

Program EVALPLOT
(Version 2018-1)

by

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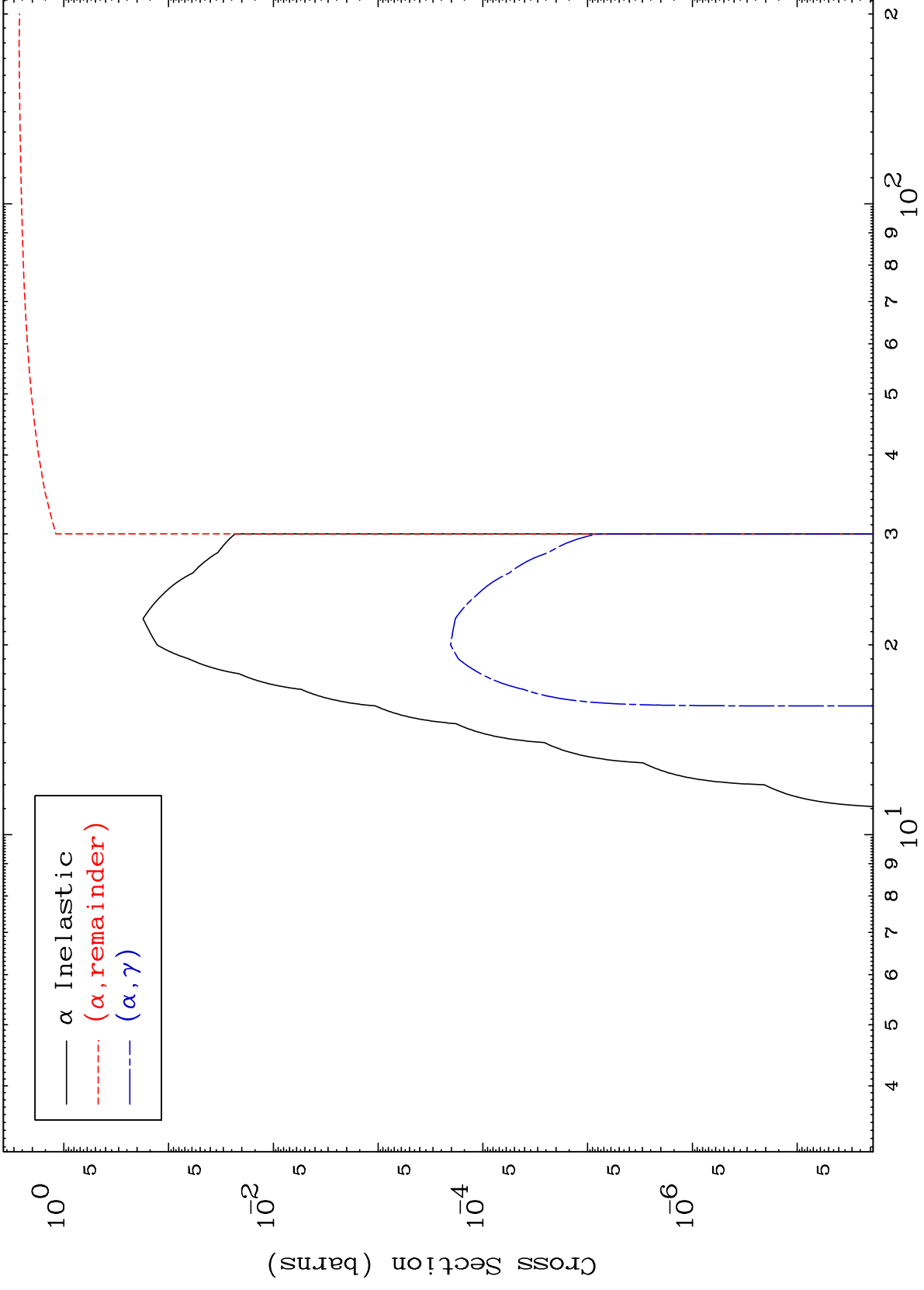
Press Mouse Button to Start

MAT 7523

α Major

75-Re-184

0 Kelvin Cross Sections



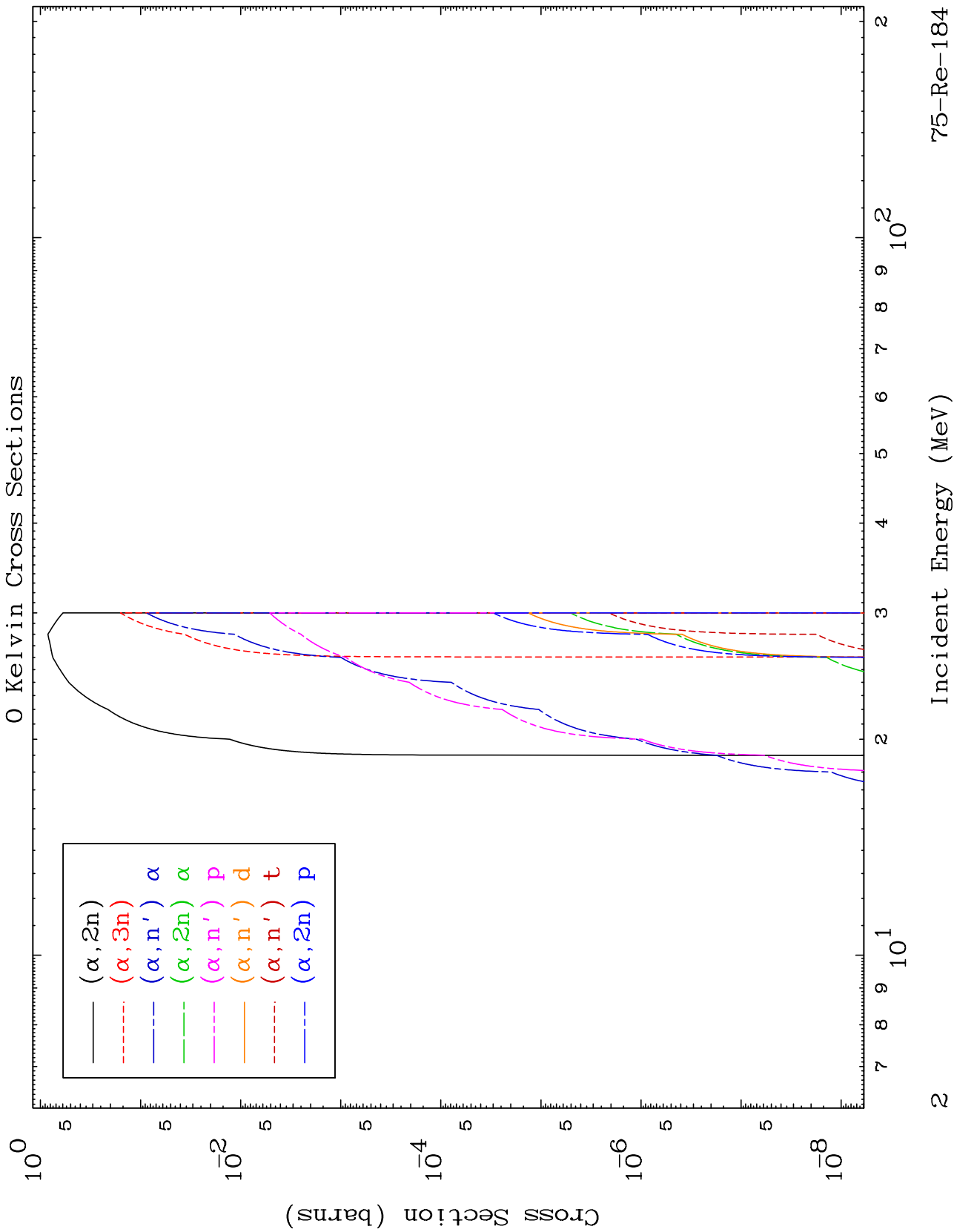
Incident Energy (MeV)

