

Program EVALPLOT  
(Version 2018-1)

by

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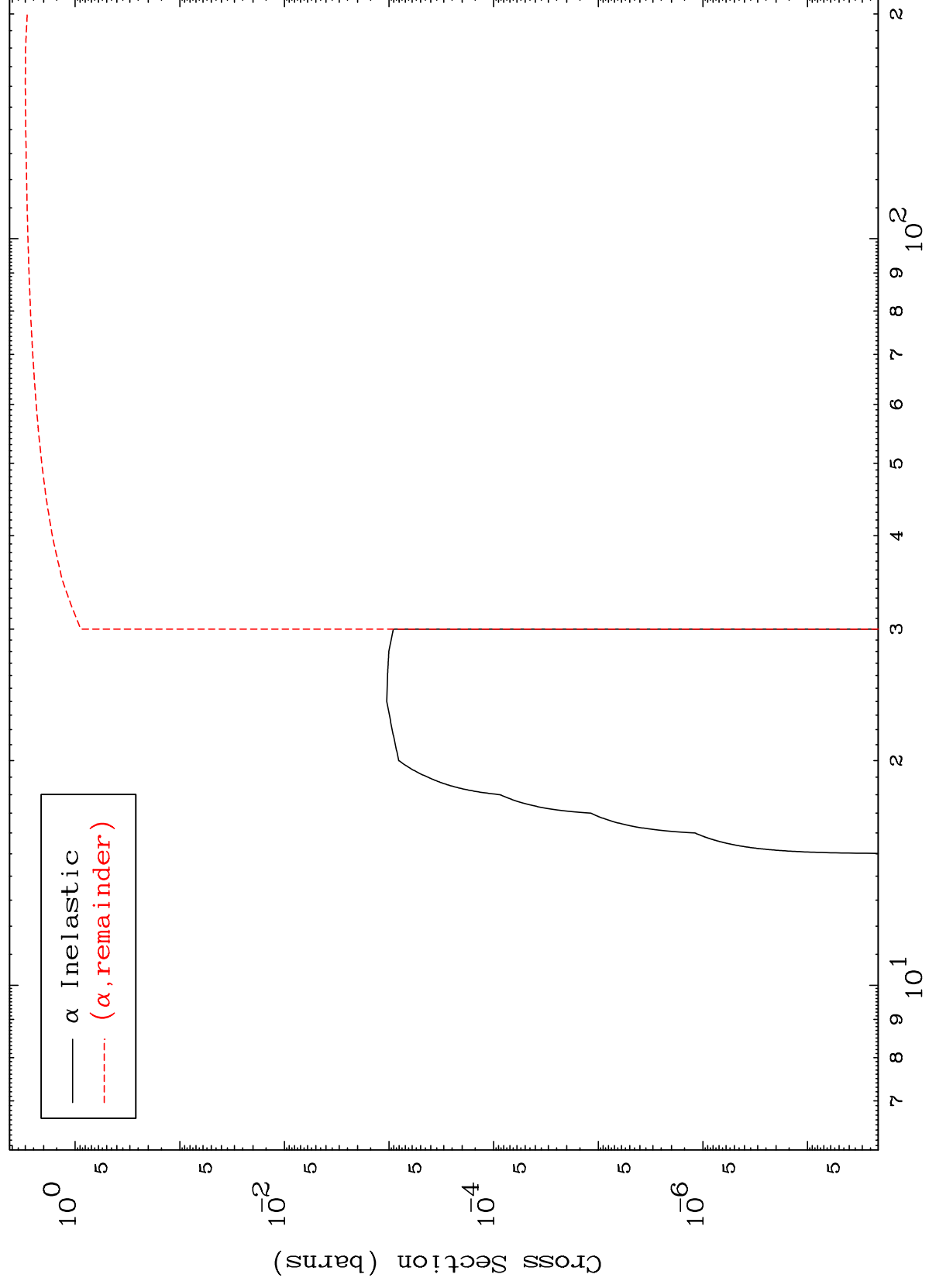
Press Mouse Button to Start

