

Program EVALPLOT  
(Version 2018-1)

by

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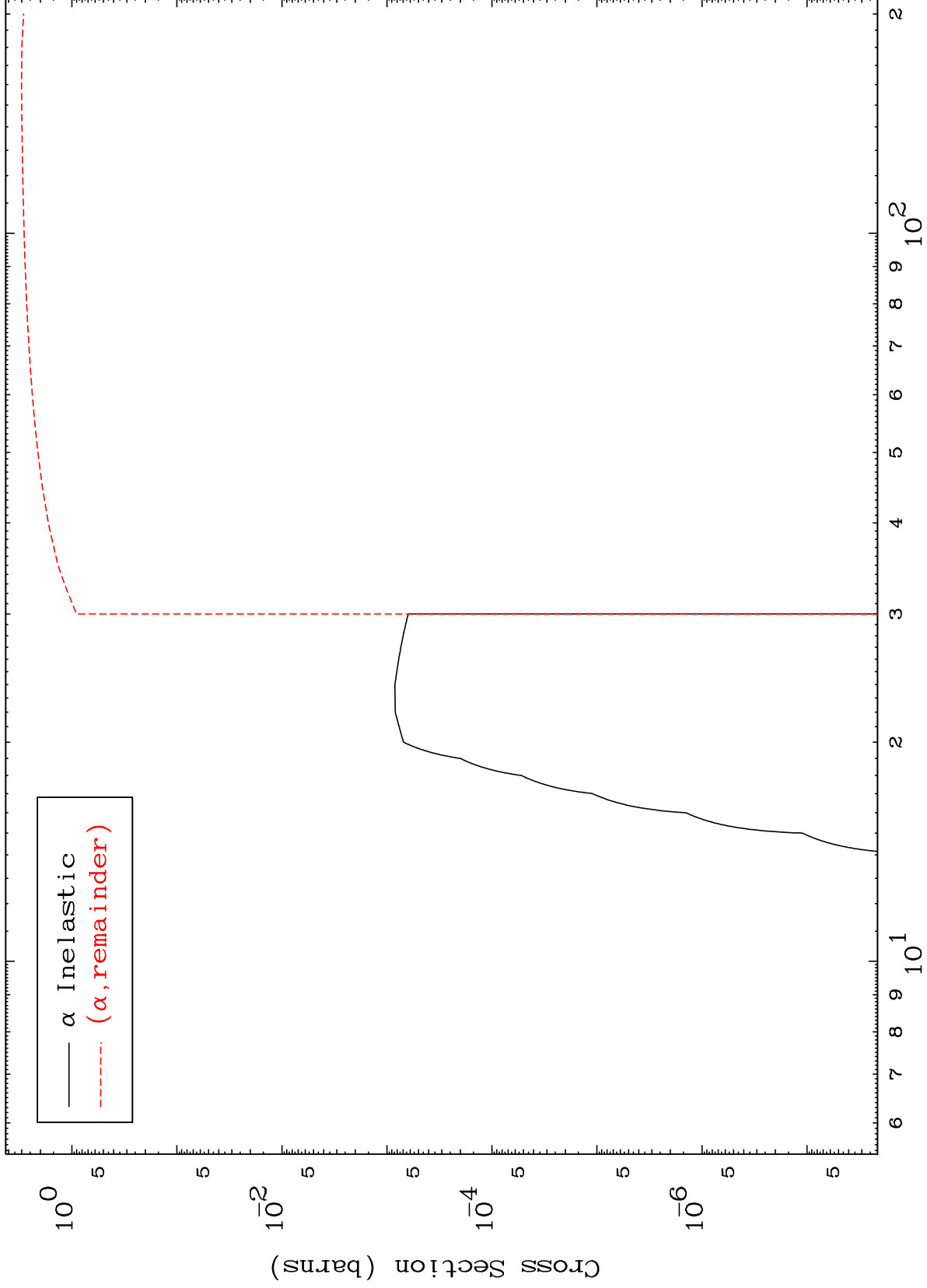
Press Mouse Button to Start

