

Program EVALPLOT  
(Version 2018-1)

by

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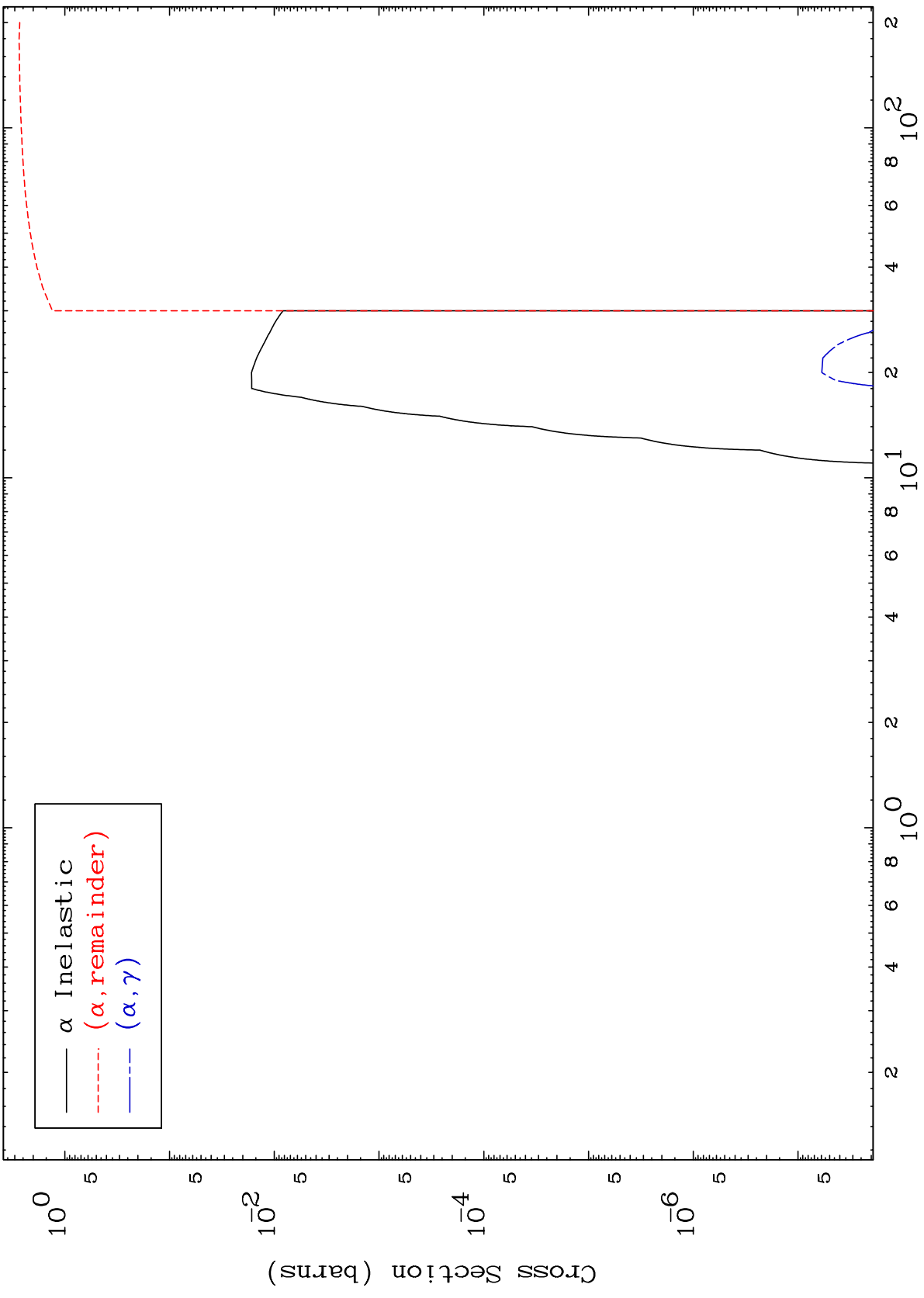
Press Mouse Button to Start

