

Program EVALPLOT  
(Version 2018-1)

by

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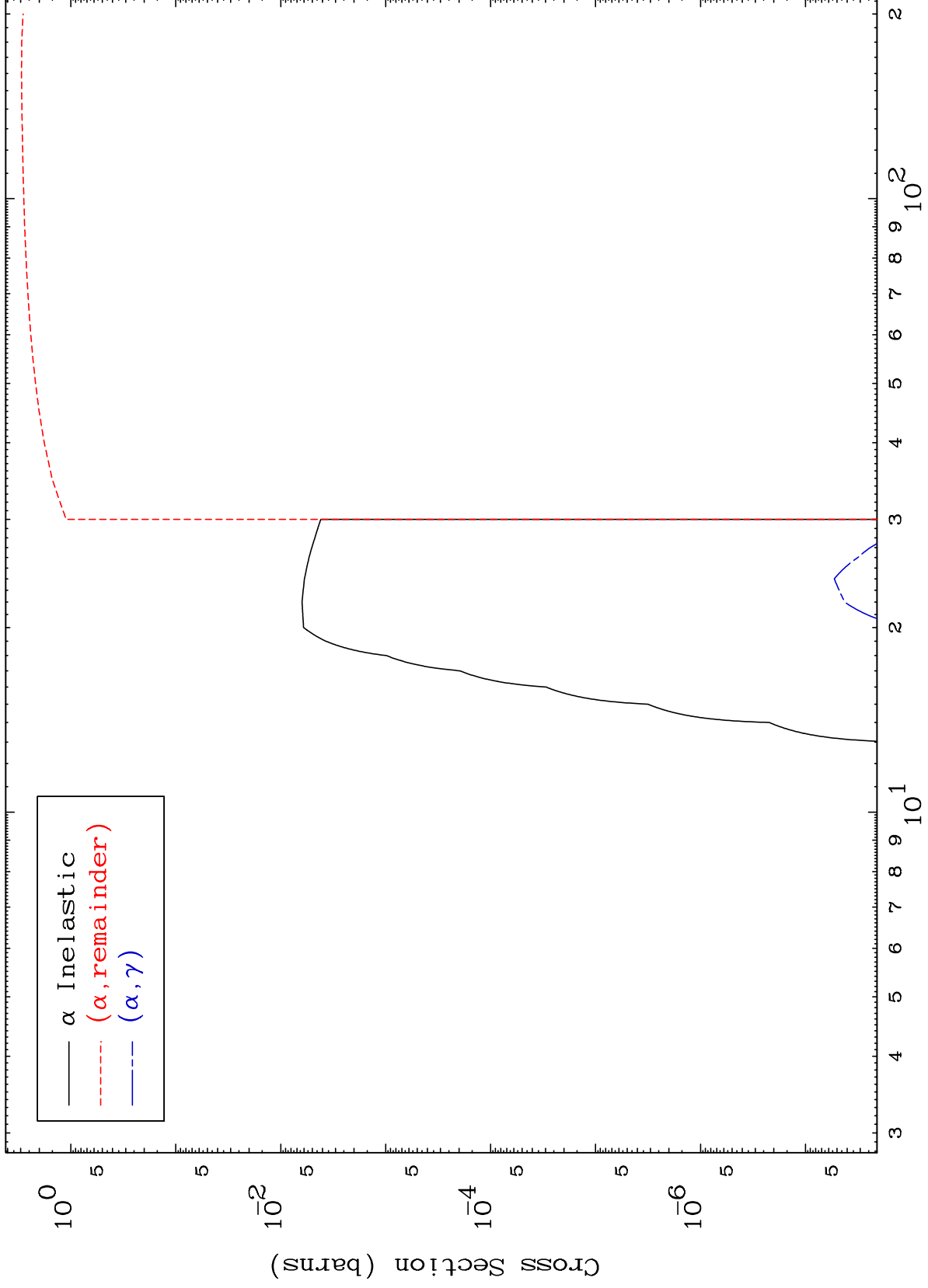
Press Mouse Button to Start