MAT 9443

0 Kelvin

$\alpha$  Major Cross Sections

94-Pu-241



1

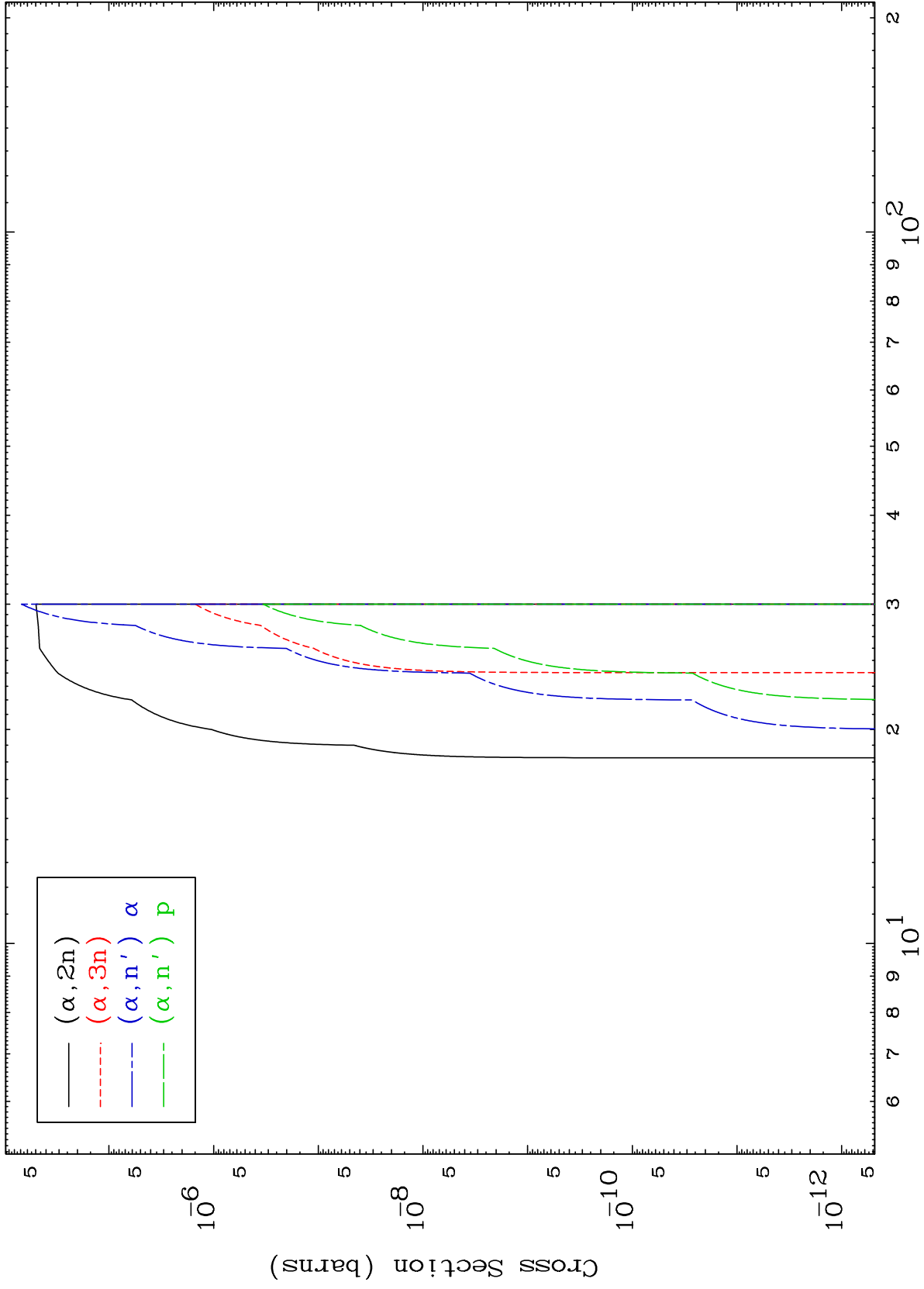
Incident Energy (MeV)