75-Re-184

MAT 7523

α Neutron Production
0 Kelvin Cross Sections

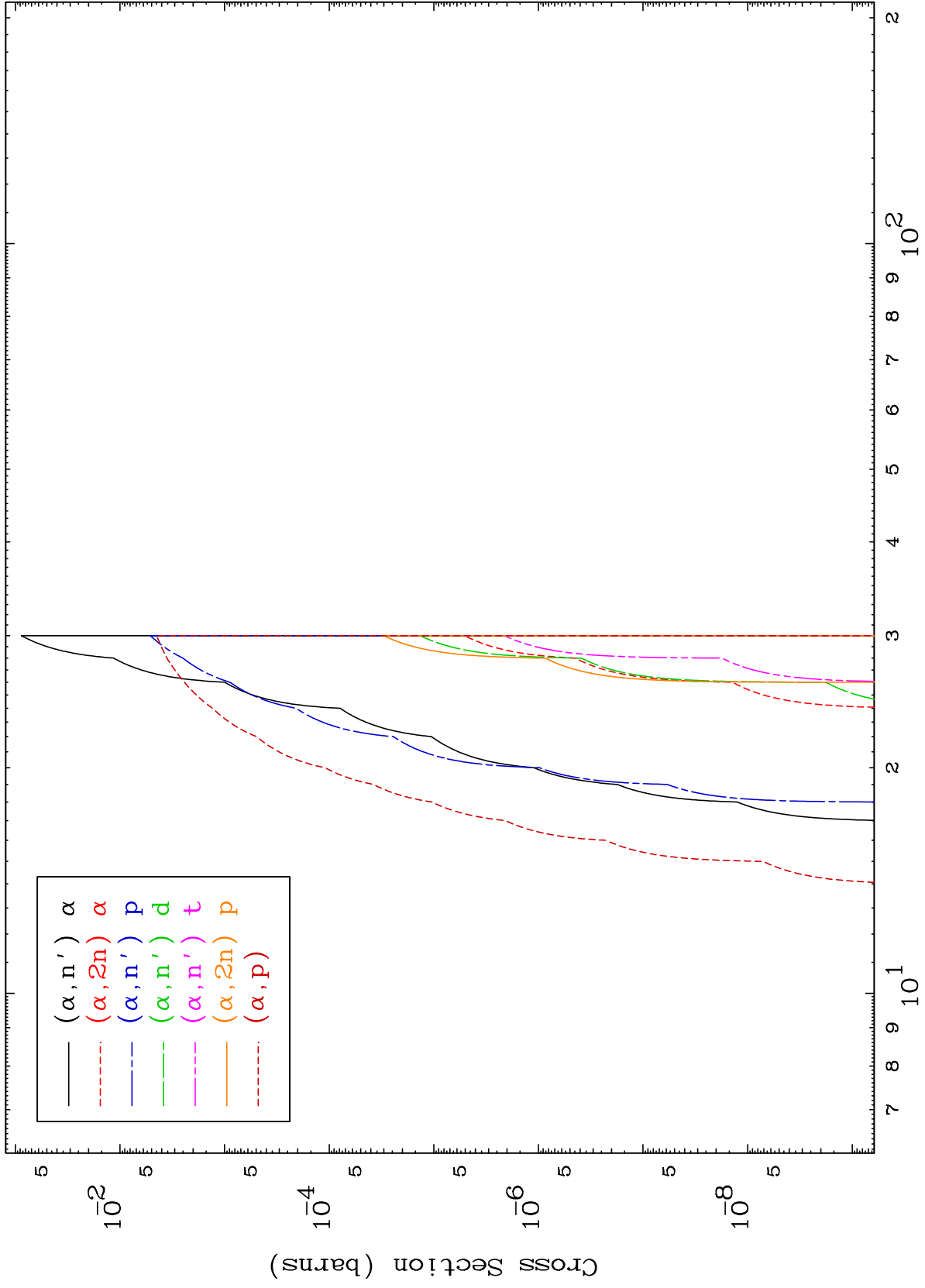
75-Re-184

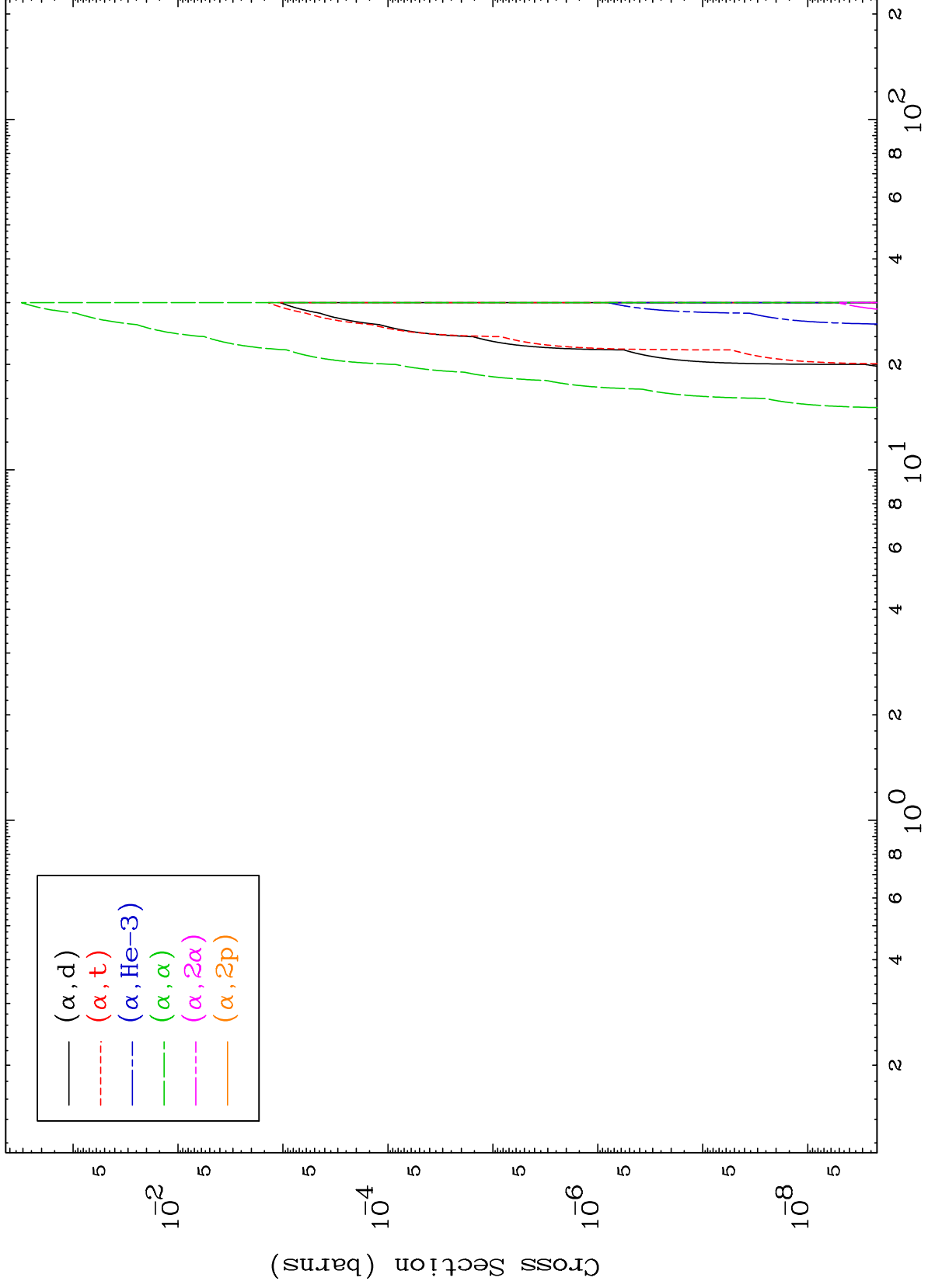


2

Incident Energy (MeV)

