

Program EVALPLOT  
(Version 2018-1)

by

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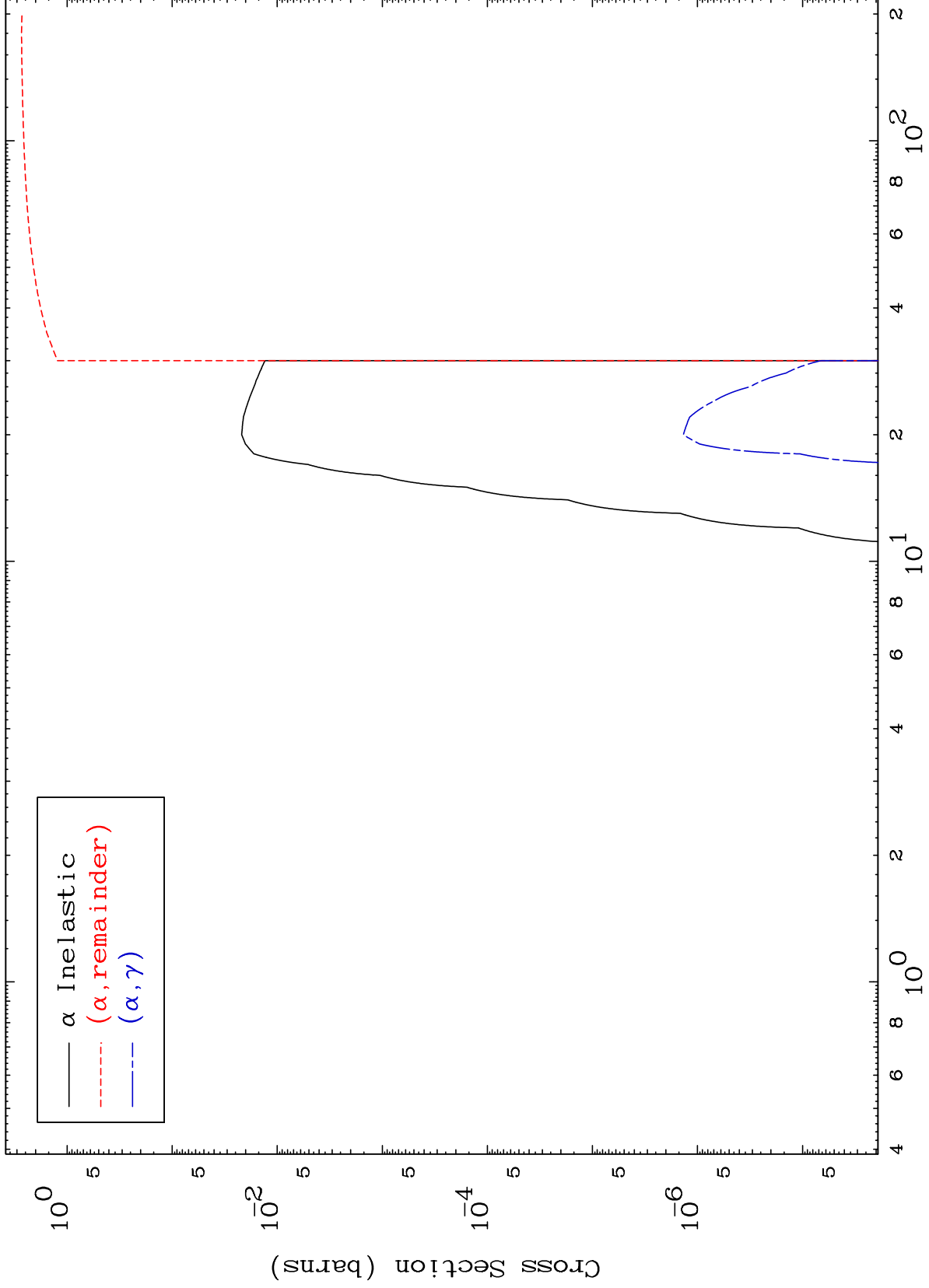
Press Mouse Button to Start

MAT 7541

$\alpha$  Major

75-Re-190

0 Kelvin Cross Sections

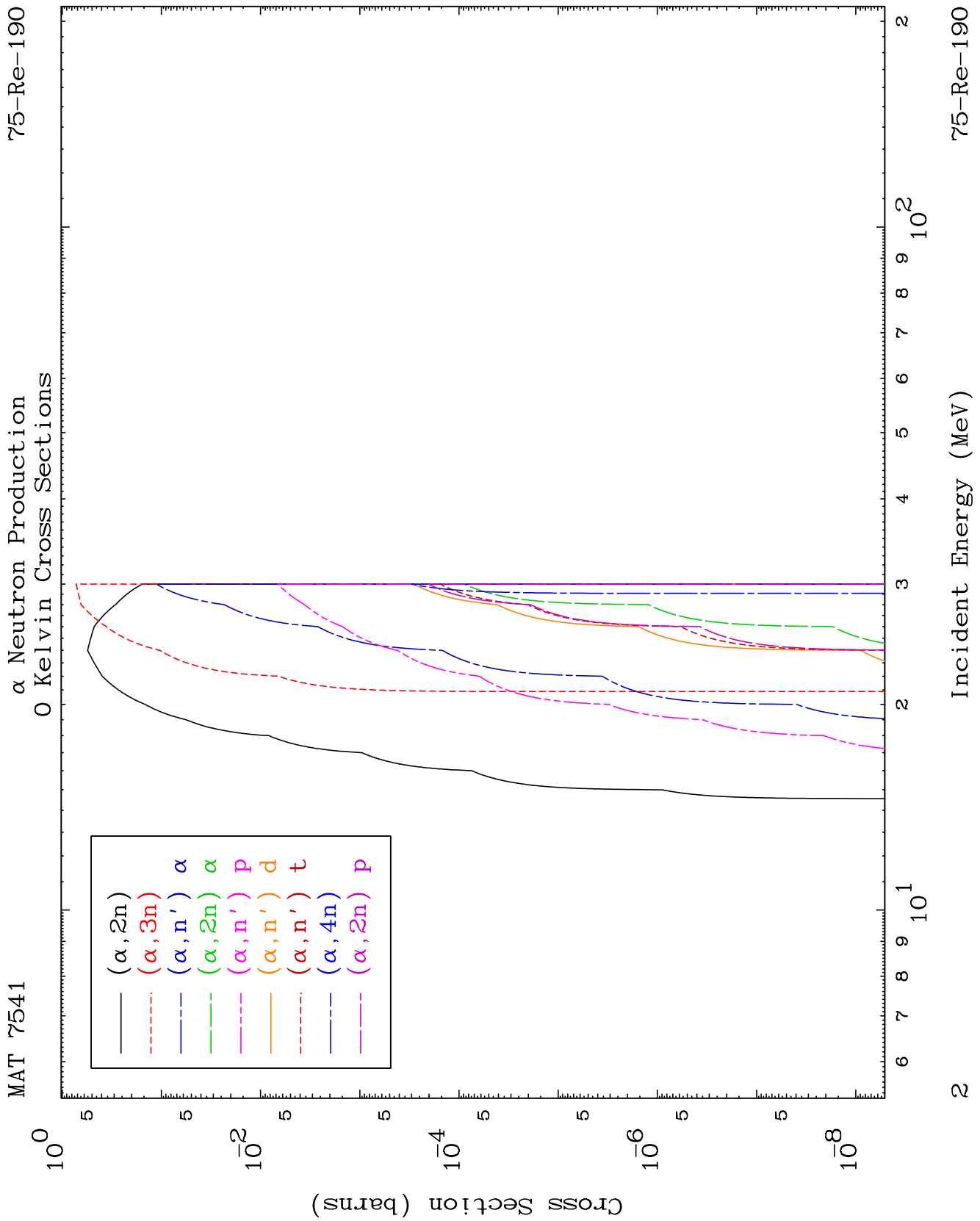


$\alpha$  Inelastic  
 $(\alpha, \text{remainder})$   
 $(\alpha, \gamma)$

Incident Energy (MeV)