MAT 8673

$\alpha$  Major

86-Rn-227

0 Kelvin Cross Sections

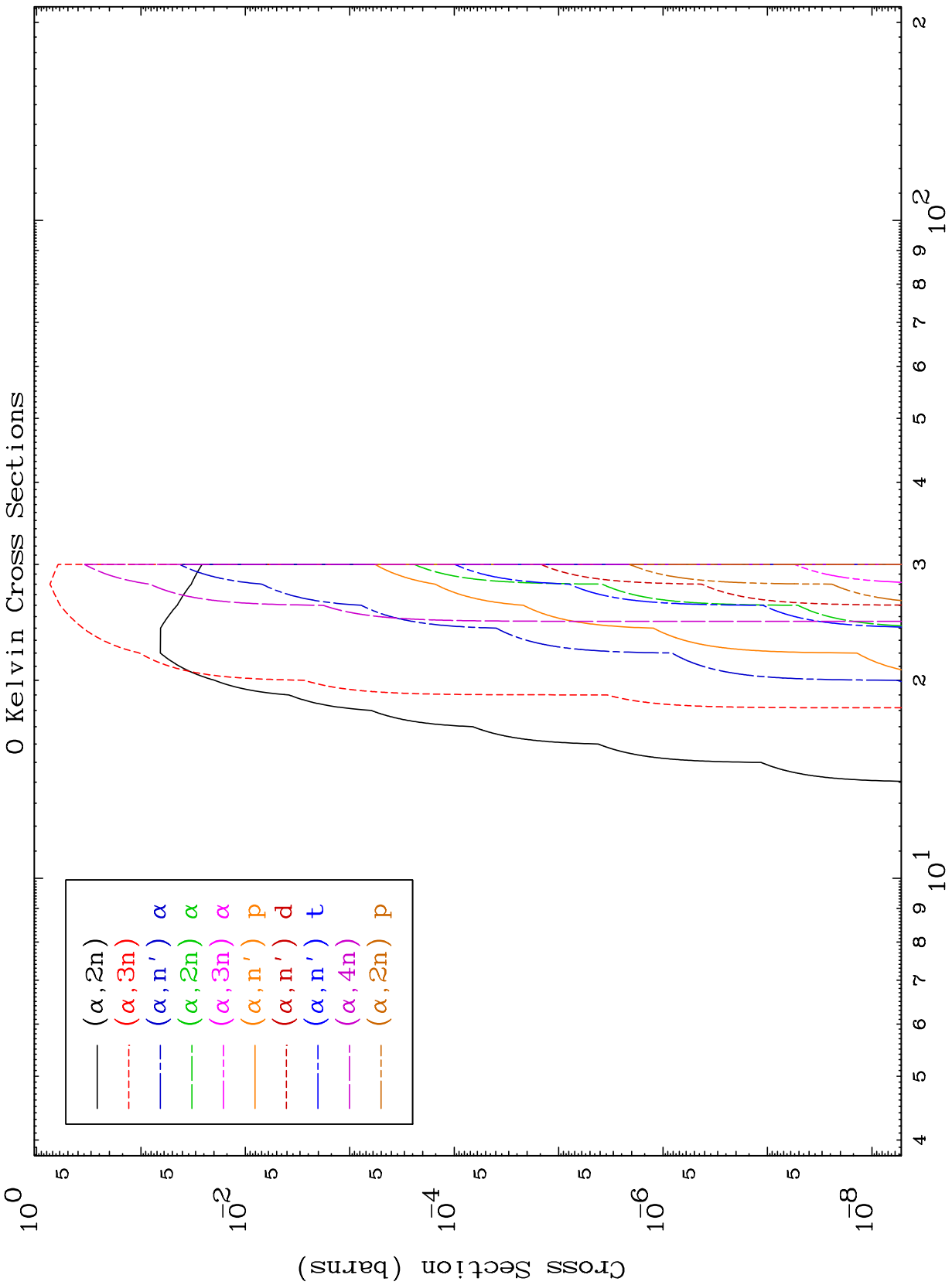


$\alpha$  Inelastic  
 $(\alpha, \text{remainder})$   
 $(\alpha, \gamma)$

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$\alpha$  Neutron Production  
0 Kelvin Cross Sections