MAT 7355

$\alpha$  Major

73-Ta-190

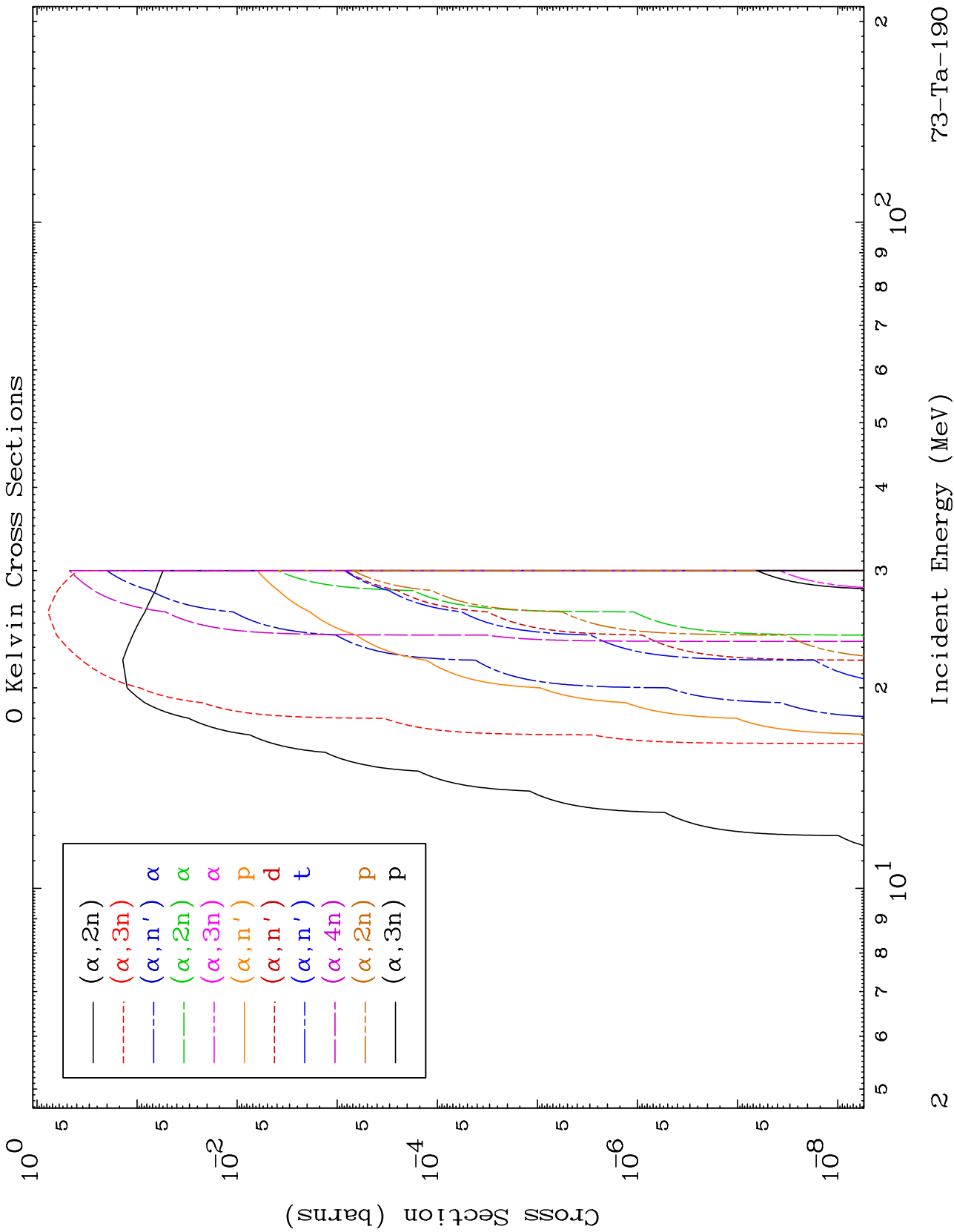
0 Kelvin Cross Sections



MAT 7355