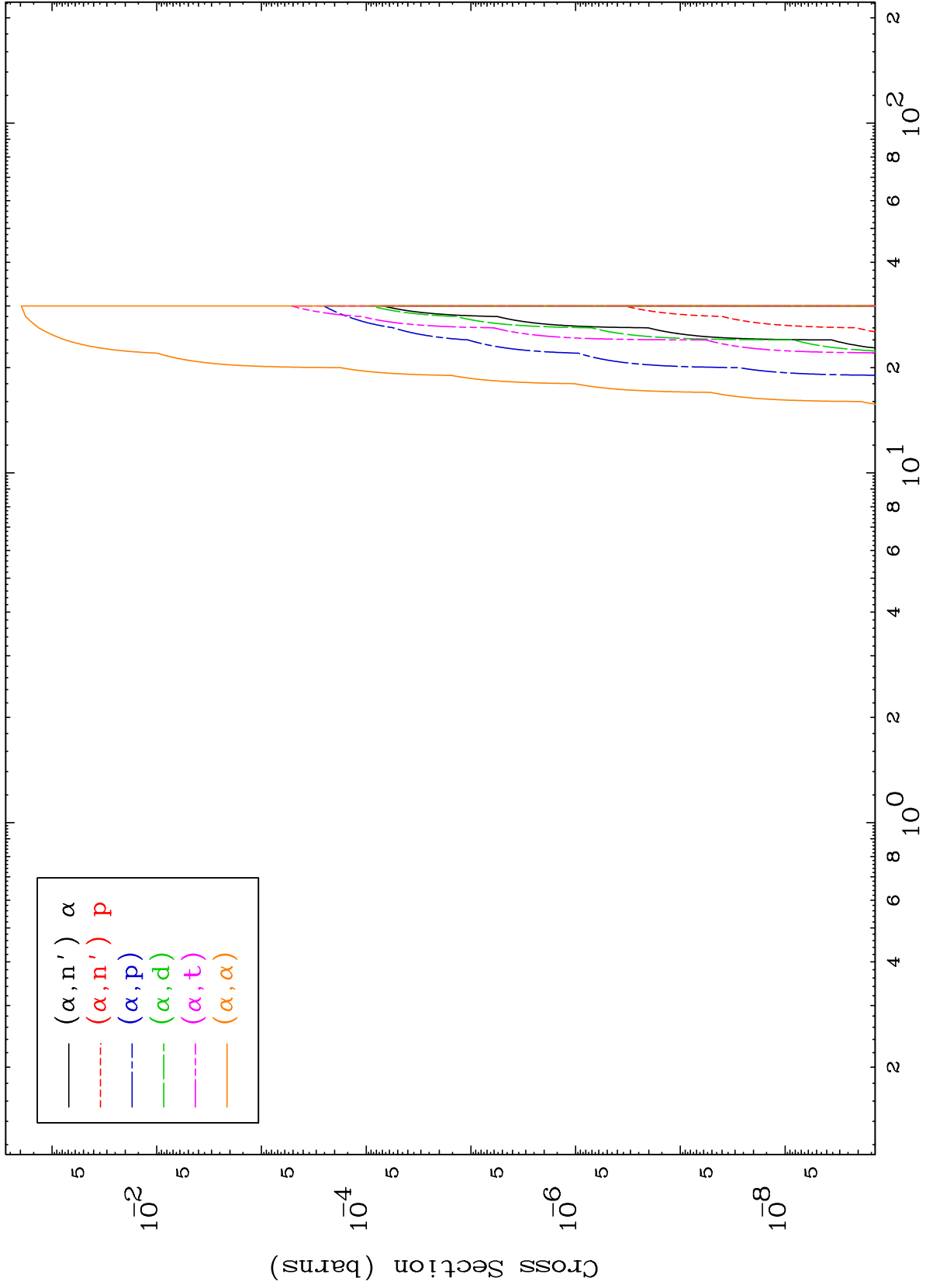
94-Pu-241

MAT 9443

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

94-Pu-241



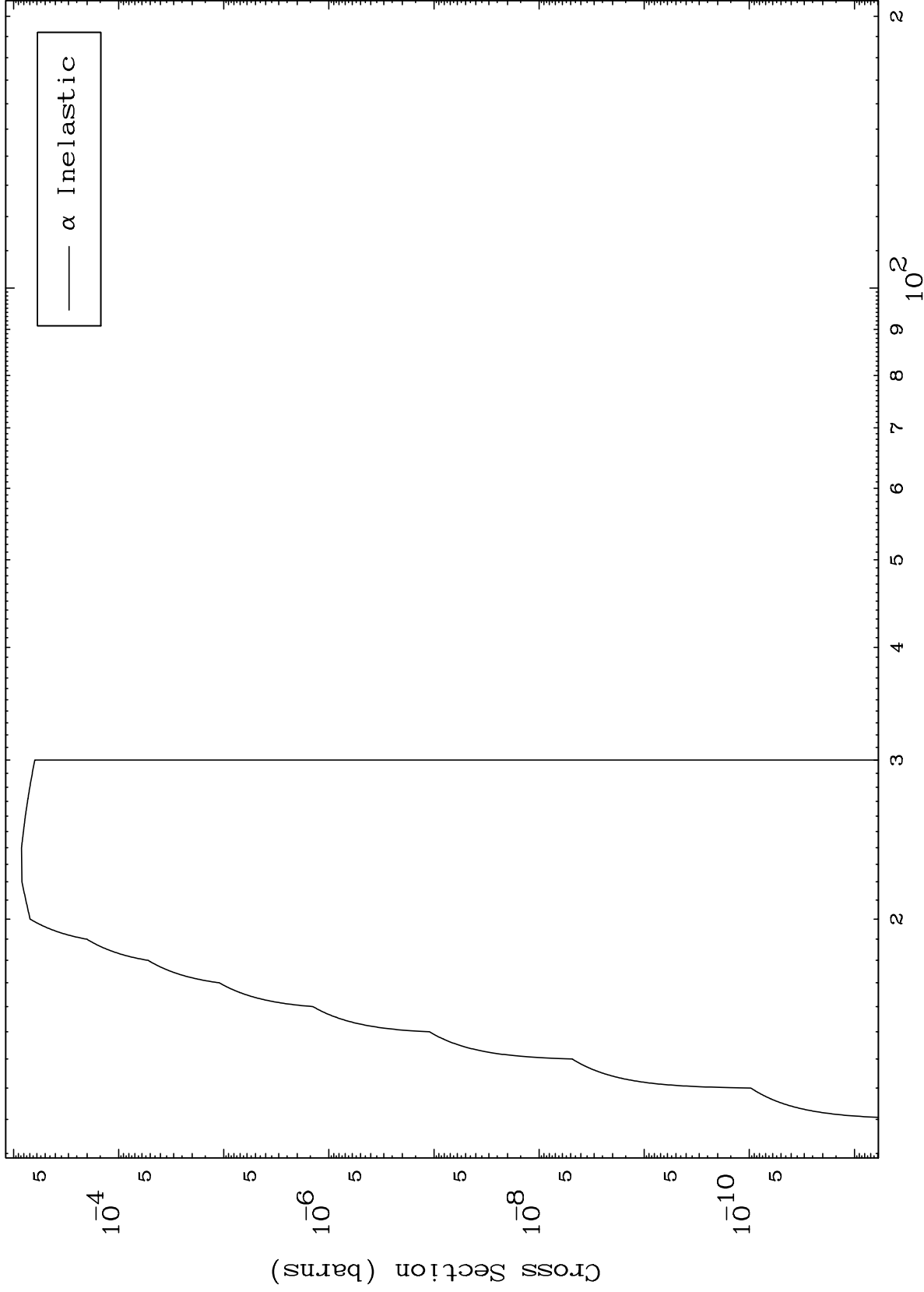