86-Rn-227



2

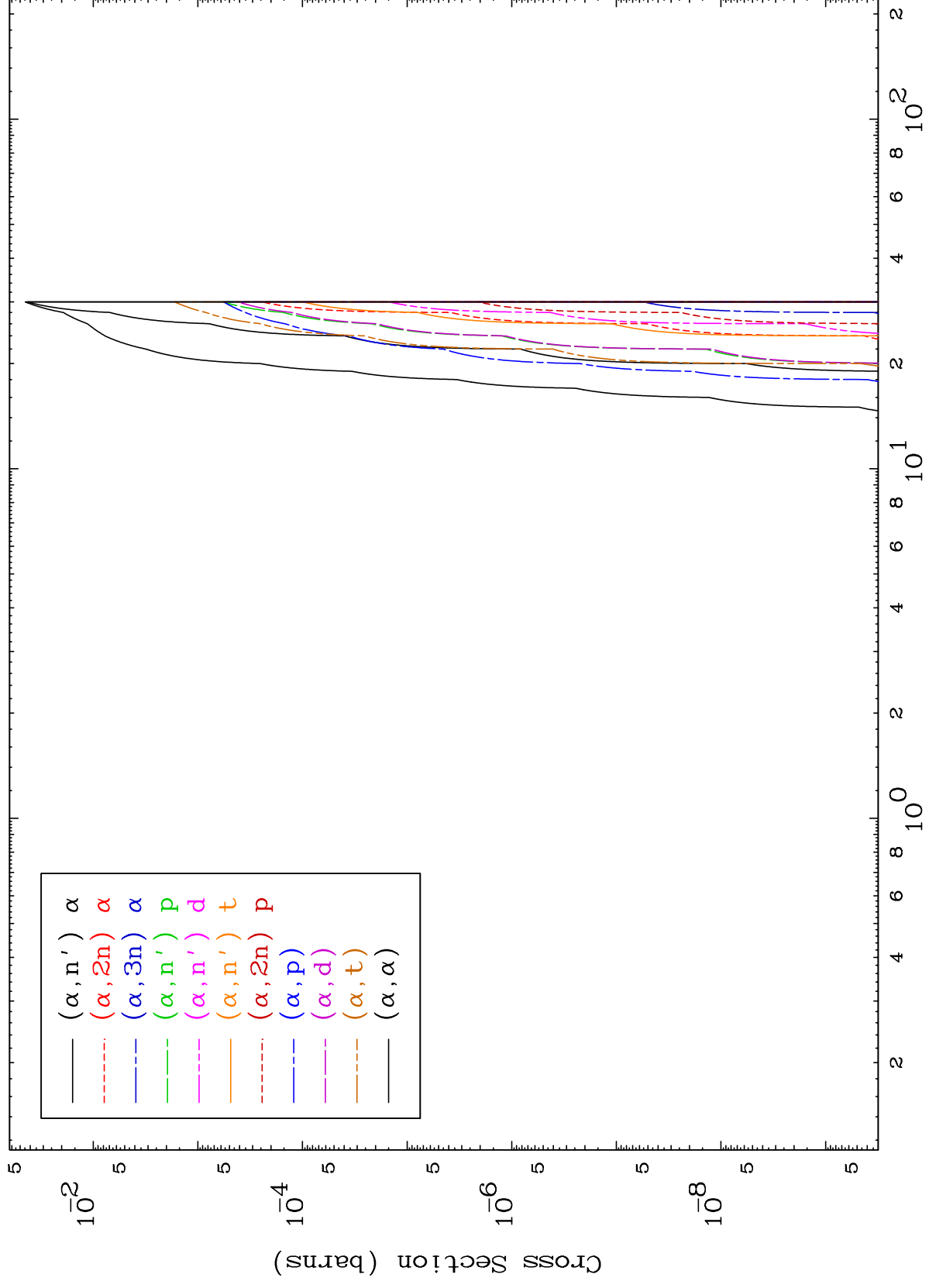
Incident Energy (MeV)