75-Re-190

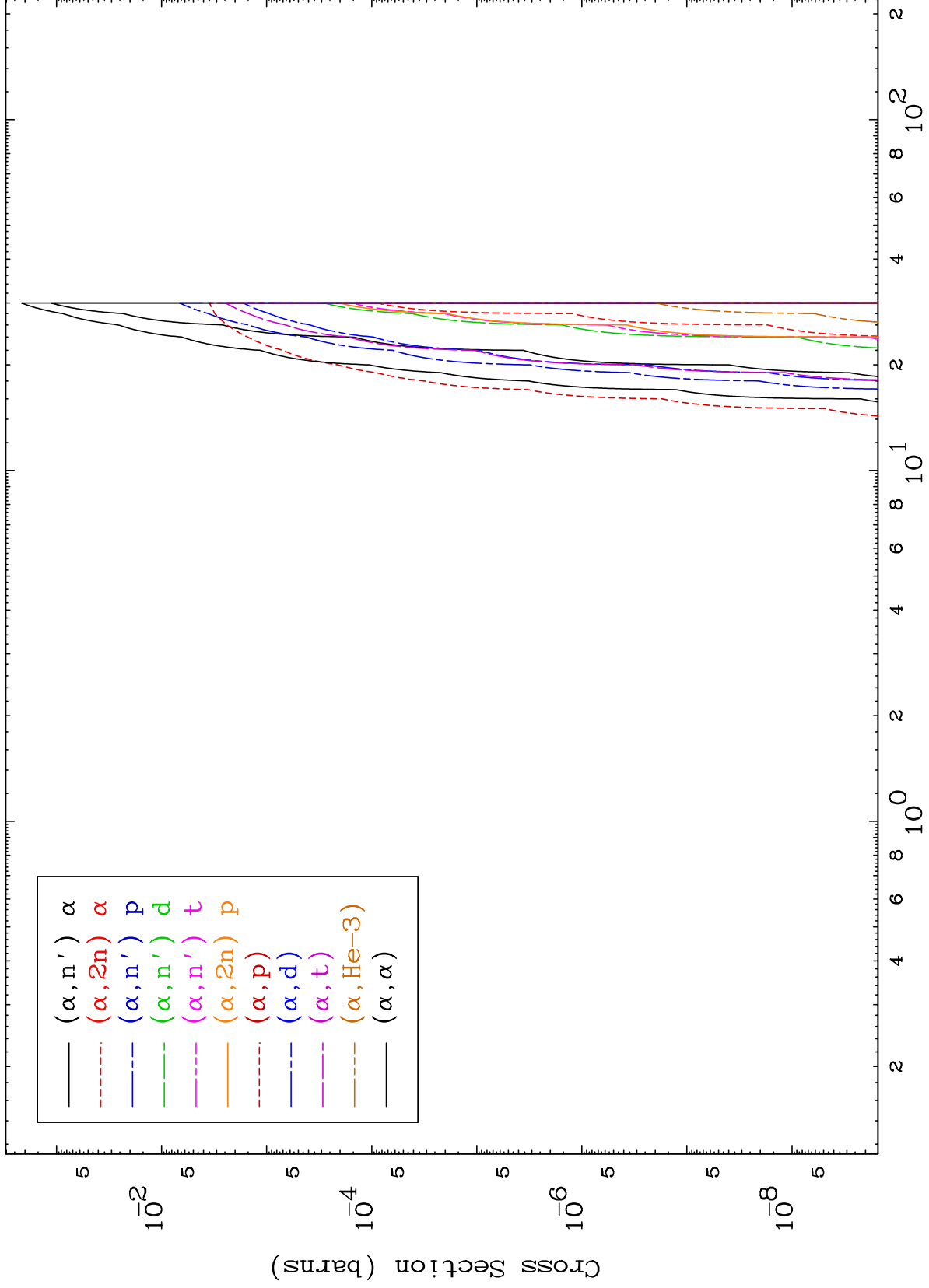
1



MAT 7541

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

75-Re-190

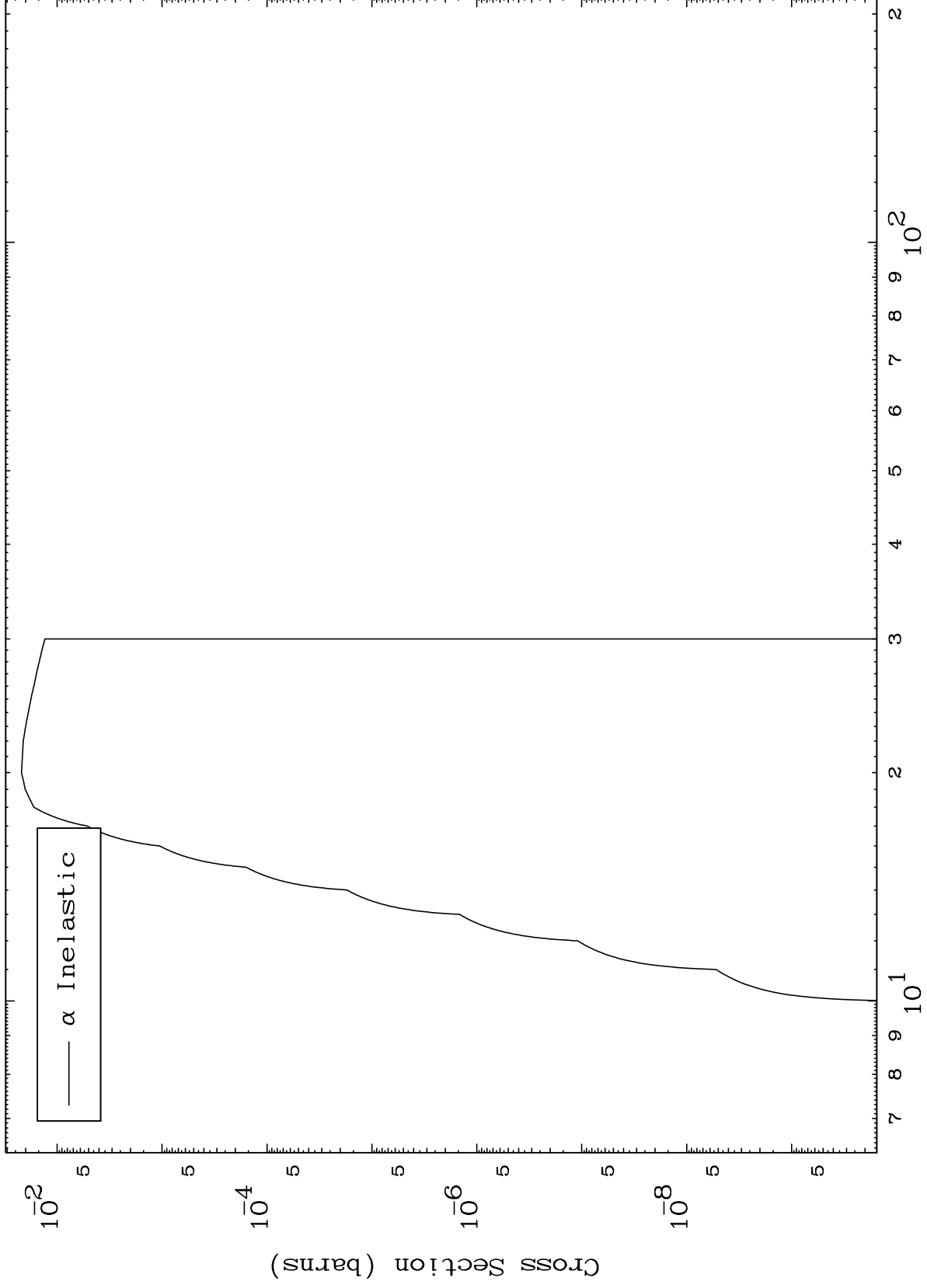


MAT 7541

( $\alpha, n'$ ) Level

75-Re-190

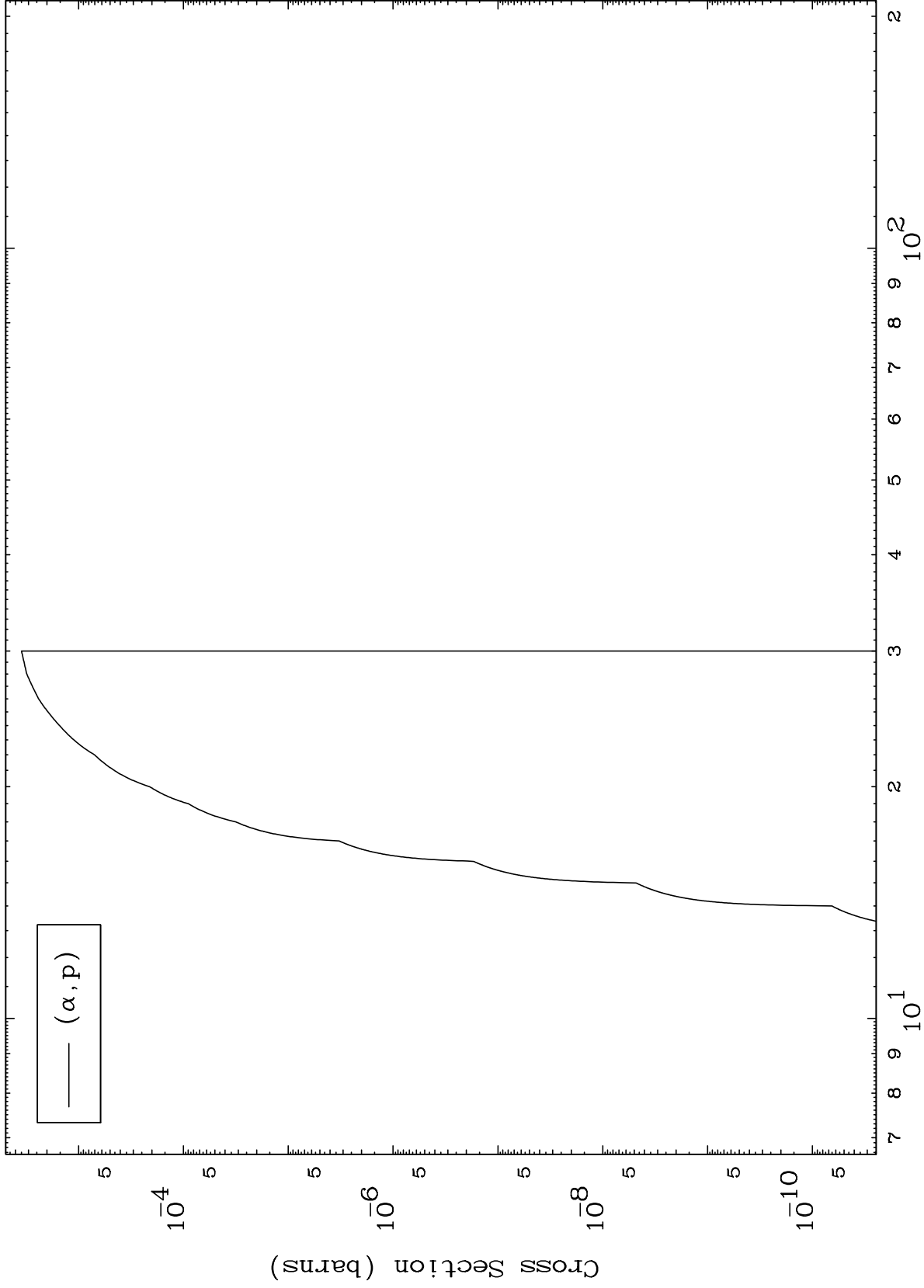
0 Kelvin Cross Sections



MAT 7541

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

75-Re-190



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Incident Energy (MeV)

