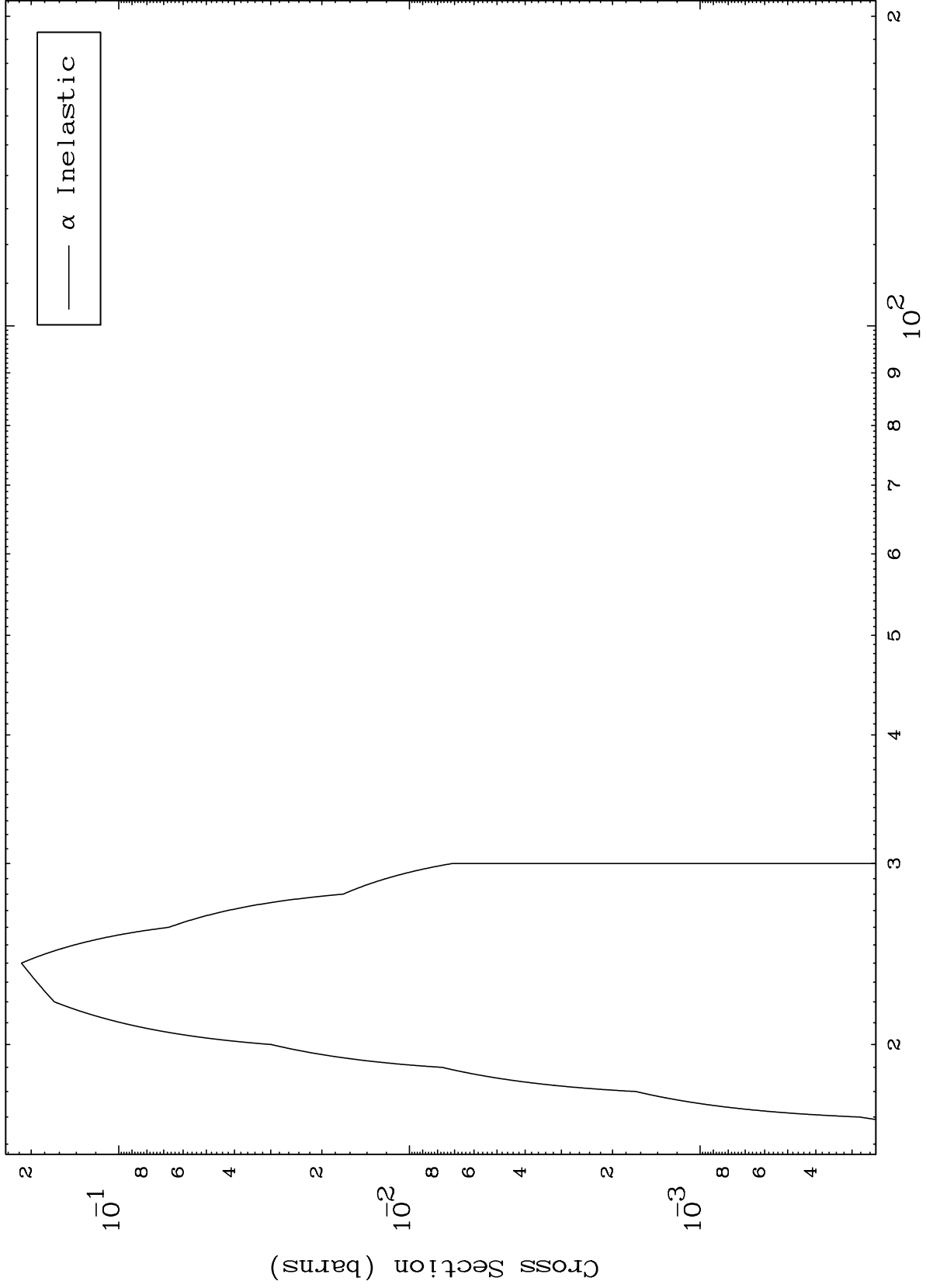


MAT 8437

( $\alpha, n'$ ) Level

84-Po-210

0 Kelvin Cross Sections



