

Program EVALPLOT  
(Version 2018-1)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

Web:redcullen1.net/HOMEPAGE.NEW

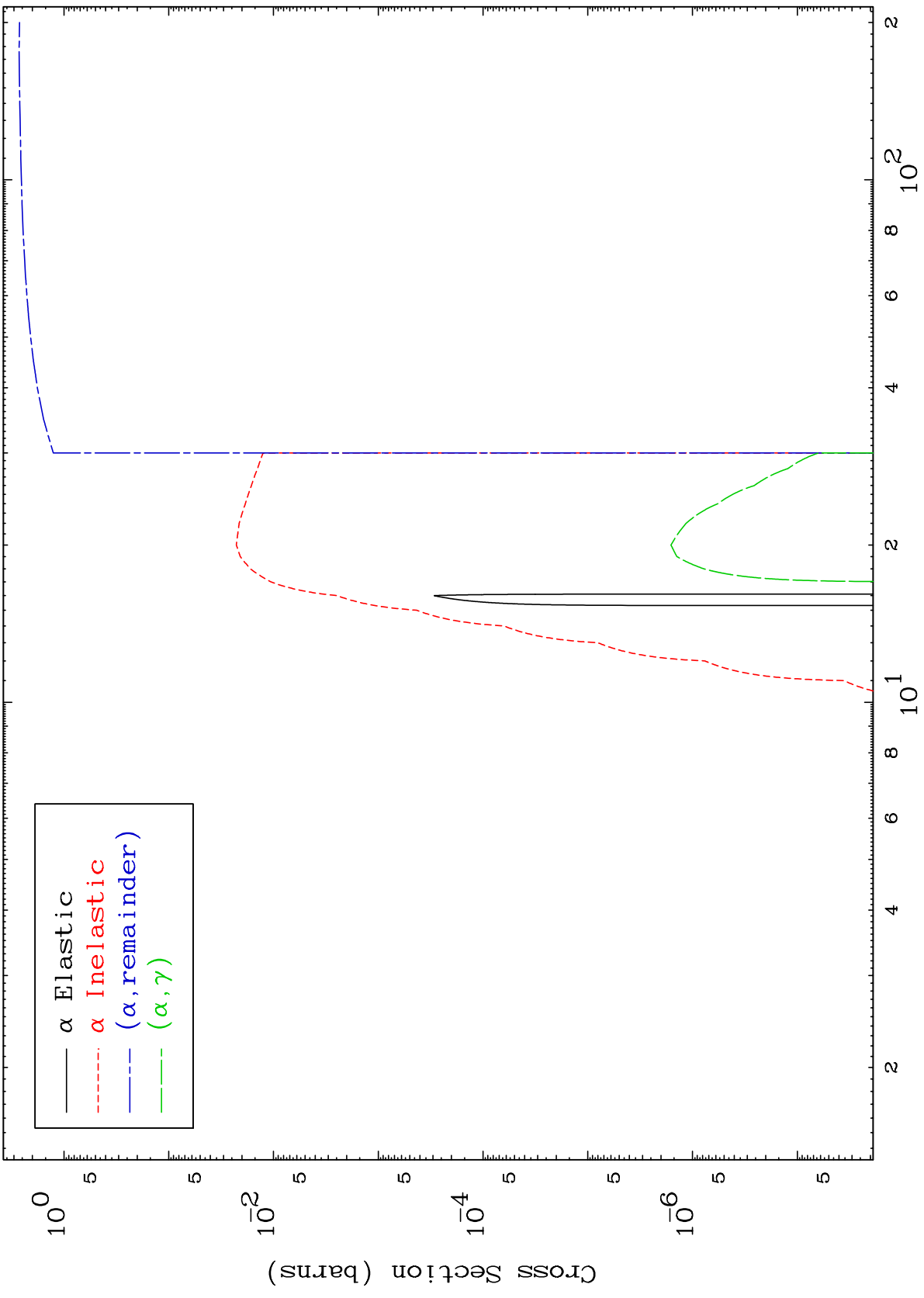
Press Mouse Button to Start

MAT 7337

$\alpha$  Major

73-Ta-184

0 Kelvin Cross Sections

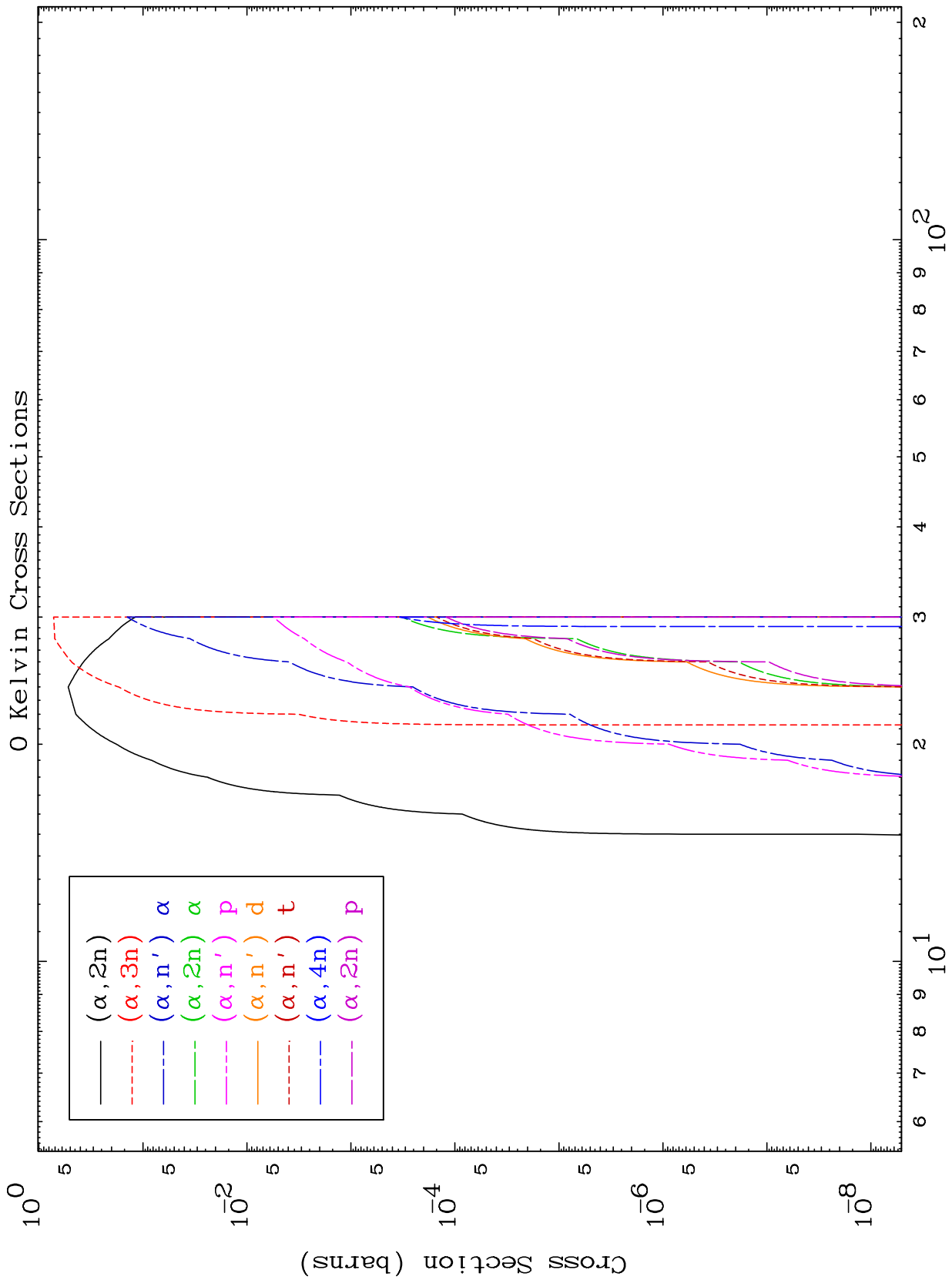


$\alpha$  Elastic  
 $\alpha$  Inelastic  
( $\alpha$ , remainder)  
( $\alpha, \gamma$ )