MAT 9434

0 Kelvin

$\alpha$  Major Cross Sections

94-Pu-238

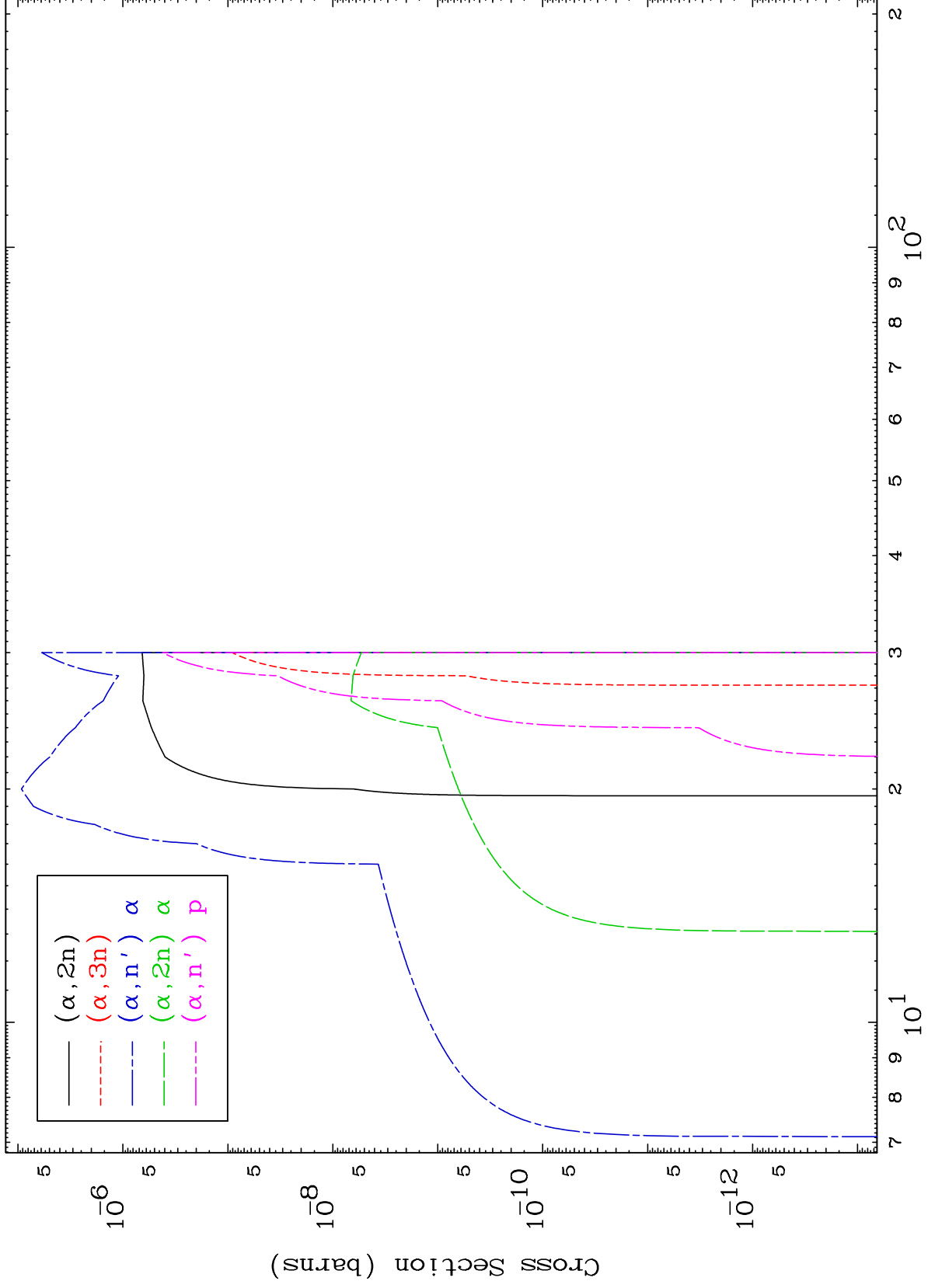


—  $\alpha$  Inelastic  
- - - ( $\alpha$ , remainder)