86-Rn-227

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$\alpha$  Charged Particle  
0 Kelvin Cross Sections

86-Rn-227

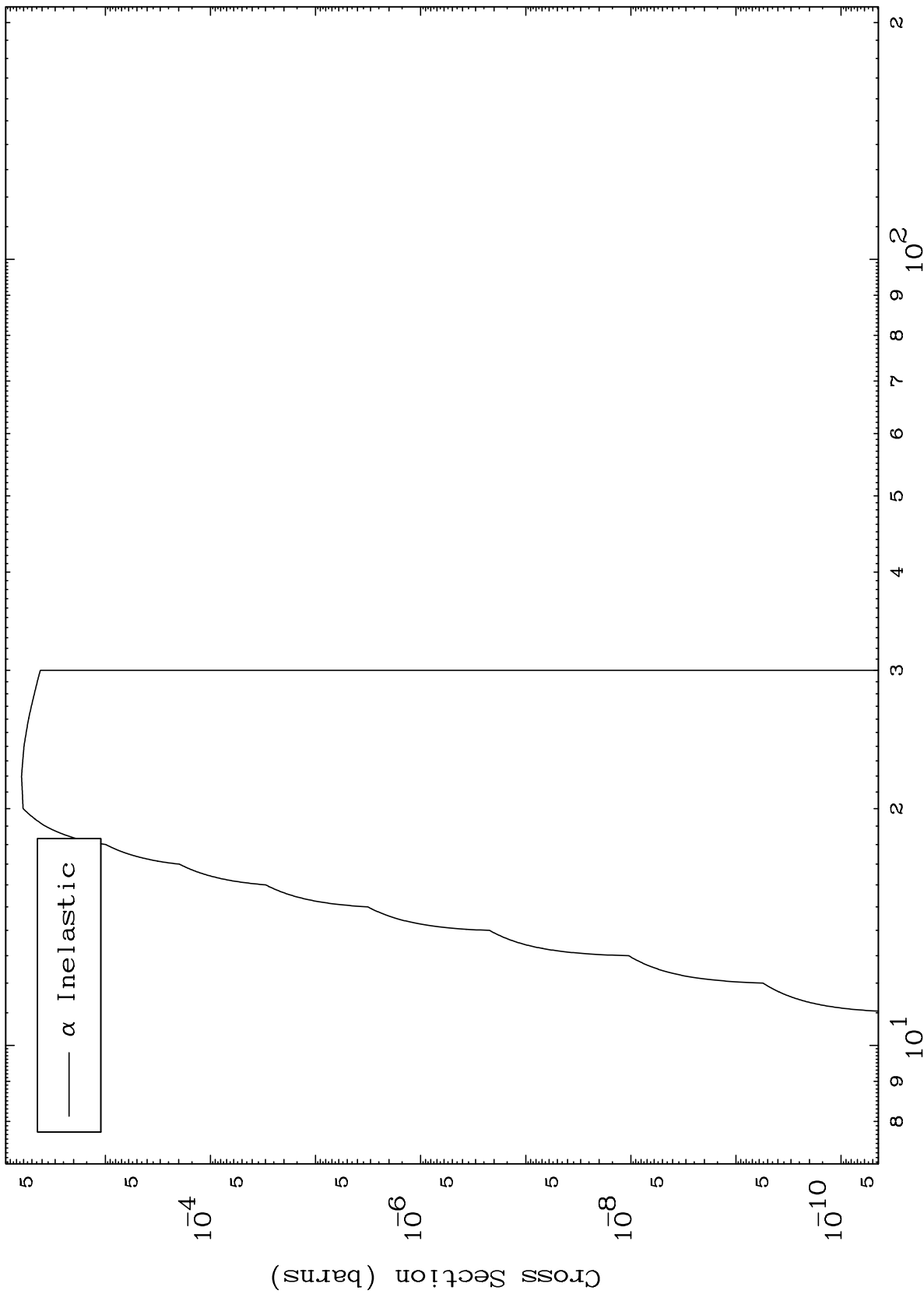


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( $\alpha, n'$ ) Level