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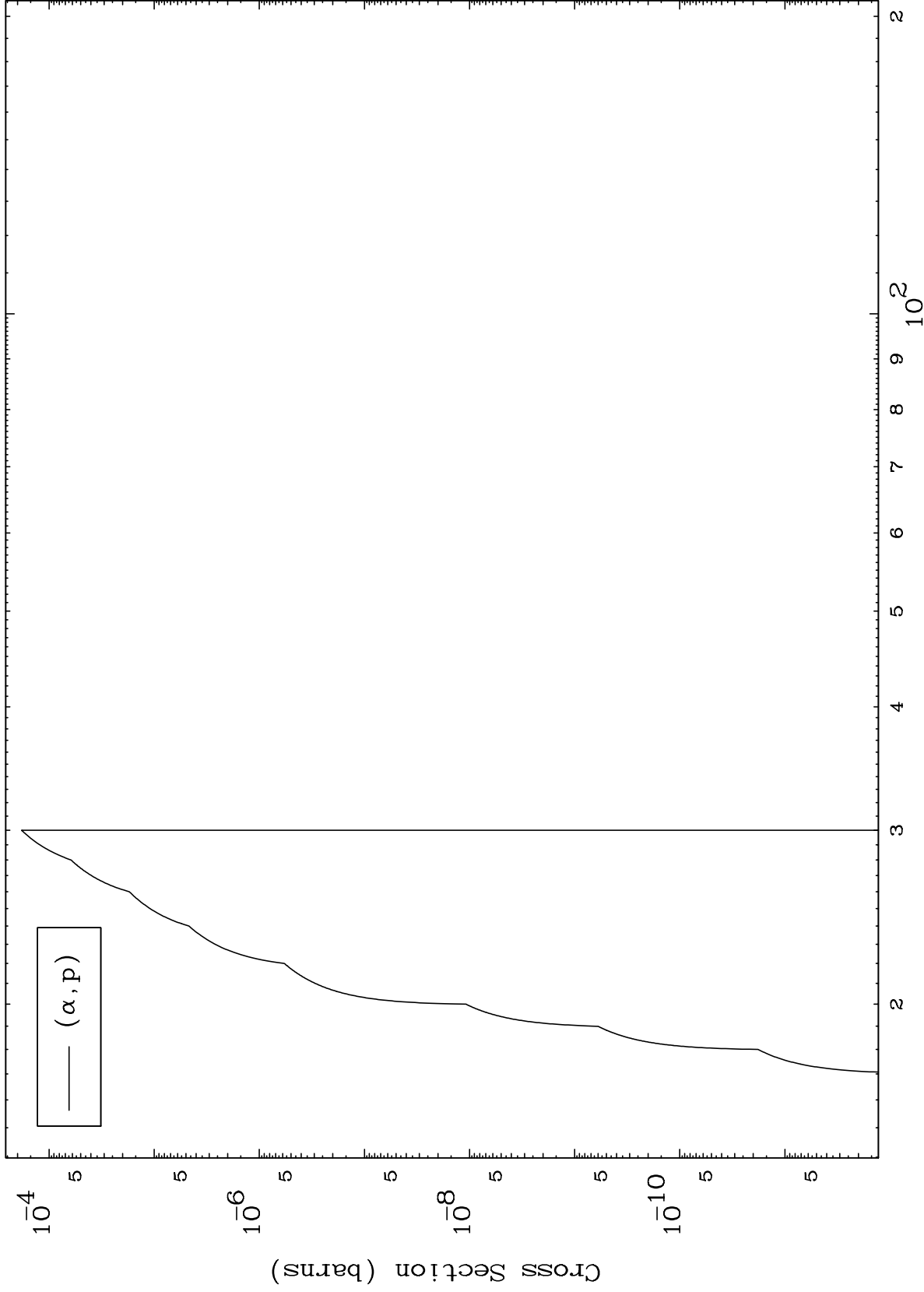
Incident Energy (MeV)