75-Re-184



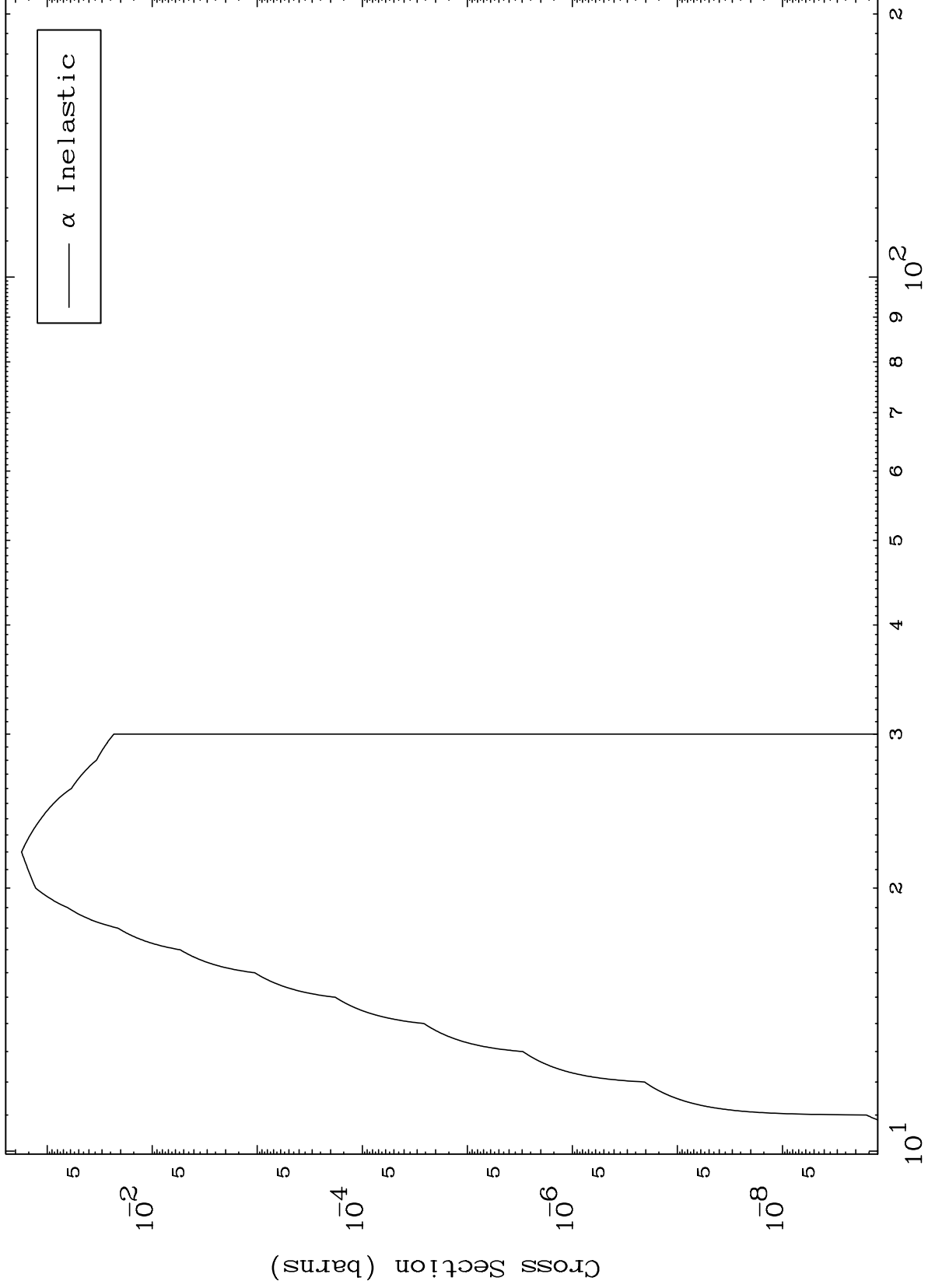


MAT 7523

(α, n') Level

75-Re-184

0 Kelvin Cross Sections



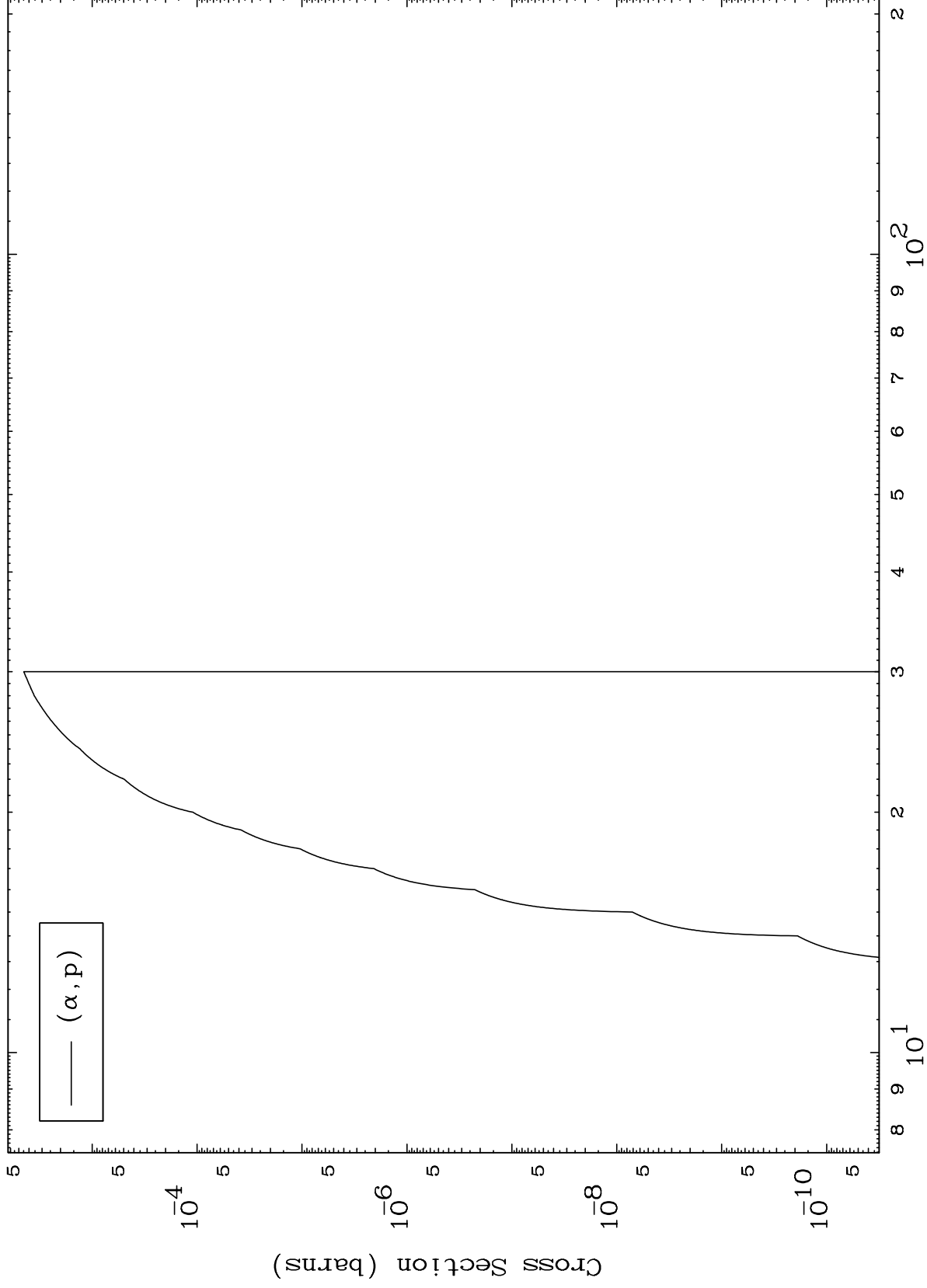
Incident Energy (MeV)