MAT 9443

( $\alpha, n'$ ) Level

94-Pu-241

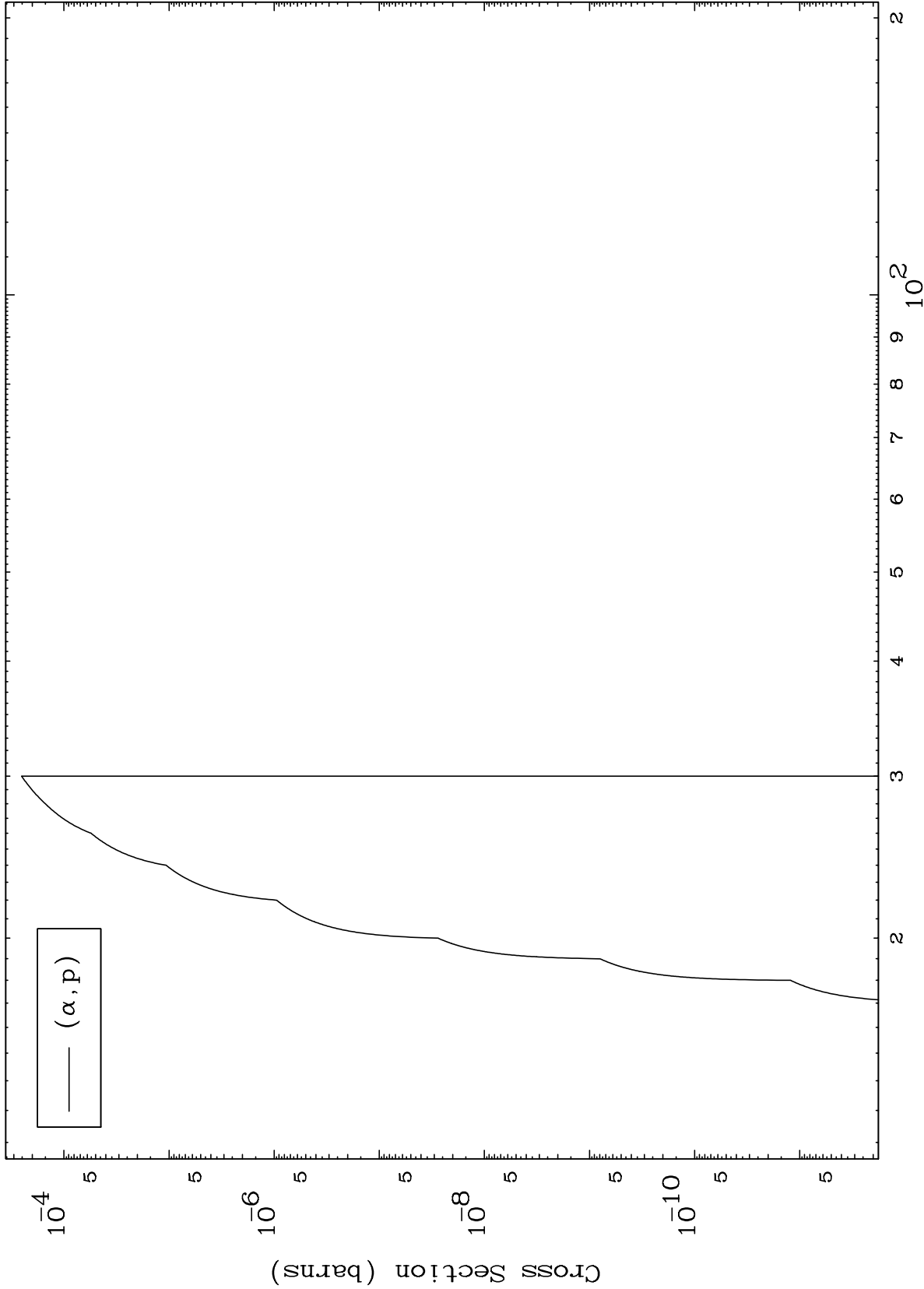
0 Kelvin Cross Sections



4

Incident Energy (MeV)