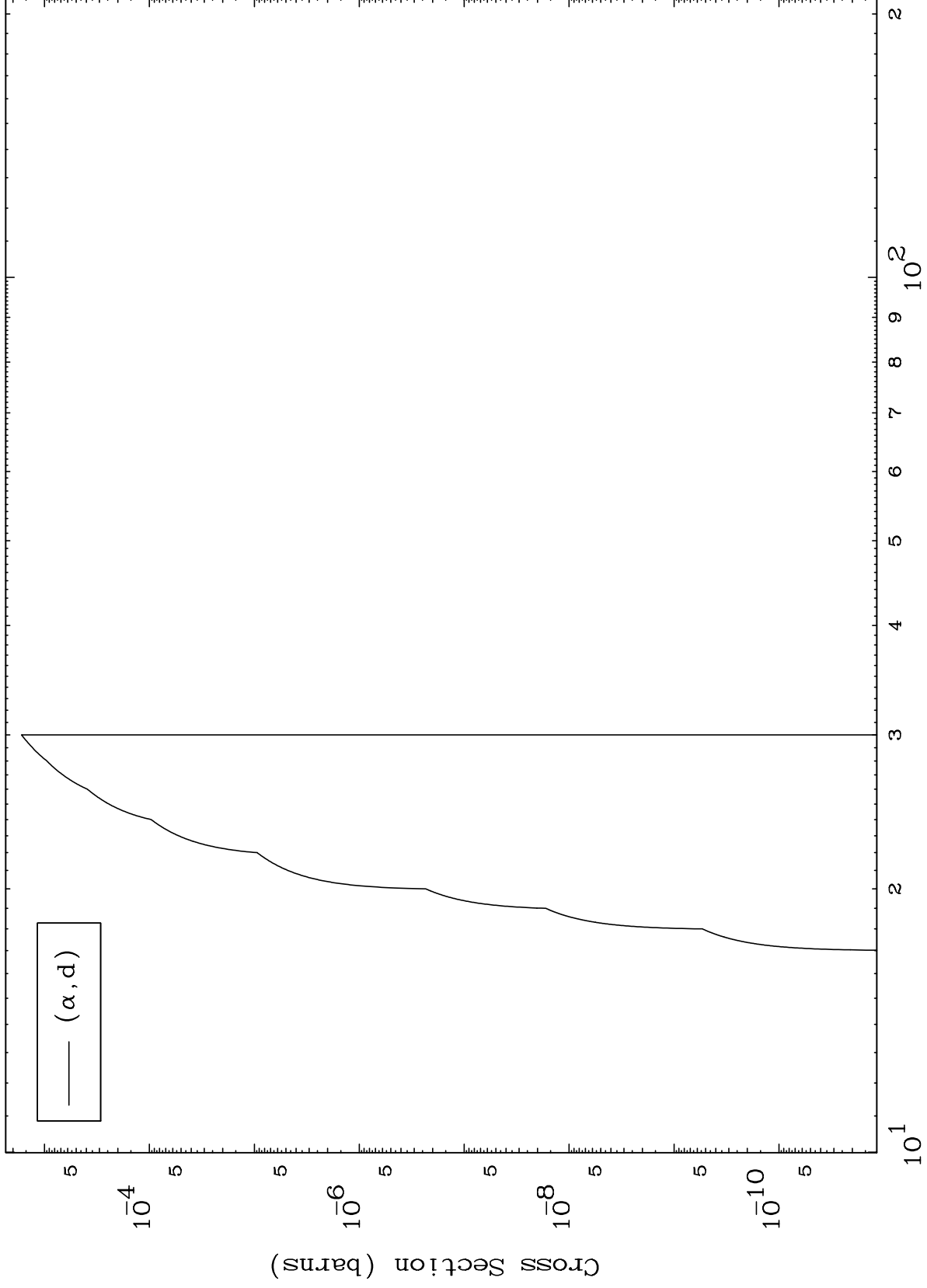
75-Re-190

MAT 7541

( $\alpha, d$ ) Levels

75-Re-190

0 Kelvin Cross Sections



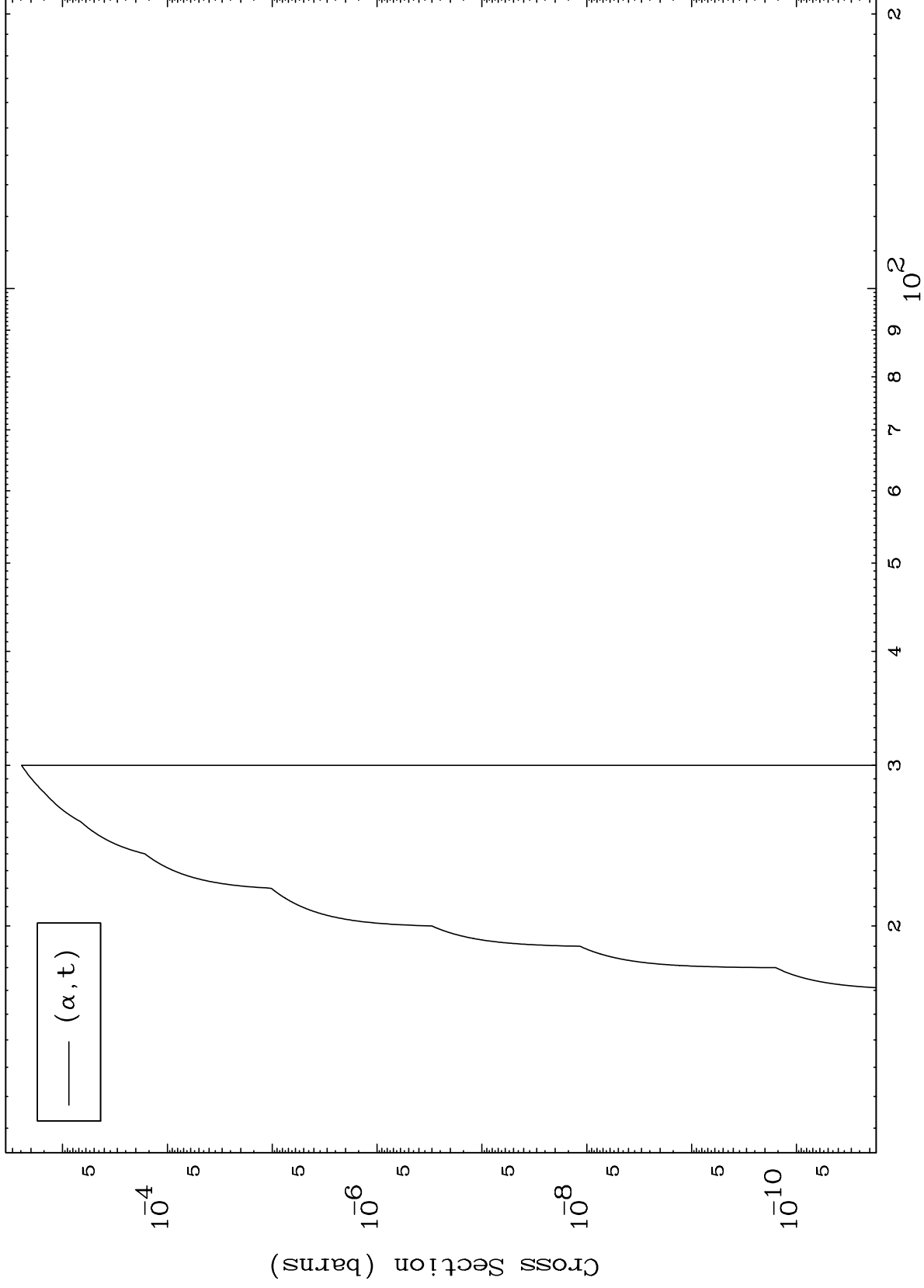
Incident Energy (MeV)

75-Re-190

MAT 7541

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

75-Re-190

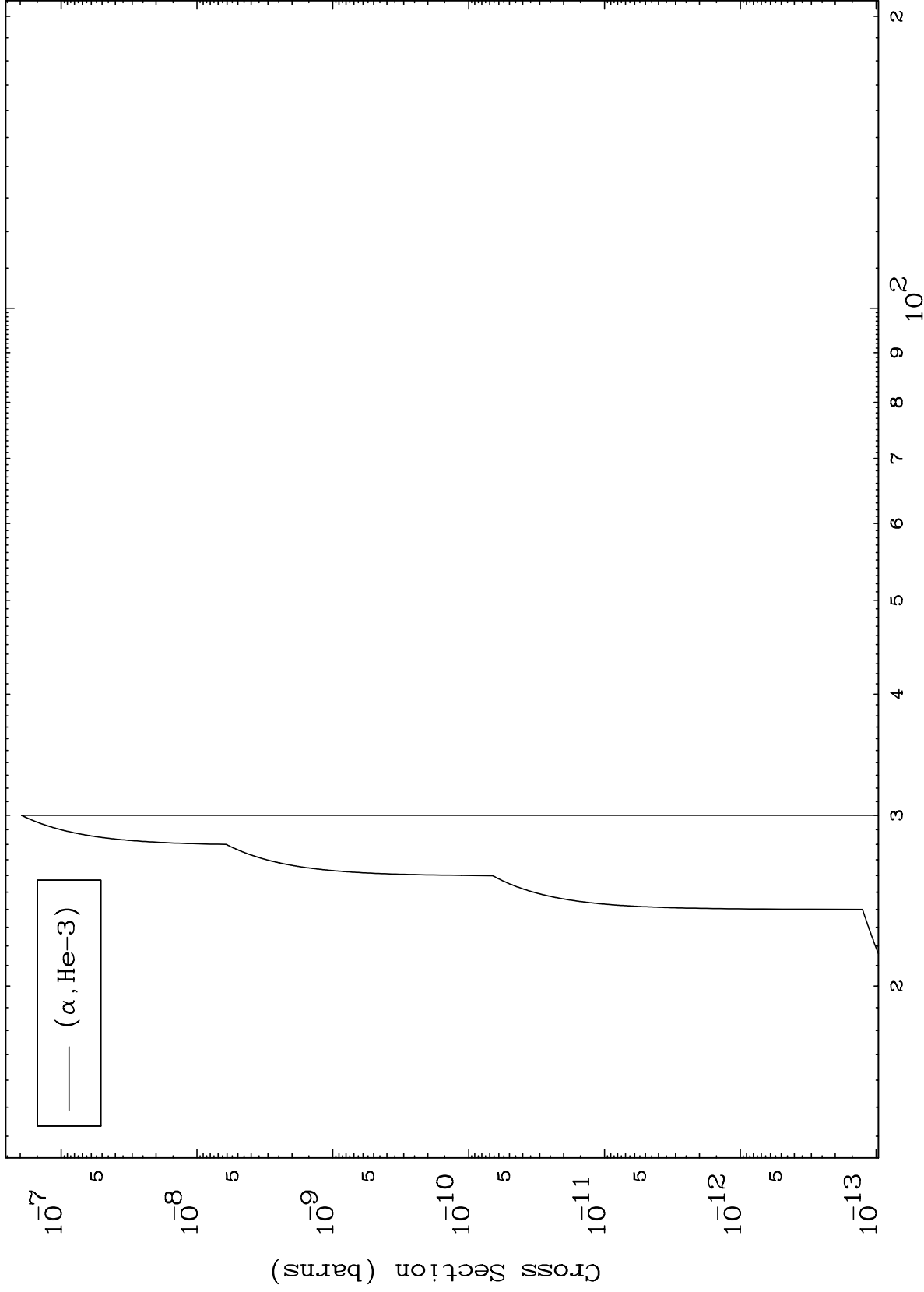


7

Incident Energy (MeV)