MAT 7337

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

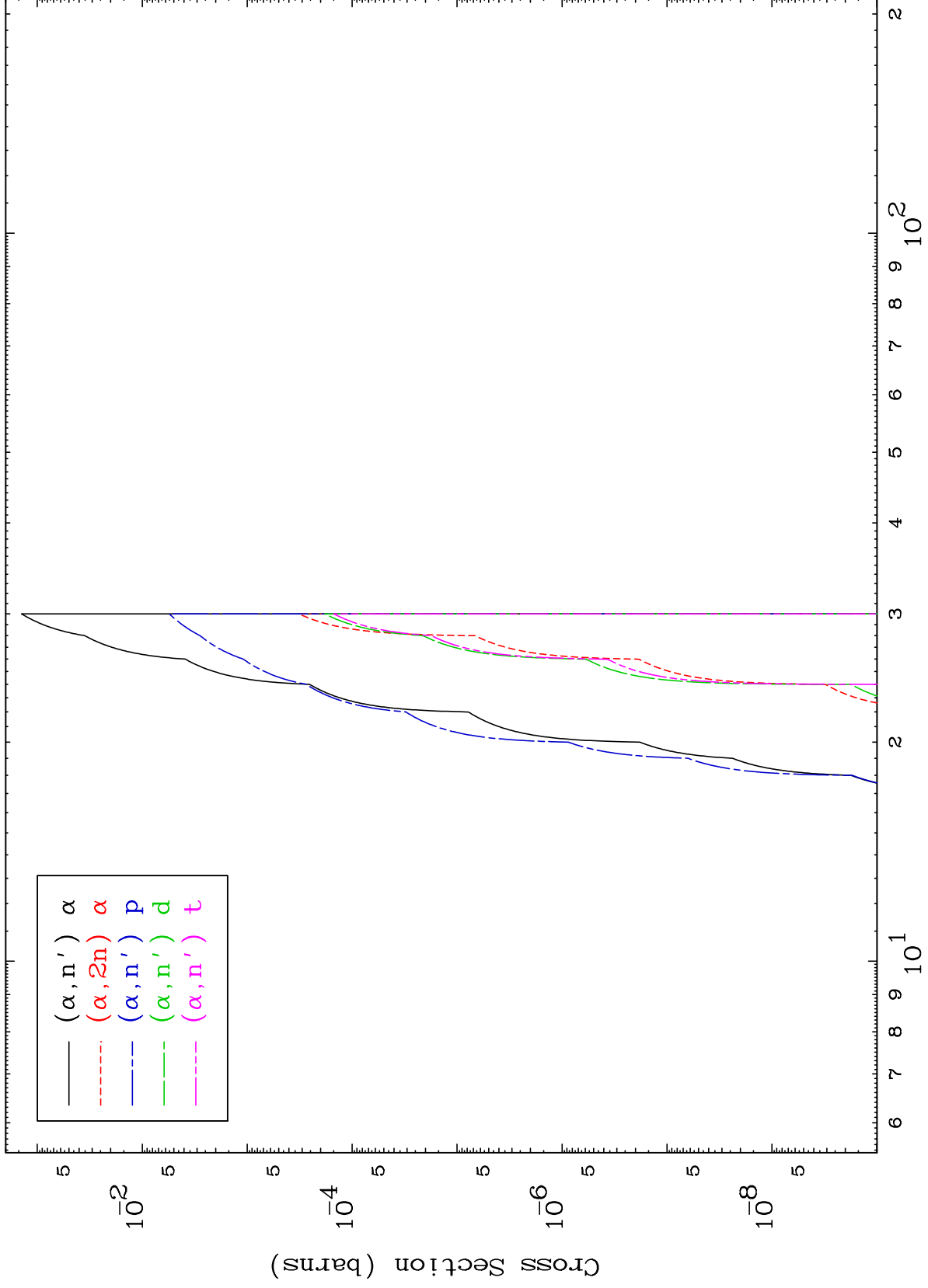
73-Ta-184

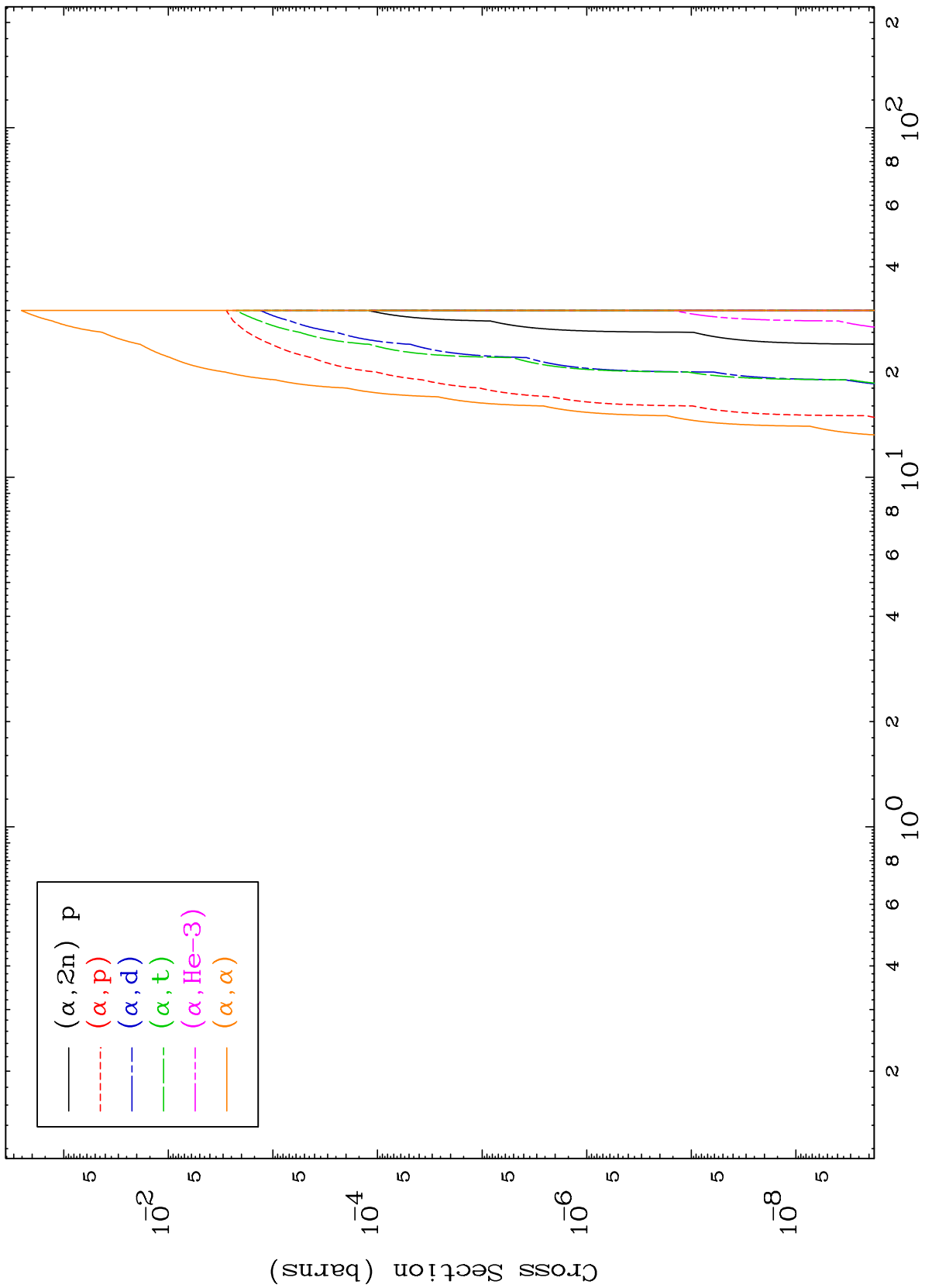


2

Incident Energy (MeV)