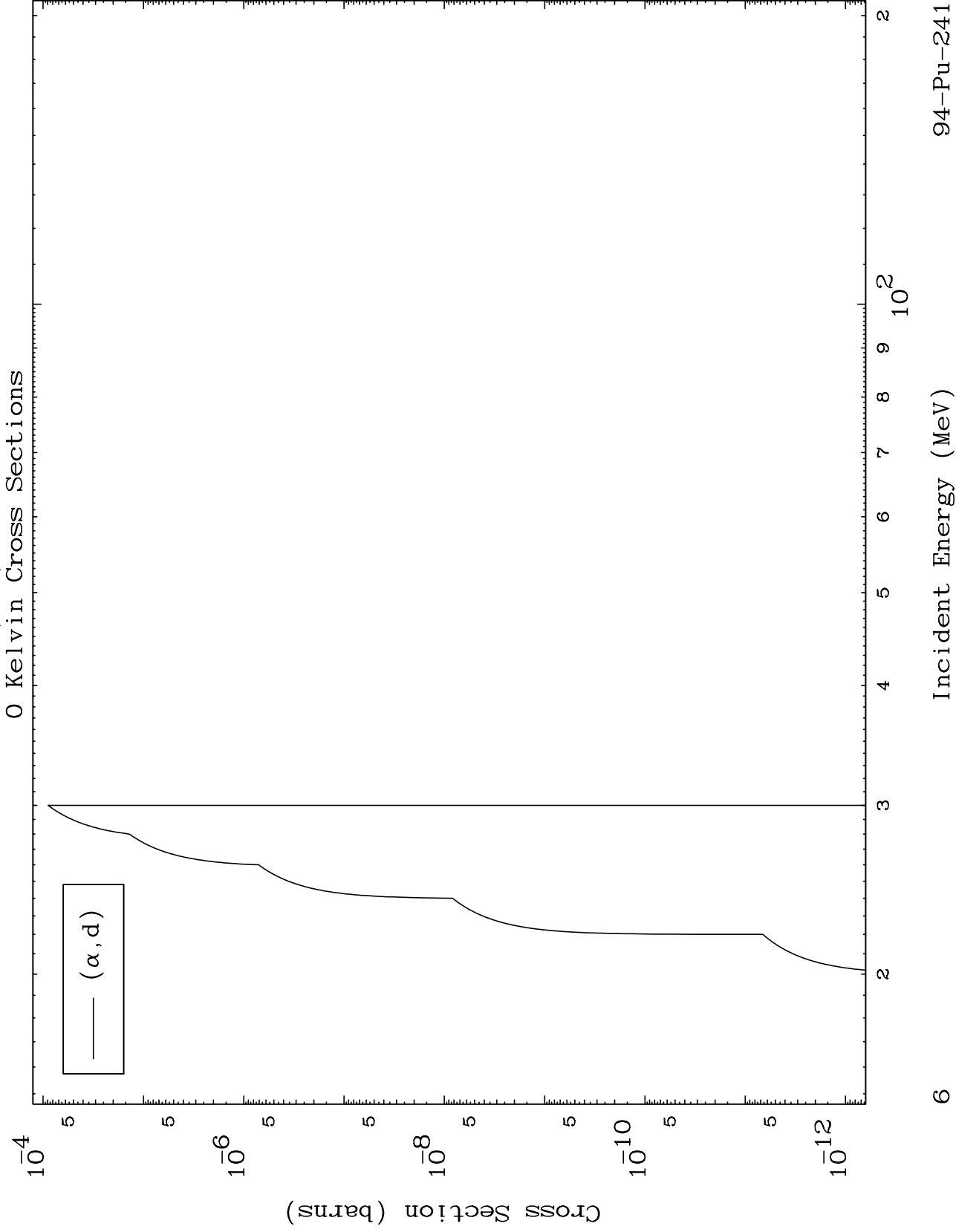
94-Pu-241



MAT 9443

( $\alpha, d$ ) Levels

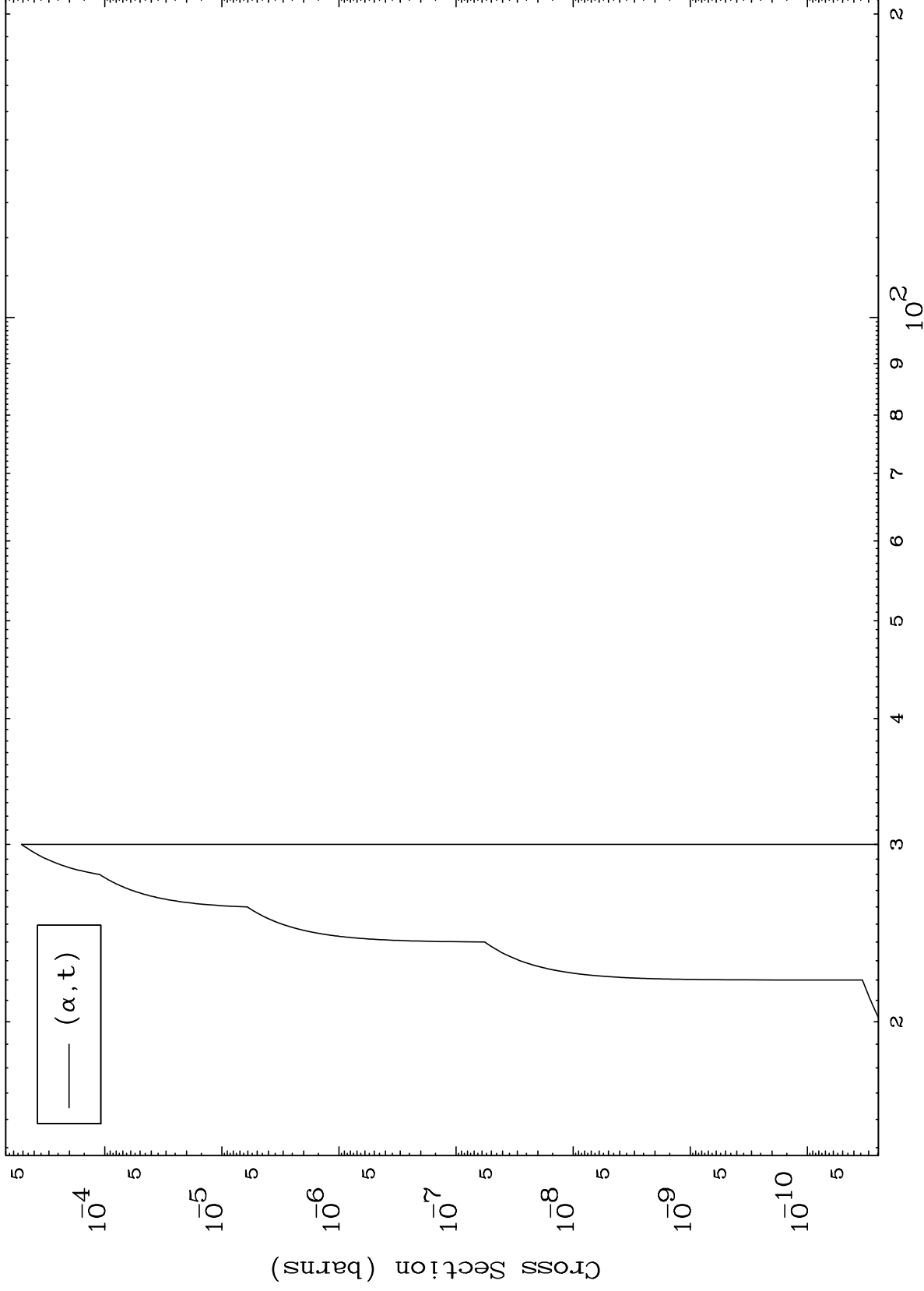
94-Pu-241



MAT 9443

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

94-Pu-241



7

Incident Energy (MeV)