75-Re-184

MAT 7523

(α, p) Levels
0 Kelvin Cross Sections

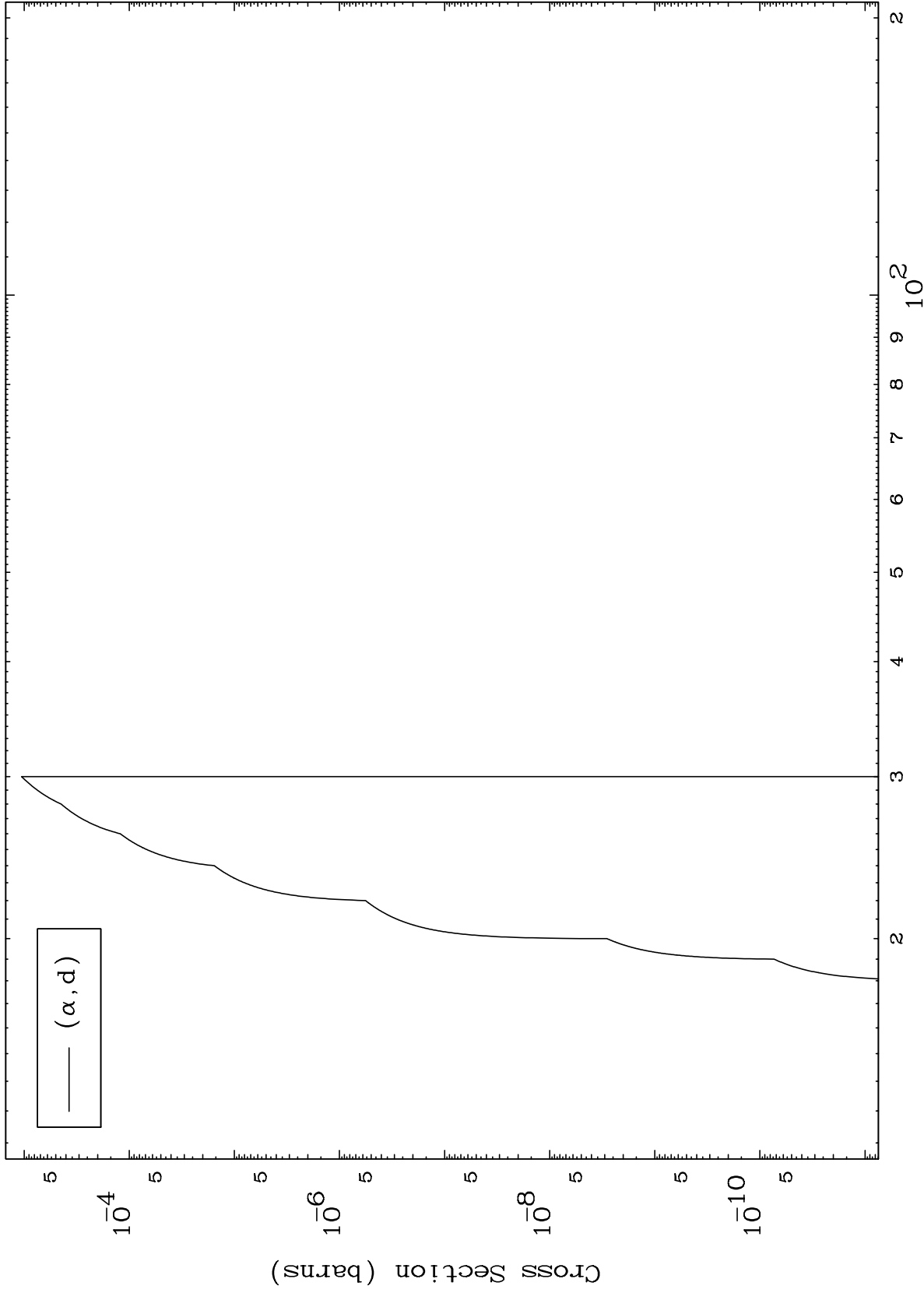
75-Re-184

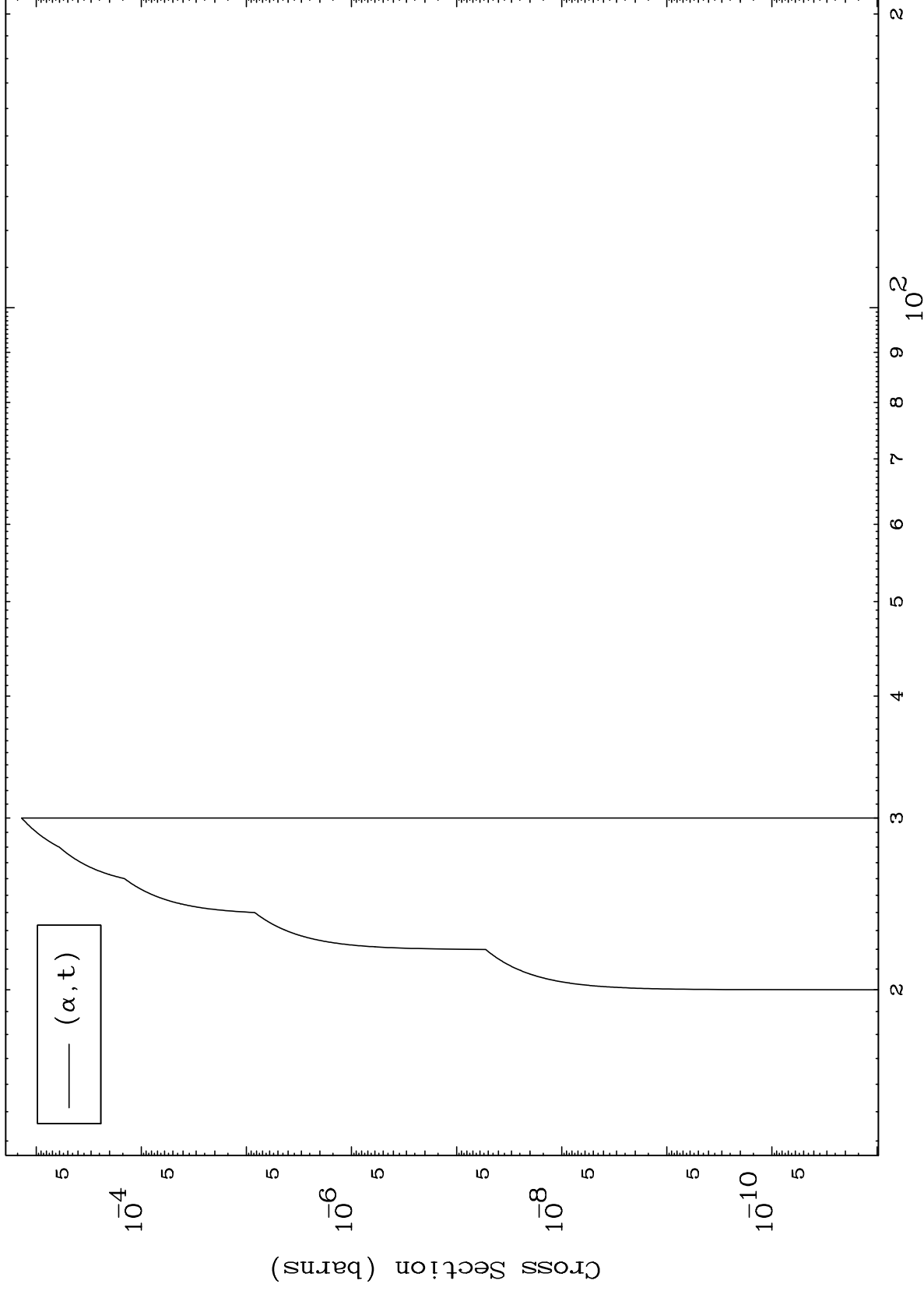


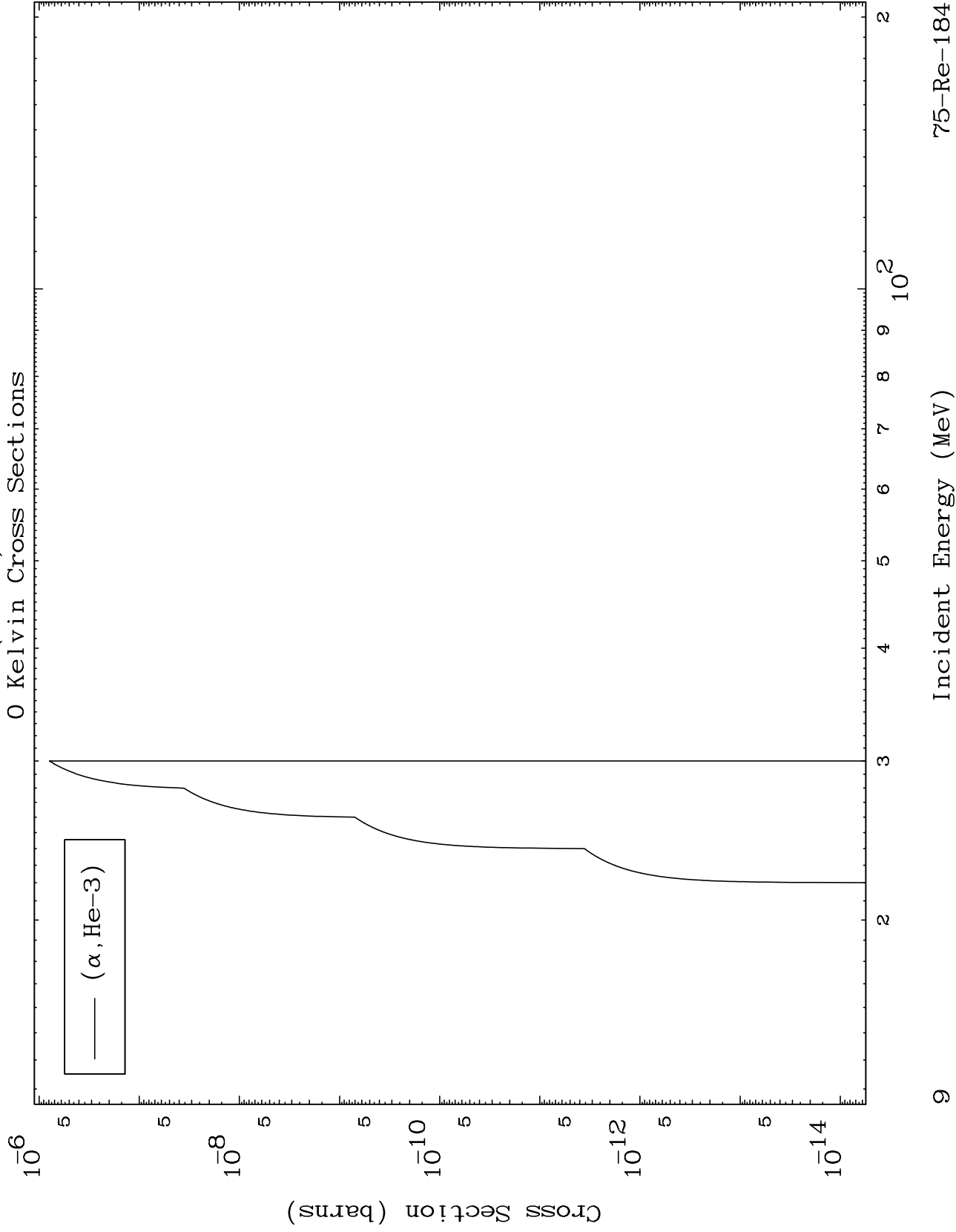
6