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

73-Ta-190



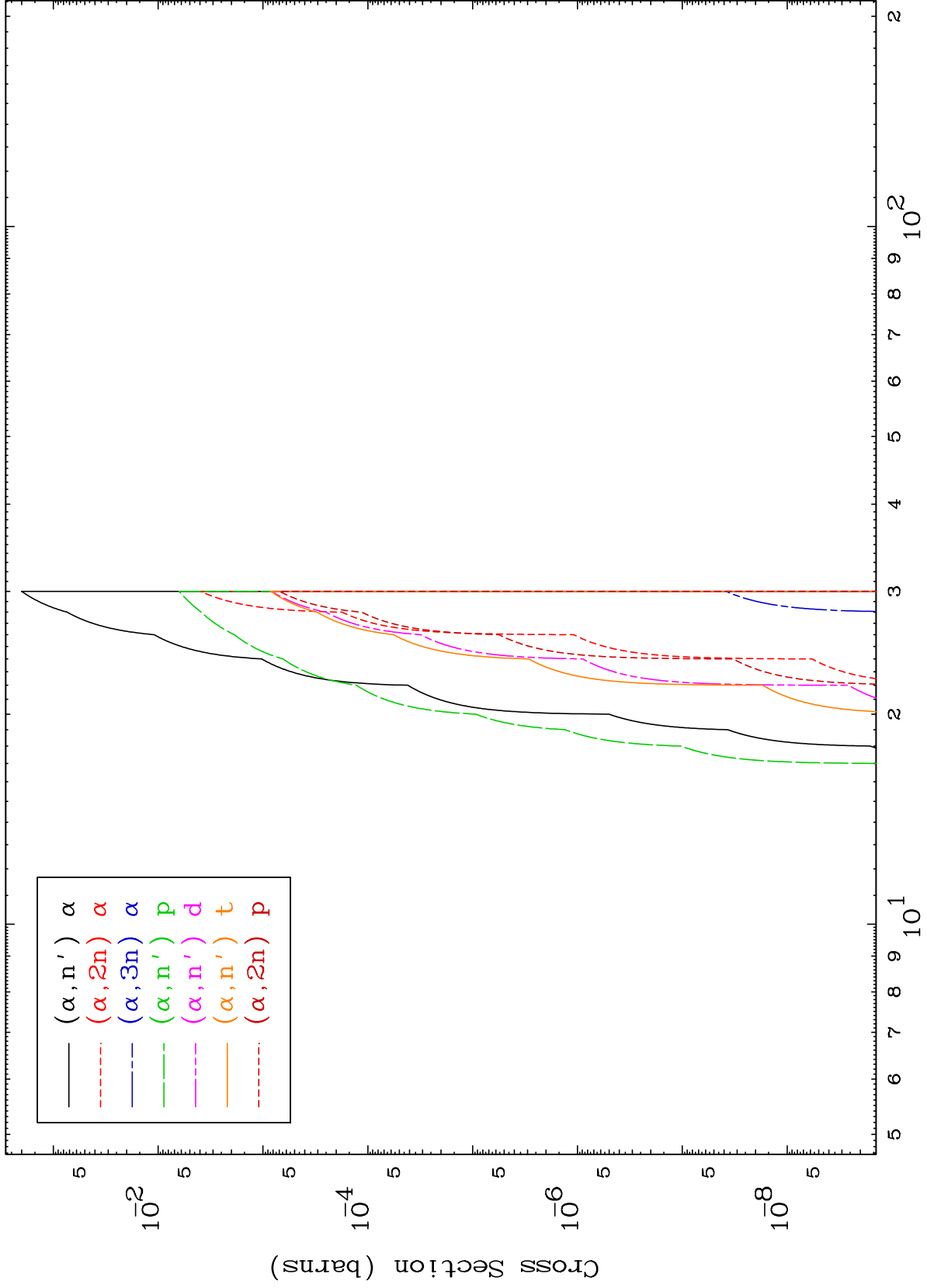
Incident Energy (MeV)