73-Ta-184



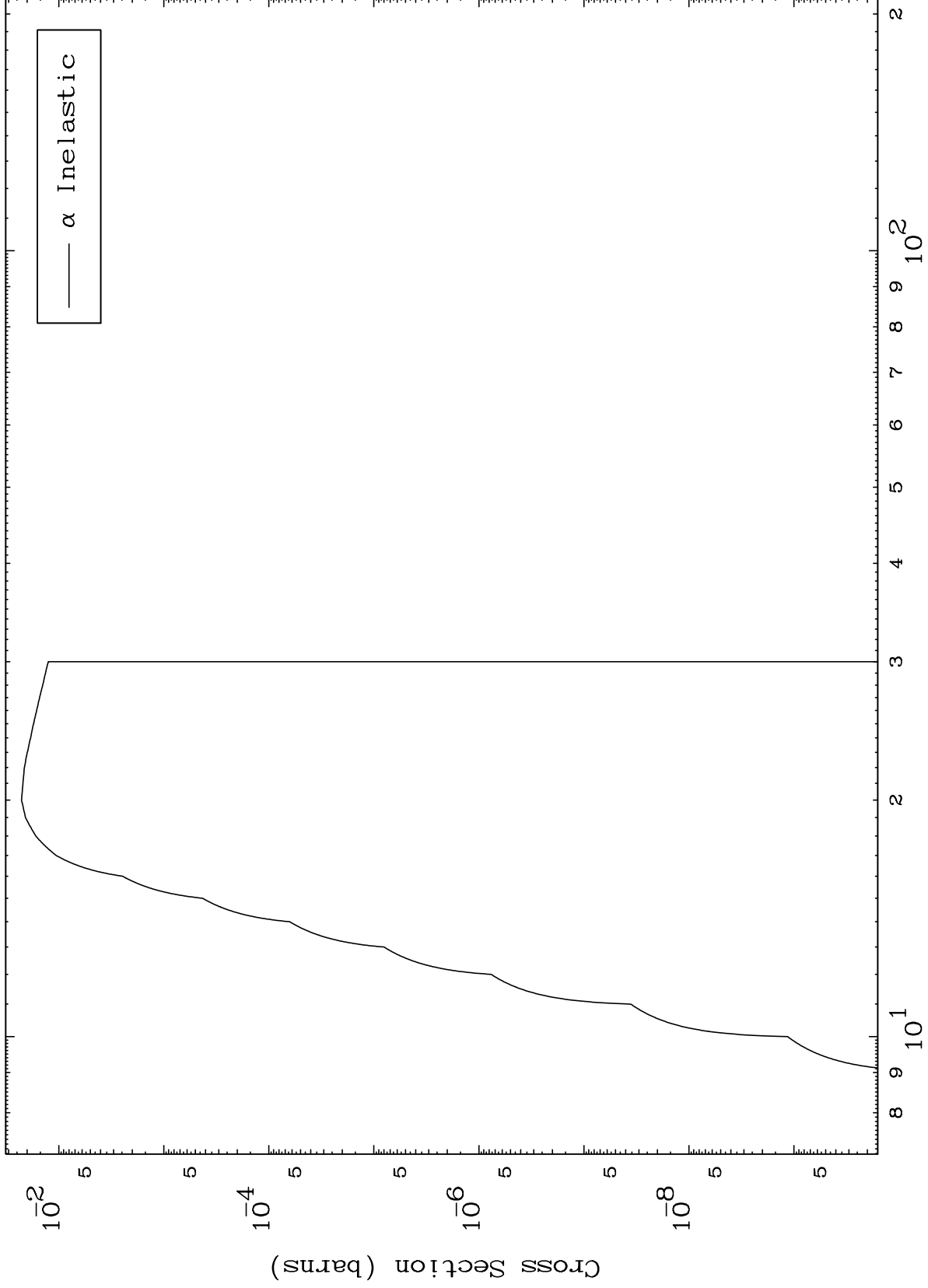


MAT 7337

( $\alpha, n'$ ) Level

73-Ta-184

0 Kelvin Cross Sections



5

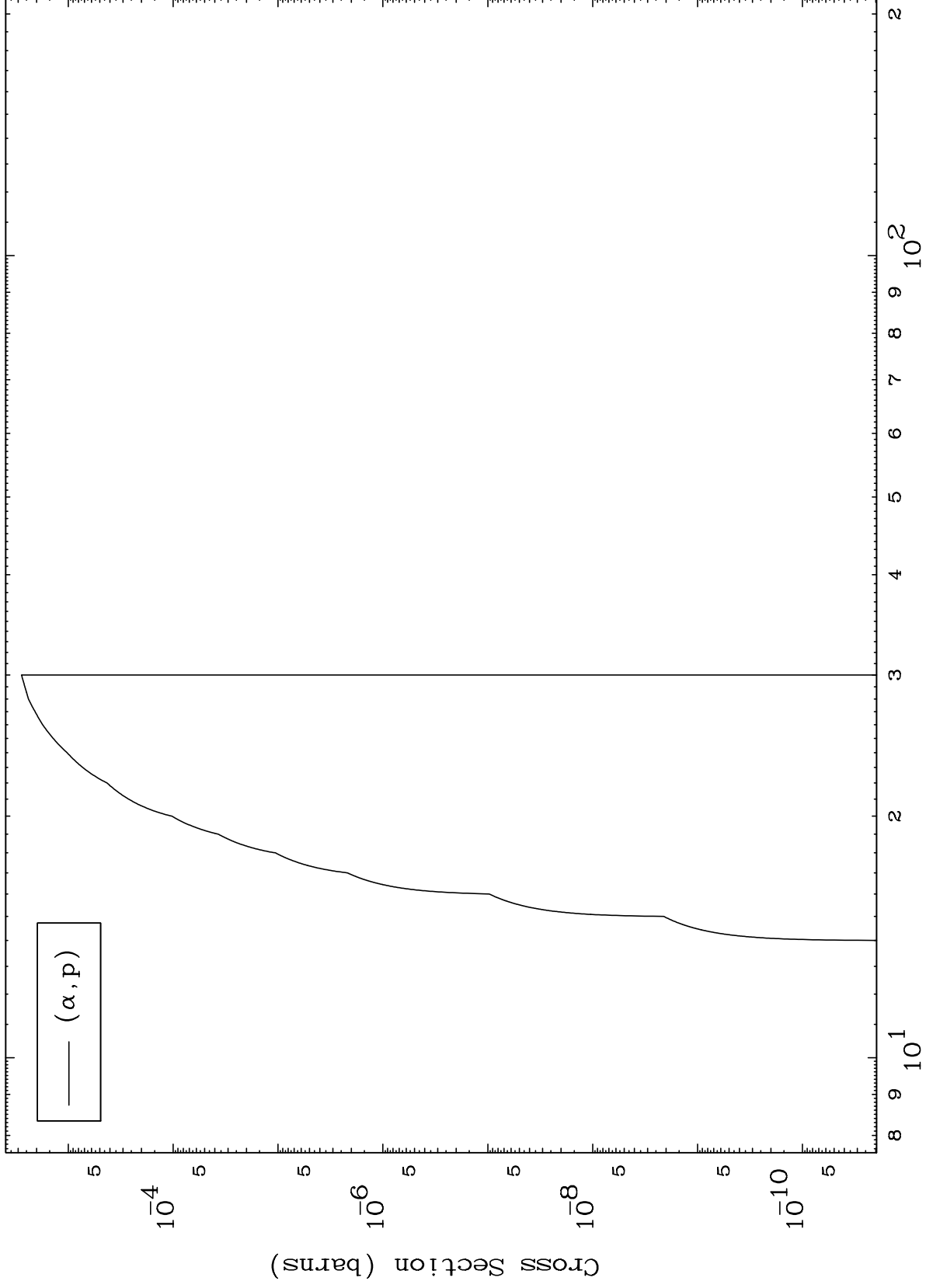
Incident Energy (MeV)

73-Ta-184

MAT 7337

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

73-Ta-184



6

Incident Energy (MeV)