86-Rn-227

0 Kelvin Cross Sections

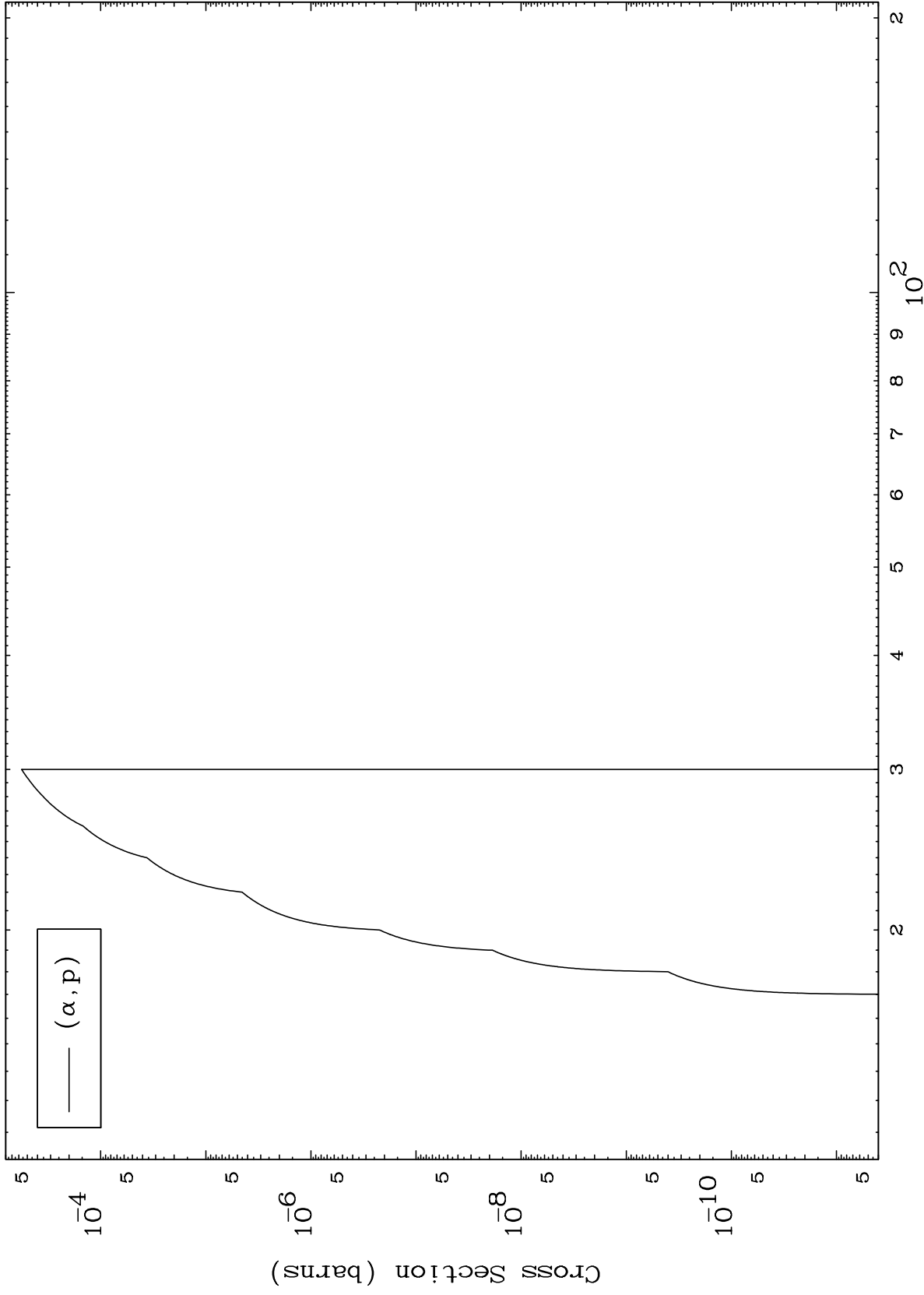


—  $\alpha$  Inelastic

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( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