75-Re-190



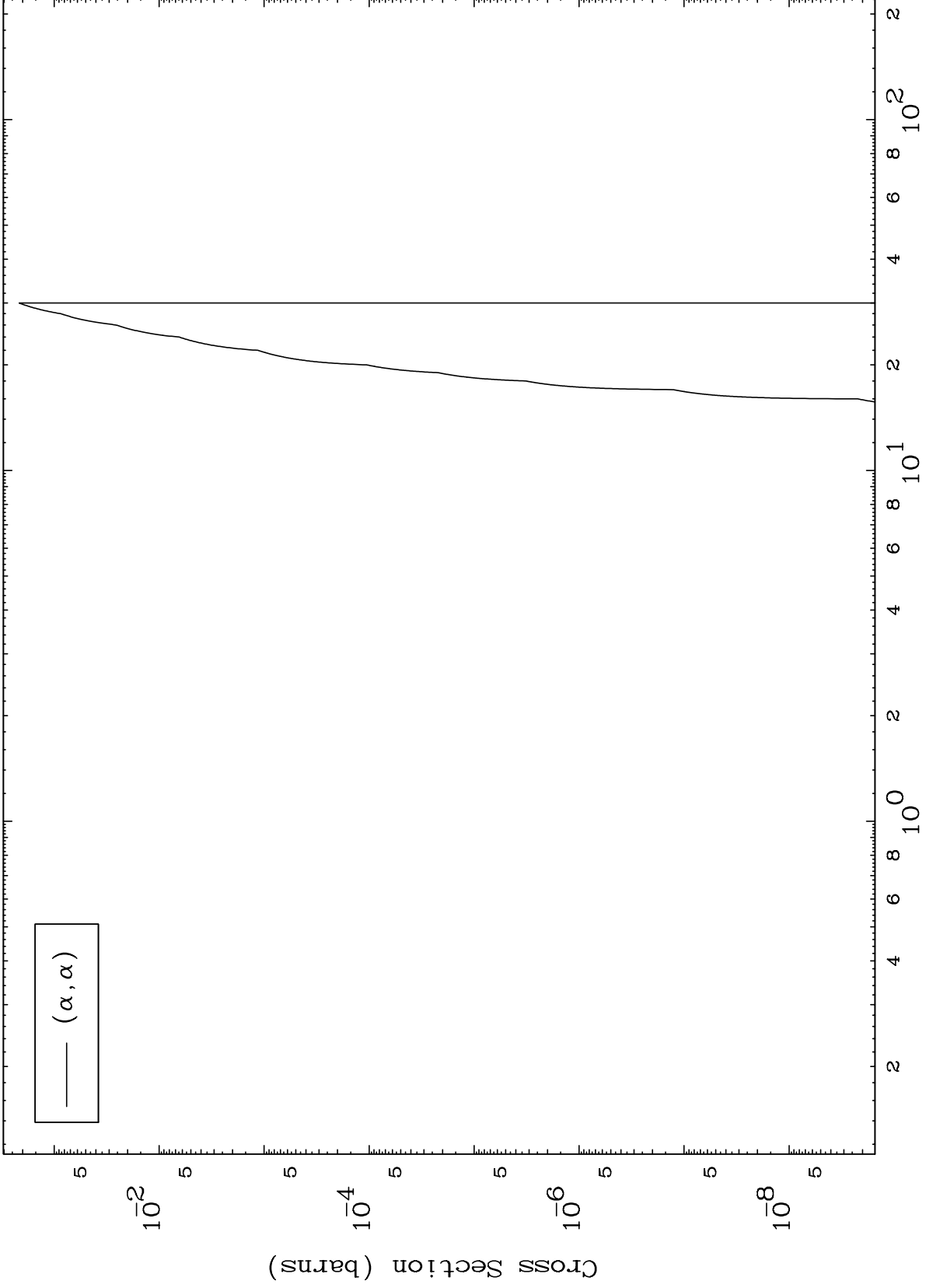


MAT 7541

( $\alpha, \alpha$ ) Levels

75-Re-190

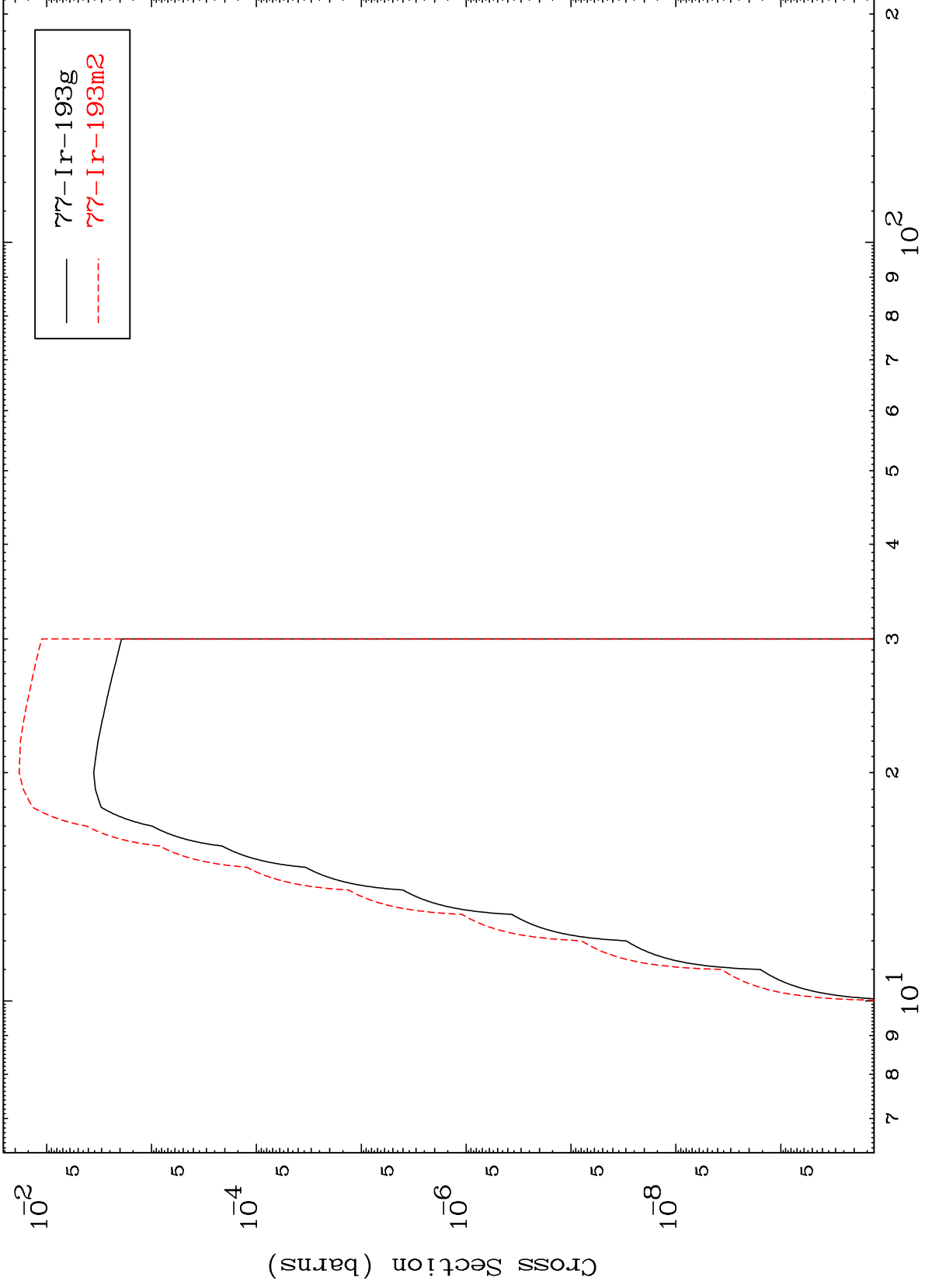
0 Kelvin Cross Sections



MAT 7541

75-Re-190

$\alpha$  Inelastic  
Radionuclide Production Cross Section



10

Incident Energy (MeV)

