

Program EVALPLOT  
(Version 2018-1)

by

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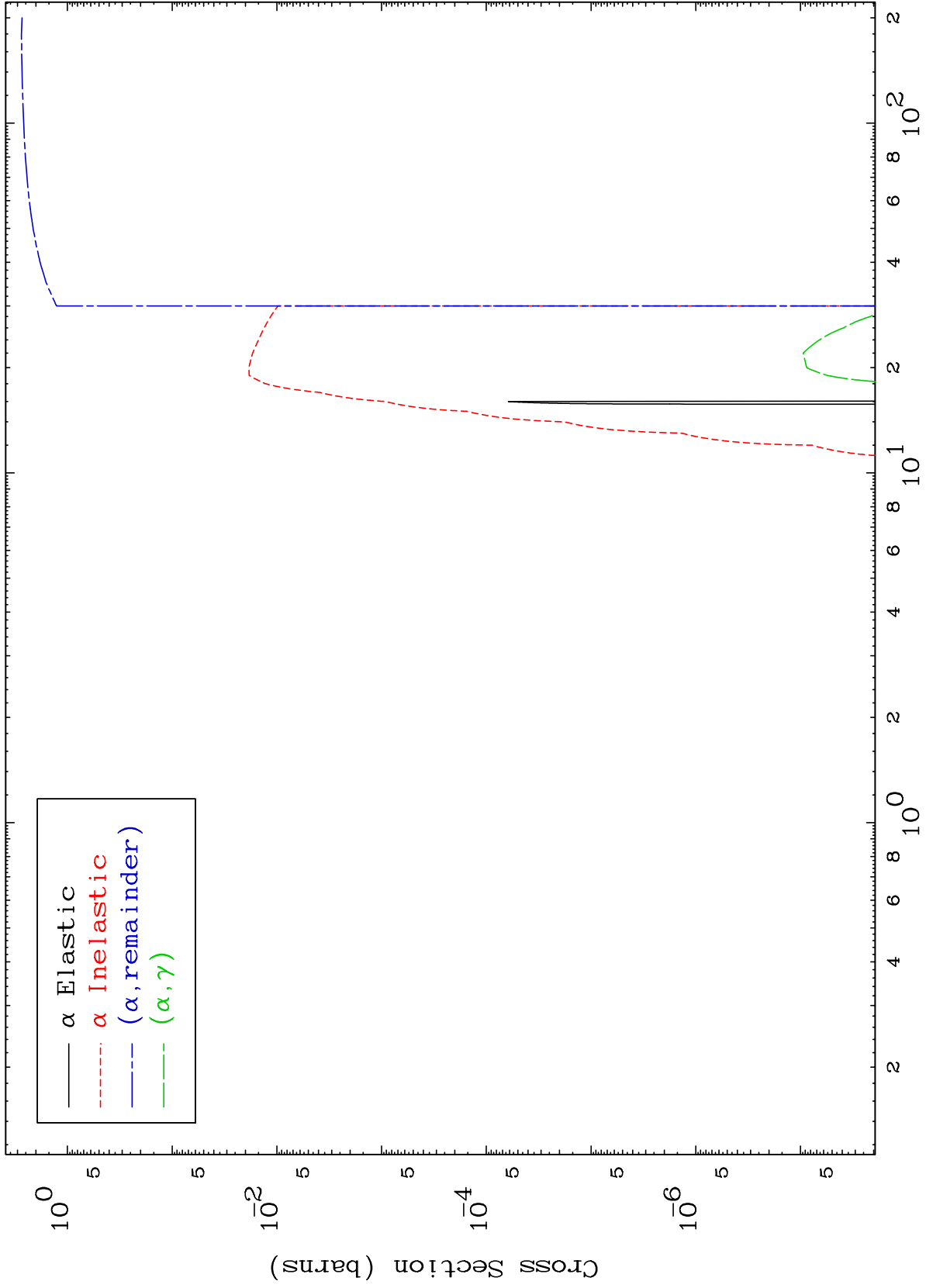
Press Mouse Button to Start