73-Ta-190

MAT 7355

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

73-Ta-190



3

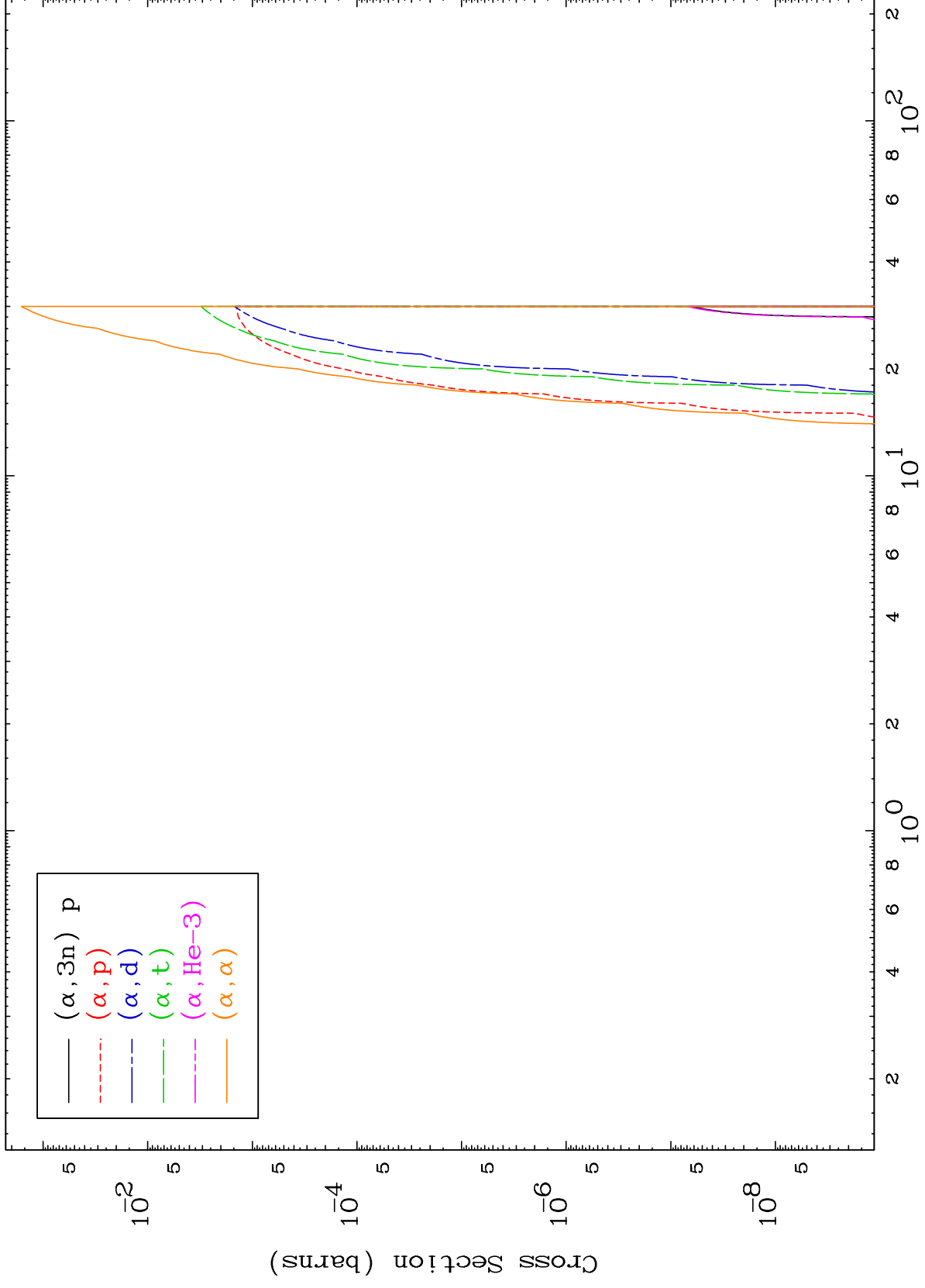
Incident Energy (MeV)

73-Ta-190

MAT 7355

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

73-Ta-190



73-Ta-190

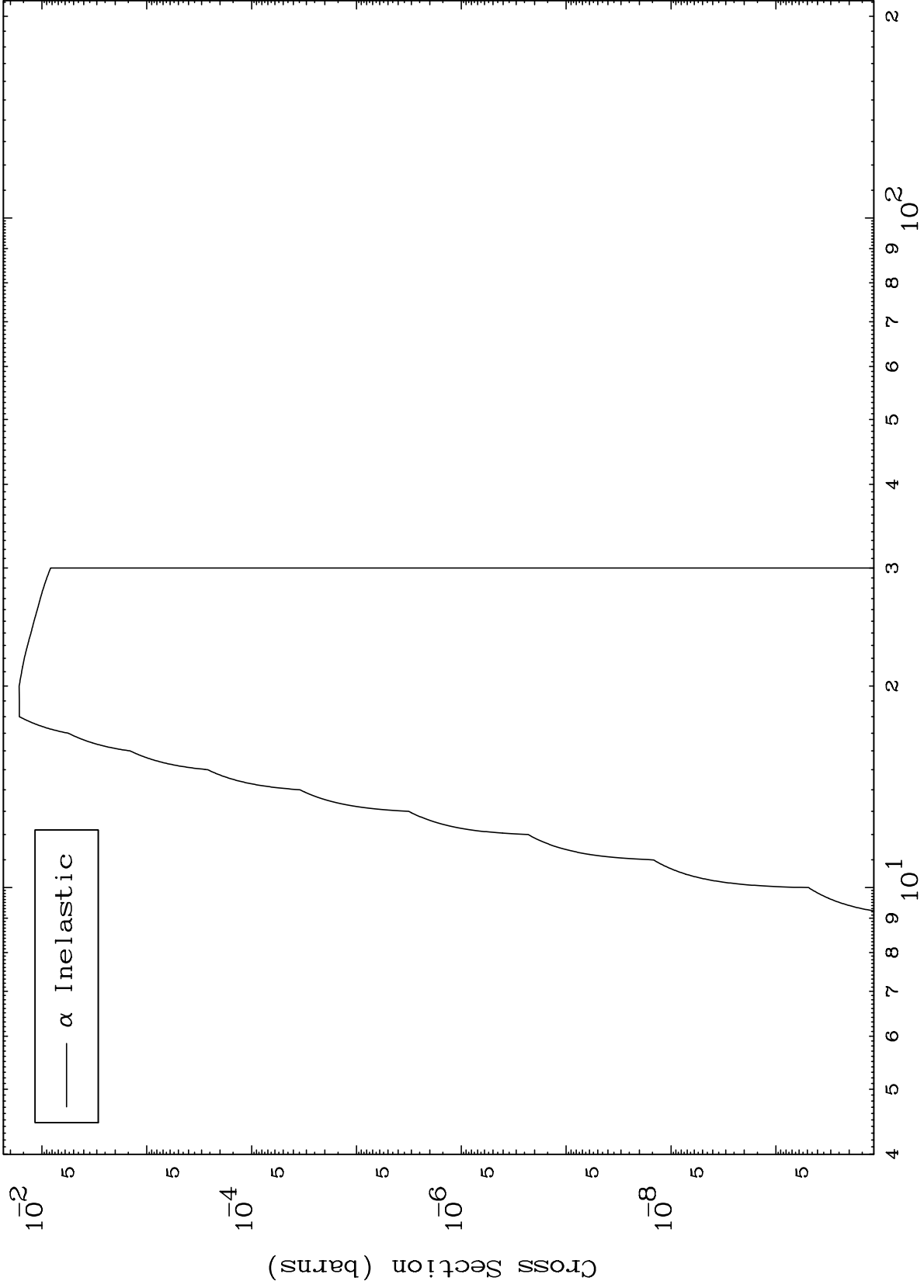
Incident Energy (MeV)