MAT 9434

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

94-Pu-238



2

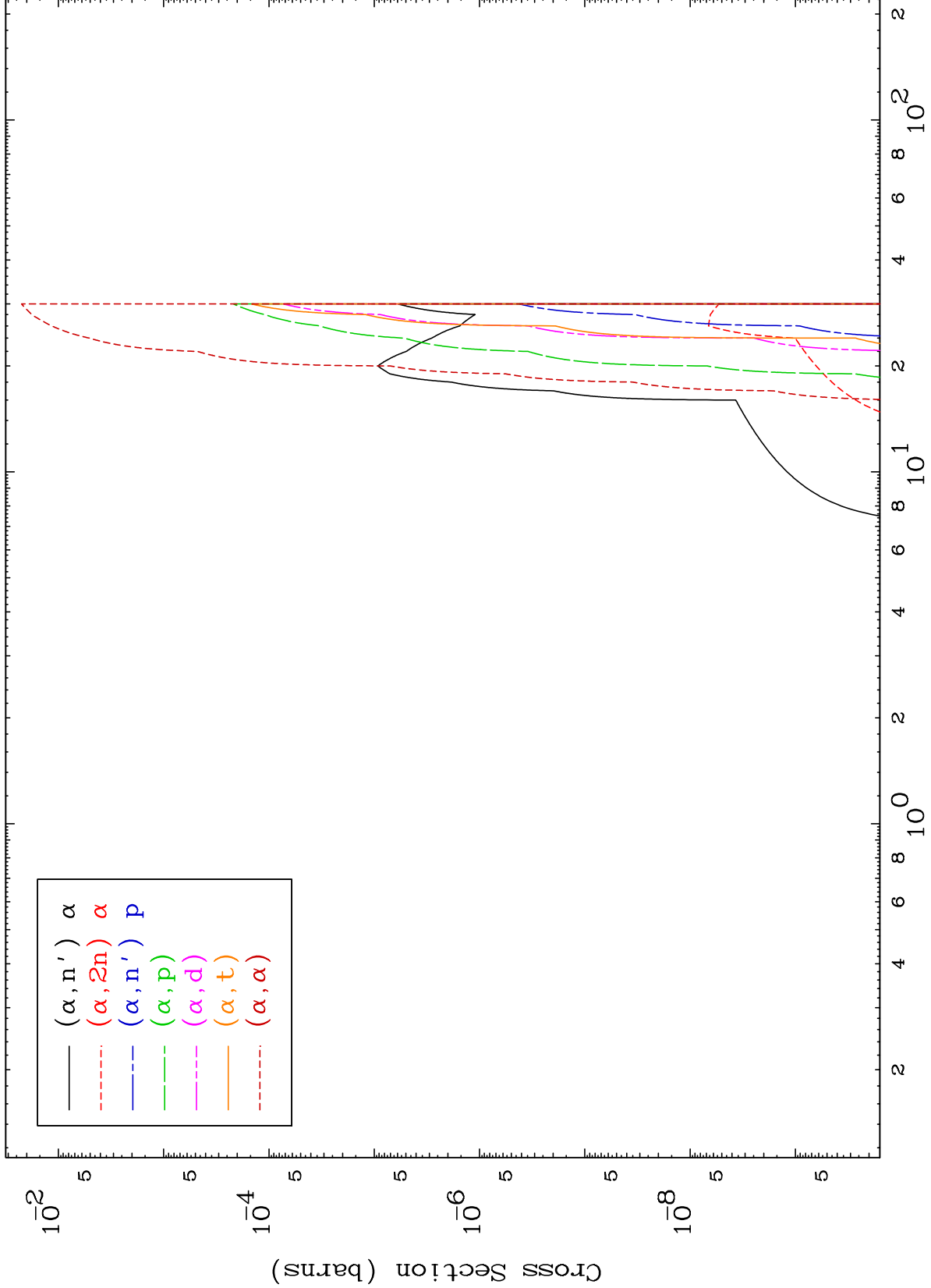
Incident Energy (MeV)