MAT 7552

$\alpha$  Major

75-Re-194

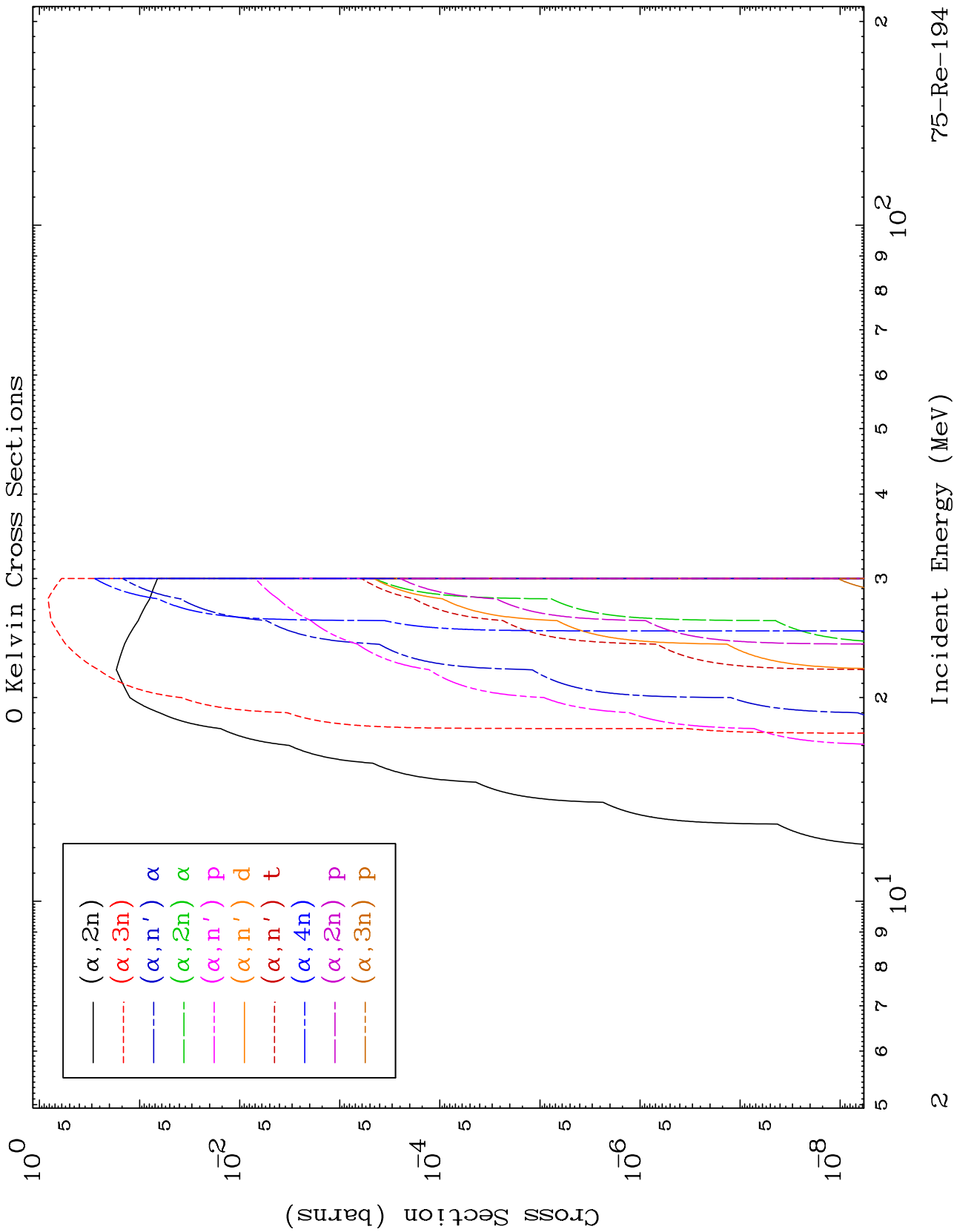
0 Kelvin Cross Sections



MAT 7552

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

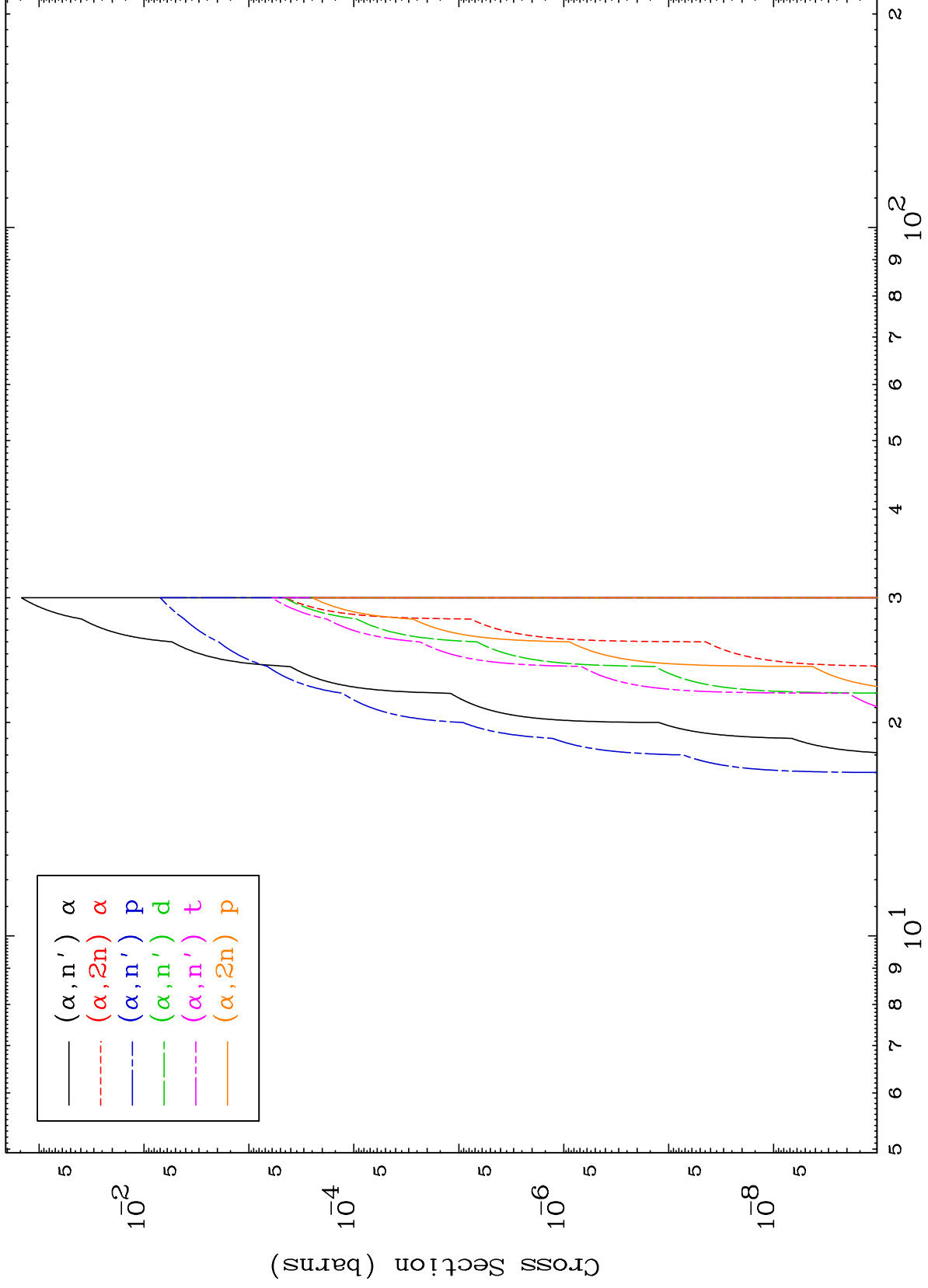
75-Re-194

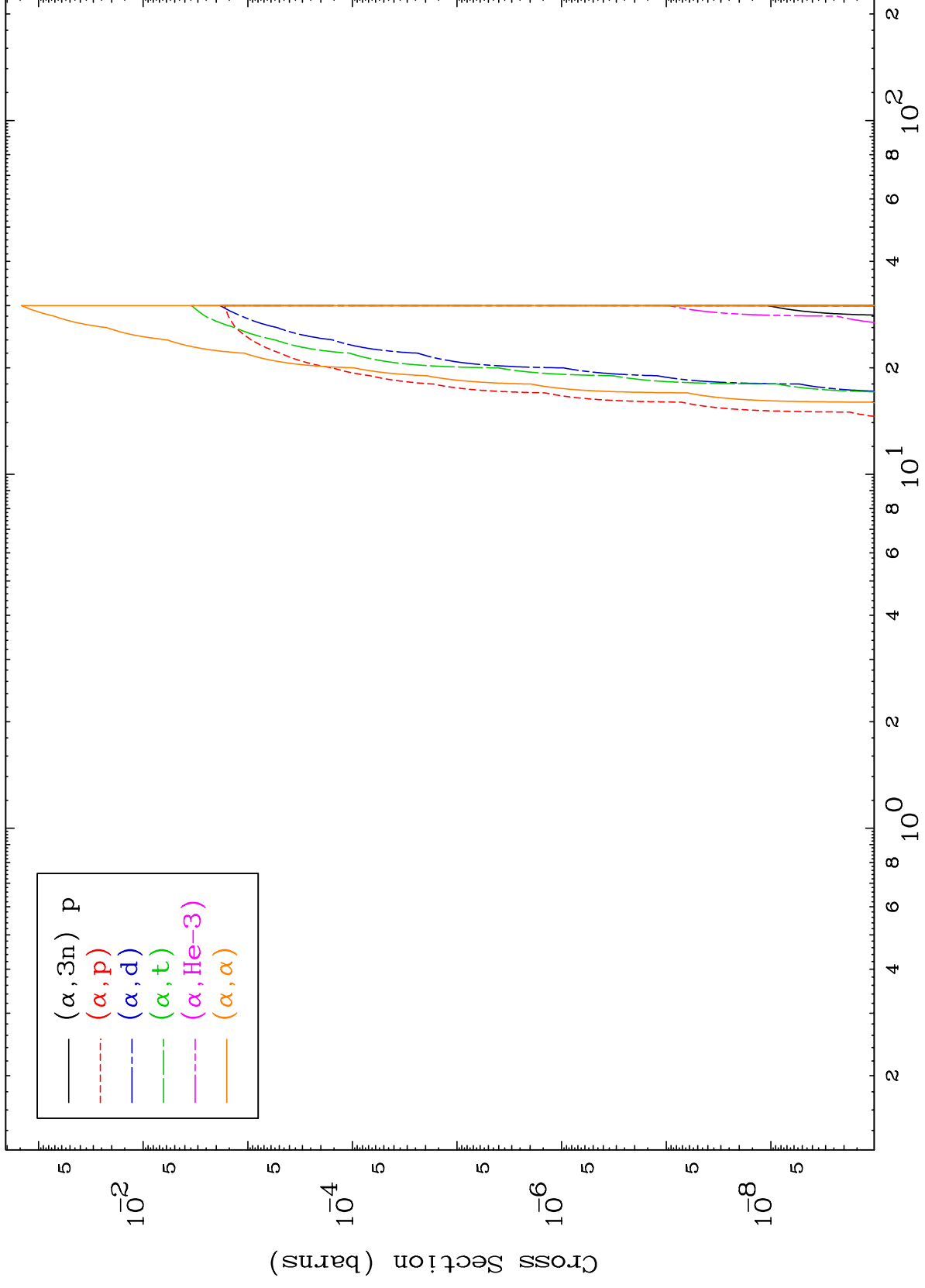


Incident Energy (MeV)