Incident Energy (MeV)

75-Re-184





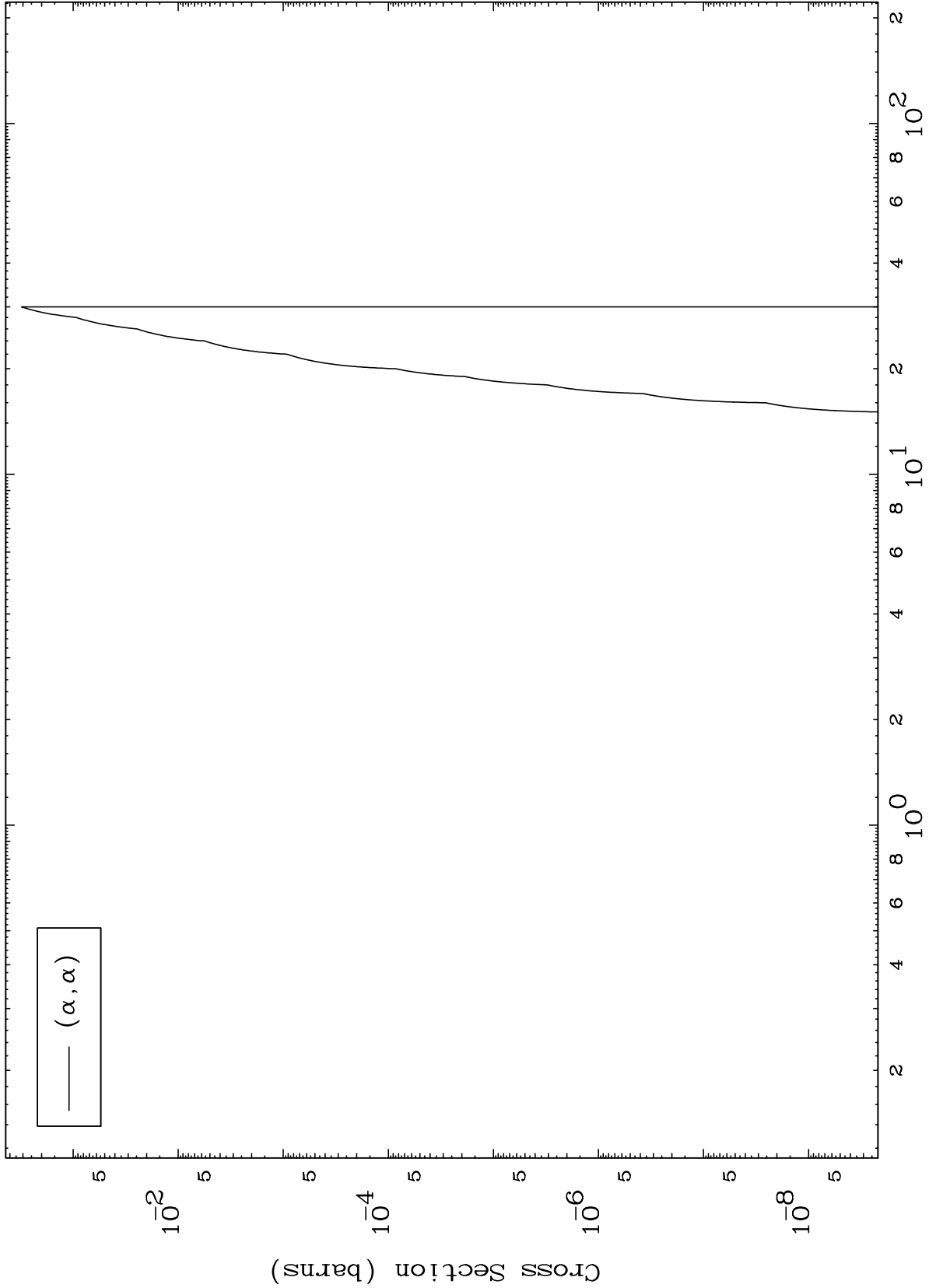


MAT 7523

(α, α) Levels

75-Re-184

0 Kelvin Cross Sections



10

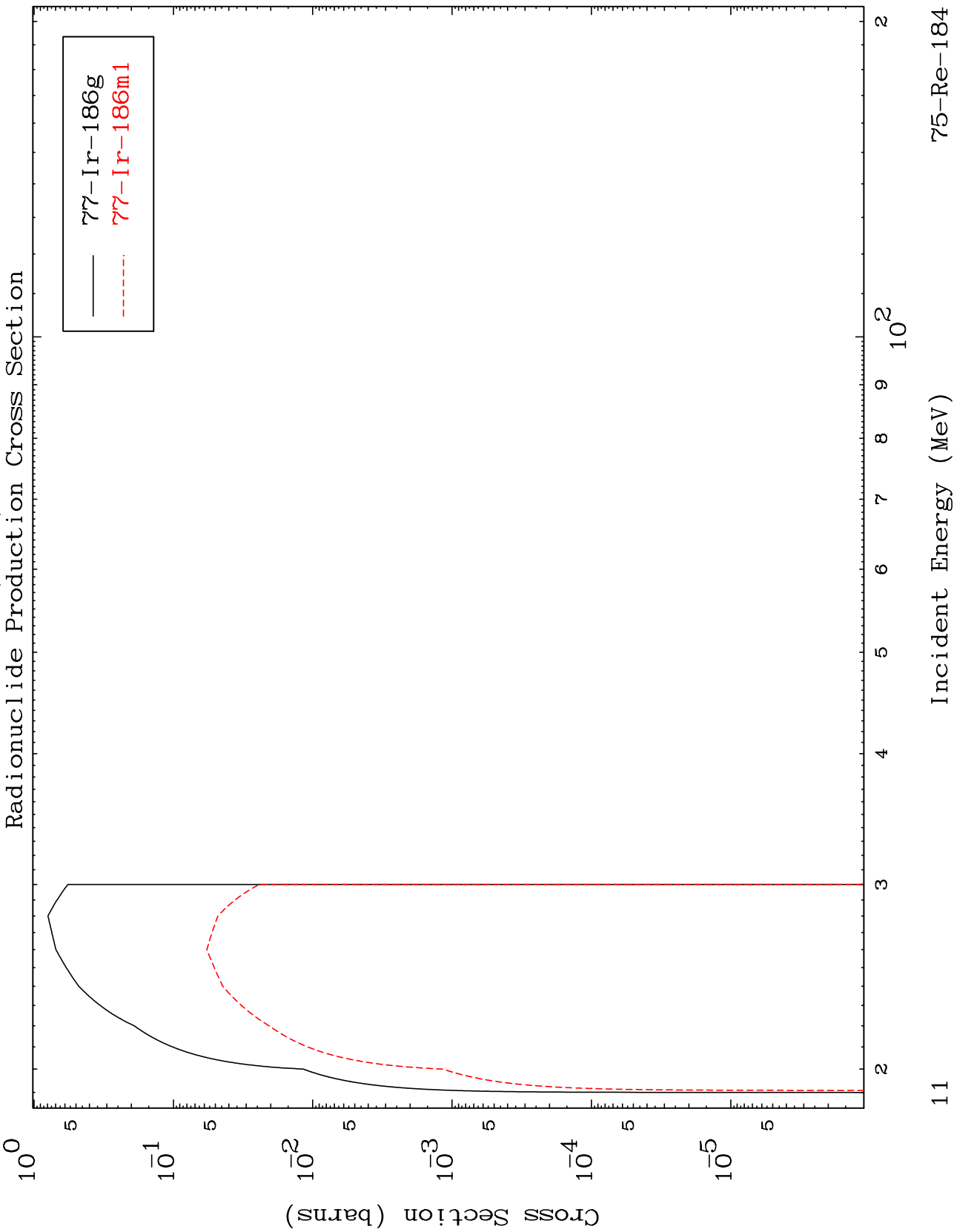
Incident Energy (MeV)