94-Pu-238

MAT 9434

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

94-Pu-238

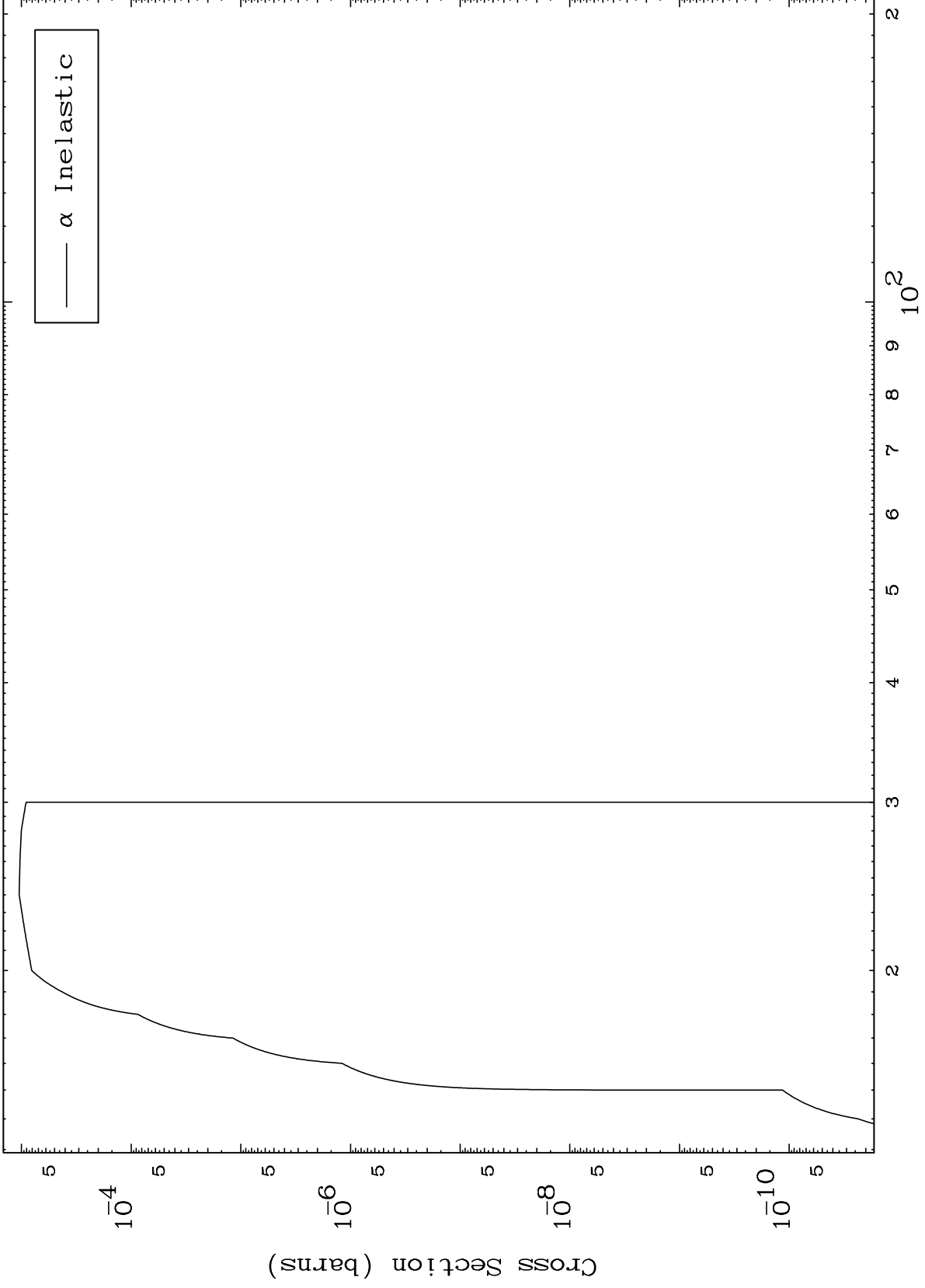


MAT 9434

( $\alpha, n'$ ) Level

94-Pu-238

0 Kelvin Cross Sections



4

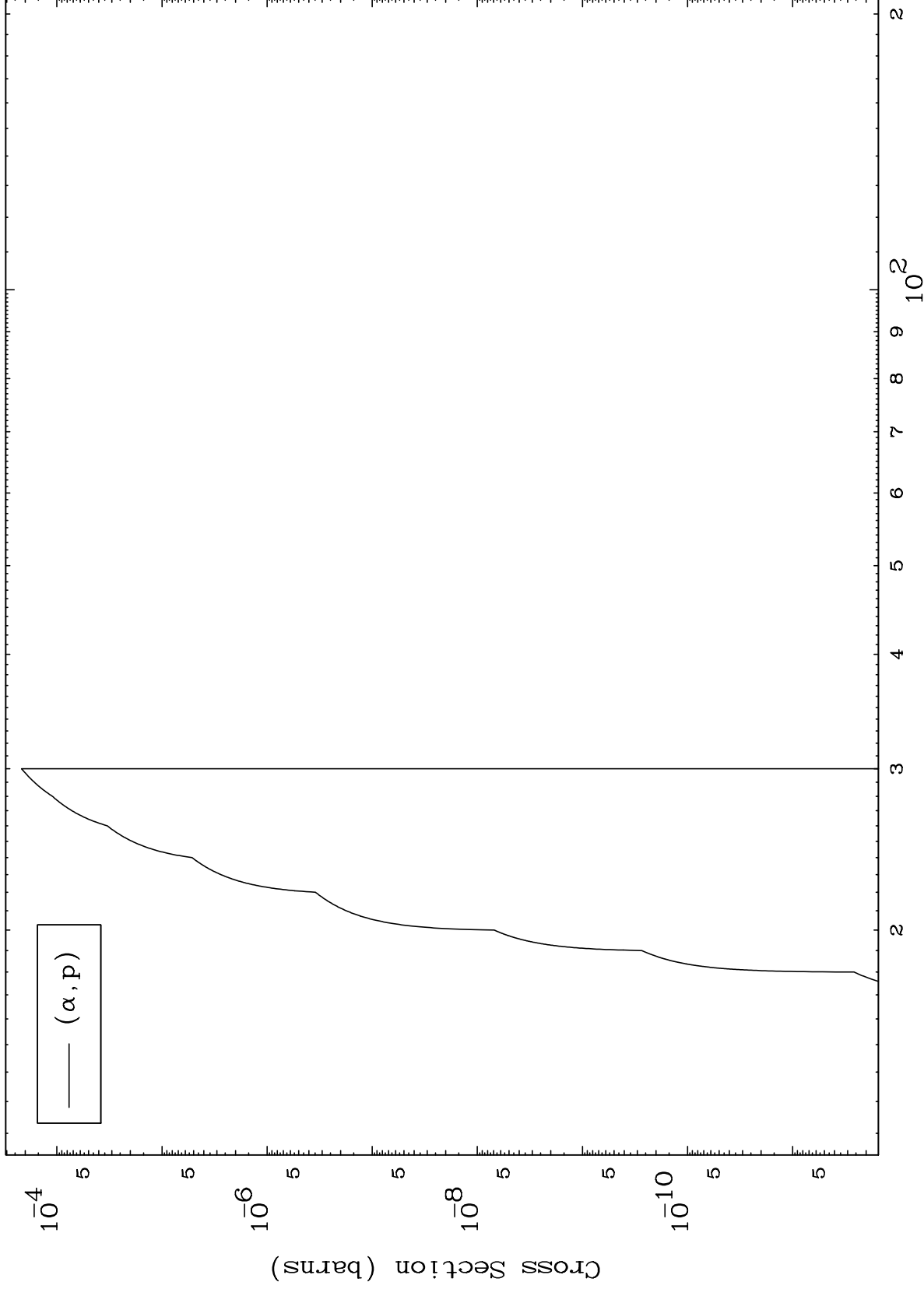
Incident Energy (MeV)