75-Re-194

2

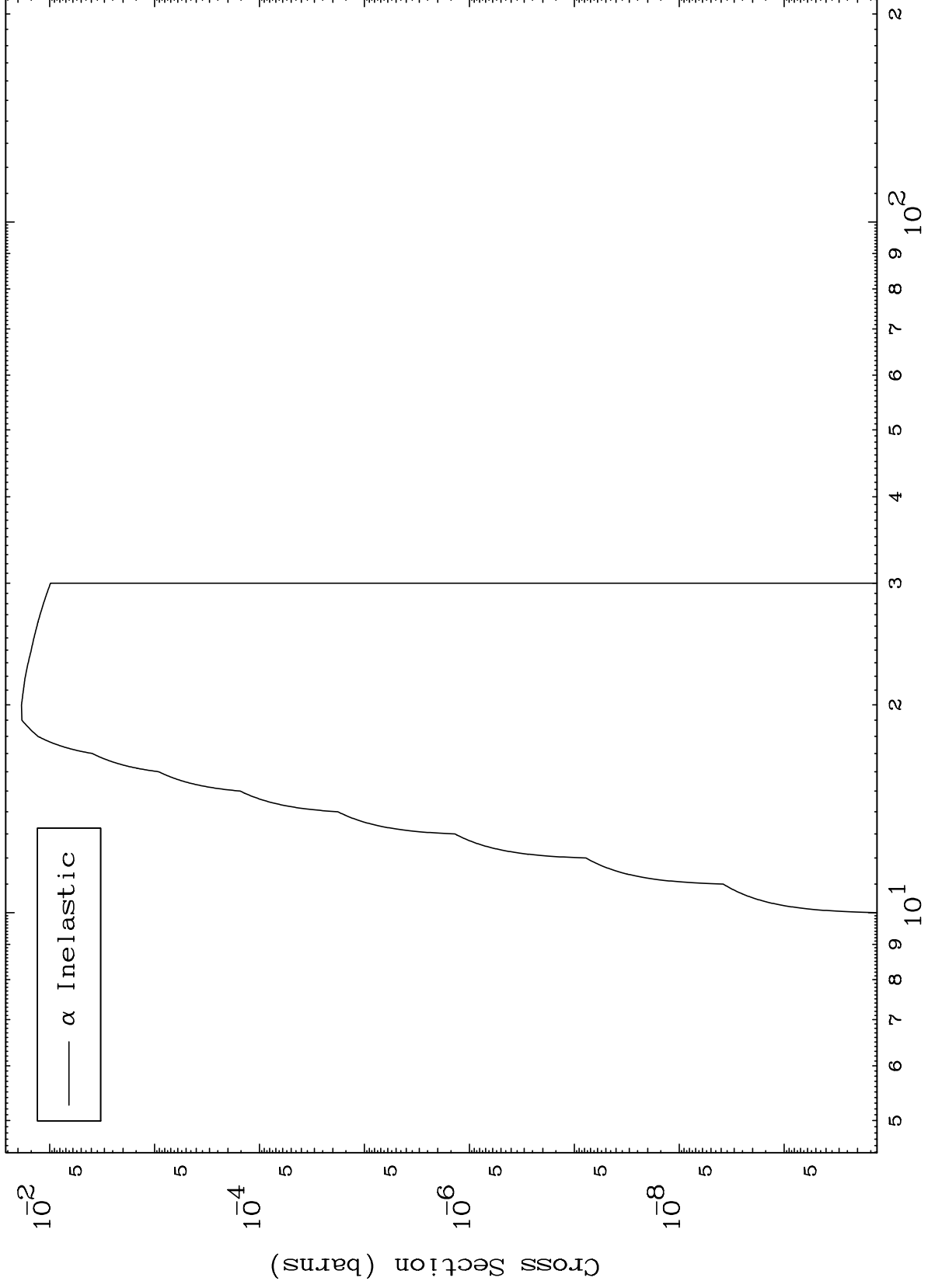




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75-Re-194

( $\alpha, n'$ ) Level  
0 Kelvin Cross Sections



5

Incident Energy (MeV)

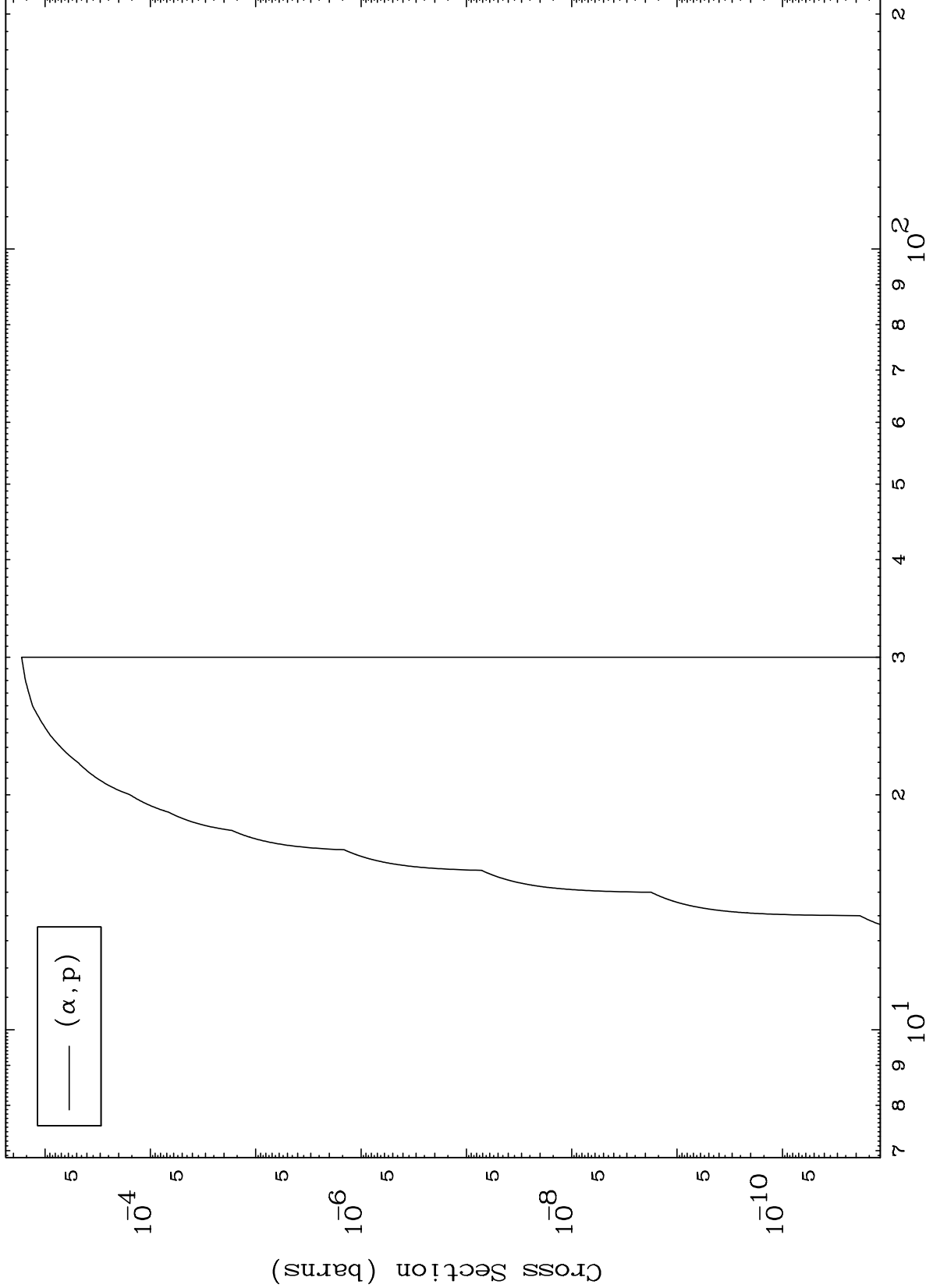
75-Re-194

MAT 7552

( $\alpha, p$ ) Levels

75-Re-194

0 Kelvin Cross Sections



6

Incident Energy (MeV)