MAT 7355

( $\alpha, n'$ ) Level

73-Ta-190

0 Kelvin Cross Sections



5

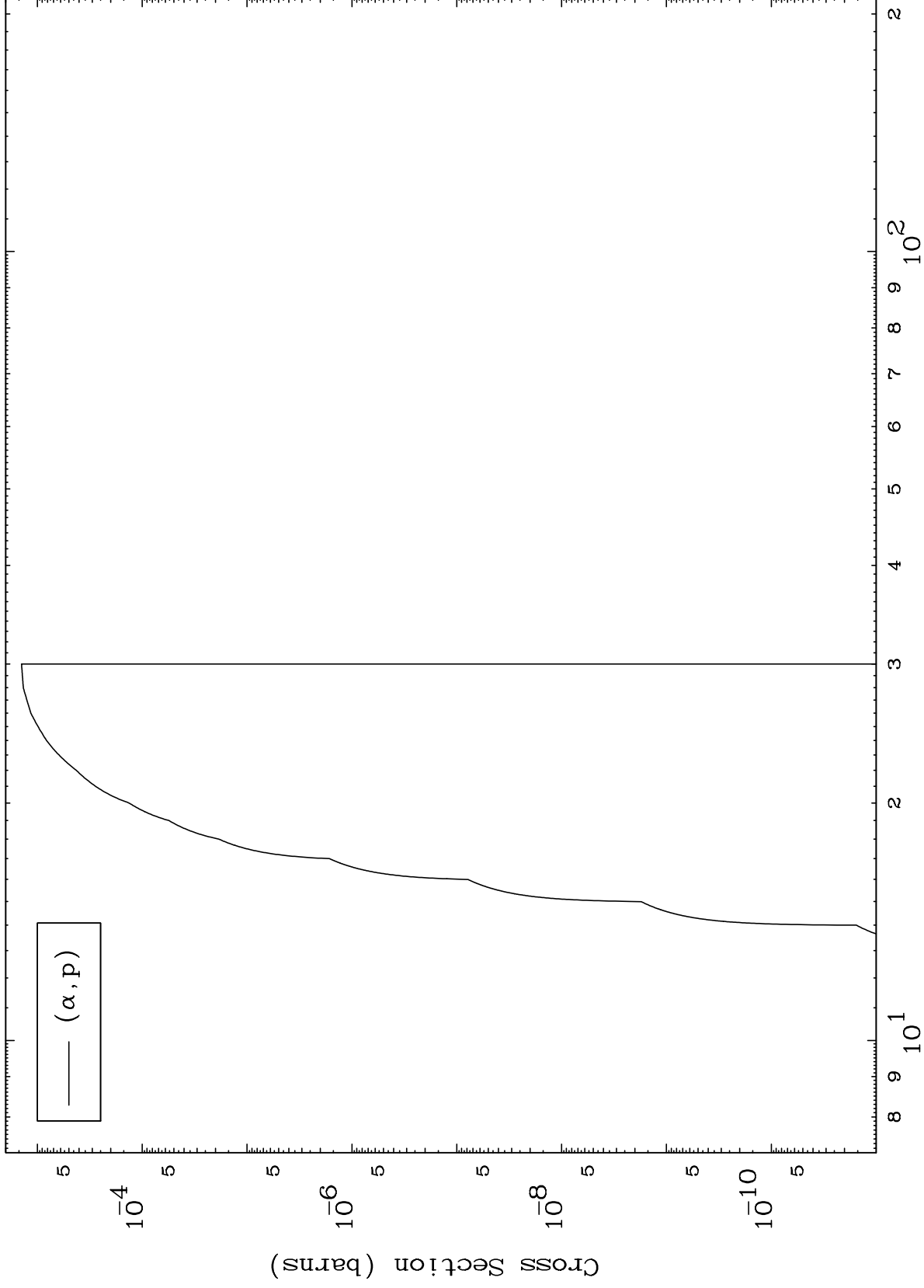
Incident Energy (MeV)