84-Po-210

MAT 8437

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

84-Po-210



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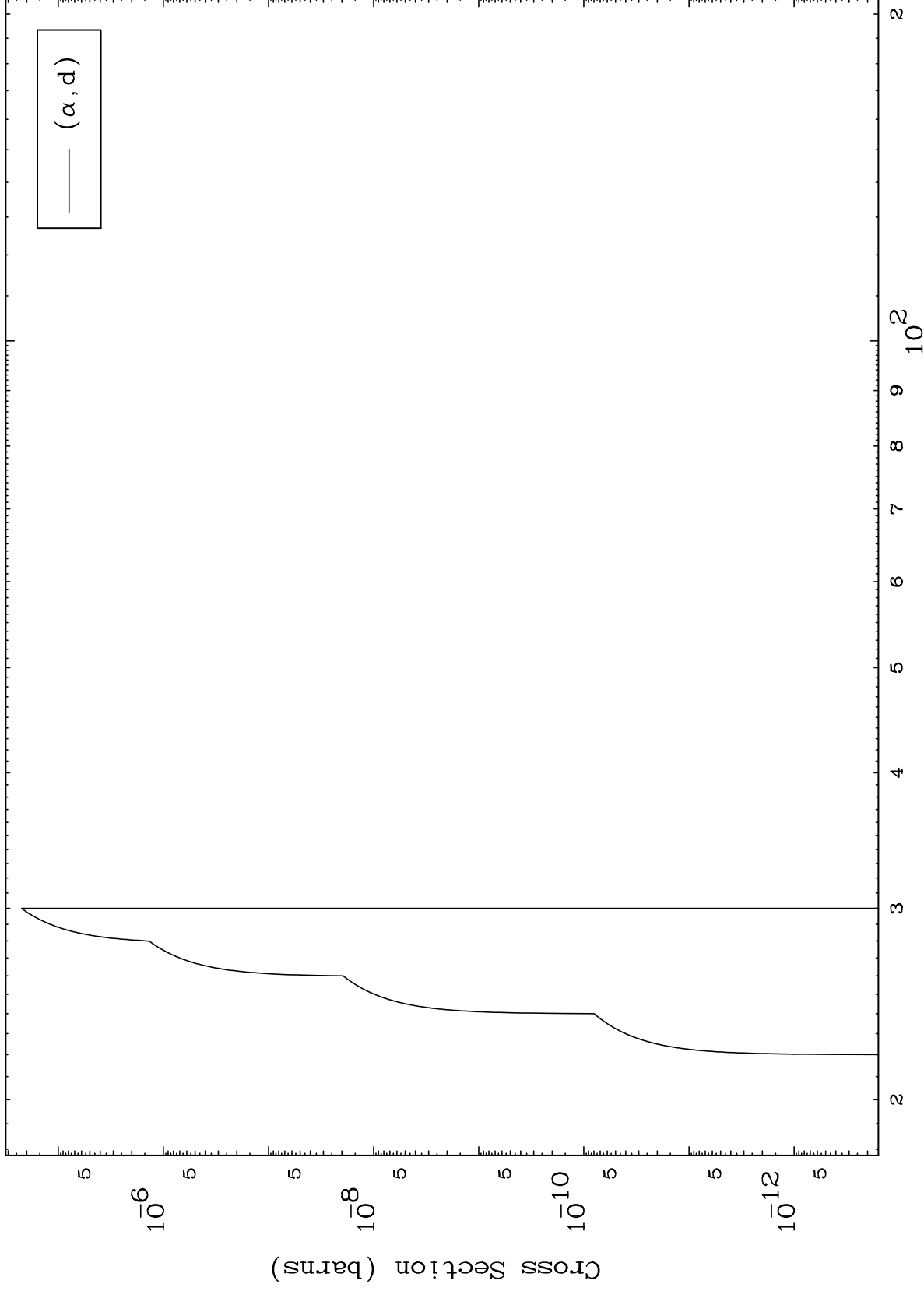
Incident Energy (MeV)