86-Rn-227



5

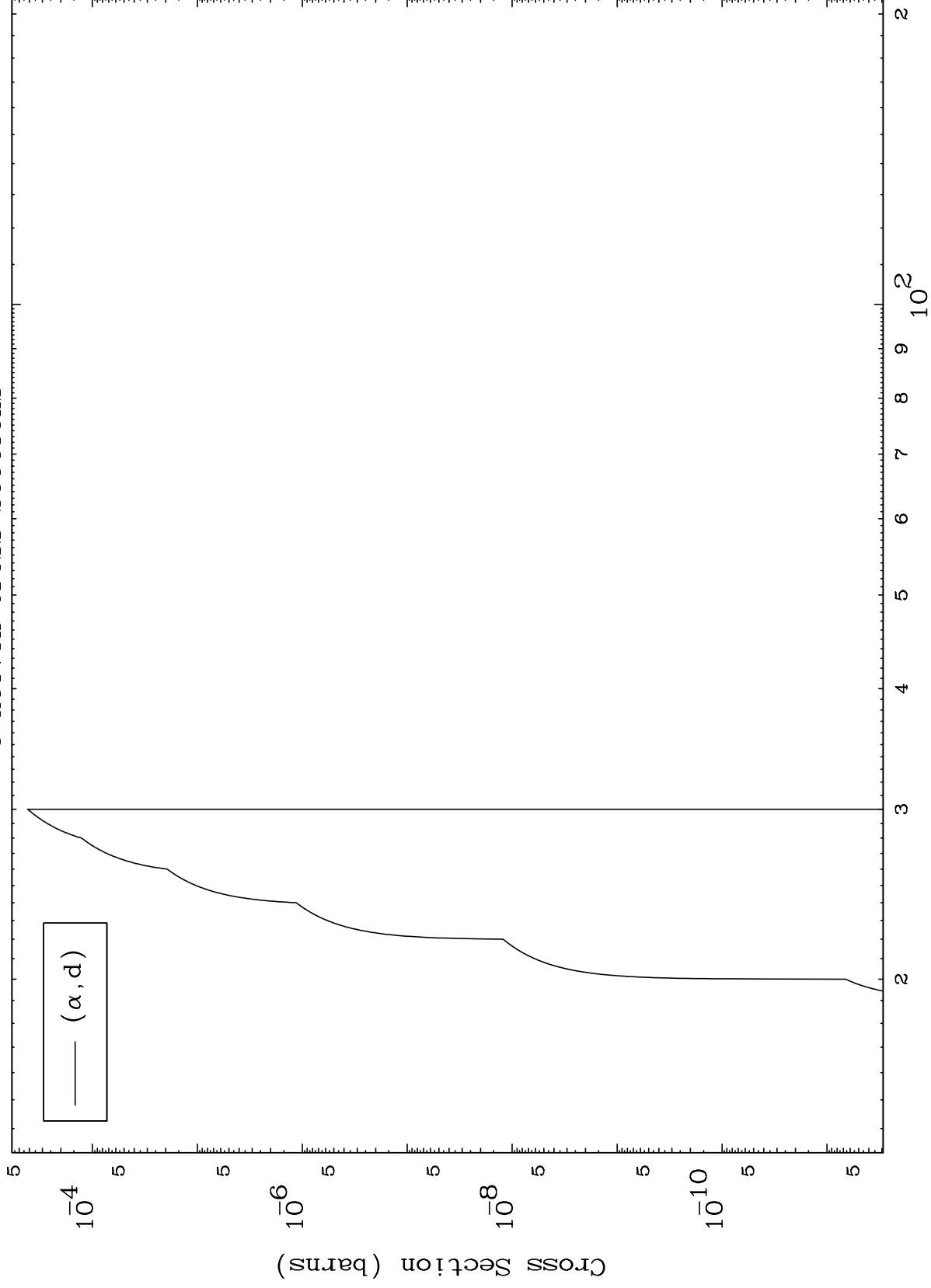
Incident Energy (MeV)