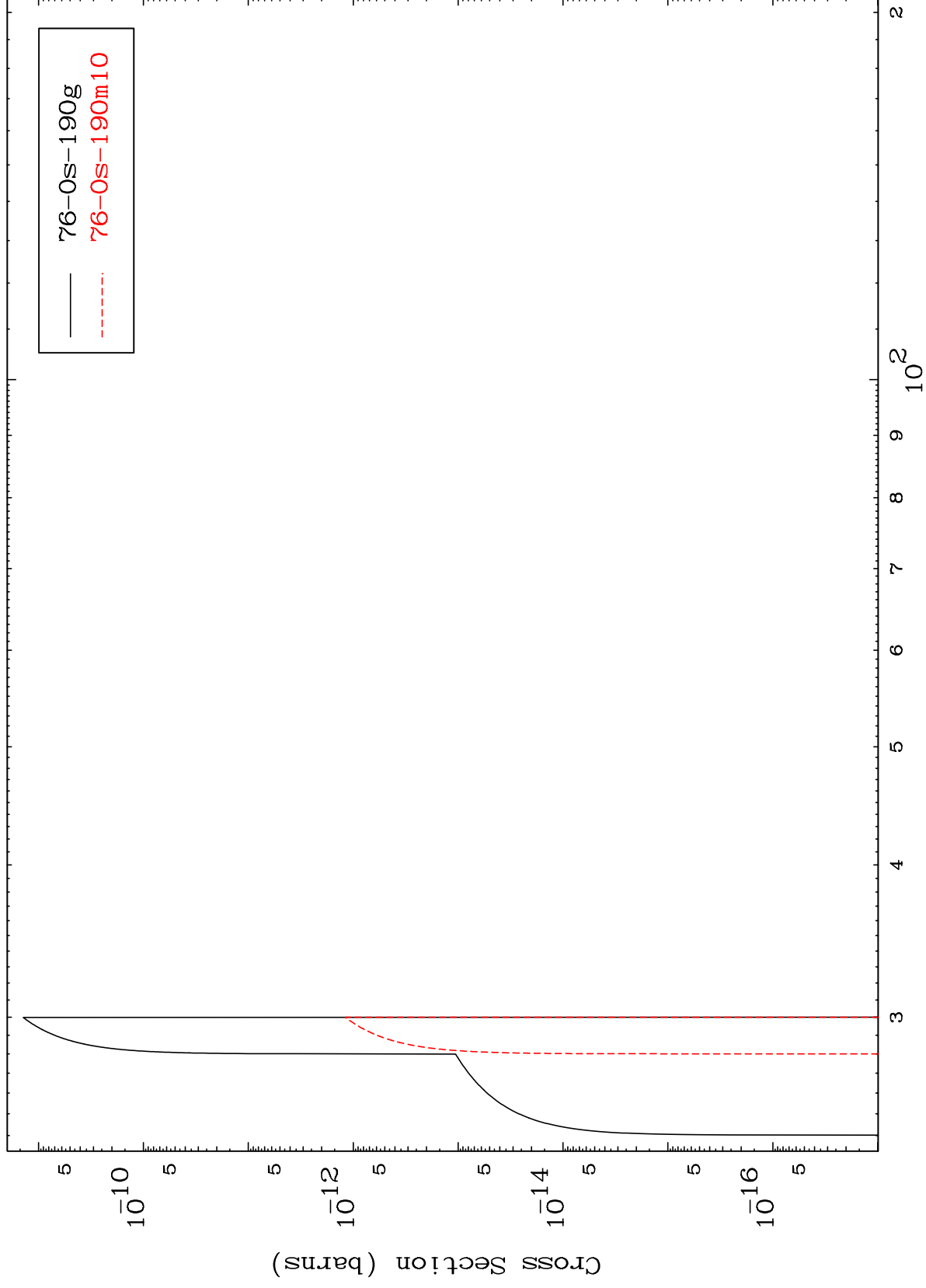
75-Re-190

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$(\alpha, 2n)$  d

75-Re-190

Radionuclide Production Cross Section

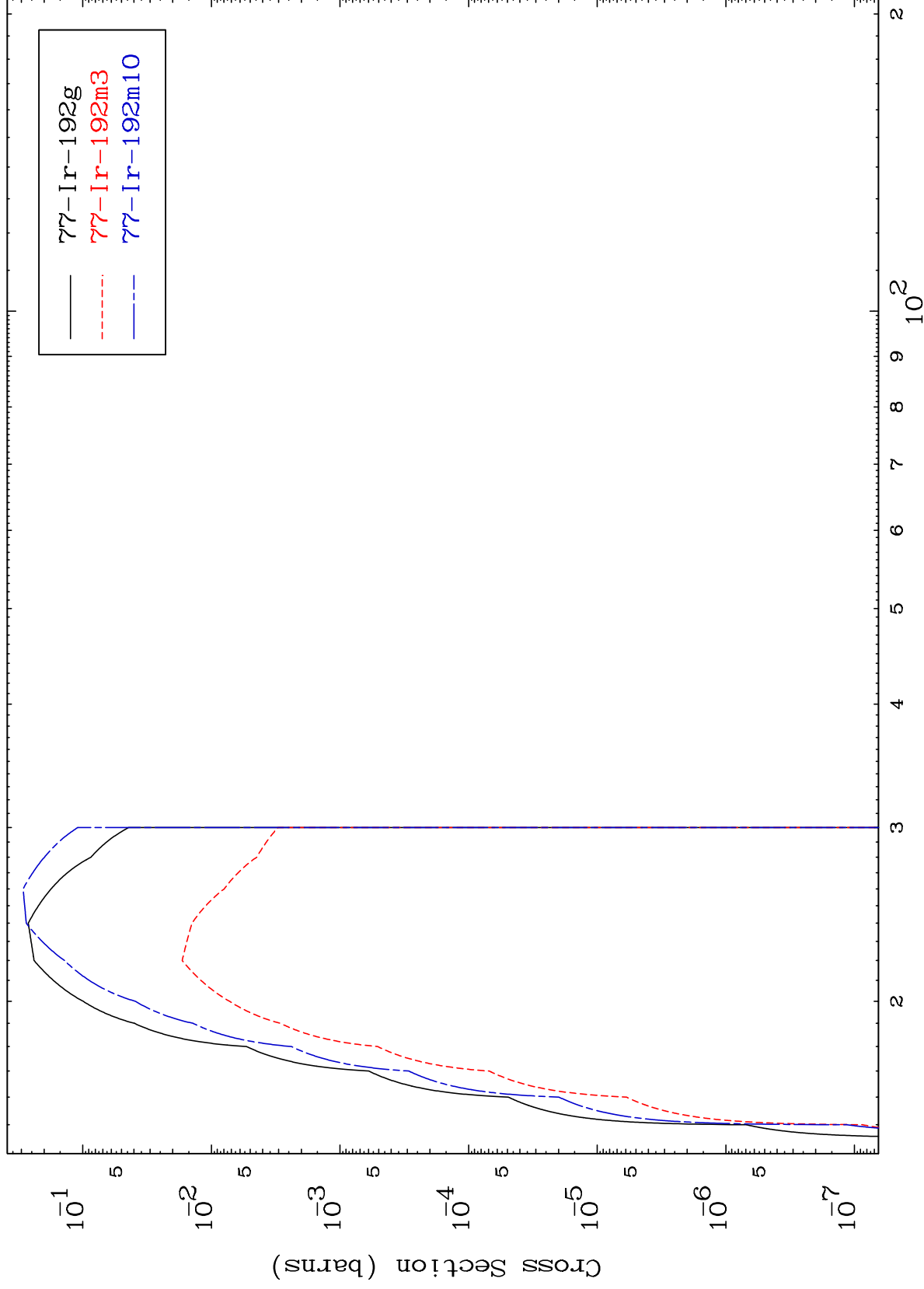


11

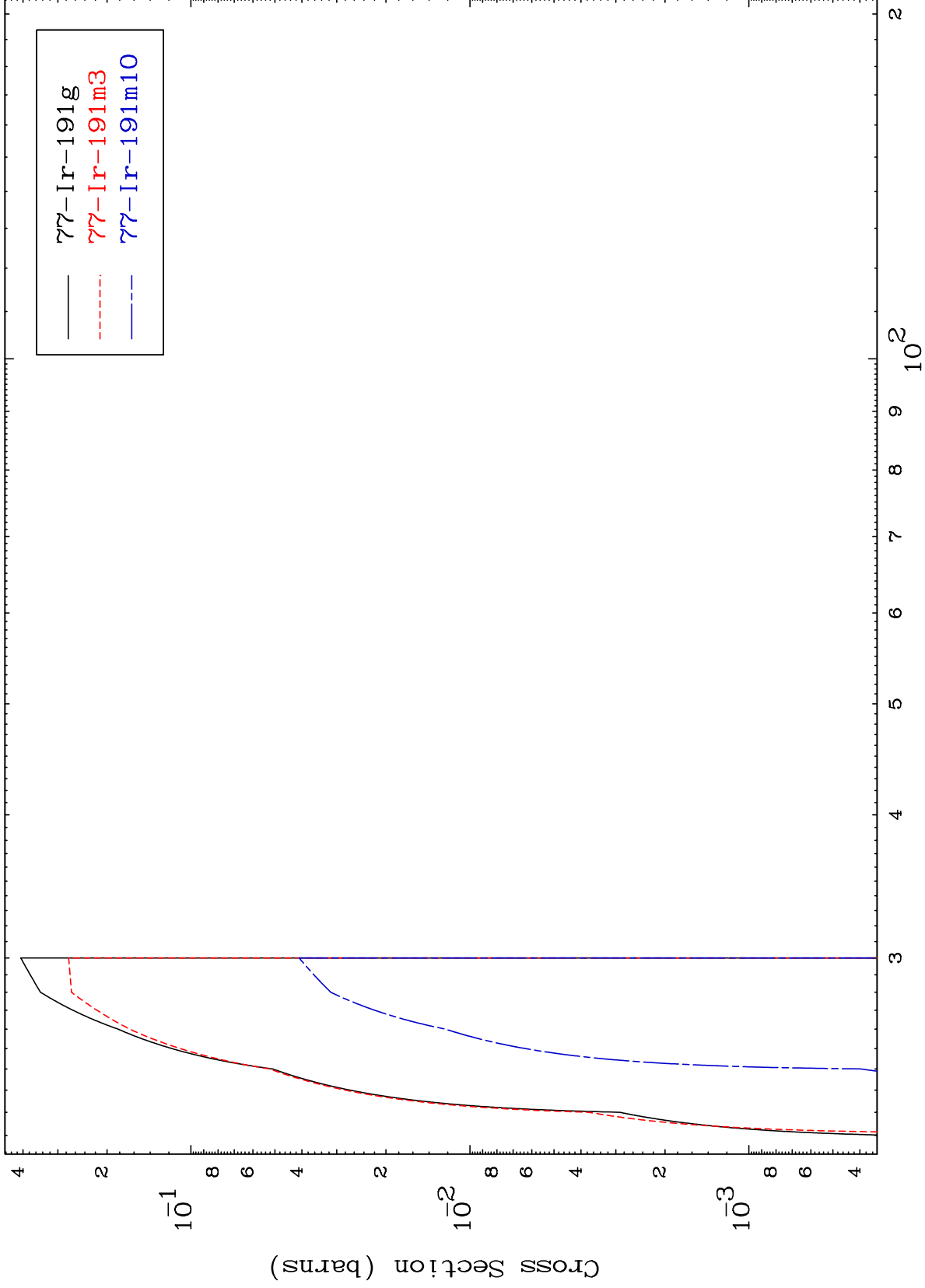
Incident Energy (MeV)

75-Re-190

( $\alpha, 2n$ )  
Radionuclide Production Cross Section



Radionuclide Production Cross Section

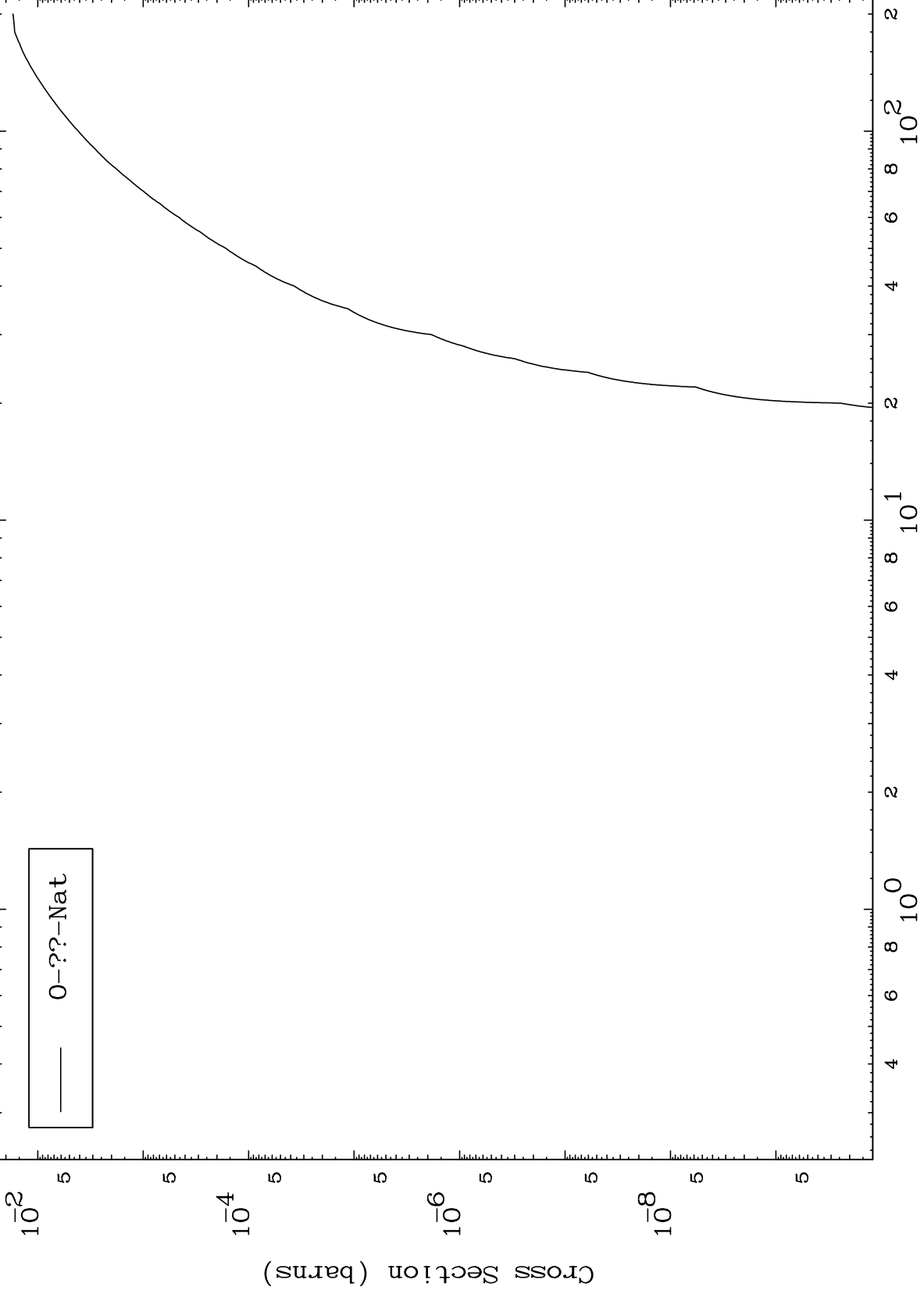


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$\alpha$  Fission

75-Re-190

Radionuclide Production Cross Section



14

Incident Energy (MeV)

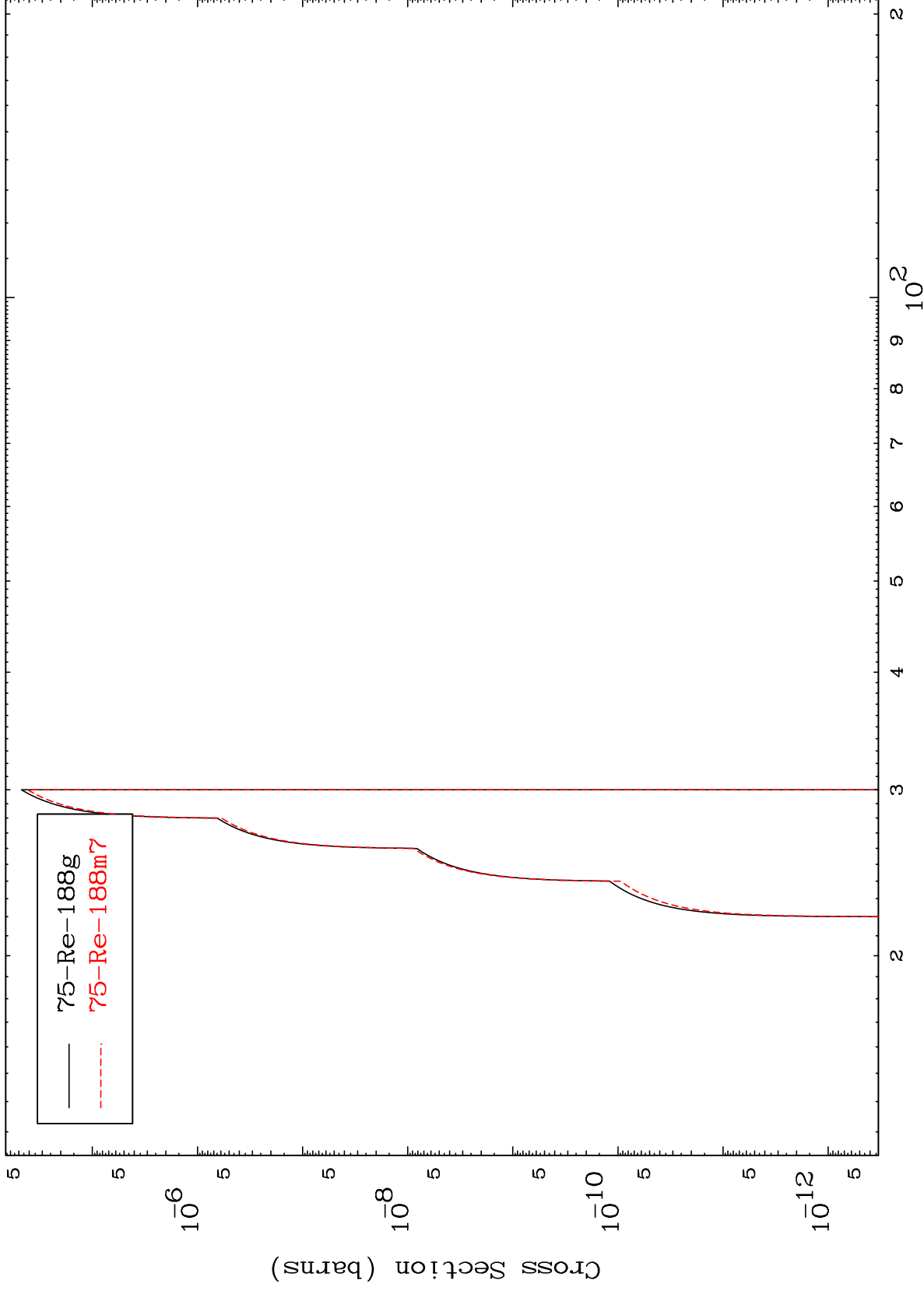
75-Re-190

MAT 7541

( $\alpha, 2n$ )  $\alpha$

75-Re-190

Radionuclide Production Cross Section



15

Incident Energy (MeV)