84-Po-210

MAT 8437

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

84-Po-210



6

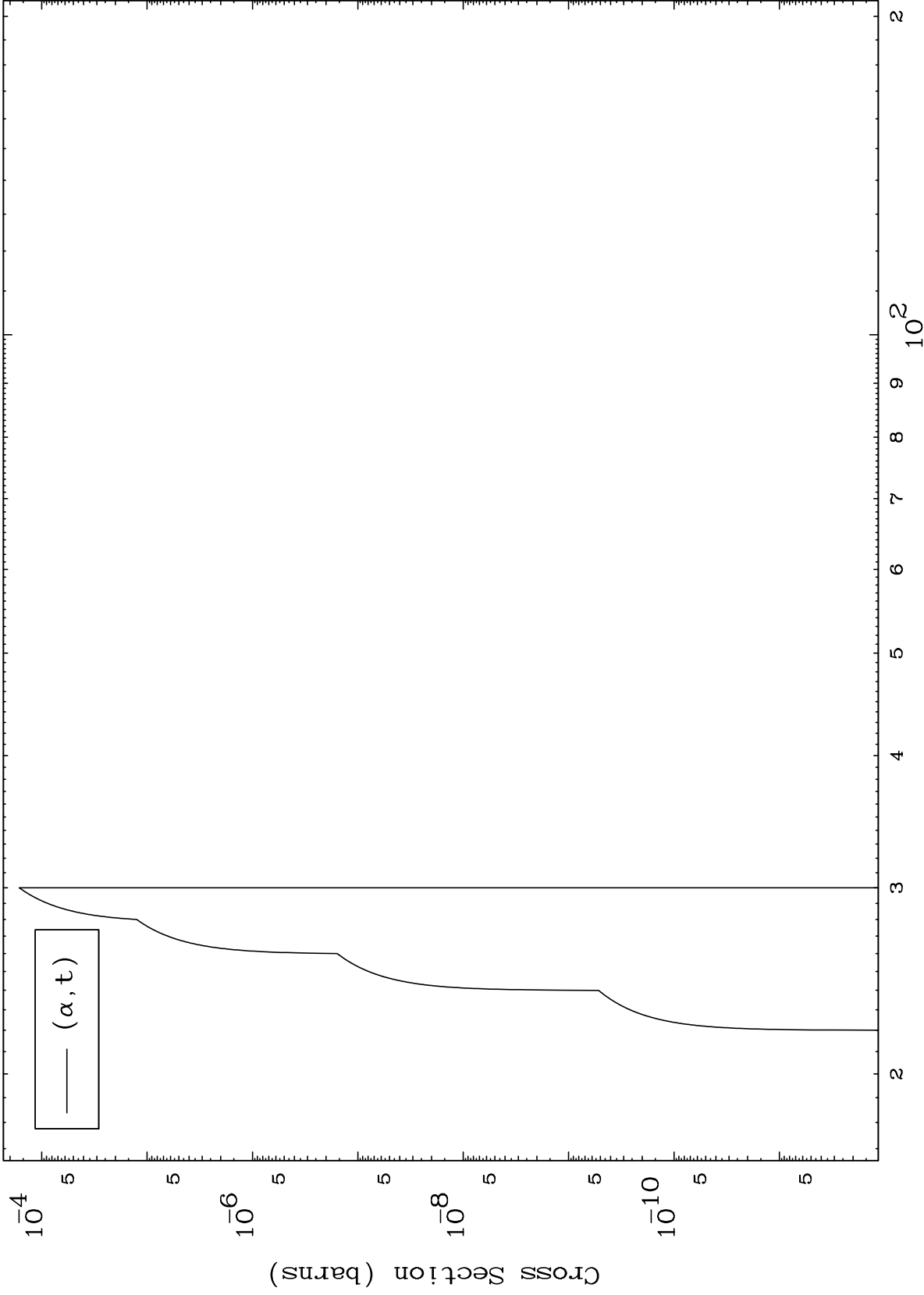
Incident Energy (MeV)

84-Po-210

MAT 8437

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

84-Po-210



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Incident Energy (MeV)