86-Rn-227

MAT 8673

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

86-Rn-227



6

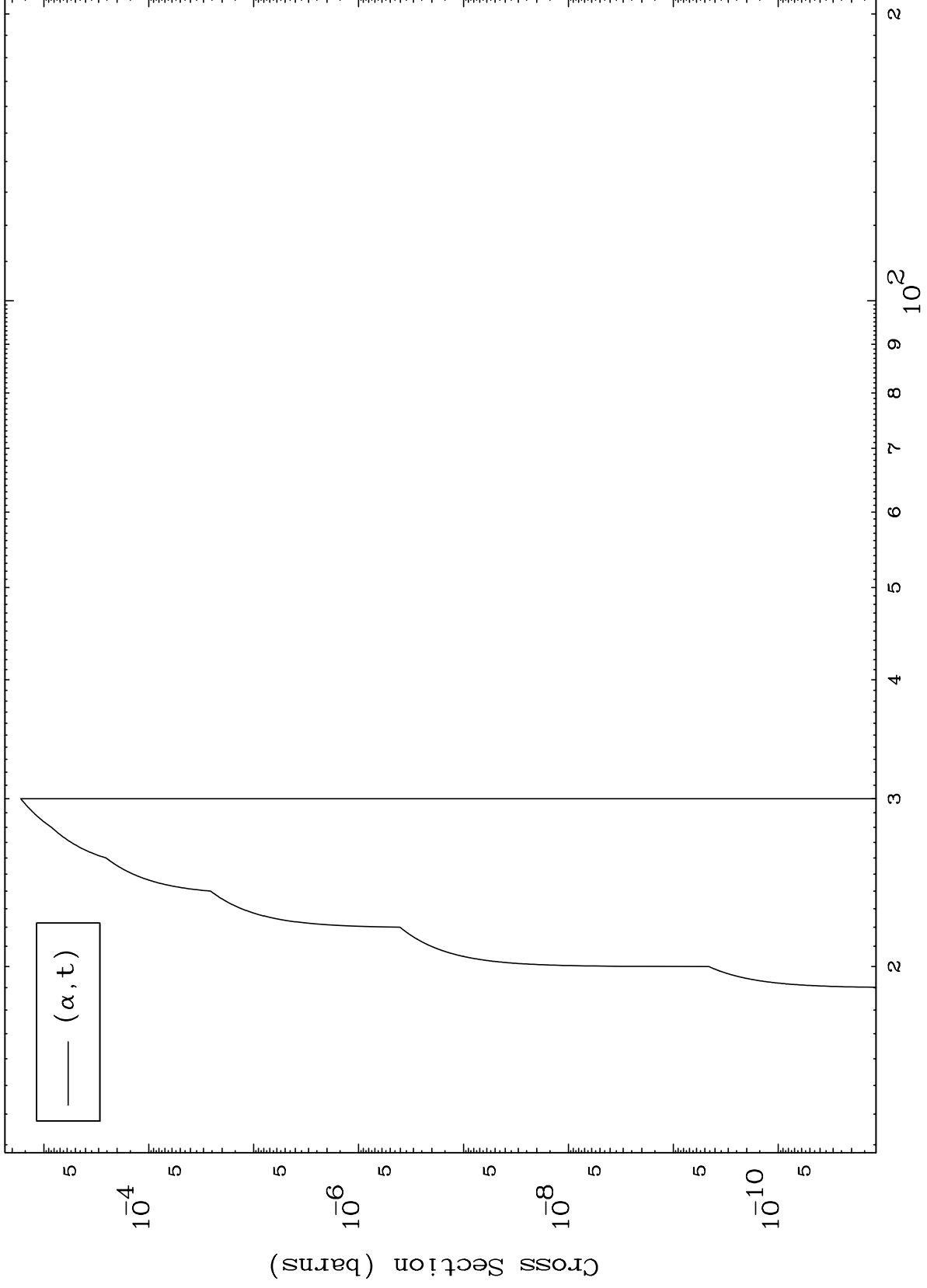
Incident Energy (MeV)

86-Rn-227

MAT 8673

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

86-Rn-227



7

Incident Energy (MeV)