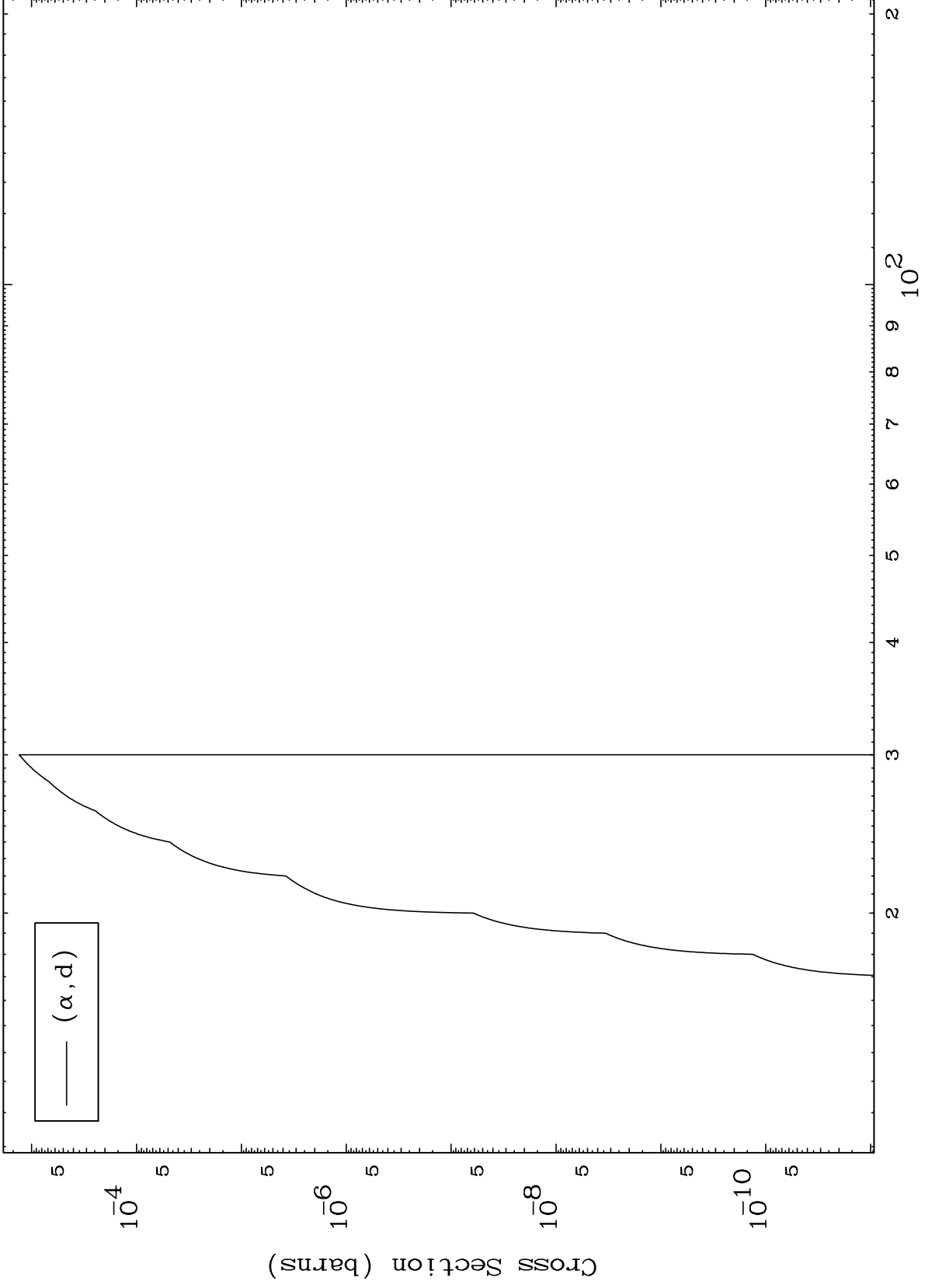
73-Ta-184

MAT 7337

( $\alpha, d$ ) Levels

$^{73}\text{Ta-184}$

0 Kelvin Cross Sections

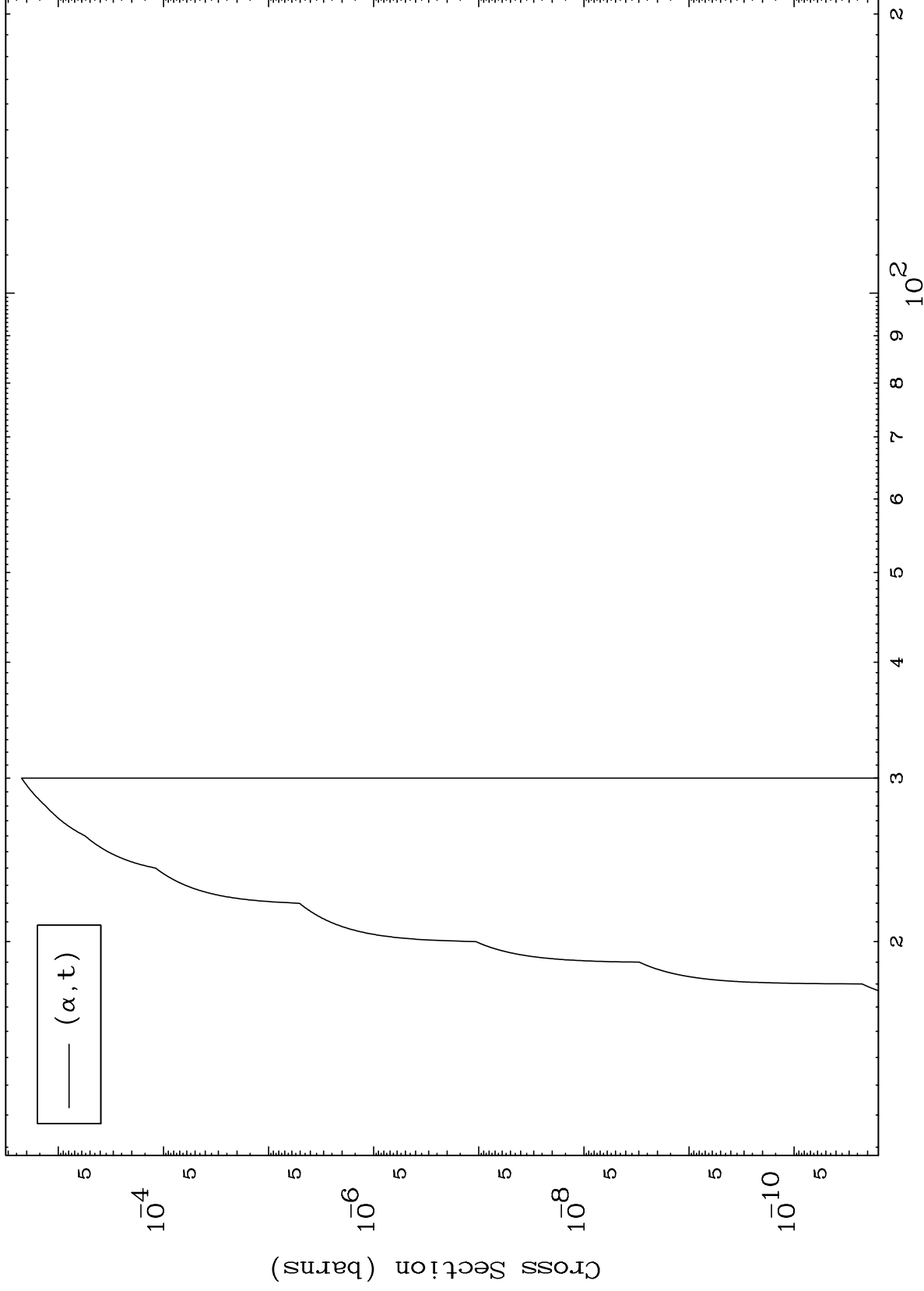


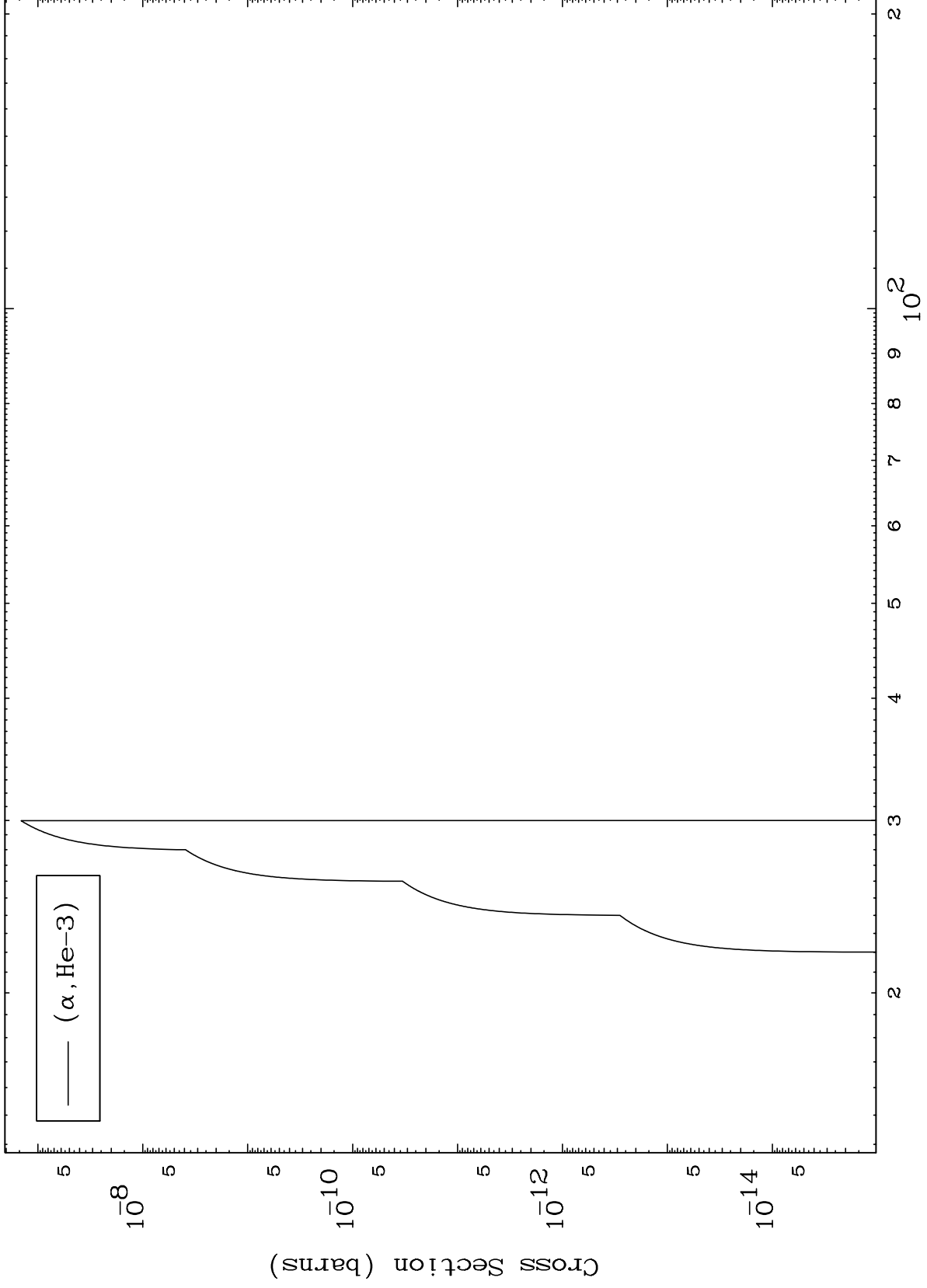
7

Incident Energy (MeV)