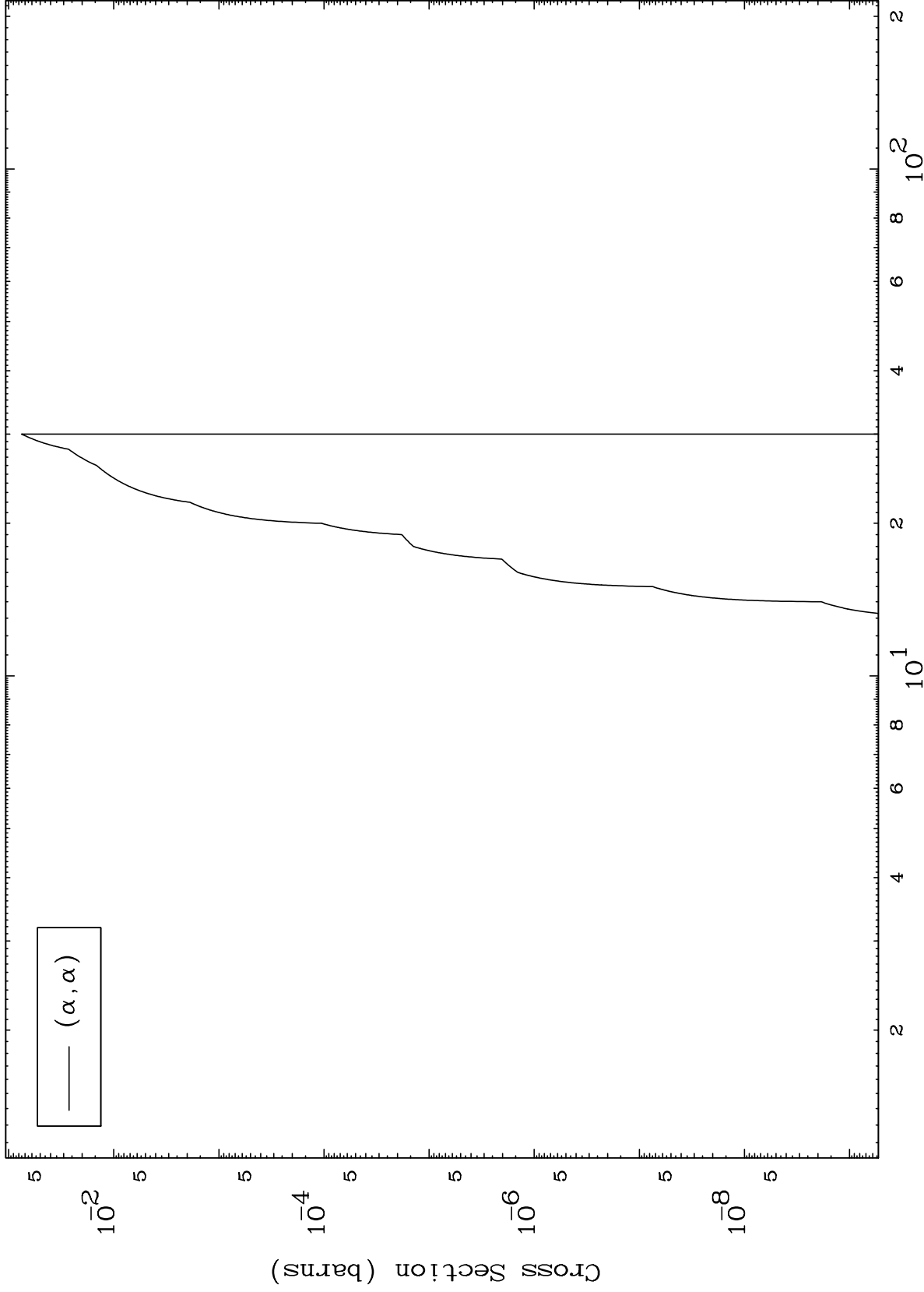
84-Po-210



MAT 8437

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