94-Pu-238

MAT 9434

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

94-Pu-238



5

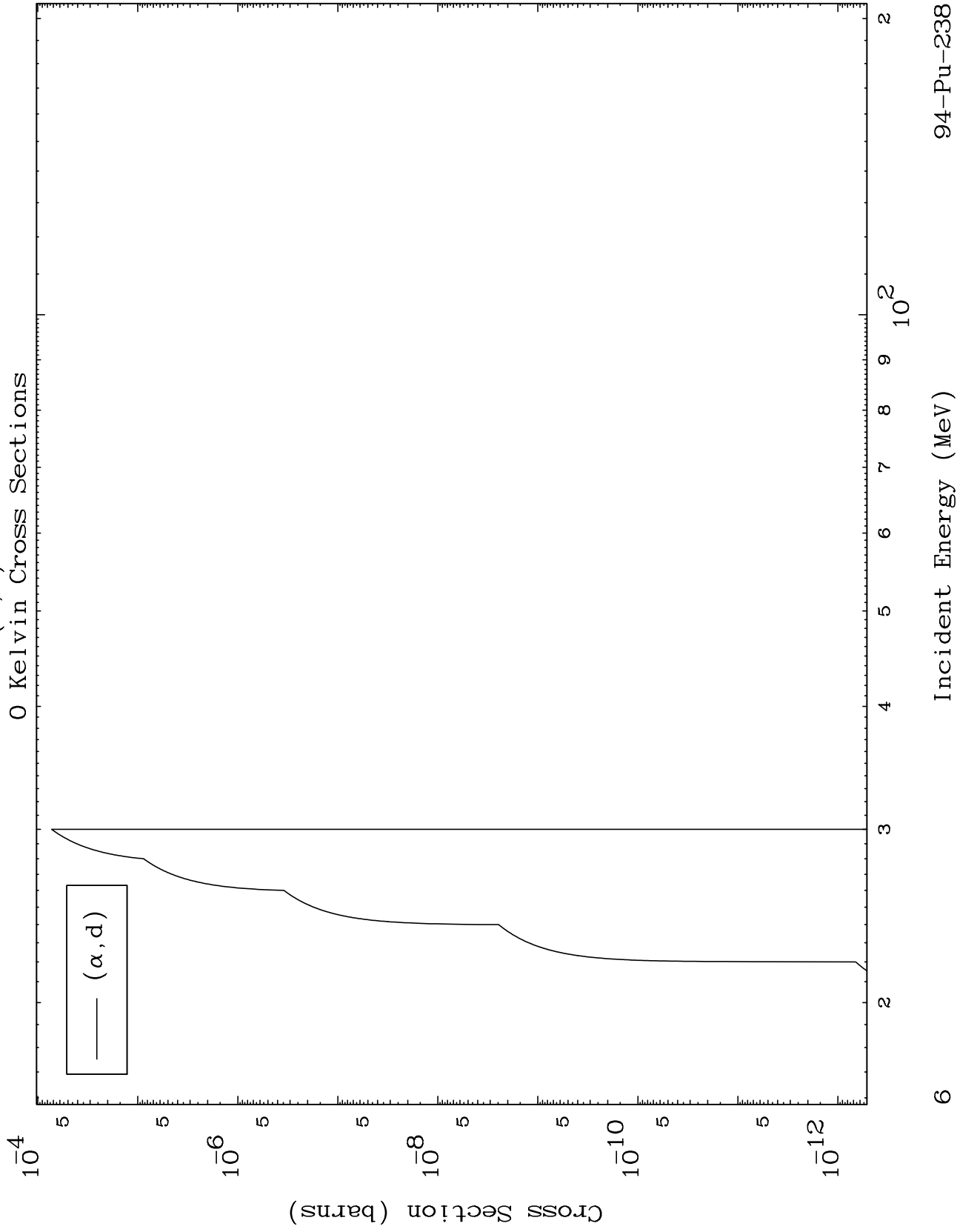
Incident Energy (MeV)