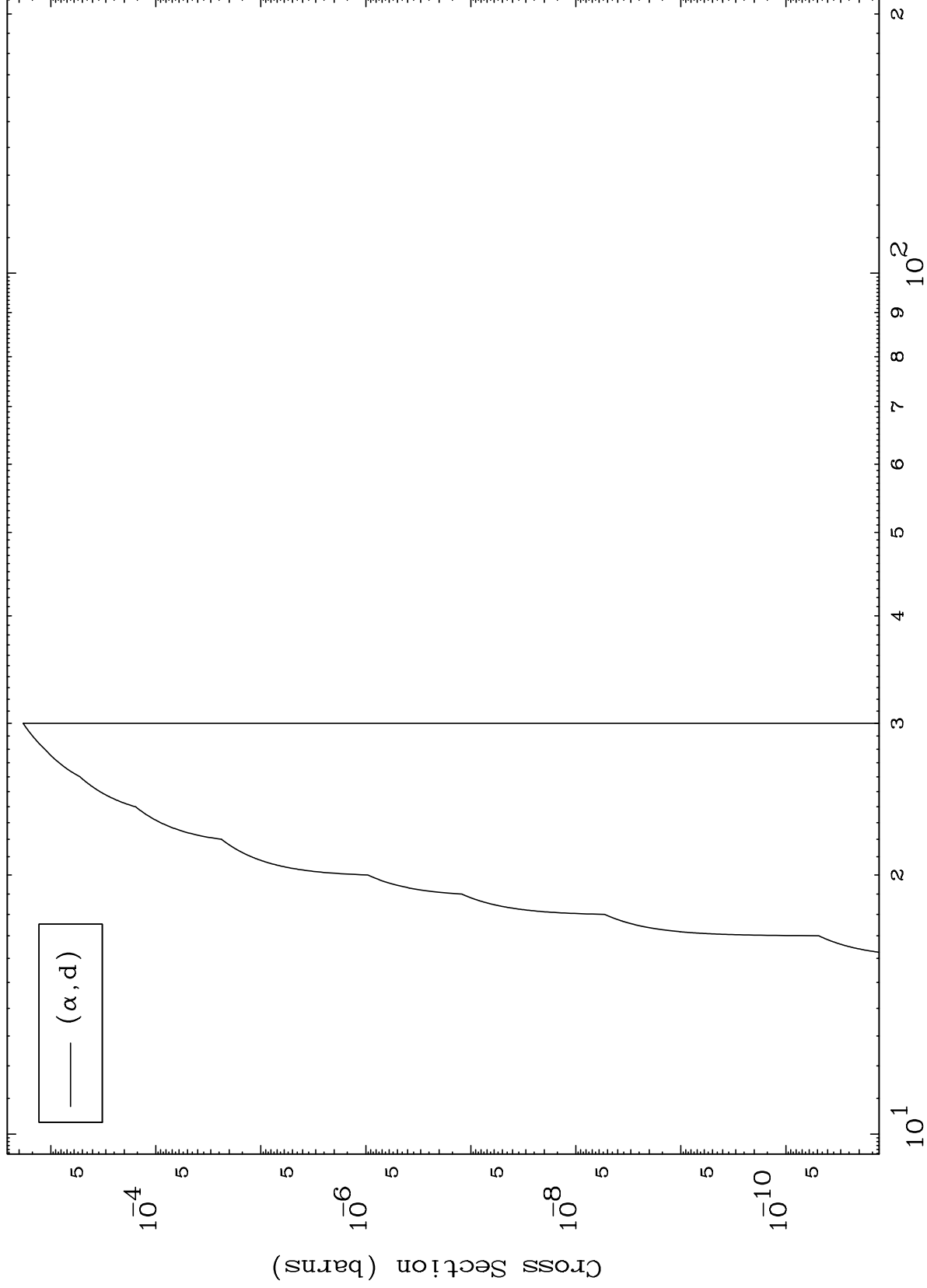
75-Re-194

MAT 7552

( $\alpha, d$ ) Levels

75-Re-194

0 Kelvin Cross Sections

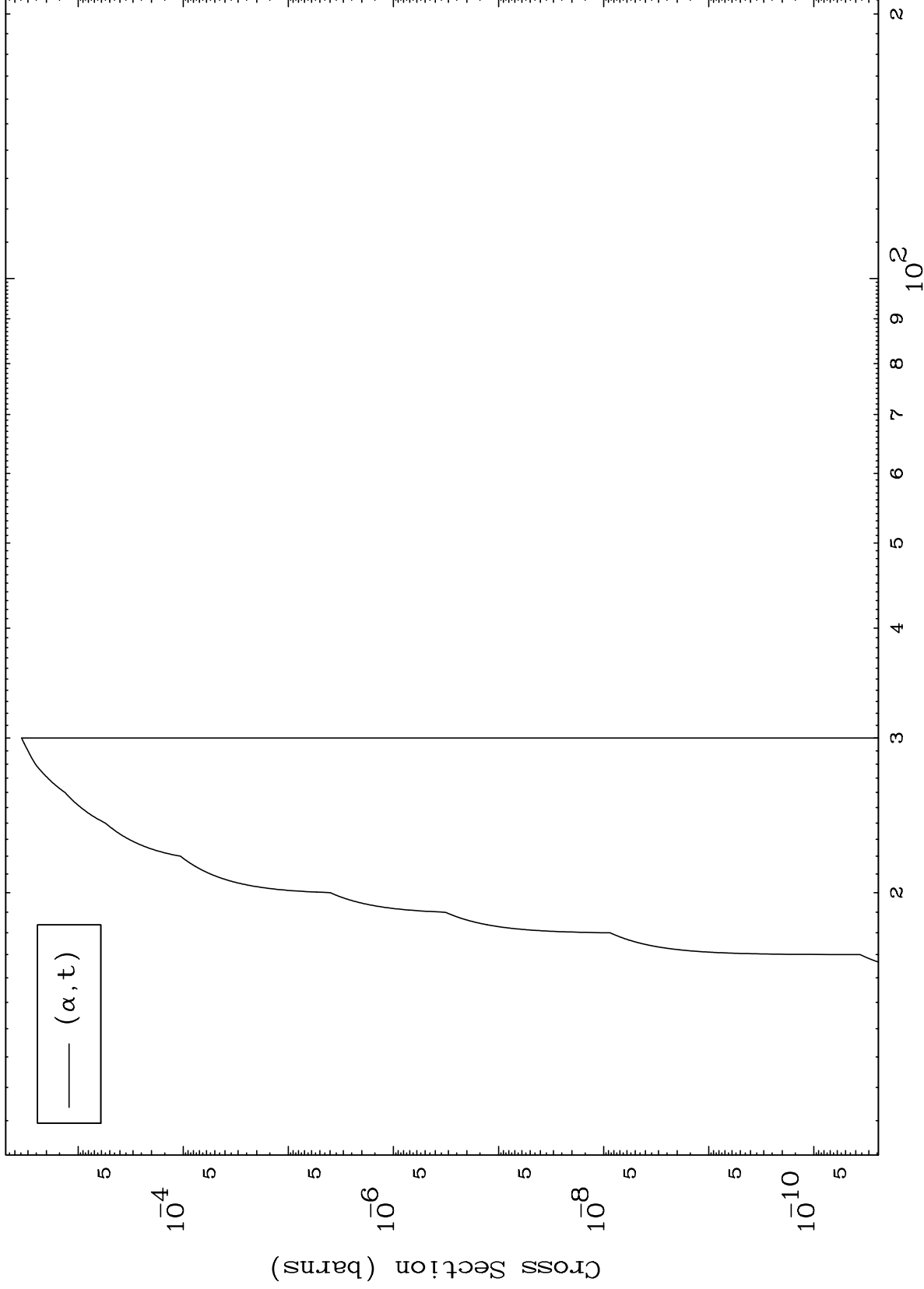


Incident Energy (MeV)