84-Po-210



8

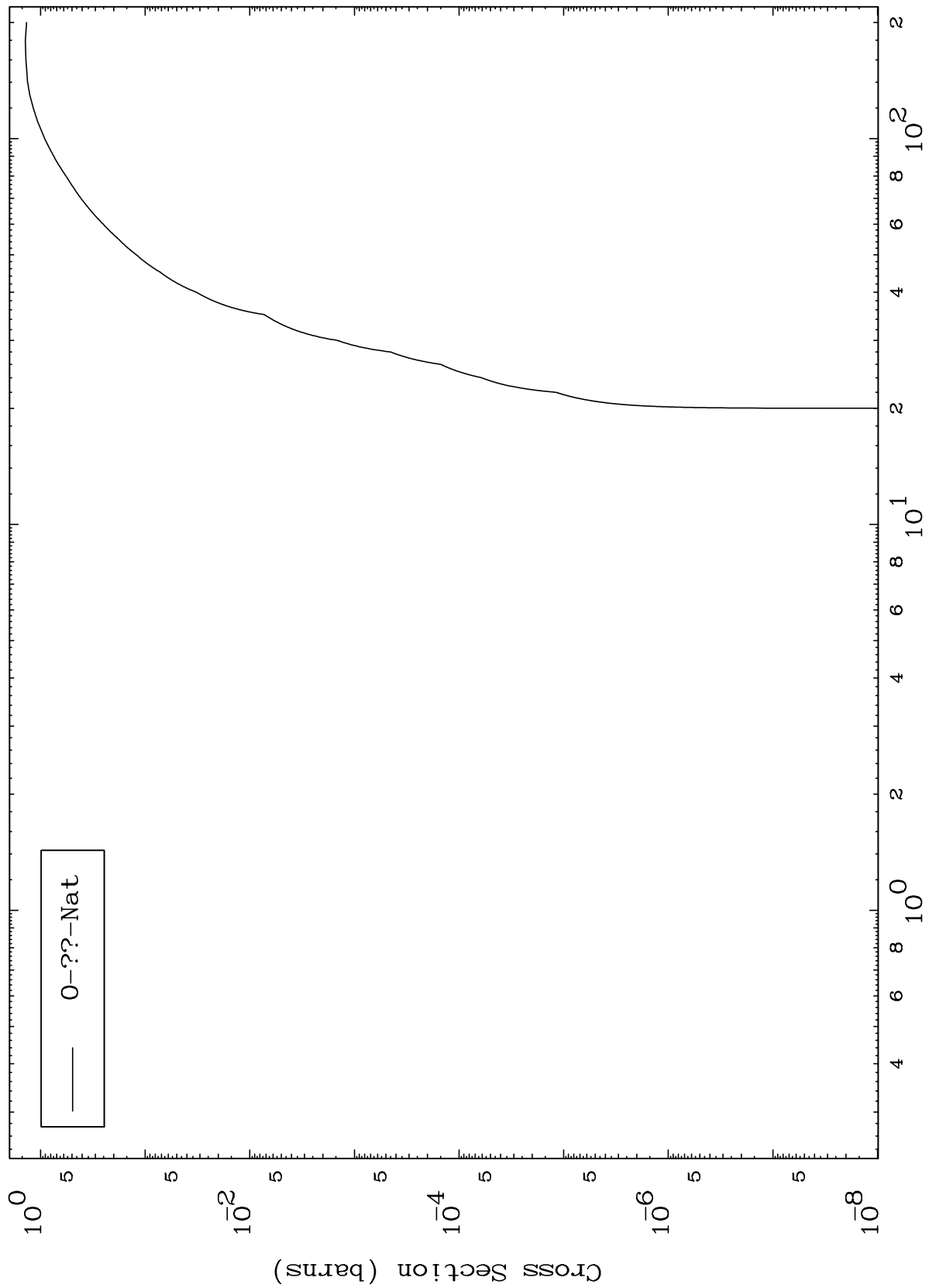
Incident Energy (MeV)