94-Pu-238

MAT 9434

( $\alpha, d$ ) Levels

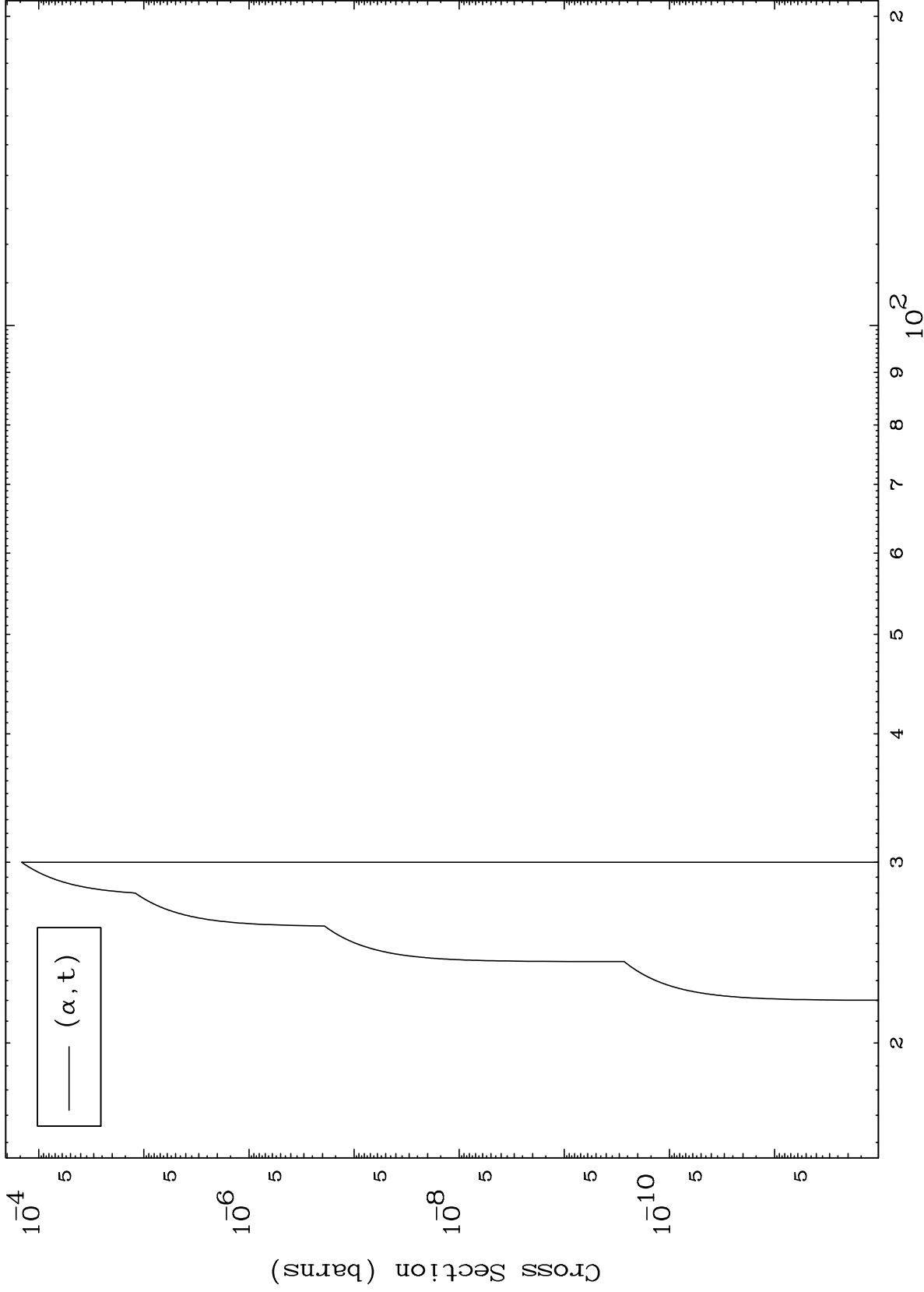
94-Pu-238



MAT 9434

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

94-Pu-238



7

Incident Energy (MeV)