75-Re-184

MAT 7523

75-Re-184

($\alpha, 2n$)

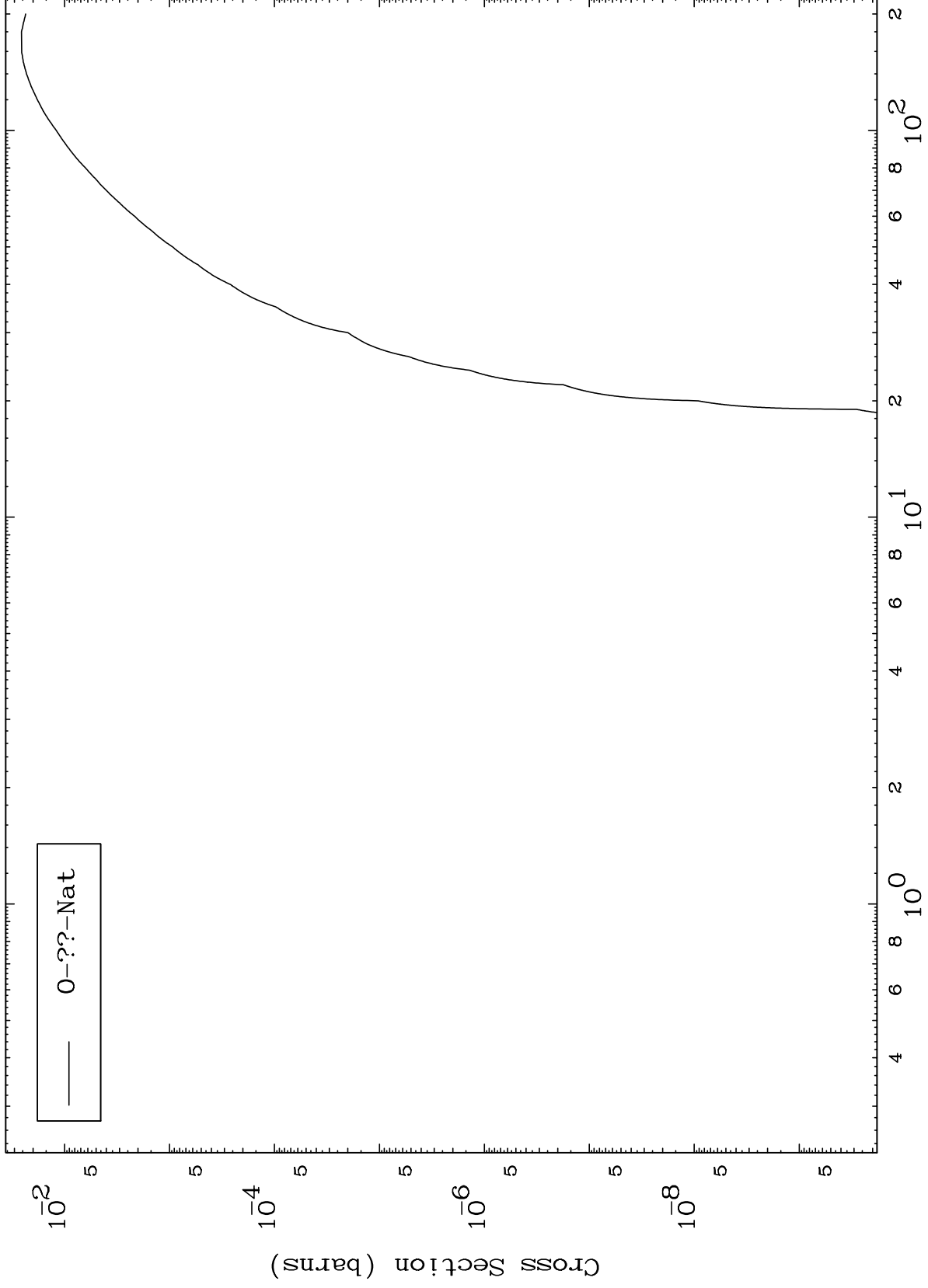


MAT 7523

α Fission

75-Re-184

Radionuclide Production Cross Section



12

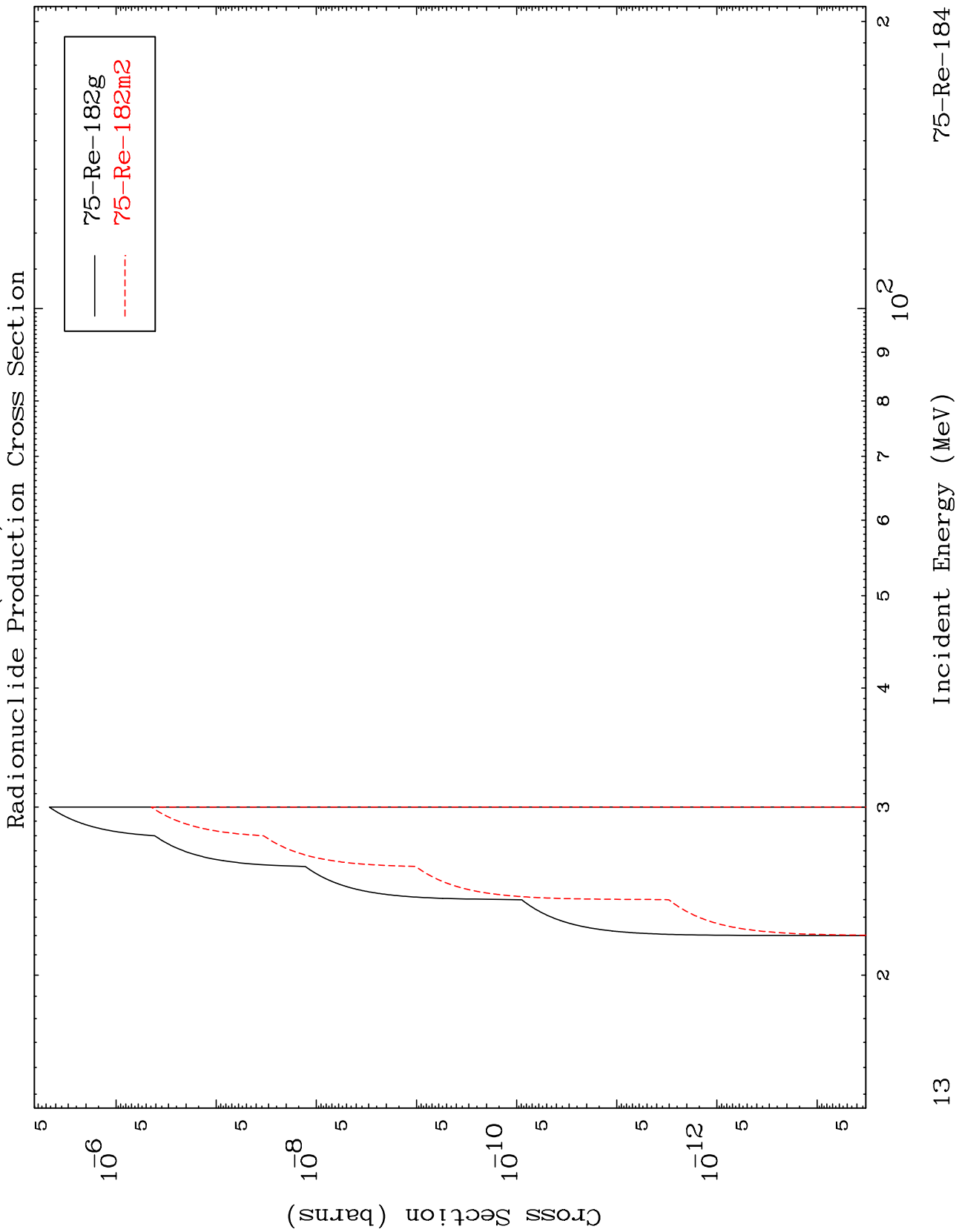
Incident Energy (MeV)