86-Rn-227

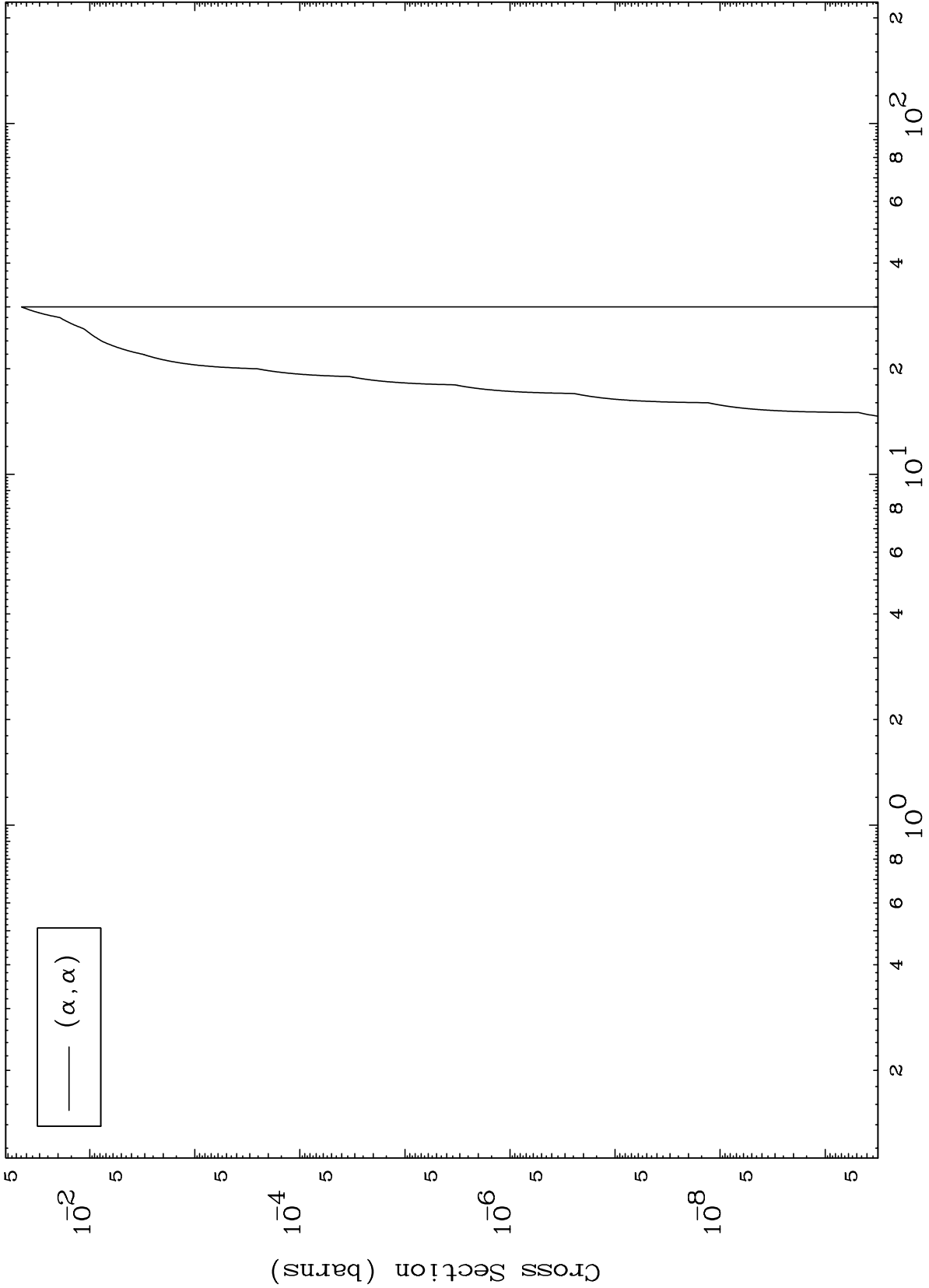


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( $\alpha, \alpha$ ) Levels

86-Rn-227

0 Kelvin Cross Sections



( $\alpha, \alpha$ )

MAT 8673

86-Rn-227

$\alpha$  Fission

Radionuclide Production Cross Section