$^{73}\text{Ta-184}$





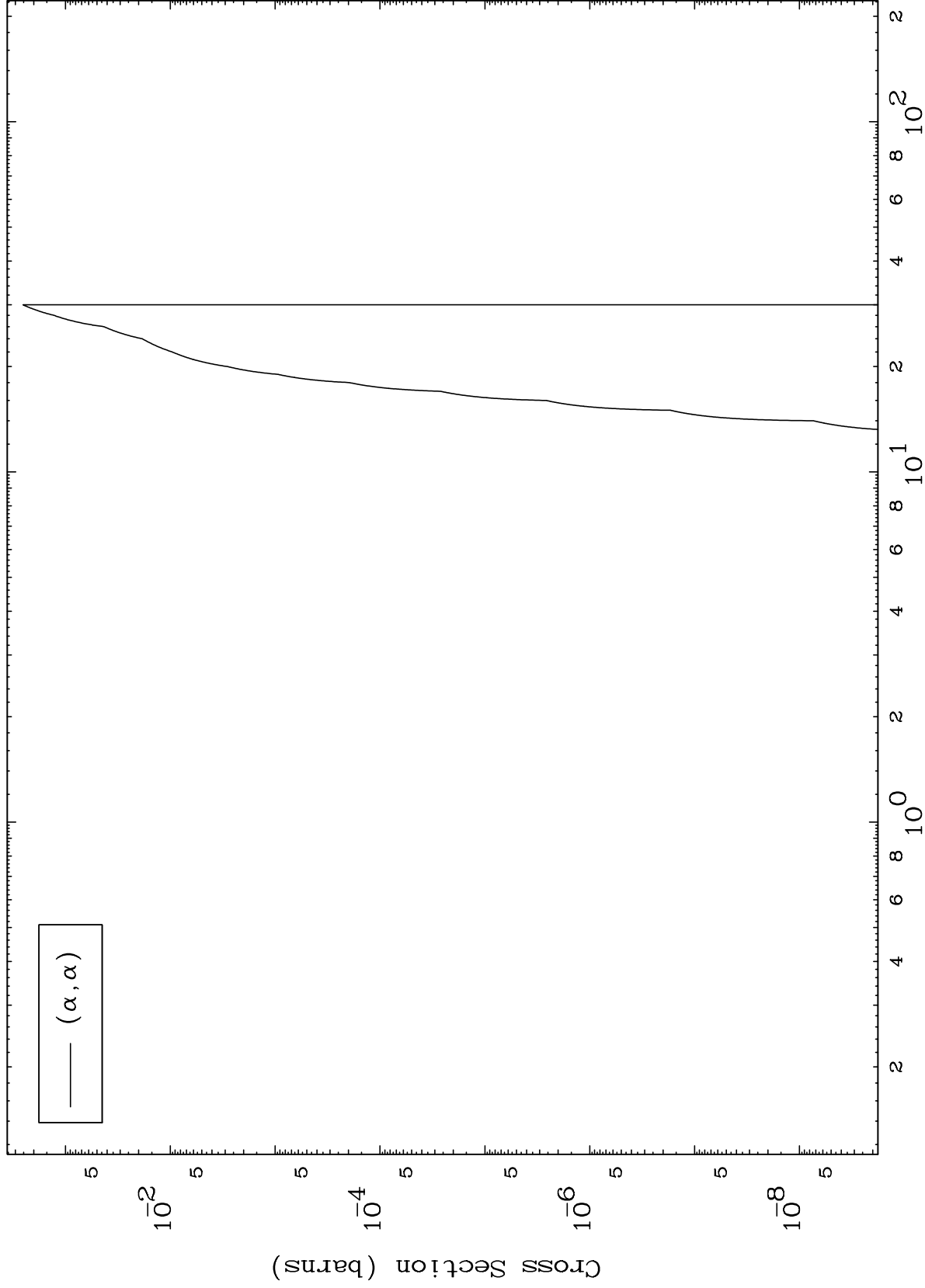


MAT 7337

( $\alpha, \alpha$ ) Levels

$^{73}\text{Ta-184}$

0 Kelvin Cross Sections



10

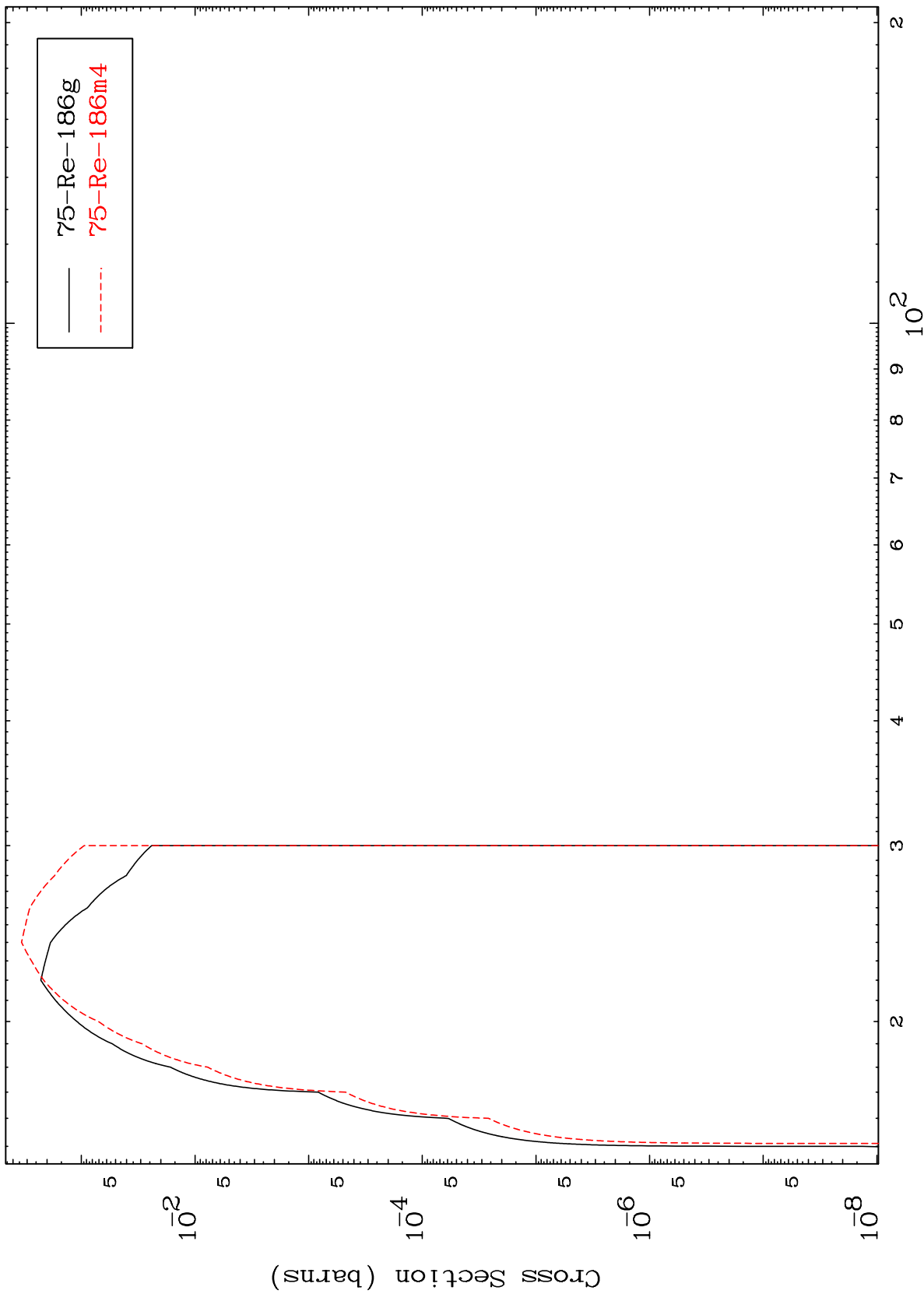
Incident Energy (MeV)