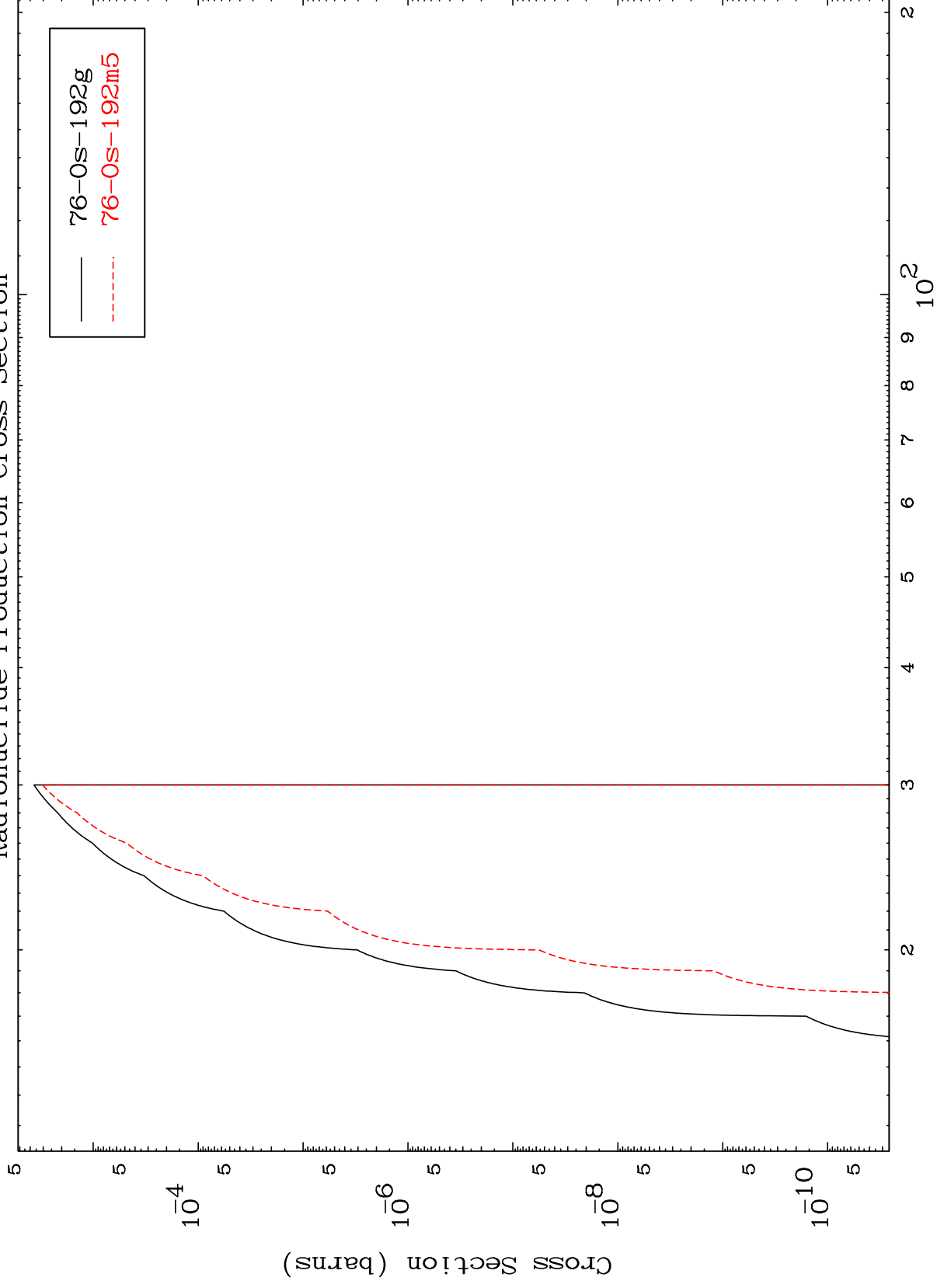
75-Re-190



MAT 7541

75-Re-190

$(\alpha, n')$  p  
Radionuclide Production Cross Section



16

Incident Energy (MeV)

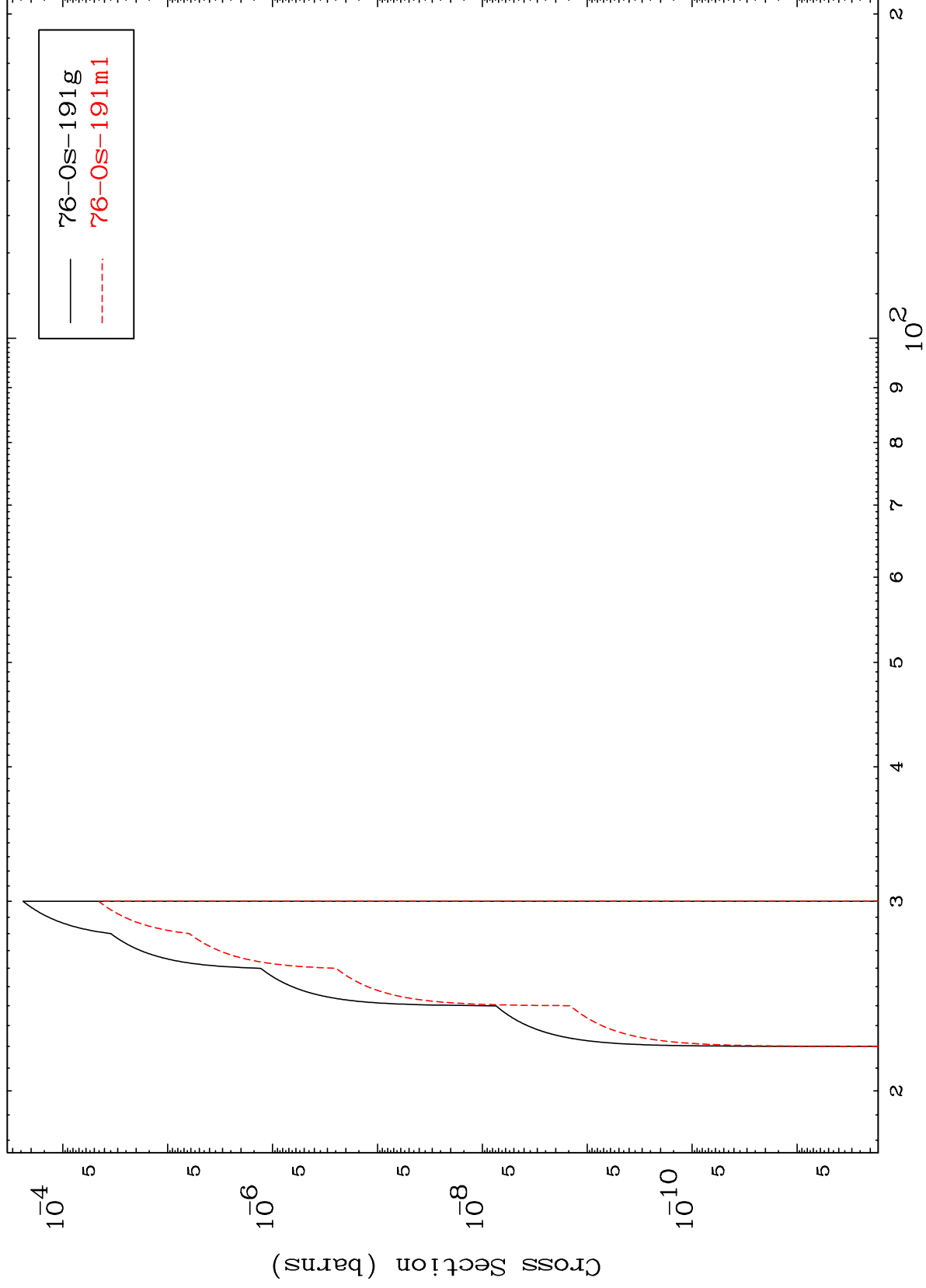
75-Re-190

MAT 7541

$(\alpha, n')$  d

75-Re-190

Radionuclide Production Cross Section



17

Incident Energy (MeV)