94-Pu-241

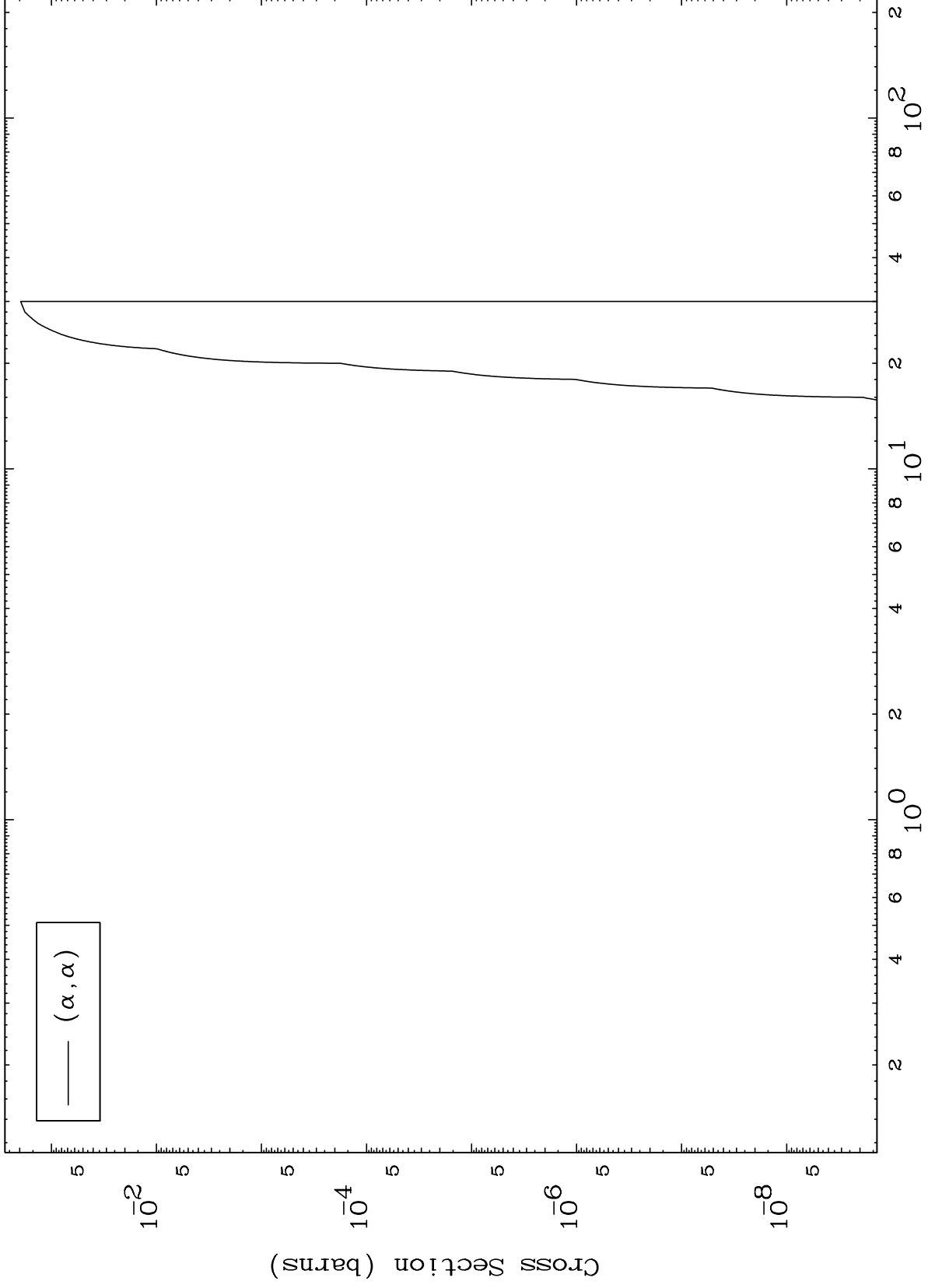


MAT 9443

( $\alpha, \alpha$ ) Levels

94-Pu-241

0 Kelvin Cross Sections

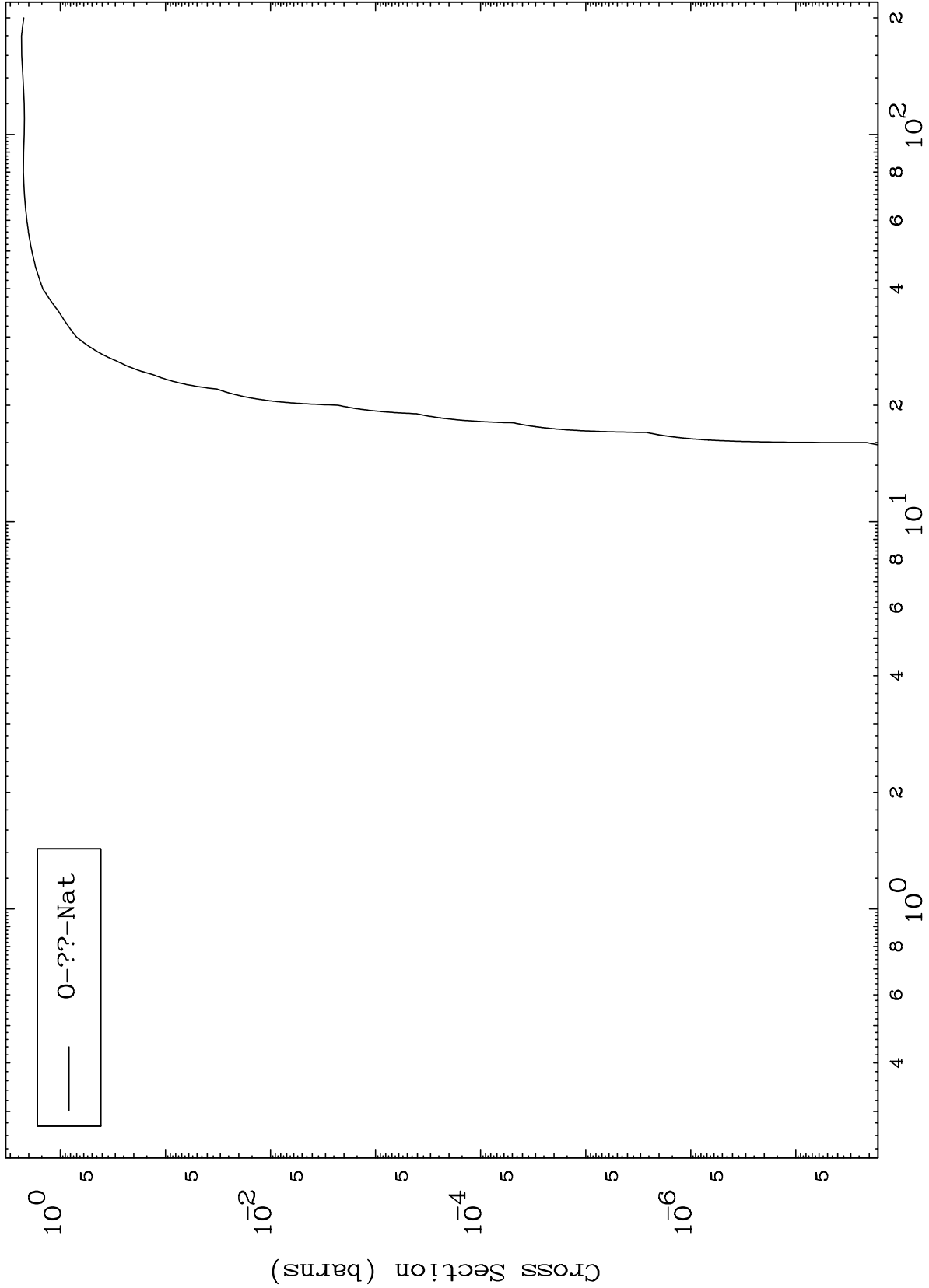


MAT 9443

94-Pu-241

$\alpha$  Fission

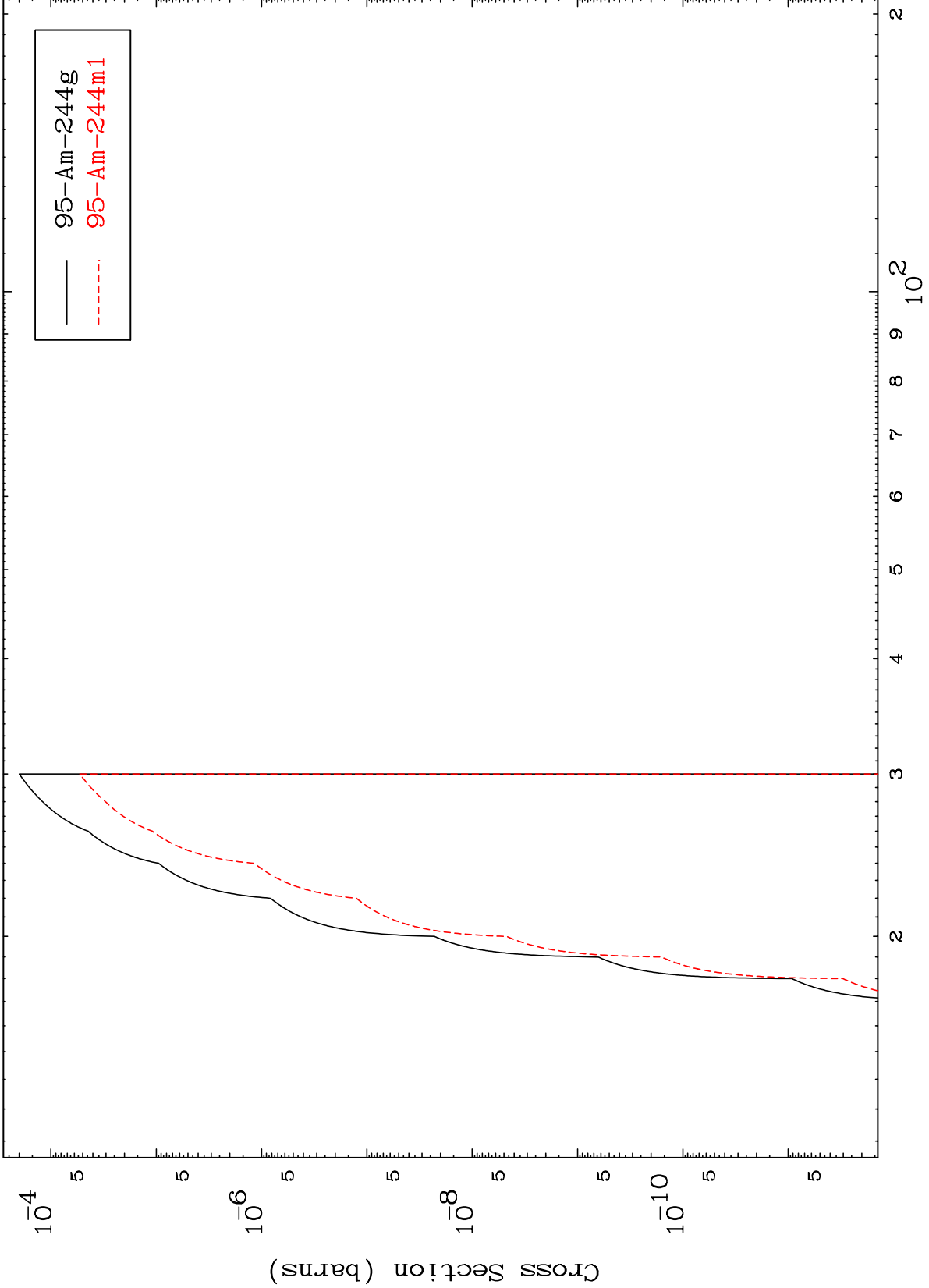
Radionuclide Production Cross Section



MAT 9443

94-Pu-241

( $\alpha, p$ )  
Radionuclide Production Cross Section



10

Incident Energy (MeV)

94-Pu-241

MAT 9443

( $\alpha, t$ )

94-Pu-241

Radionuclide Production Cross Section