75-Re-184

MAT 7523

$(\alpha, 2n)$ α

⁷⁵Re-¹⁸⁴



13

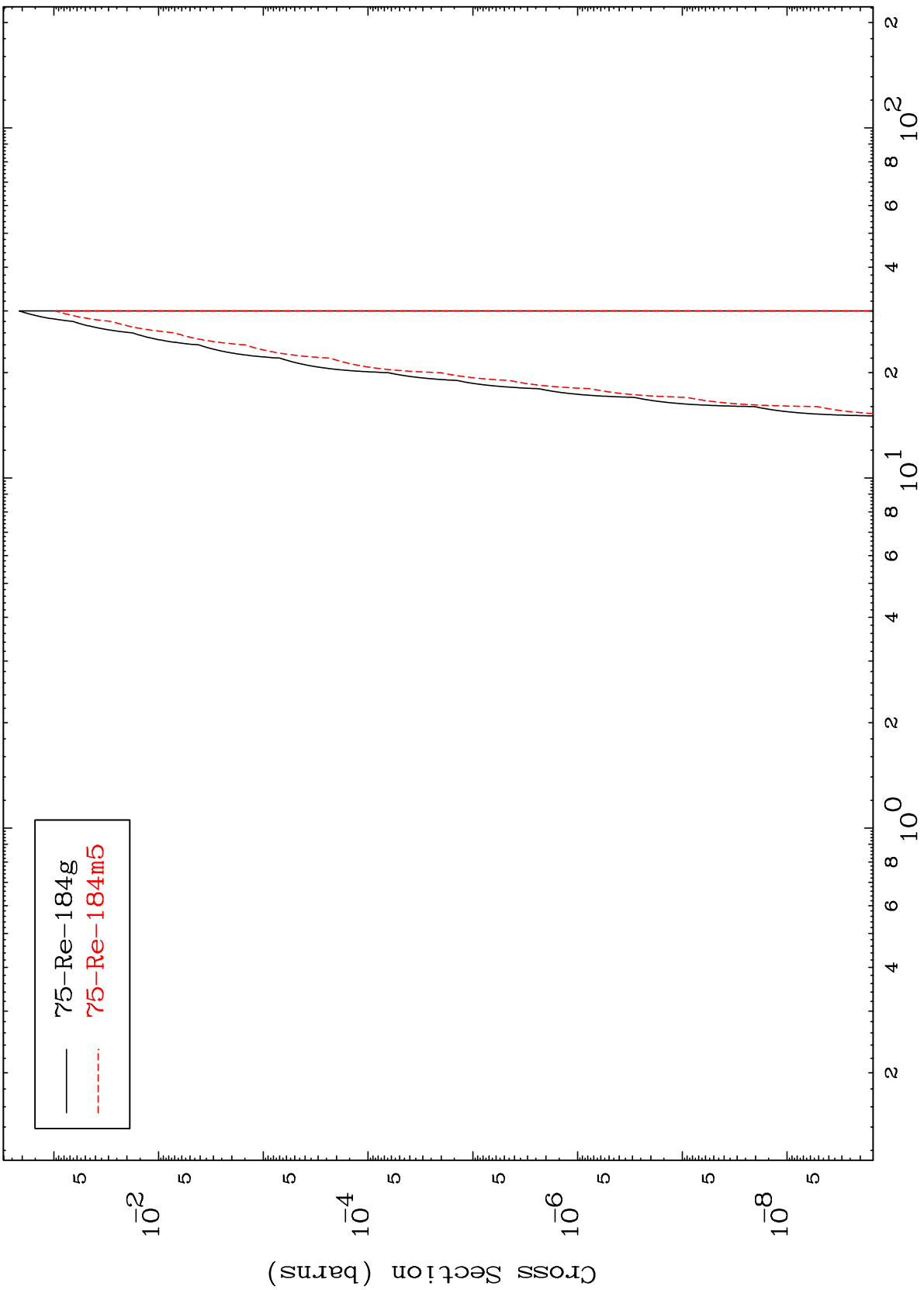
⁷⁵Re-¹⁸⁴

MAT 7523

(α, α)

75-Re-184

Radionuclide Production Cross Section



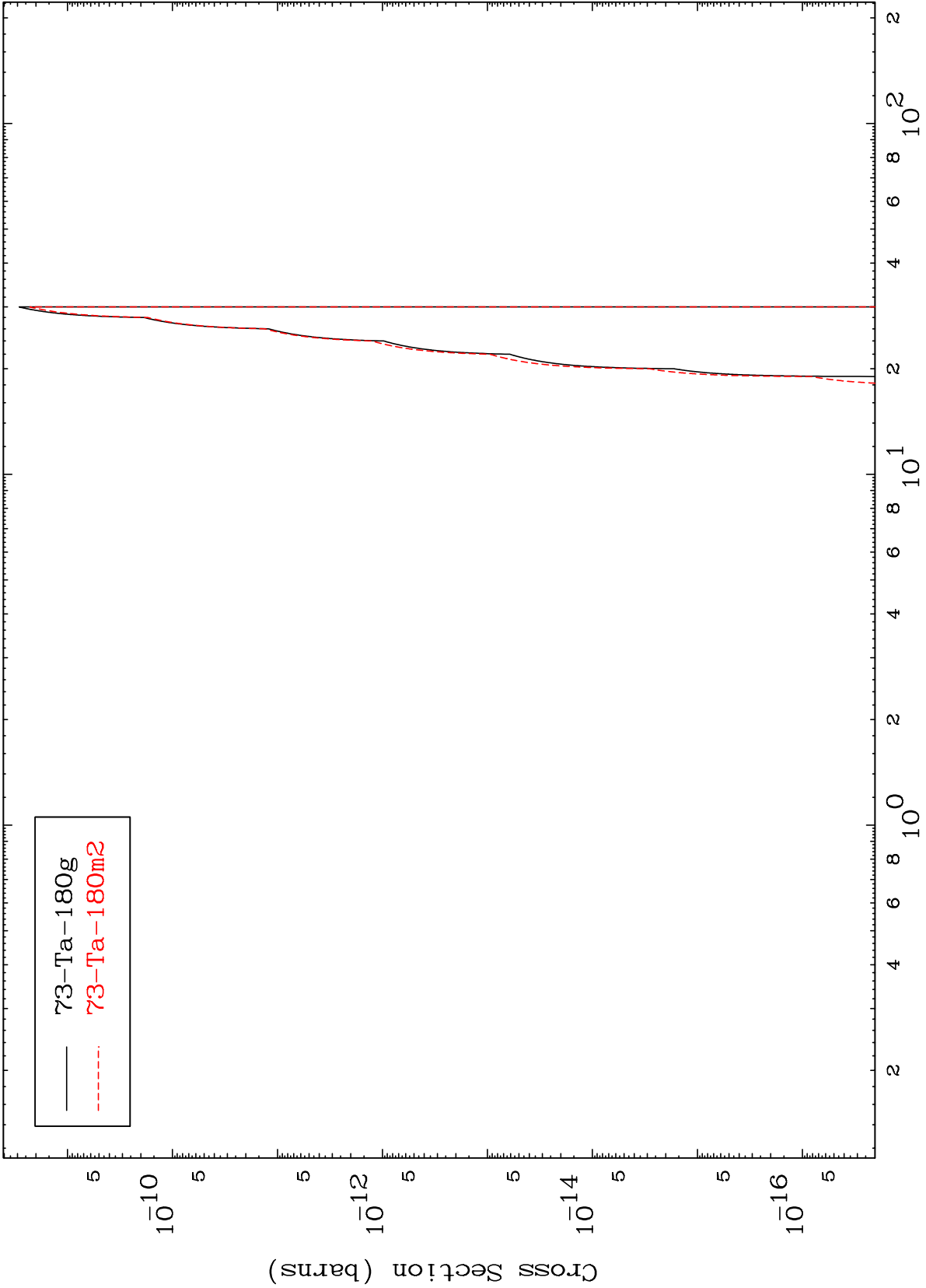
75-Re-184g
75-Re-184m5

MAT 7523

($\alpha, 2\alpha$)