84-Po-210

MAT 8437

84-Po-210

$\alpha$  Fission  
Radionuclide Production Cross Section



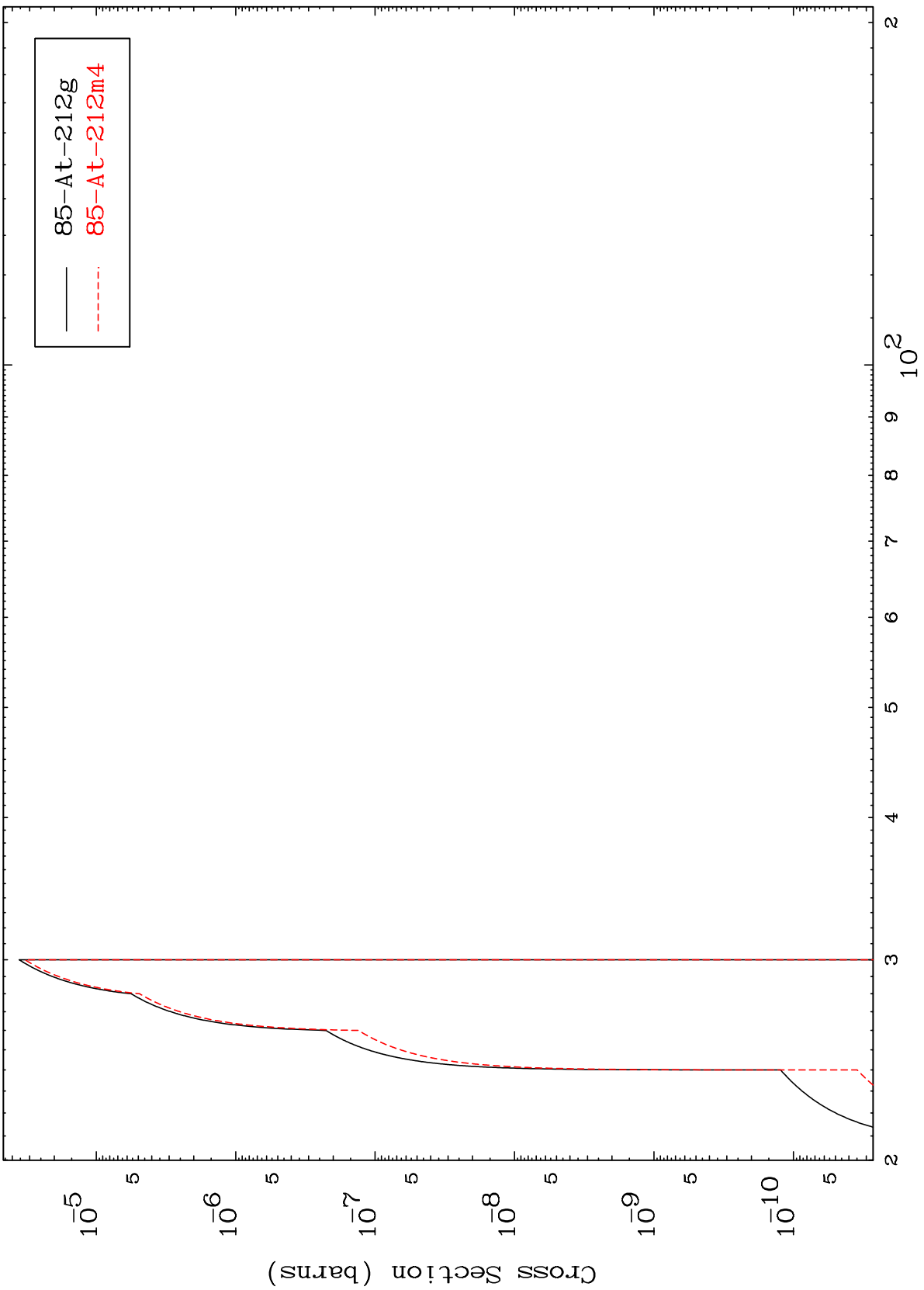
0-??-Nat

MAT 8437

$(\alpha, n')$  p

84-Po-210

Radionuclide Production Cross Section



10

Incident Energy (MeV)