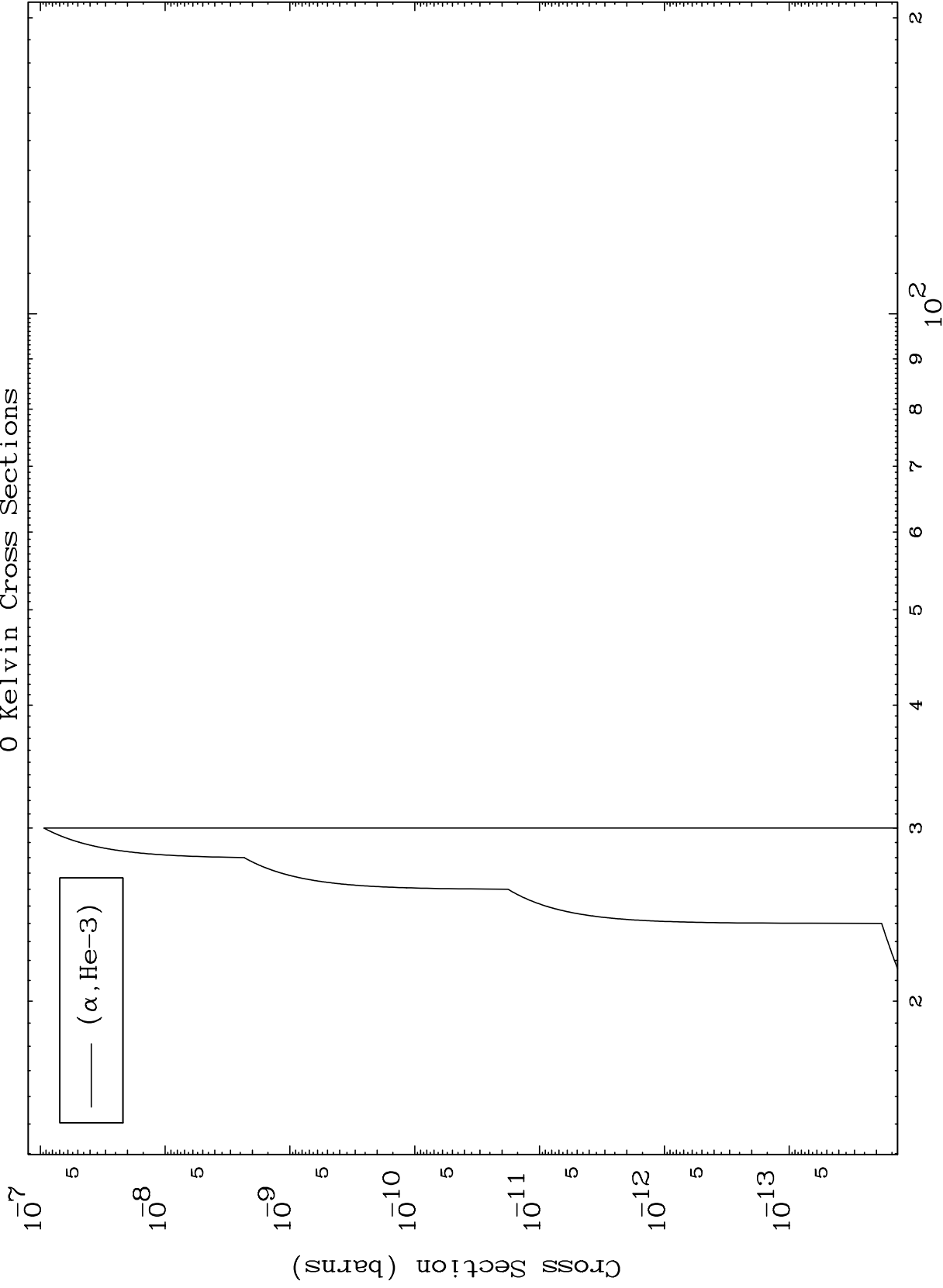
75-Re-194



0 Kelvin Cross Sections



0 Kelvin Cross Sections

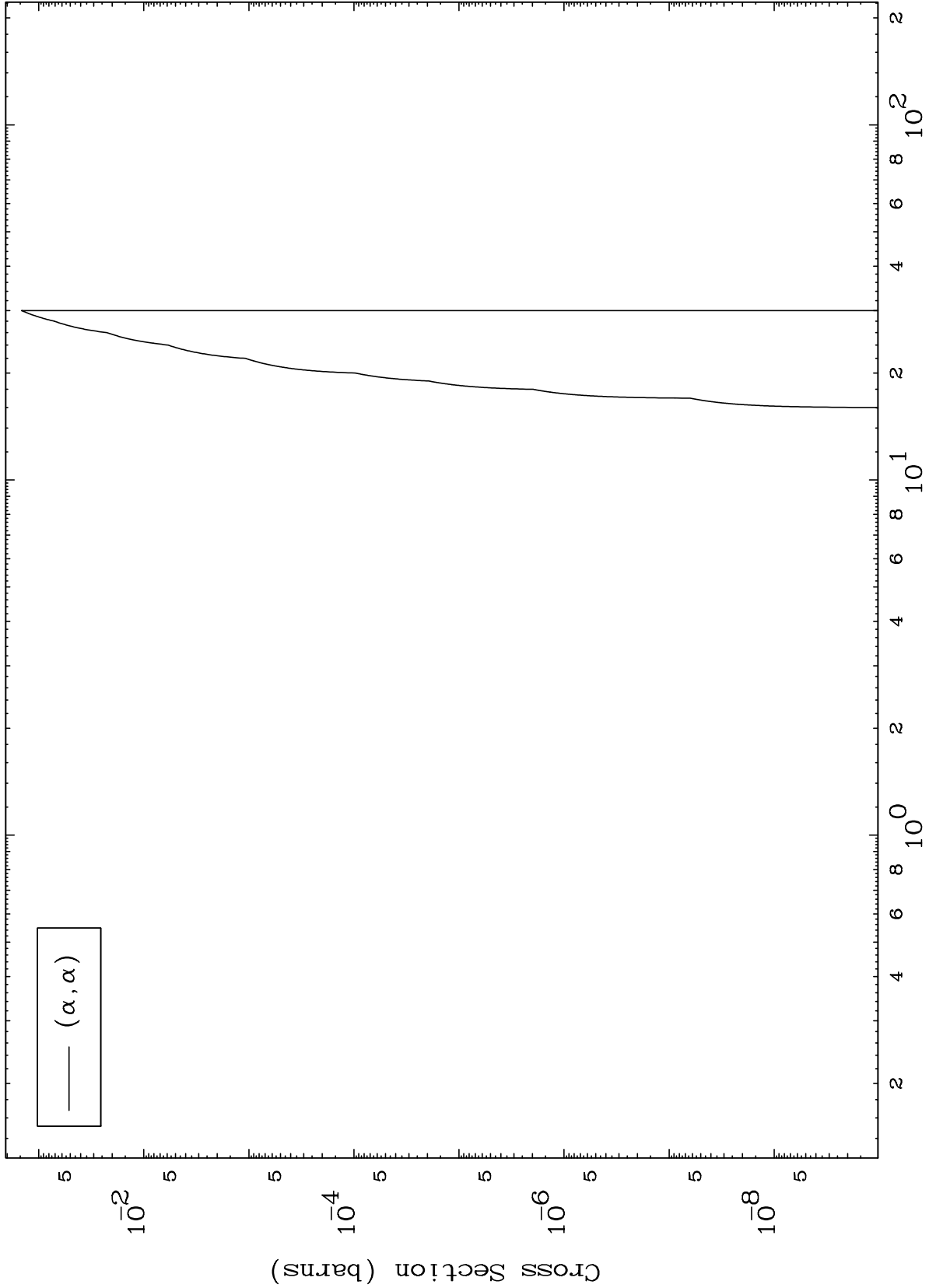


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( $\alpha, \alpha$ ) Levels

75-Re-194

0 Kelvin Cross Sections