73-Ta-190

MAT 7355

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

73-Ta-190



6

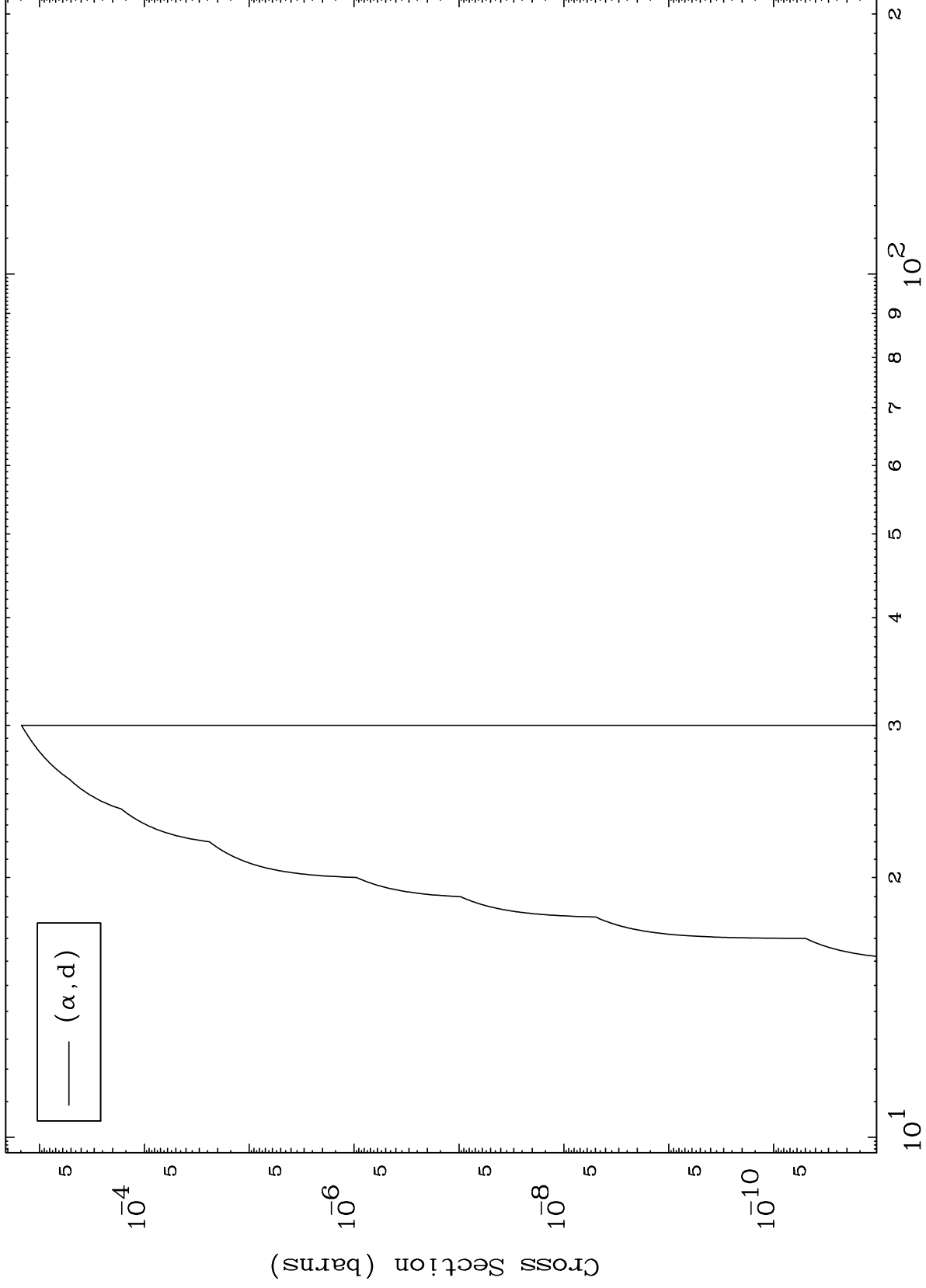
Incident Energy (MeV)