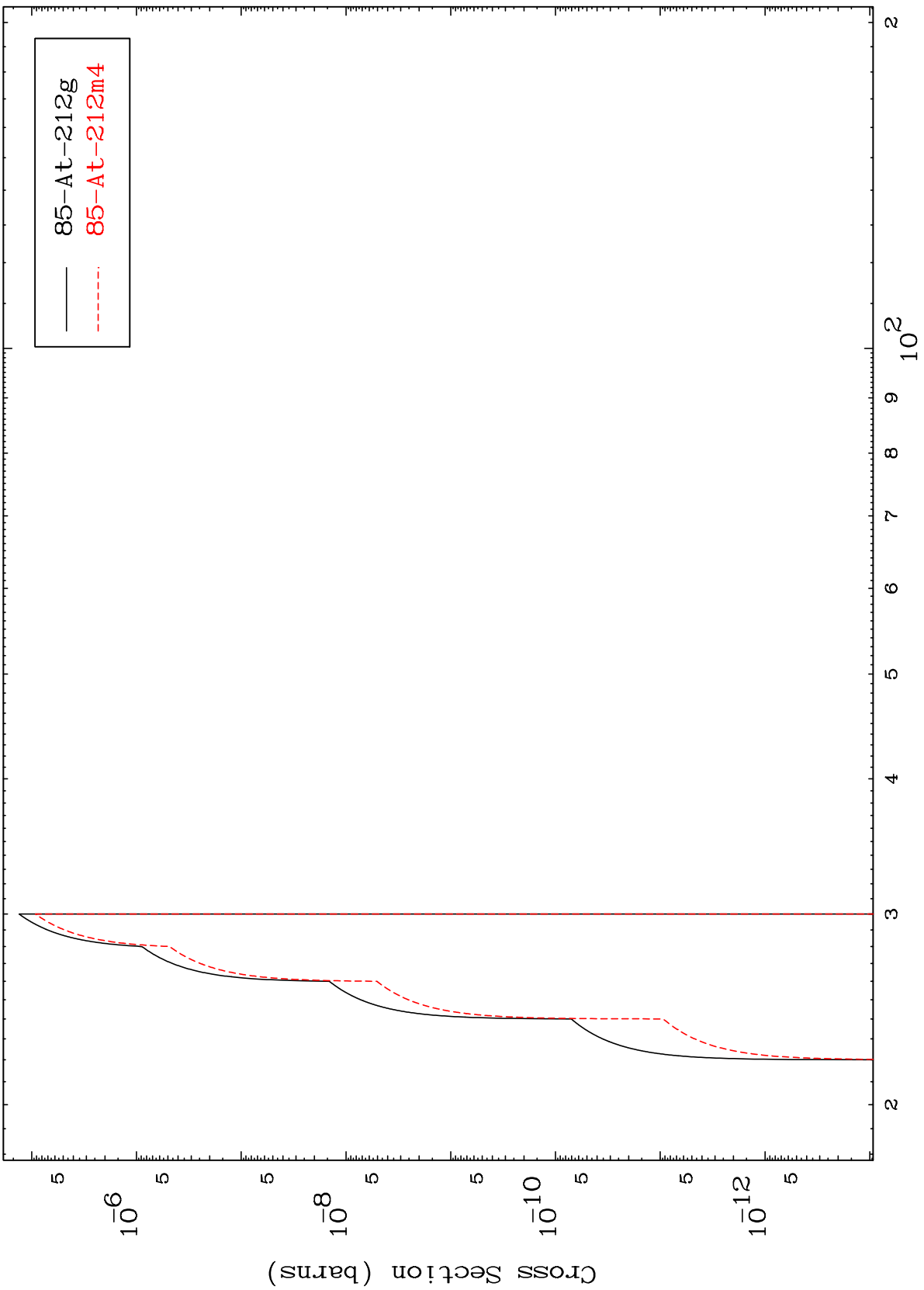
84-Po-210

MAT 8437

( $\alpha, d$ )

84-Po-210

Radionuclide Production Cross Section



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Incident Energy (MeV)

84-Po-210