10

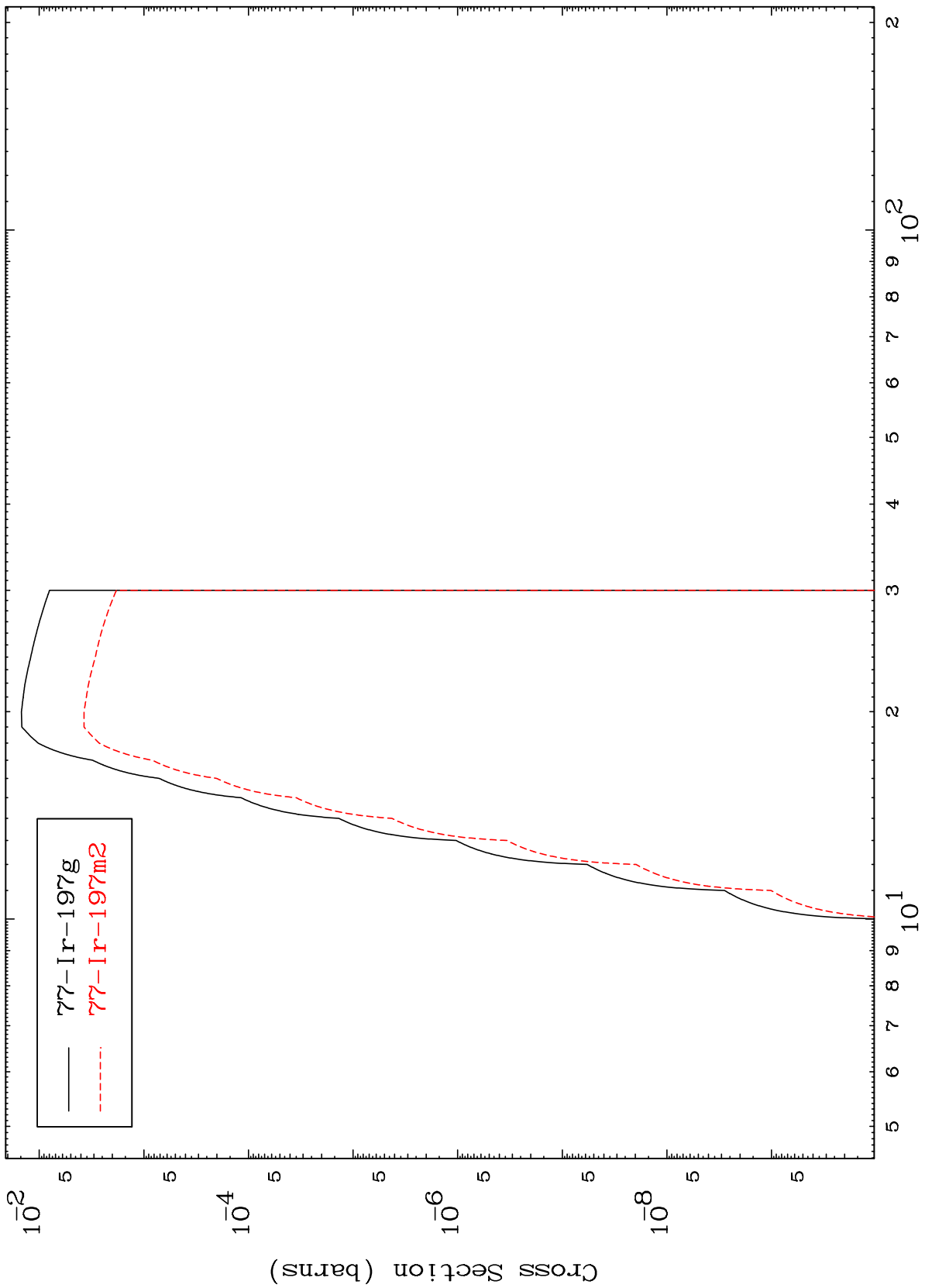
Incident Energy (MeV)

75-Re-194

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75-Re-194

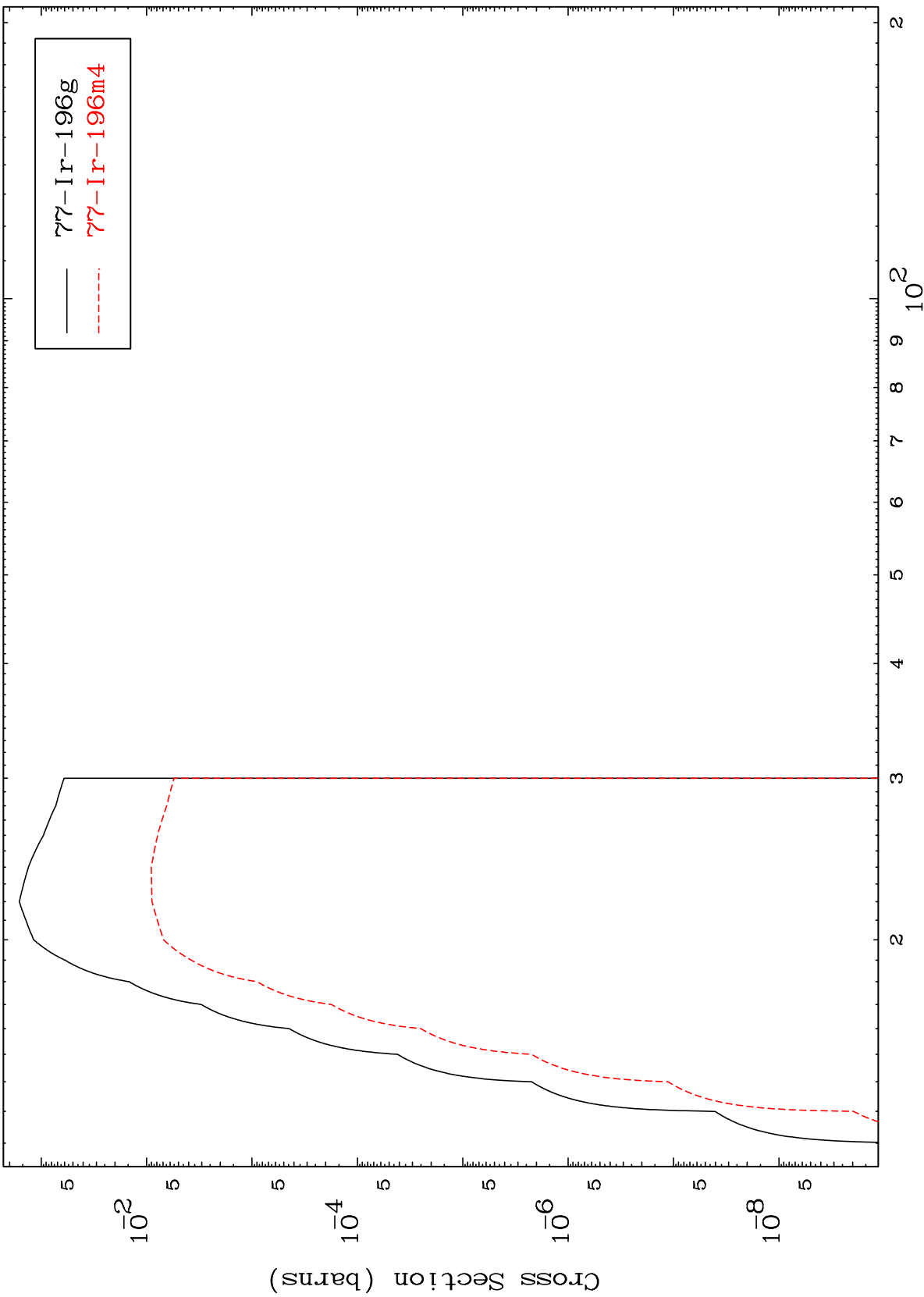
Radionuclide Production Cross Section  
 $\alpha$  Inelastic