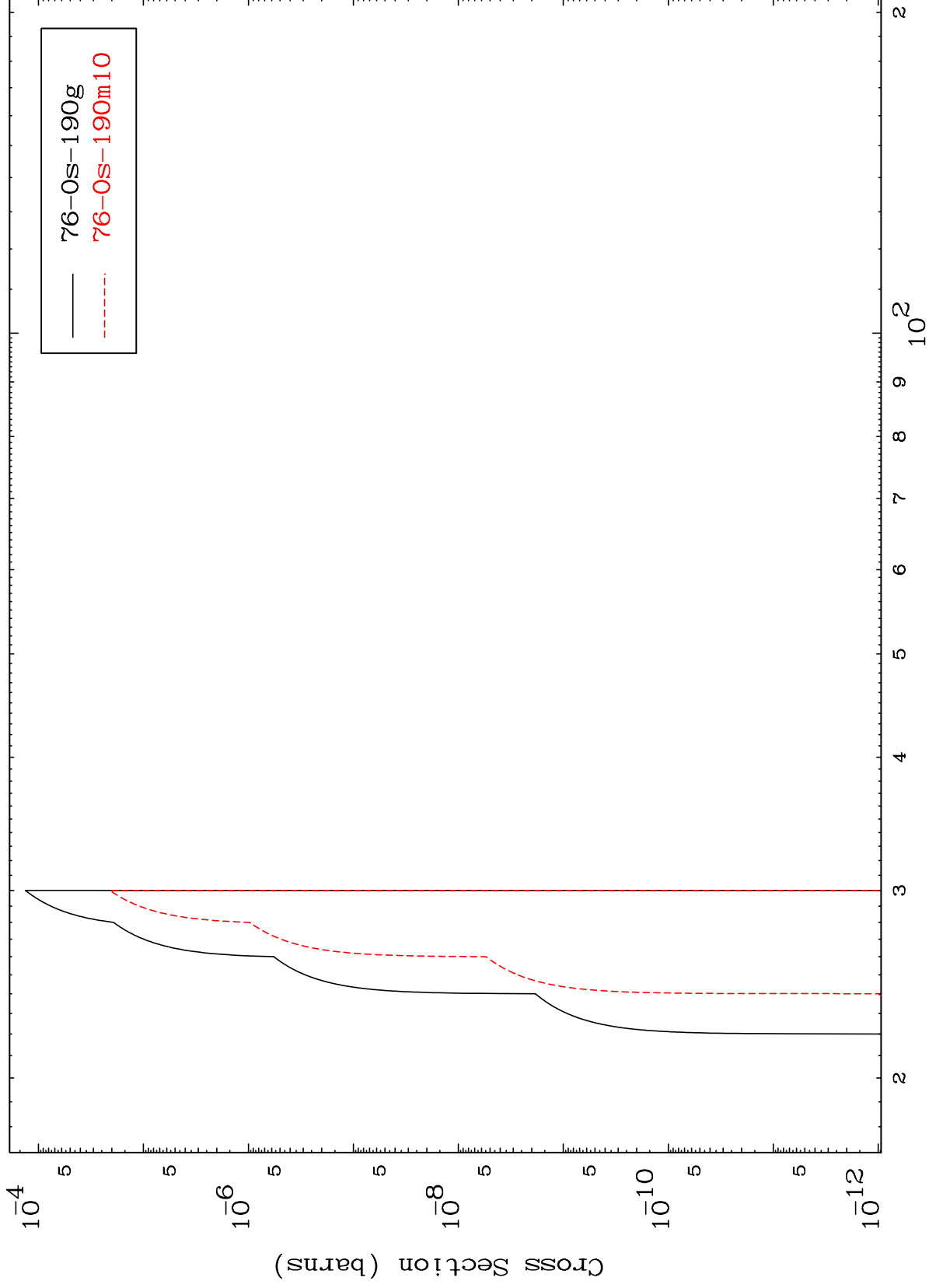
75-Re-190

MAT 7541

$(\alpha, n')$  t

75-Re-190

Radionuclide Production Cross Section



18

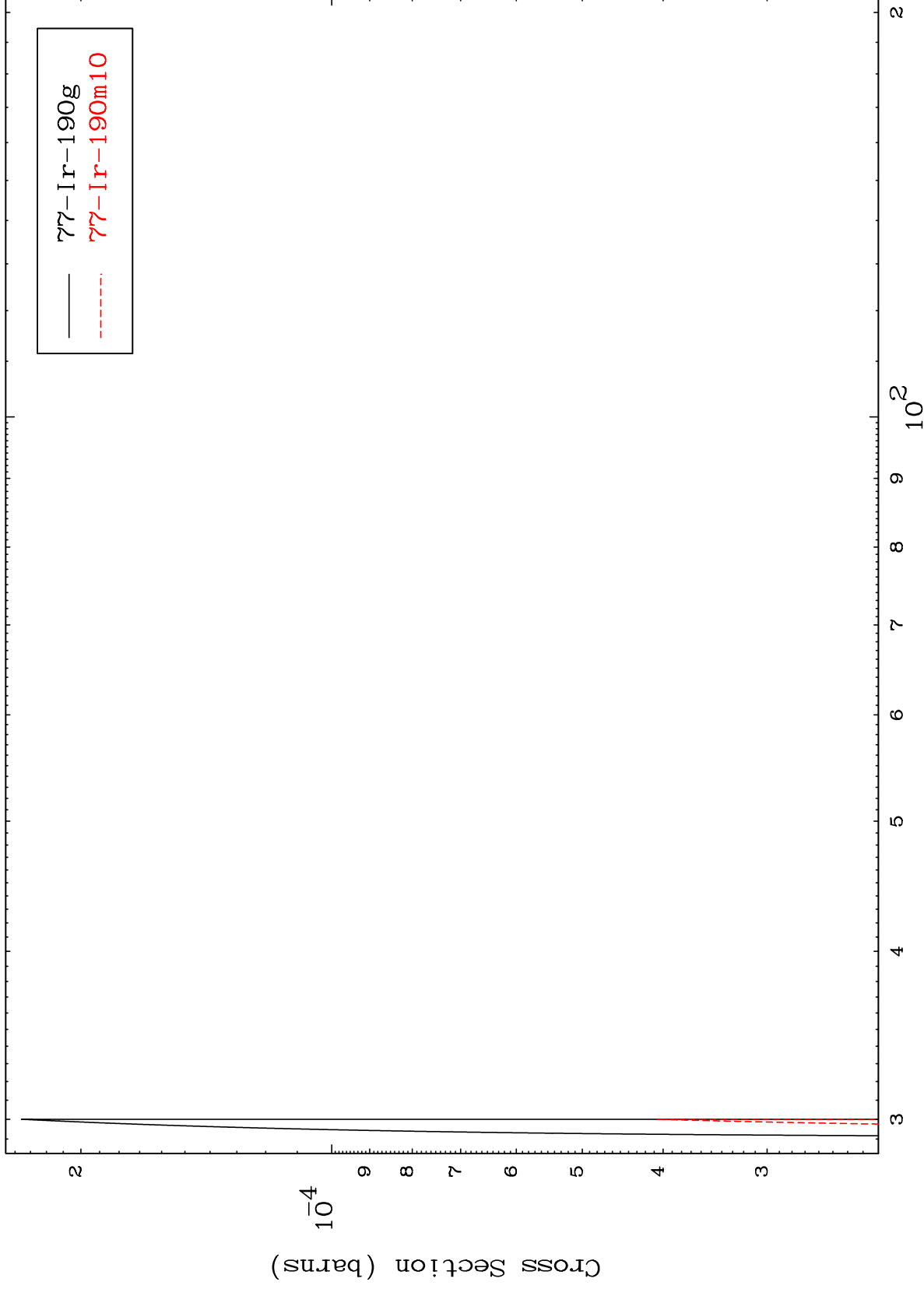
Incident Energy (MeV)

75-Re-190

MAT 7541

75-Re-190

( $\alpha, 4n$ )  
Radionuclide Production Cross Section



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75-Re-190

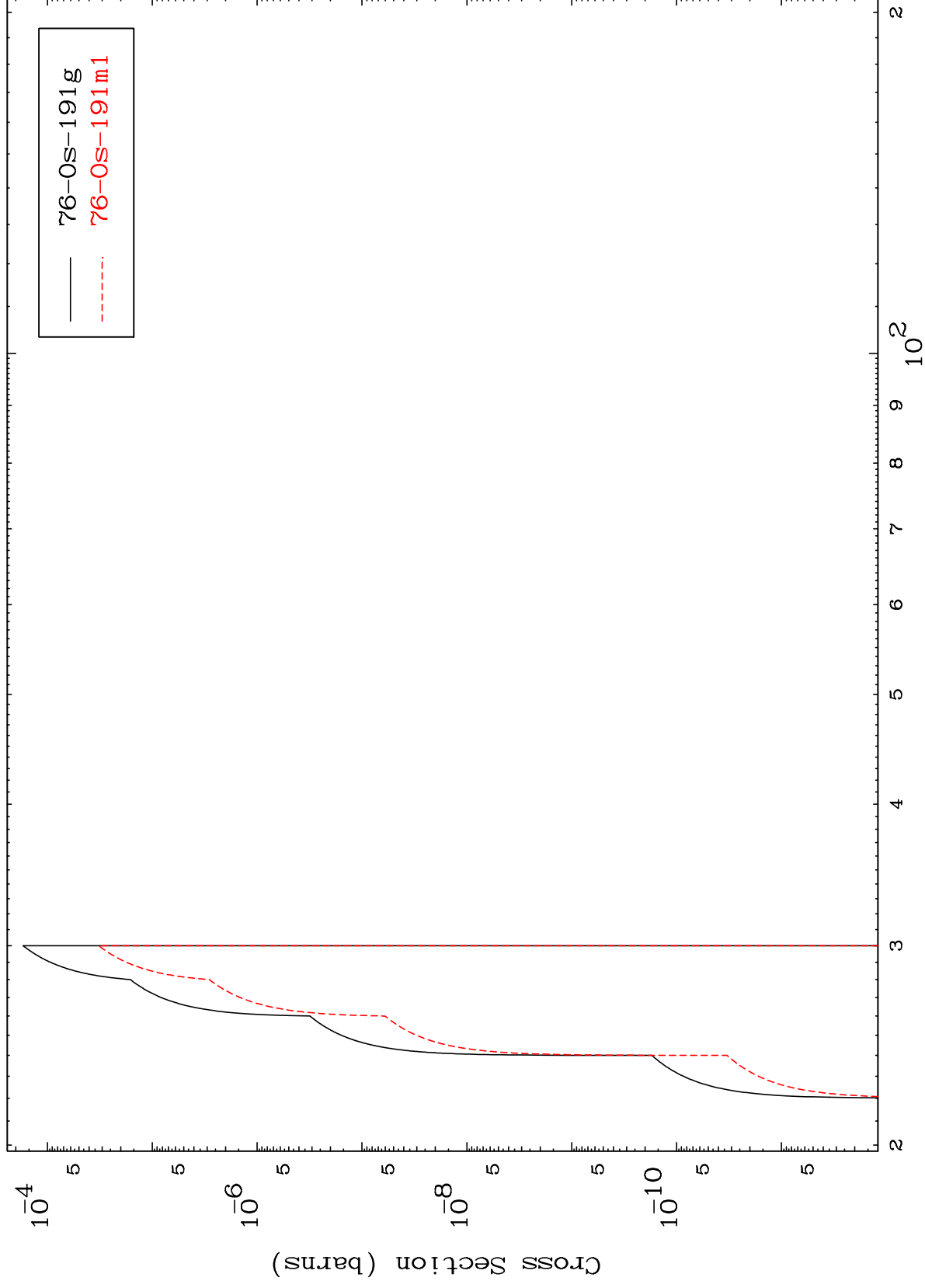
Incident Energy (MeV)

MAT 7541

$(\alpha, 2n)$  p

75-Re-190

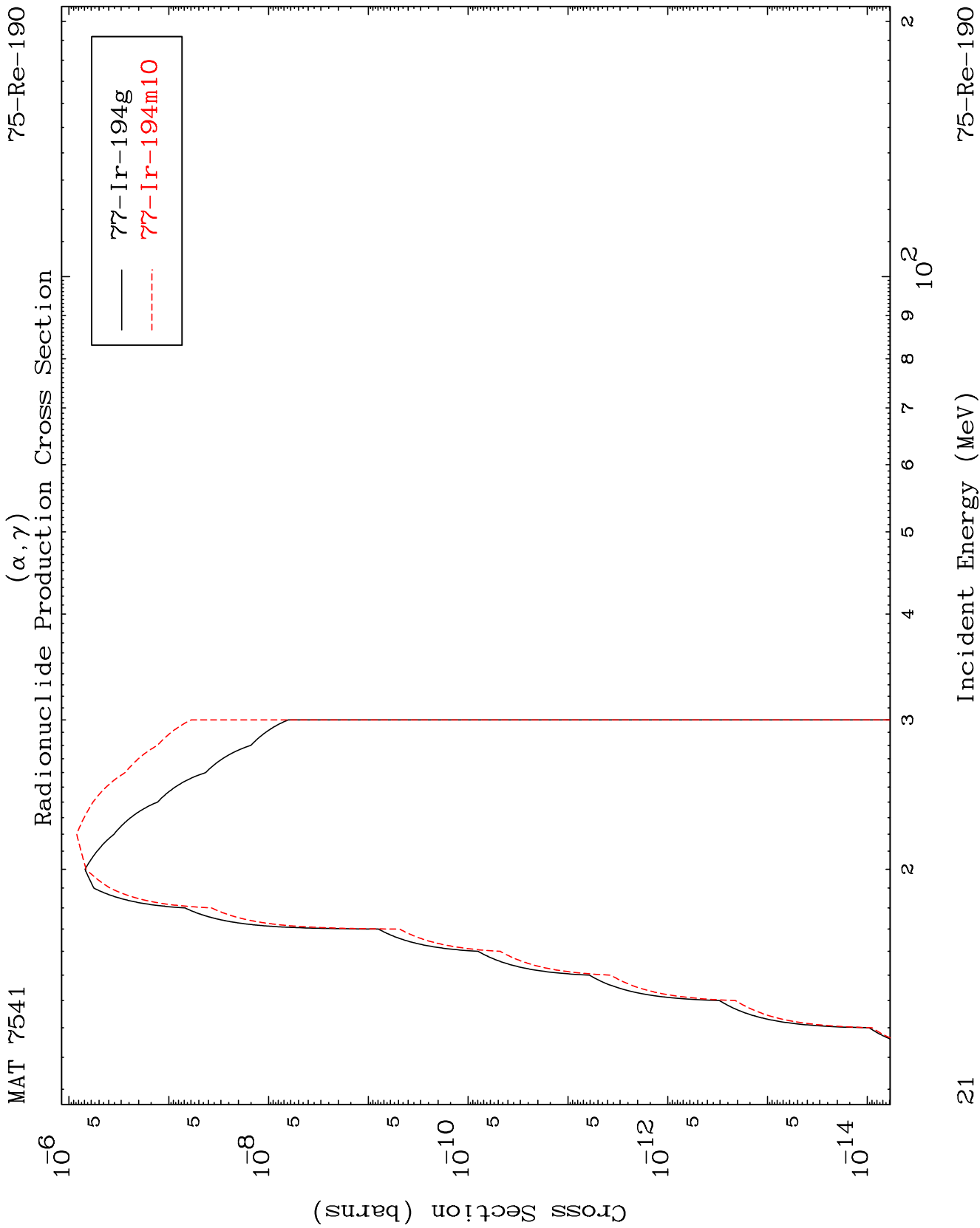
Radionuclide Production Cross Section



20

Incident Energy (MeV)