73-Ta-190

MAT 7355

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

$^{73}\text{Ta-190}$



Incident Energy (MeV)

$^{73}\text{Ta-190}$

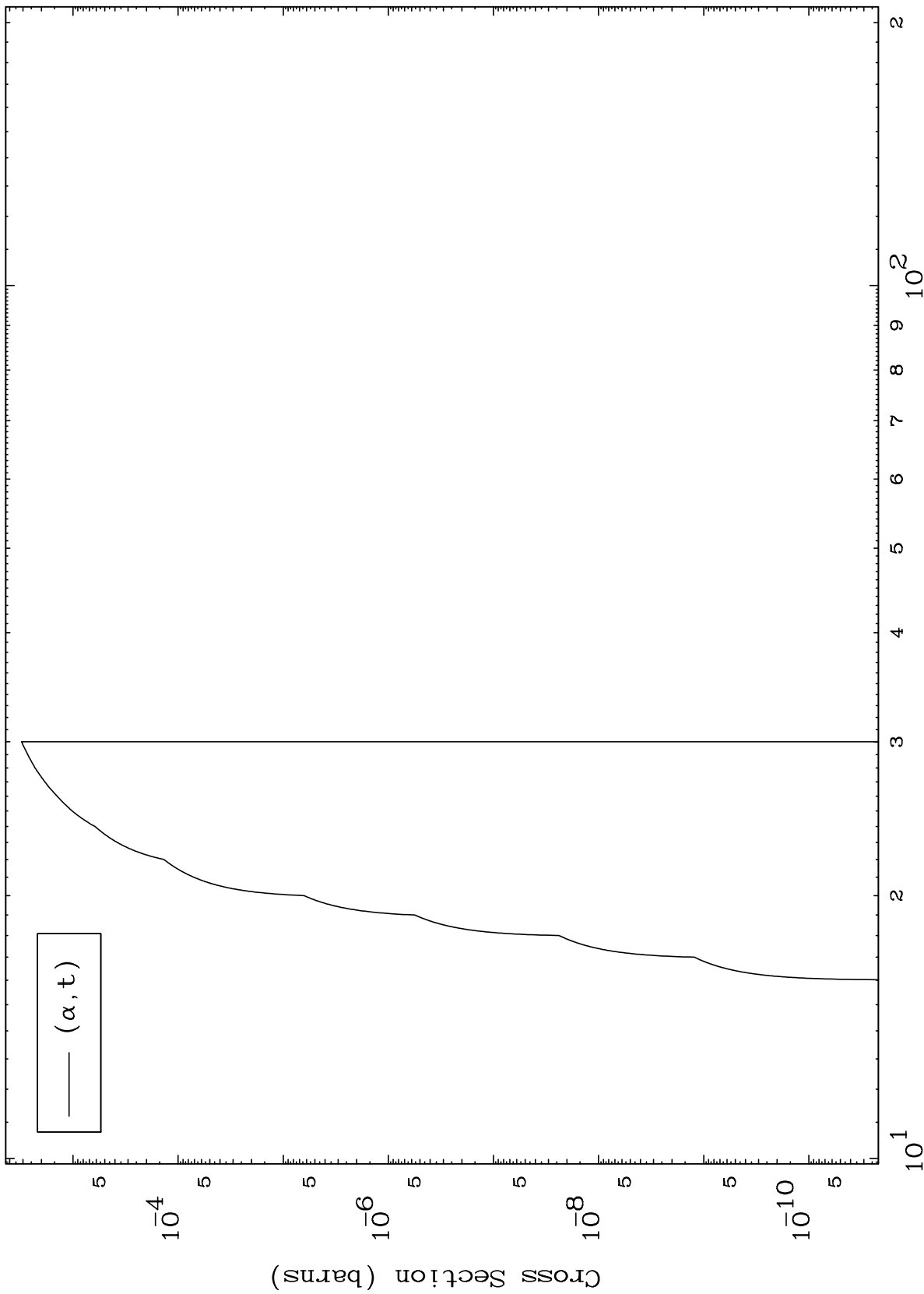