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75-Re-194

$(\alpha, 2n)$   
Radionuclide Production Cross Section



12

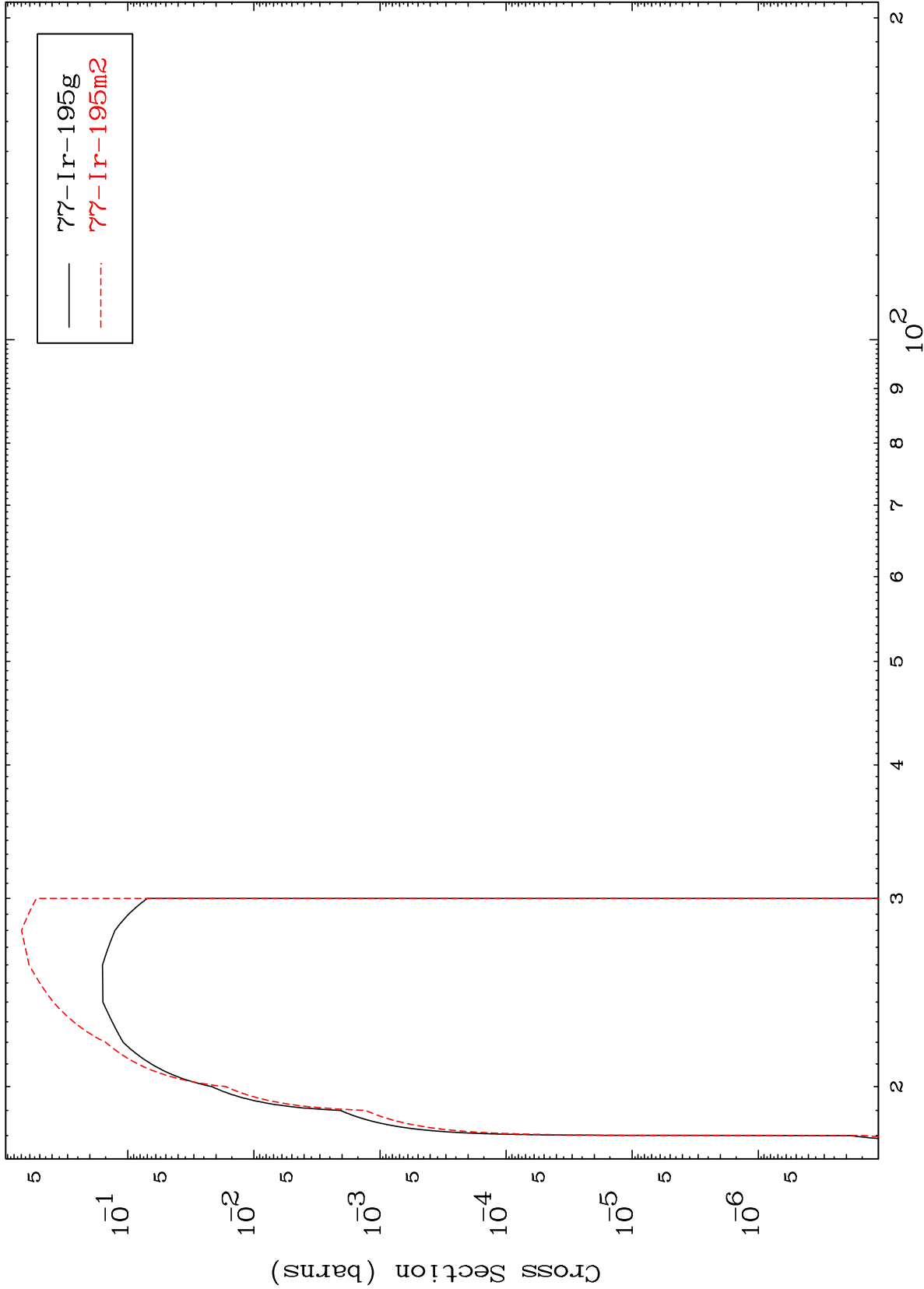
Incident Energy (MeV)