75-Re-184

Radionuclide Production Cross Section



15

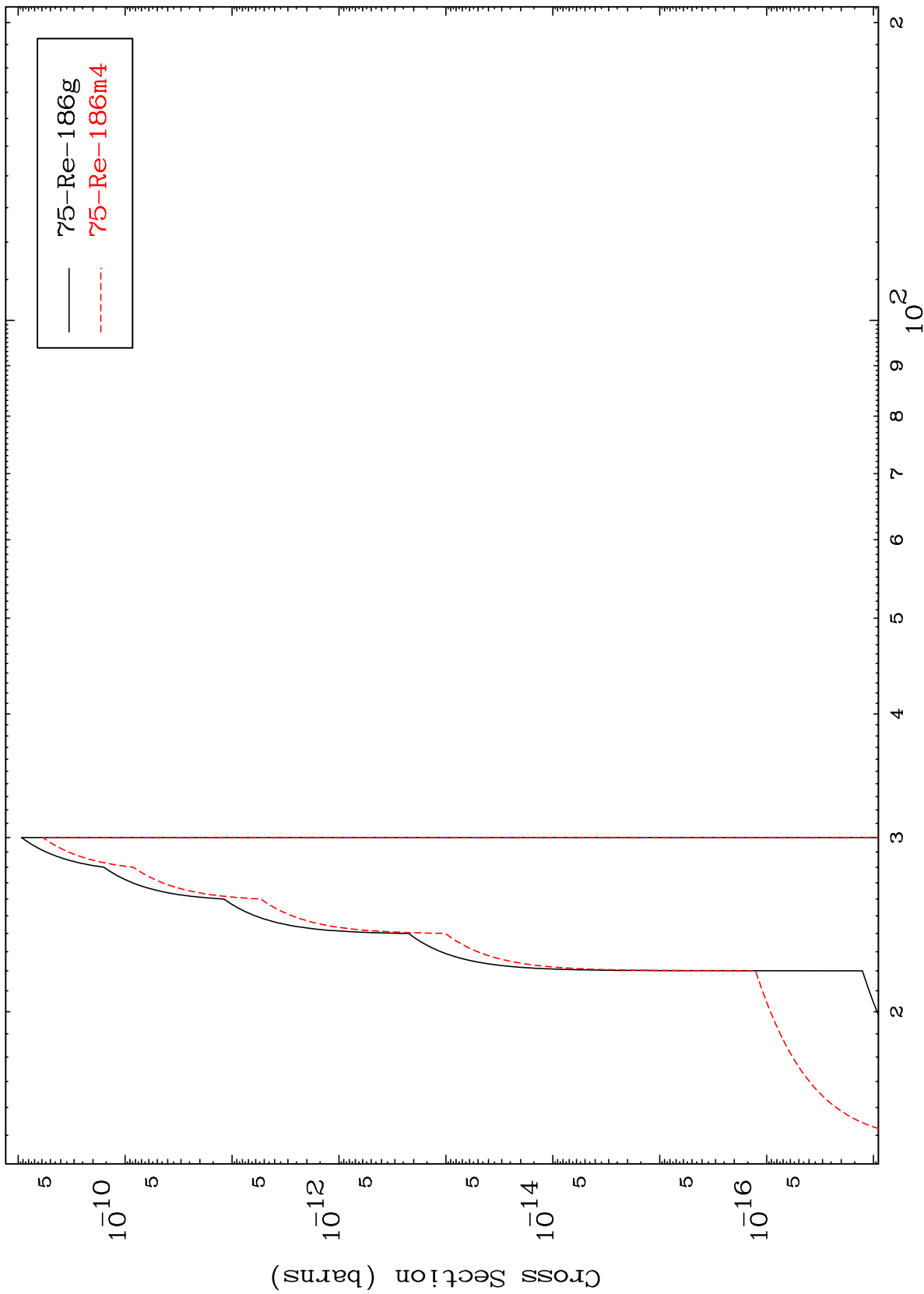
Incident Energy (MeV)

75-Re-184

MAT 7523

75-Re-184

($\alpha, 2p$)
Radionuclide Production Cross Section



16

75-Re-184

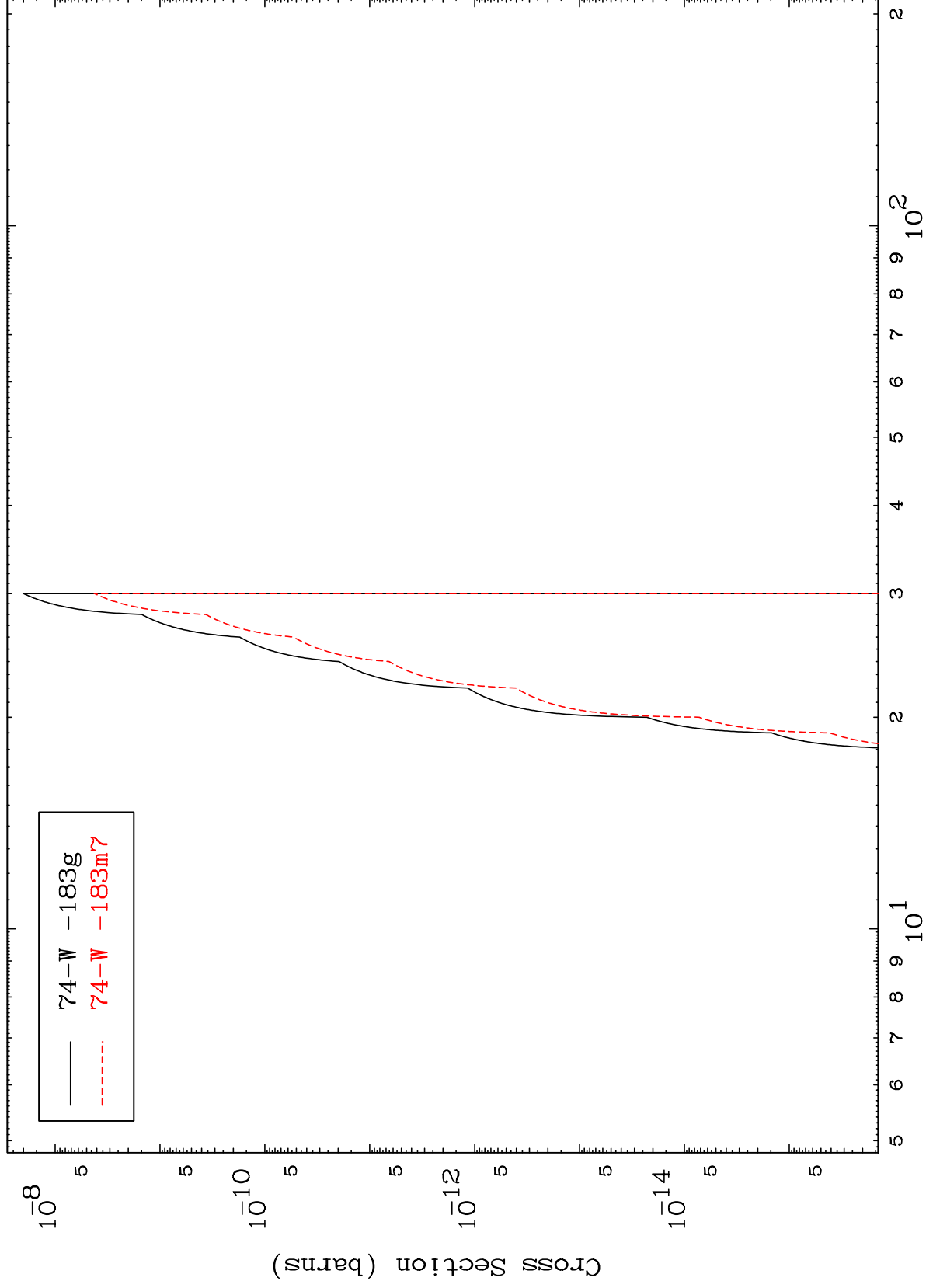
Incident Energy (MeV)

MAT 7523

(α, p) α

75-Re-184

Radionuclide Production Cross Section



17

Incident Energy (MeV)

75-Re-184