94-Pu-238

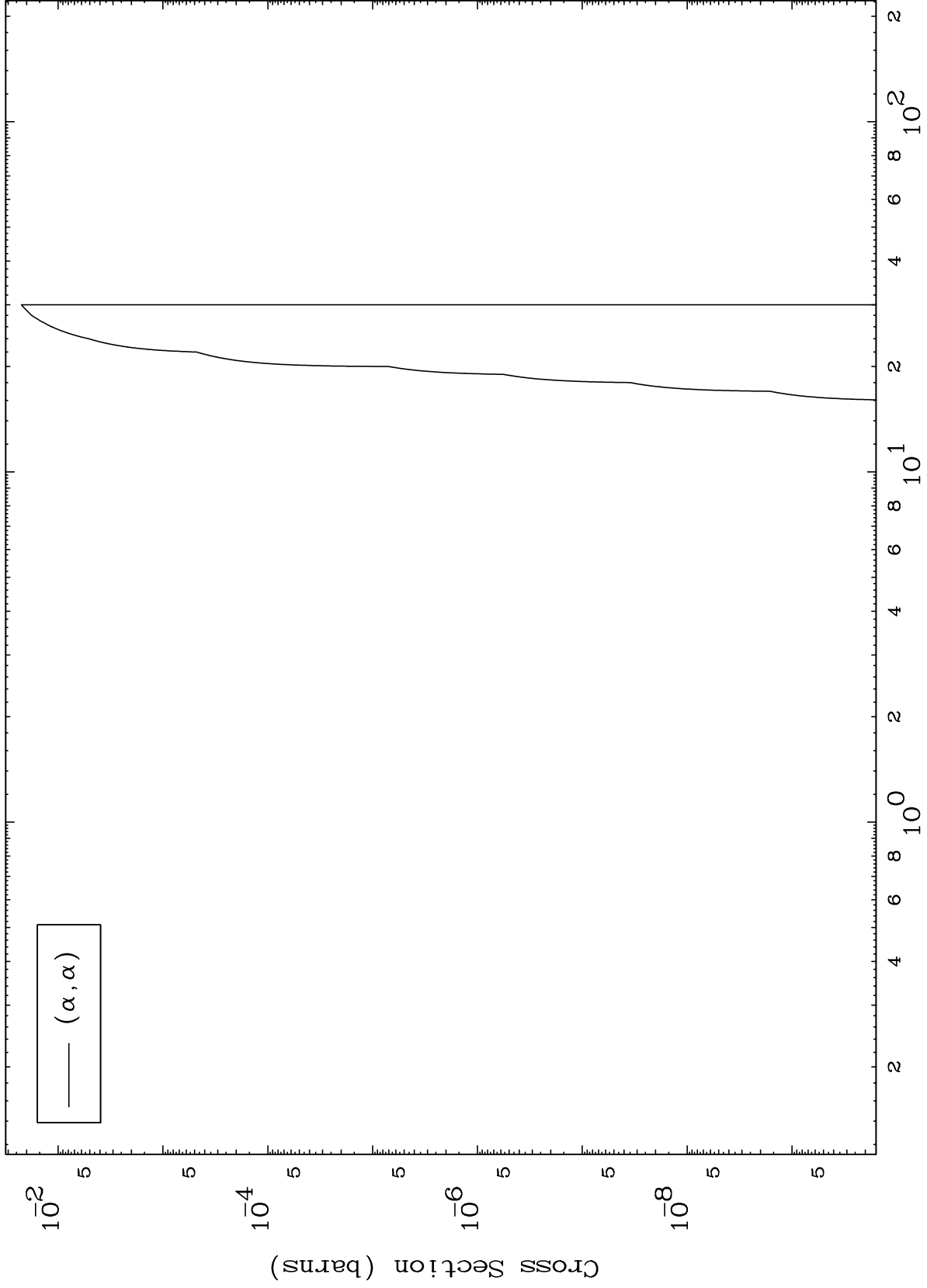


MAT 9434

( $\alpha, \alpha$ ) Levels

94-Pu-238

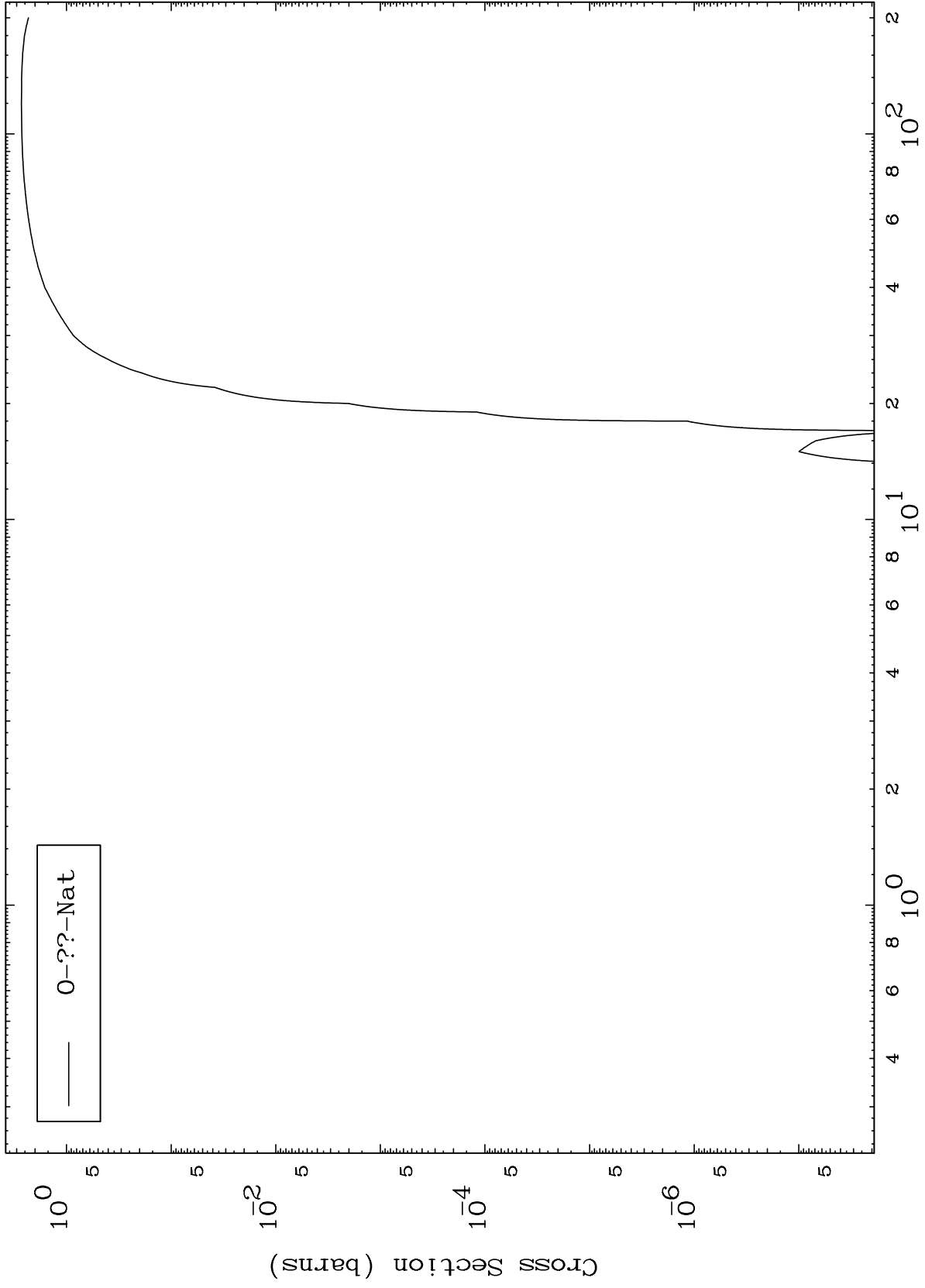
0 Kelvin Cross Sections



MAT 9434

94-Pu-238

$\alpha$  Fission  