75-Re-194

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75-Re-194

( $\alpha, 3n$ )  
Radionuclide Production Cross Section

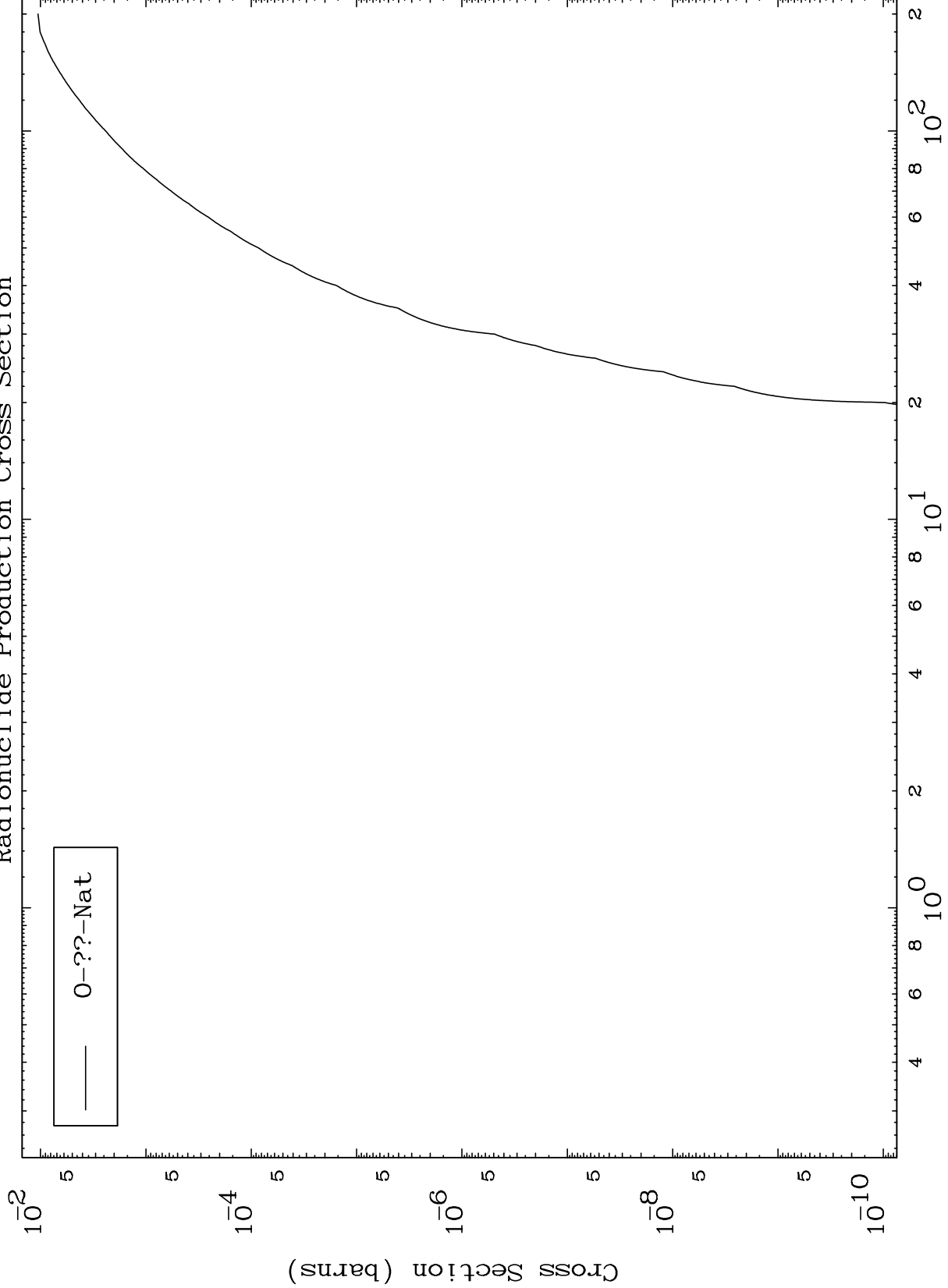


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$\alpha$  Fission

75-Re-194

Radionuclide Production Cross Section

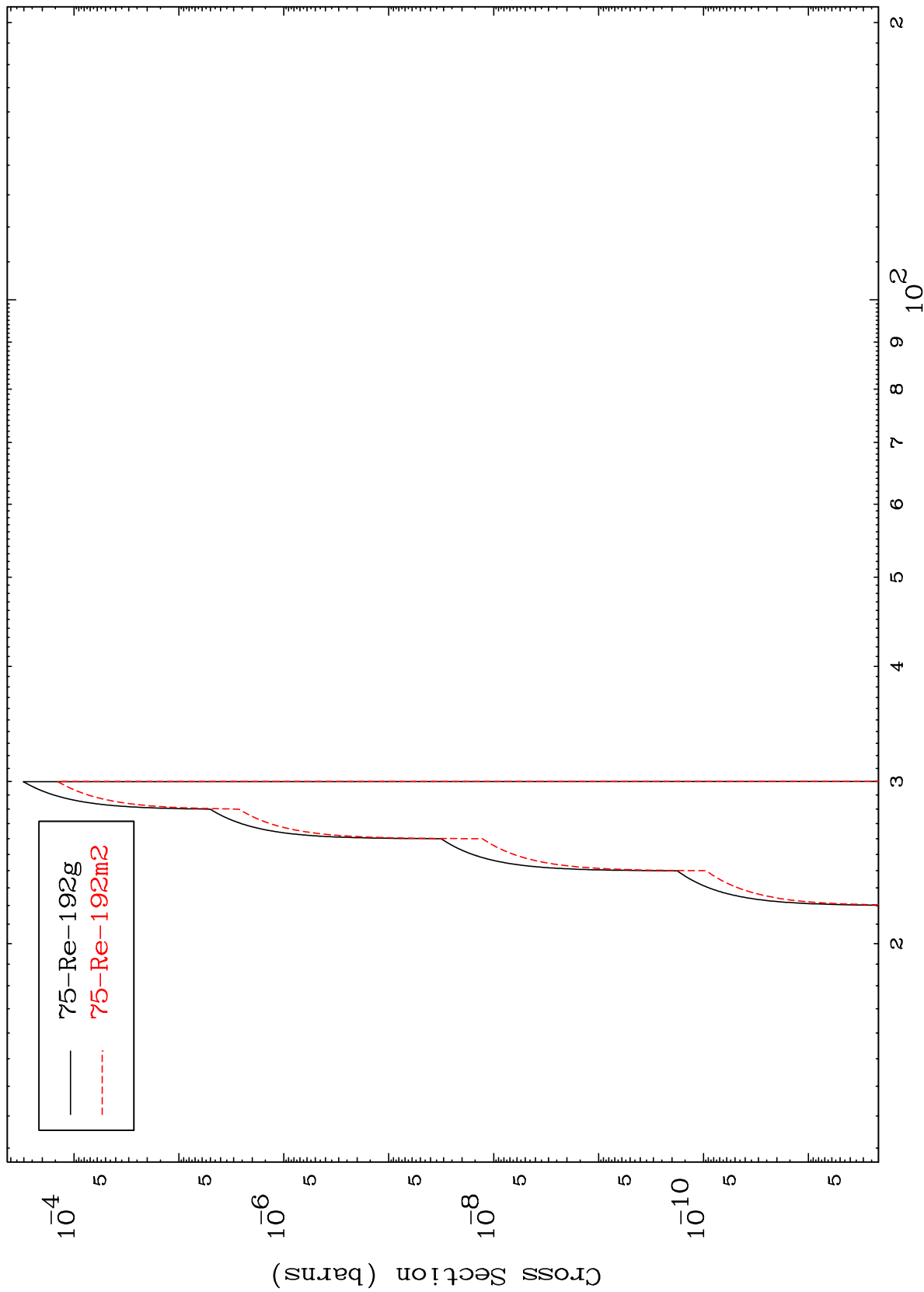


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$(\alpha, 2n)$   $\alpha$

$^{75}\text{Re-194}$

Radionuclide Production Cross Section



15

Incident Energy (MeV)

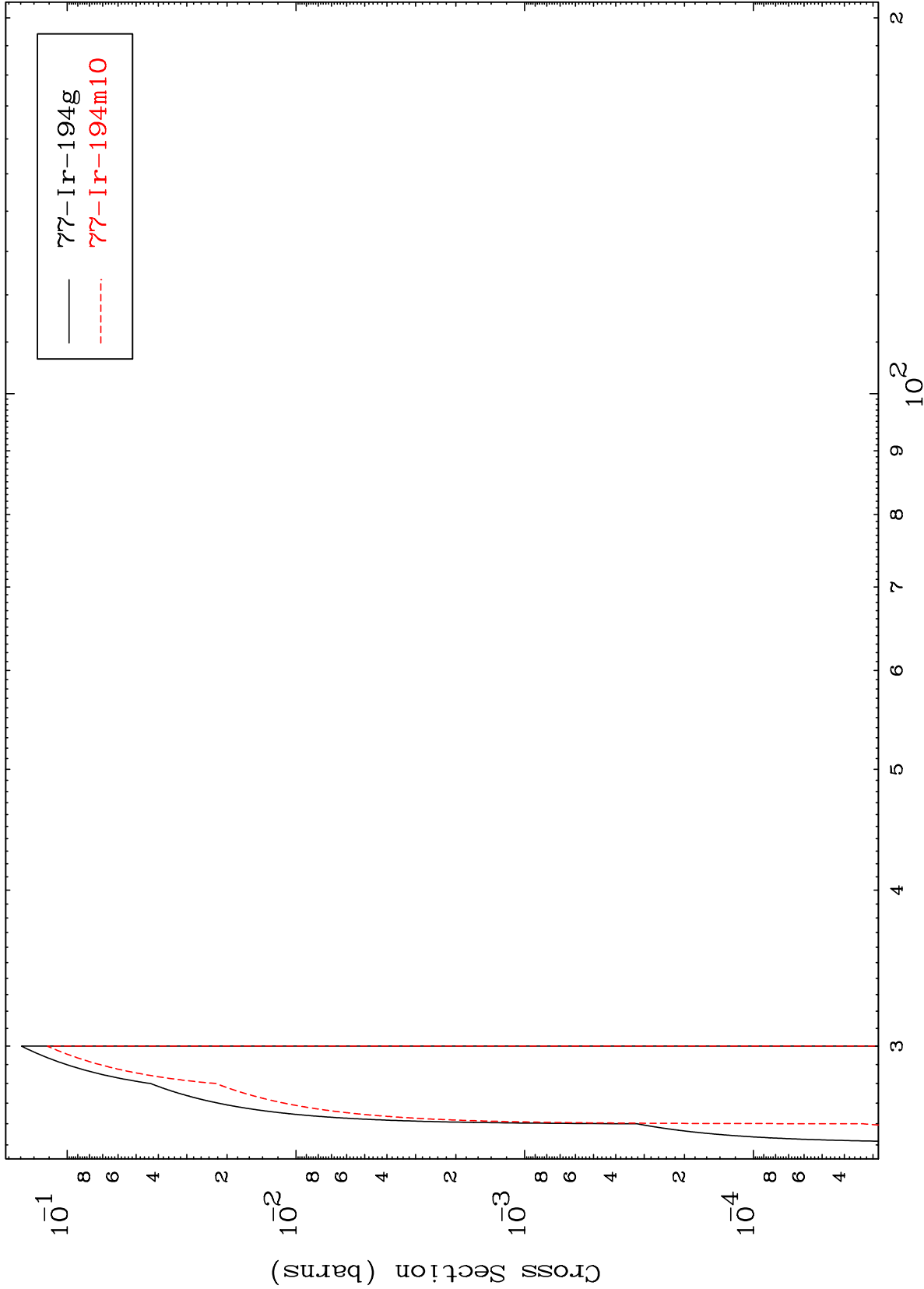
$^{75}\text{Re-194}$



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75-Re-194

( $\alpha, 4n$ )  
Radionuclide Production Cross Section



16

75-Re-194

Incident Energy (MeV)