75-Re-190

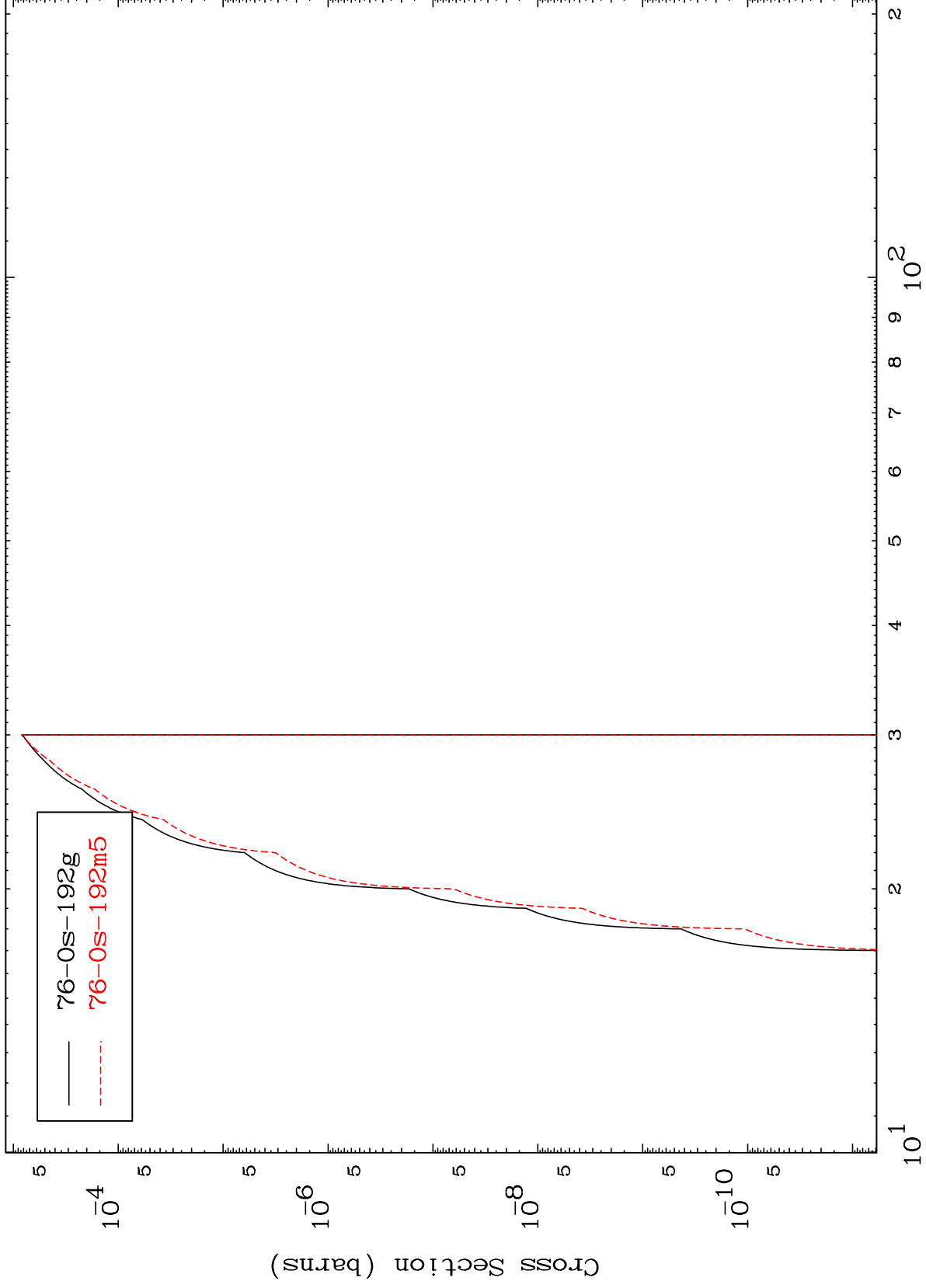


MAT 7541

( $\alpha, d$ )

75-Re-190

Radionuclide Production Cross Section



Incident Energy (MeV)

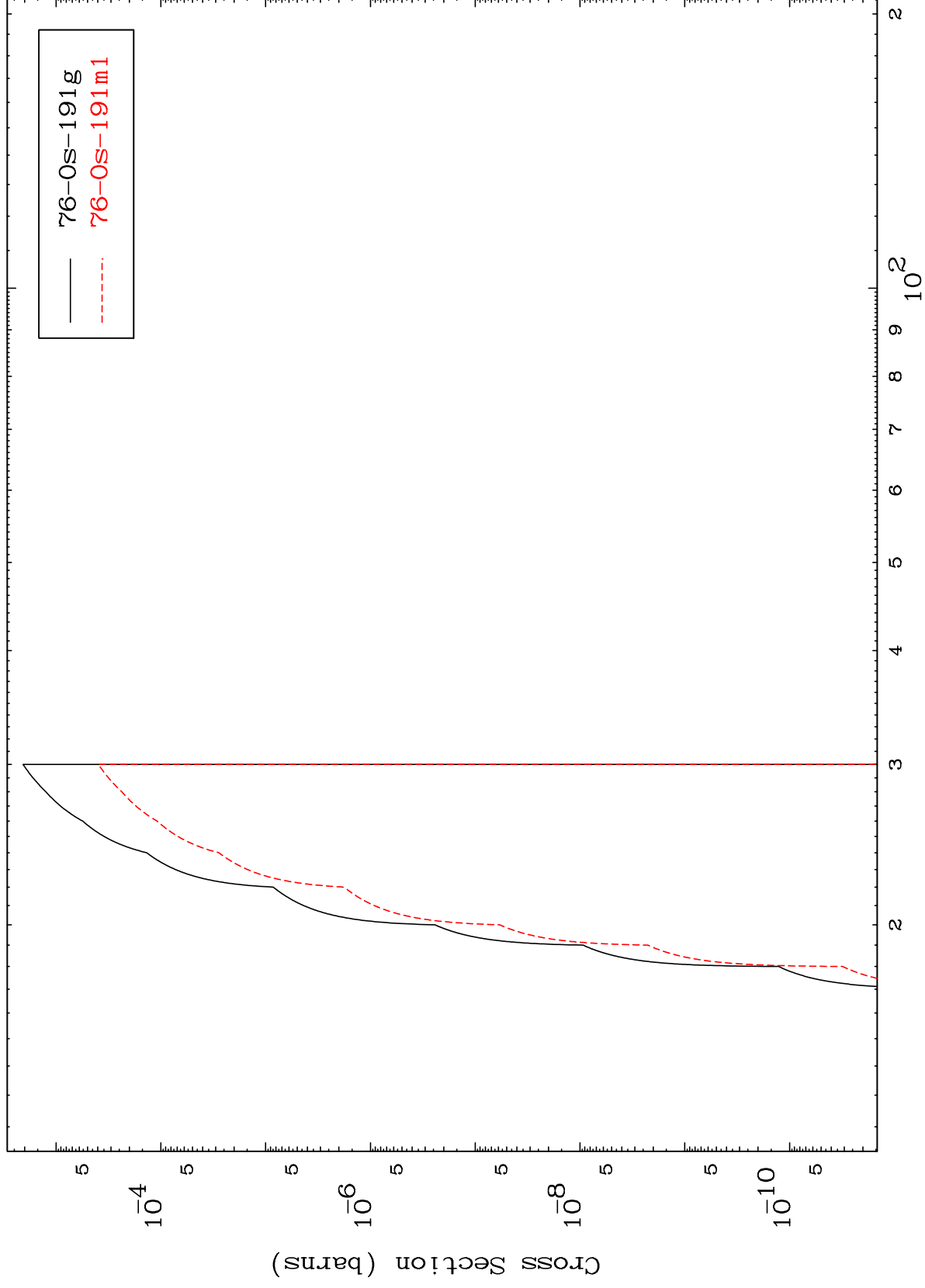
75-Re-190

22

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75-Re-190

( $\alpha, t$ )  
Radionuclide Production Cross Section



23

75-Re-190

Incident Energy (MeV)

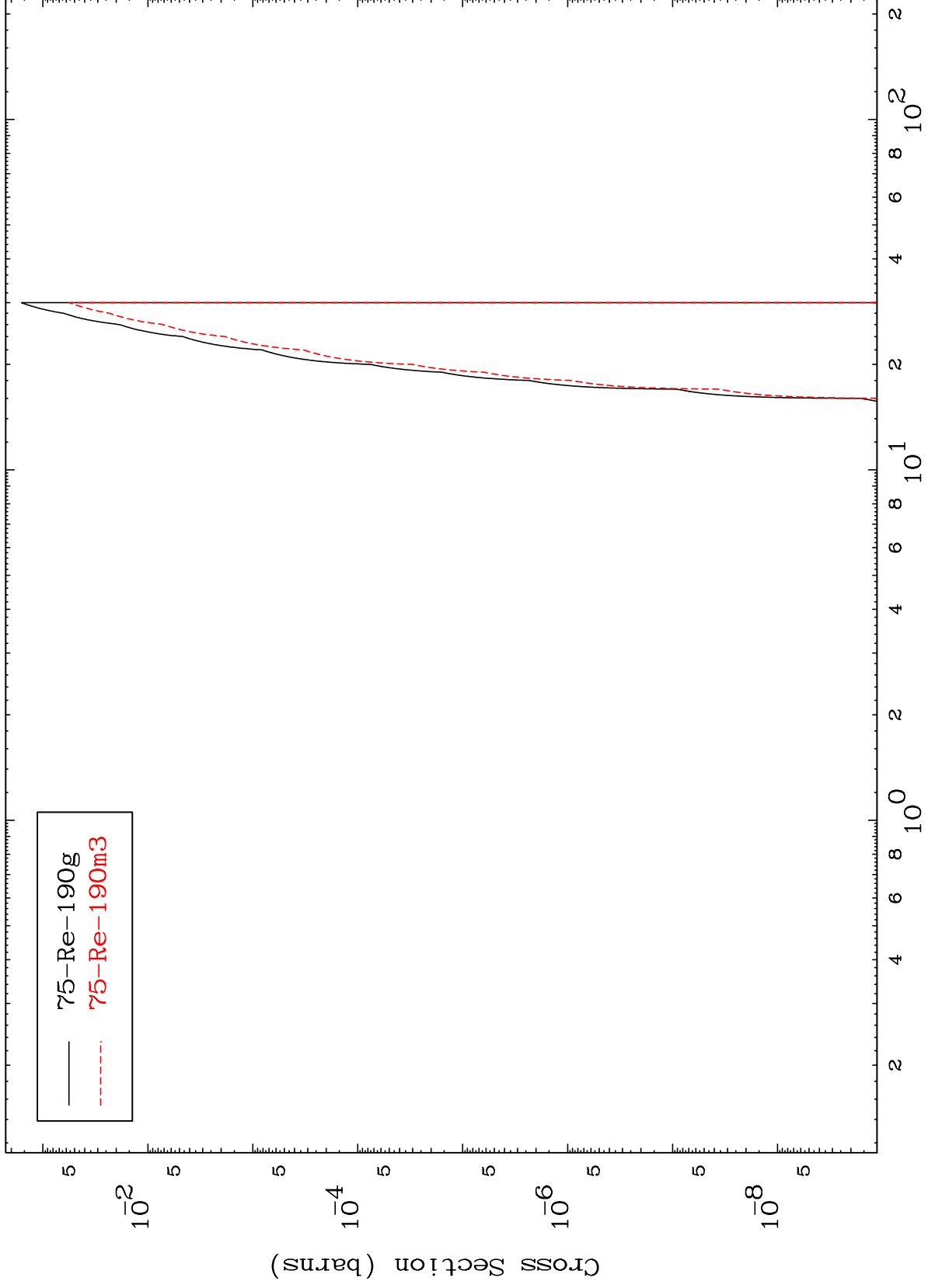


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( $\alpha, \alpha$ )

<sup>75</sup>Re-<sup>190</sup>

Radionuclide Production Cross Section



24

Incident Energy (MeV)

<sup>75</sup>Re-<sup>190</sup>