Radionuclide Production Cross Section

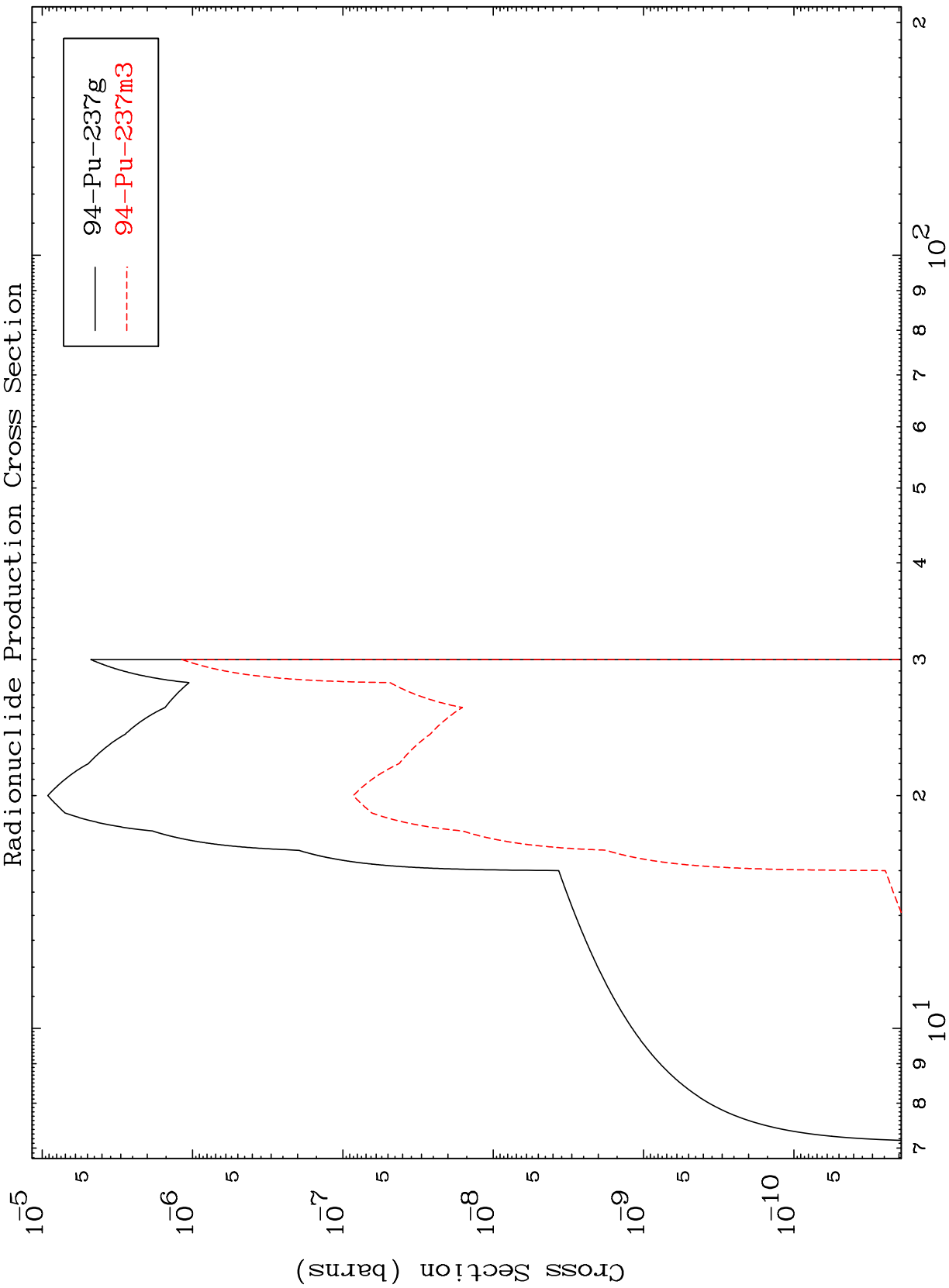


MAT 9434

$(\alpha, n')$   $\alpha$

94-Pu-238

Radionuclide Production Cross Section



94-Pu-238

Incident Energy (MeV)

10