$^{73}\text{Ta-184}$

MAT 7337

73-Ta-184

$(\alpha, 2n)$   
Radionuclide Production Cross Section



11

73-Ta-184

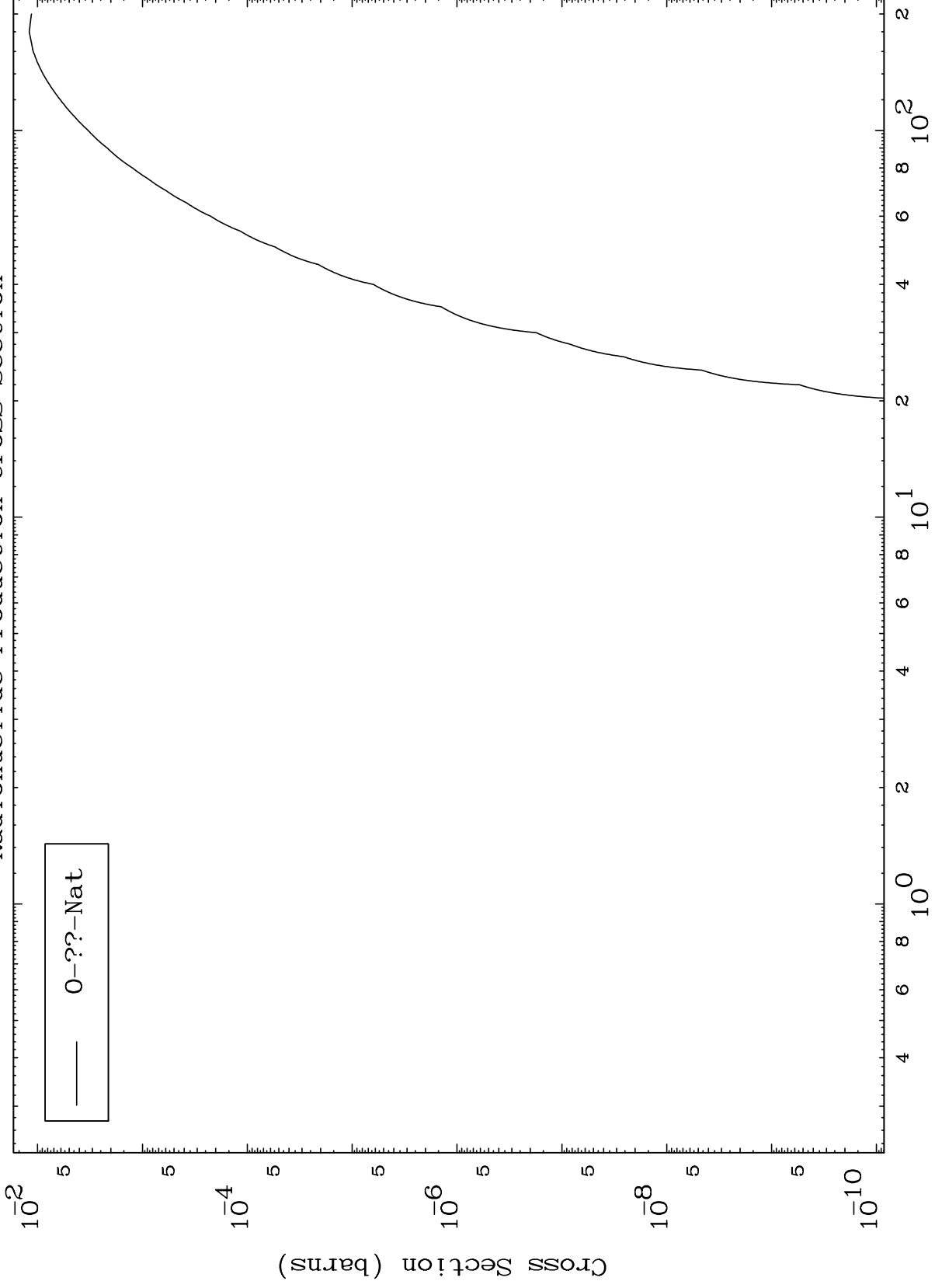
Incident Energy (MeV)

MAT 7337

$\alpha$  Fission

<sup>73</sup>Ta-184

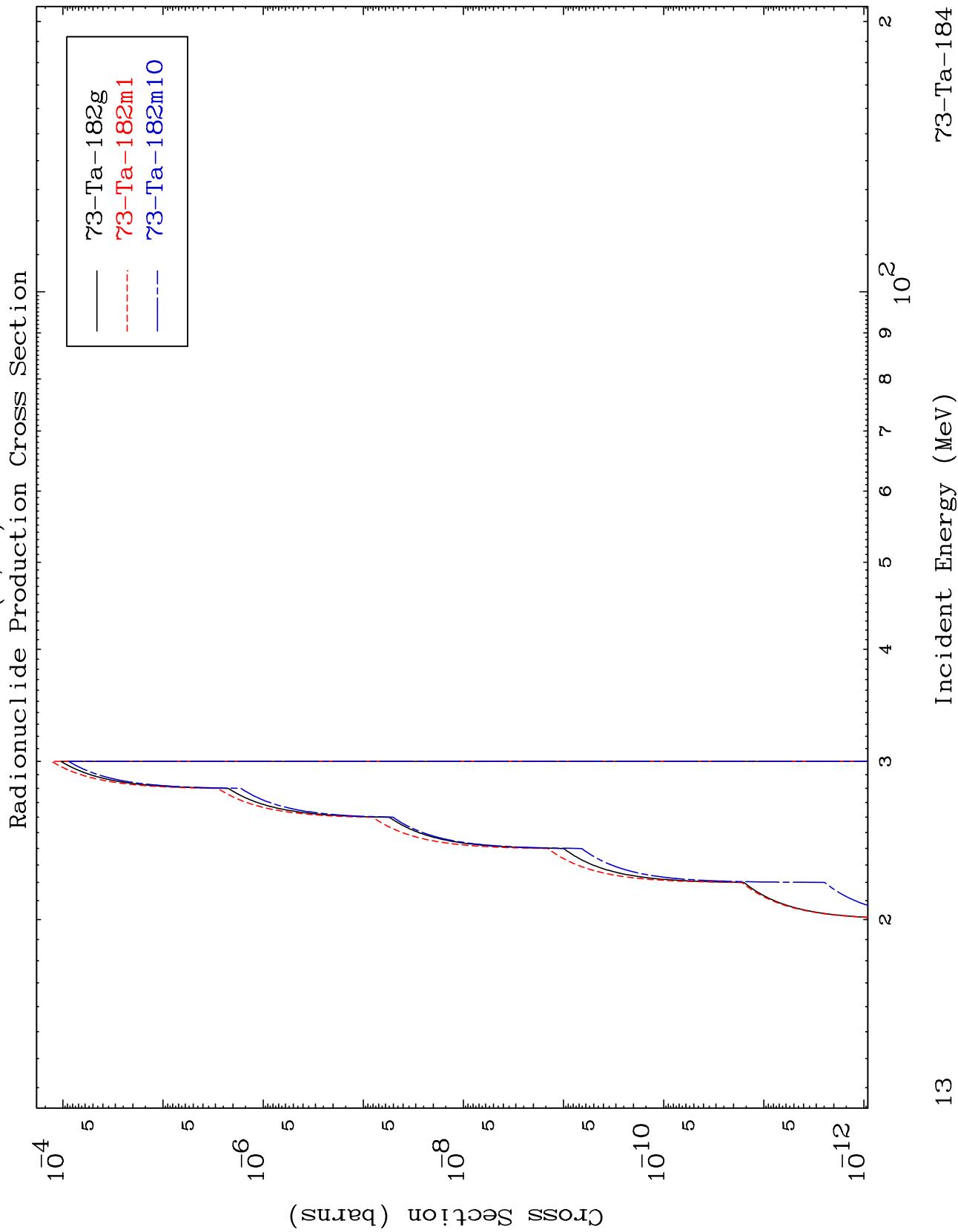
Radionuclide Production Cross Section



12

Incident Energy (MeV)

<sup>73</sup>Ta-184

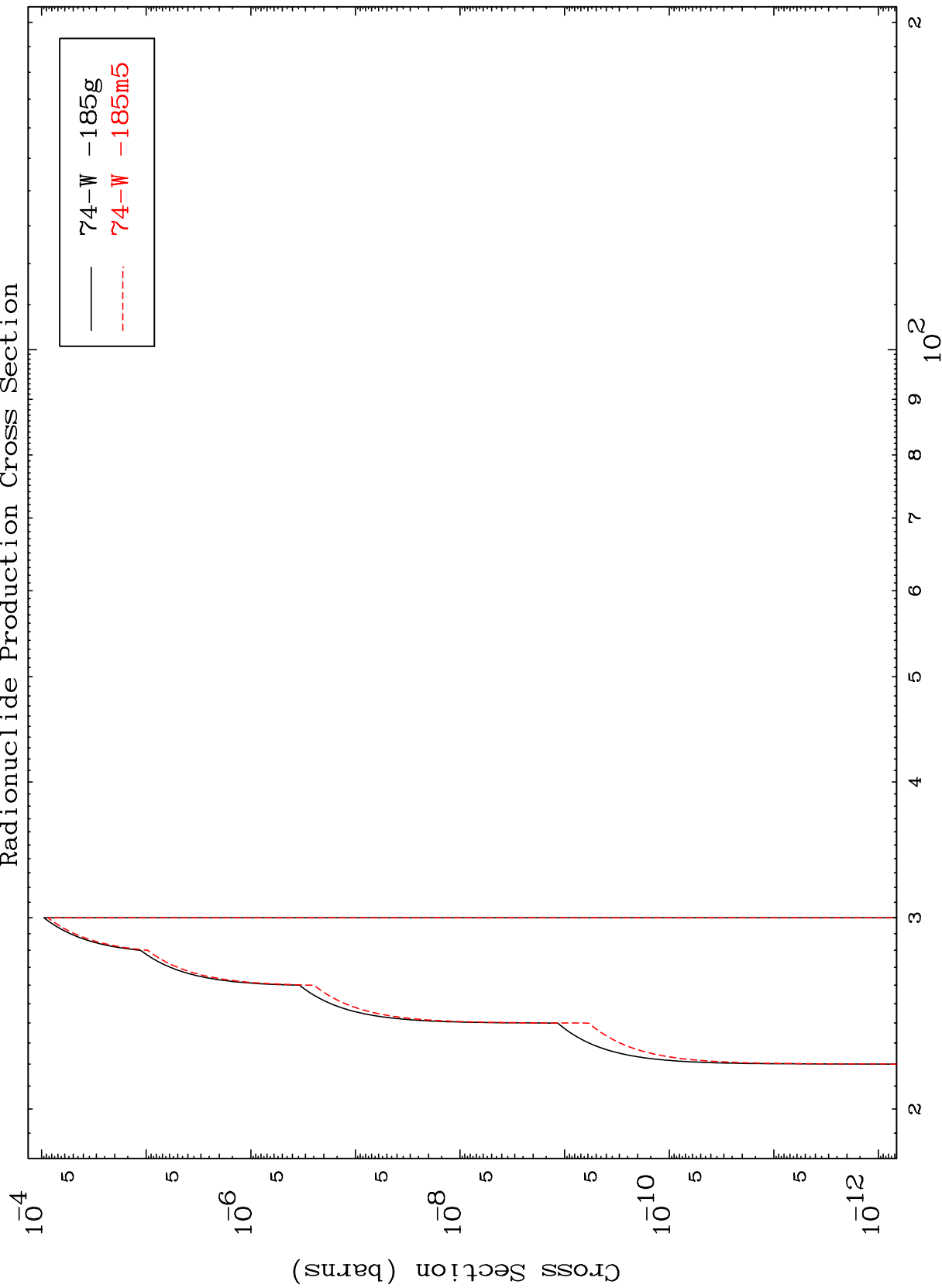


MAT 7337

$(\alpha, n')$  d

<sup>73</sup>Ta-184

Radionuclide Production Cross Section



14

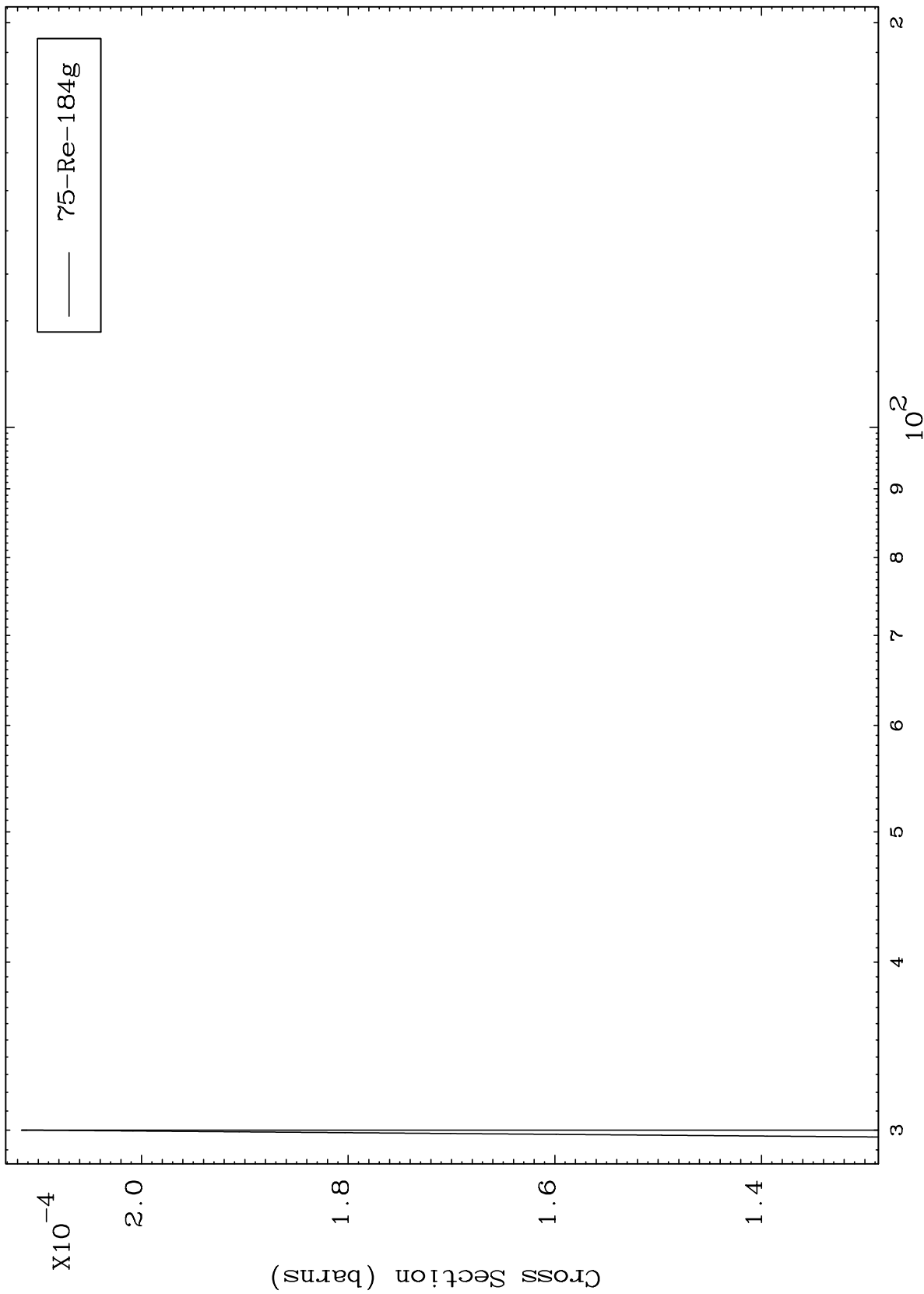
Incident Energy (MeV)

<sup>73</sup>Ta-184

MAT 7337

73-Ta-184

( $\alpha, 4n$ )  
Radionuclide Production Cross Section



15

73-Ta-184

Incident Energy (MeV)

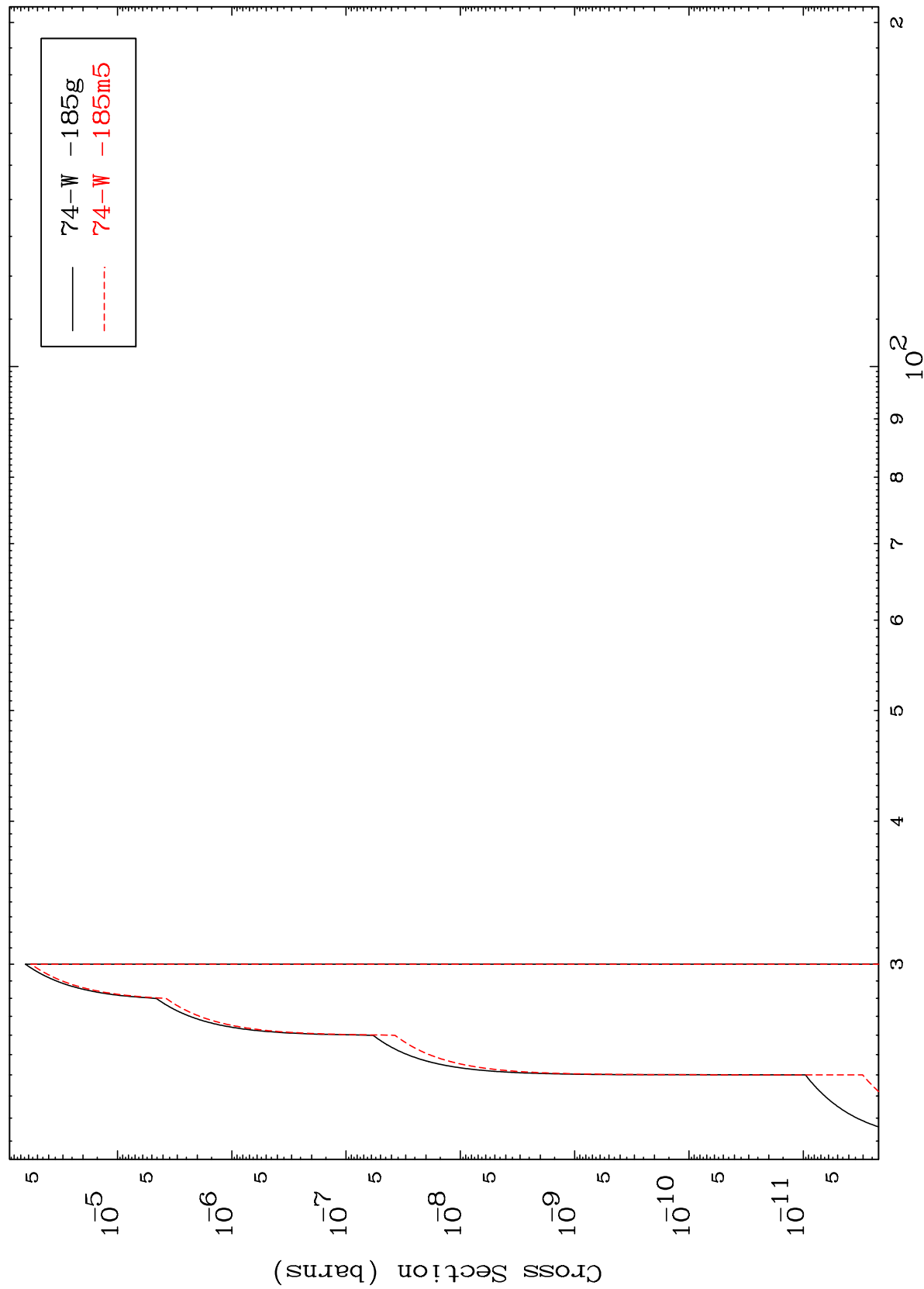


MAT 7337

$(\alpha, 2n)$  p

$^{73}\text{Ta-184}$

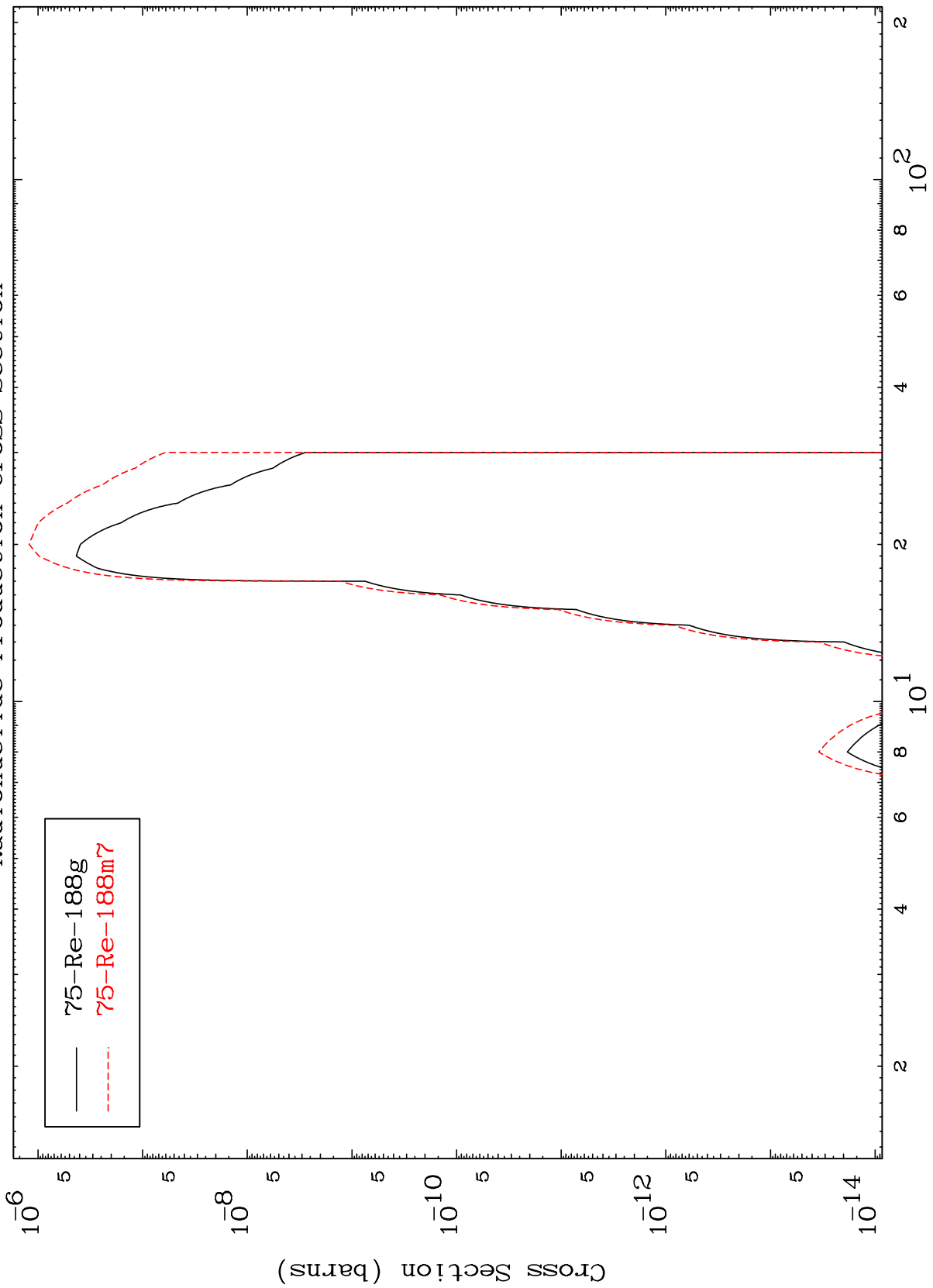
Radionuclide Production Cross Section



MAT 7337

73-Ta-184

( $\alpha, \gamma$ )  
Radionuclide Production Cross Section

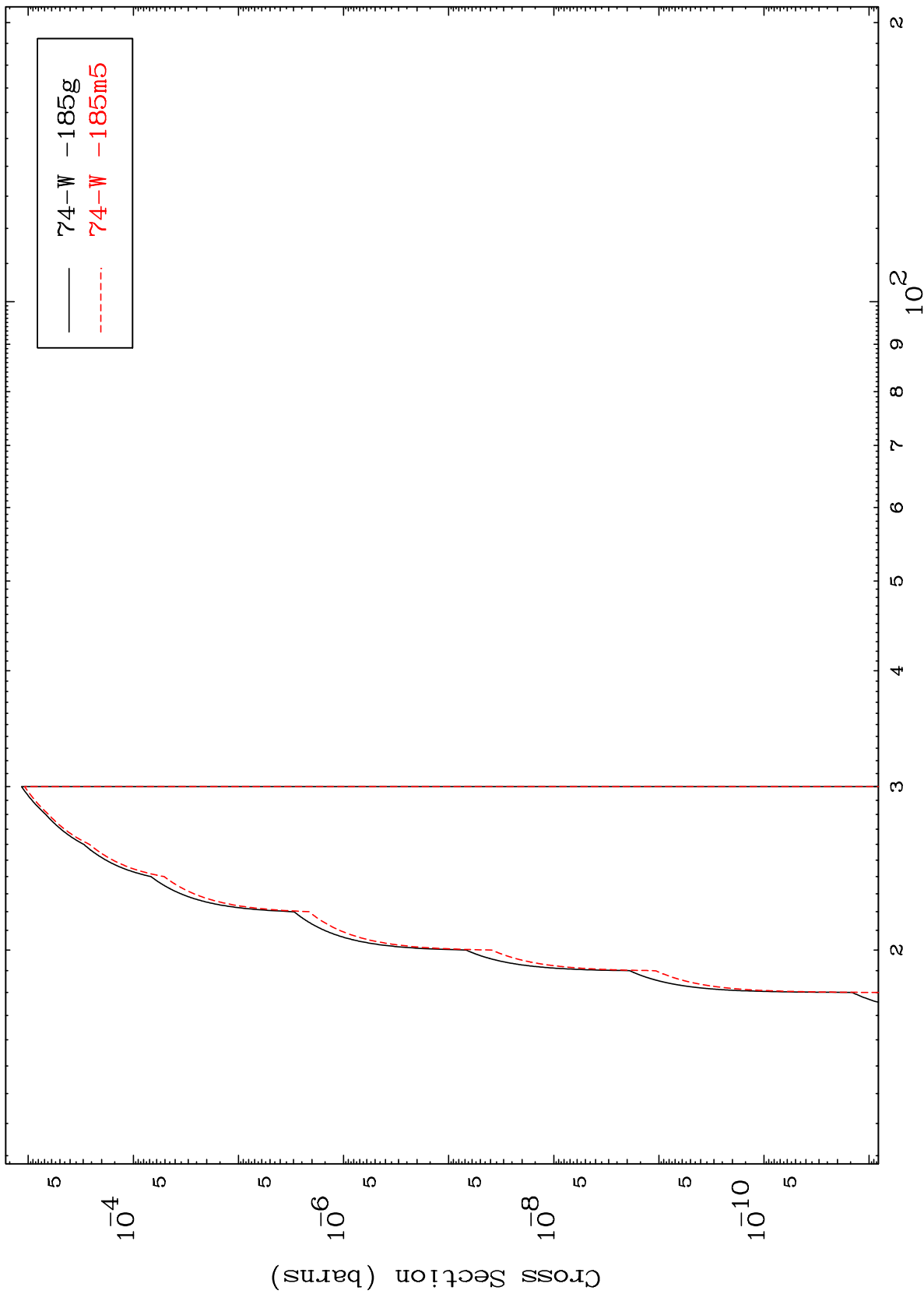


75-Re-188g  
75-Re-188m7

MAT 7337

73-Ta-184

( $\alpha, t$ )  
Radionuclide Production Cross Section



18

Incident Energy (MeV)

73-Ta-184