MAT 7355

( $\alpha, t$ ) Levels

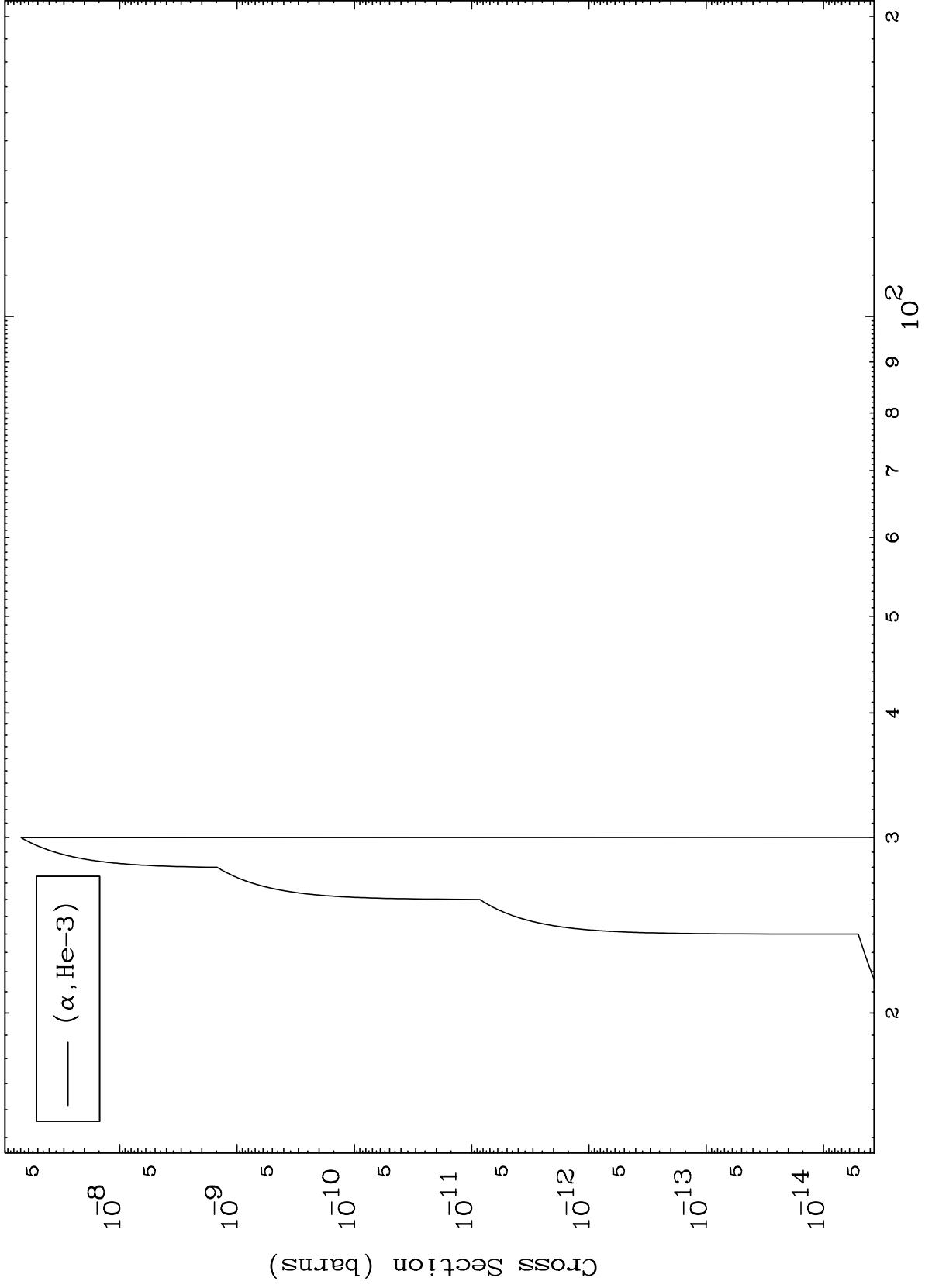
73-Ta-190

0 Kelvin Cross Sections



Incident Energy (MeV)

73-Ta-190

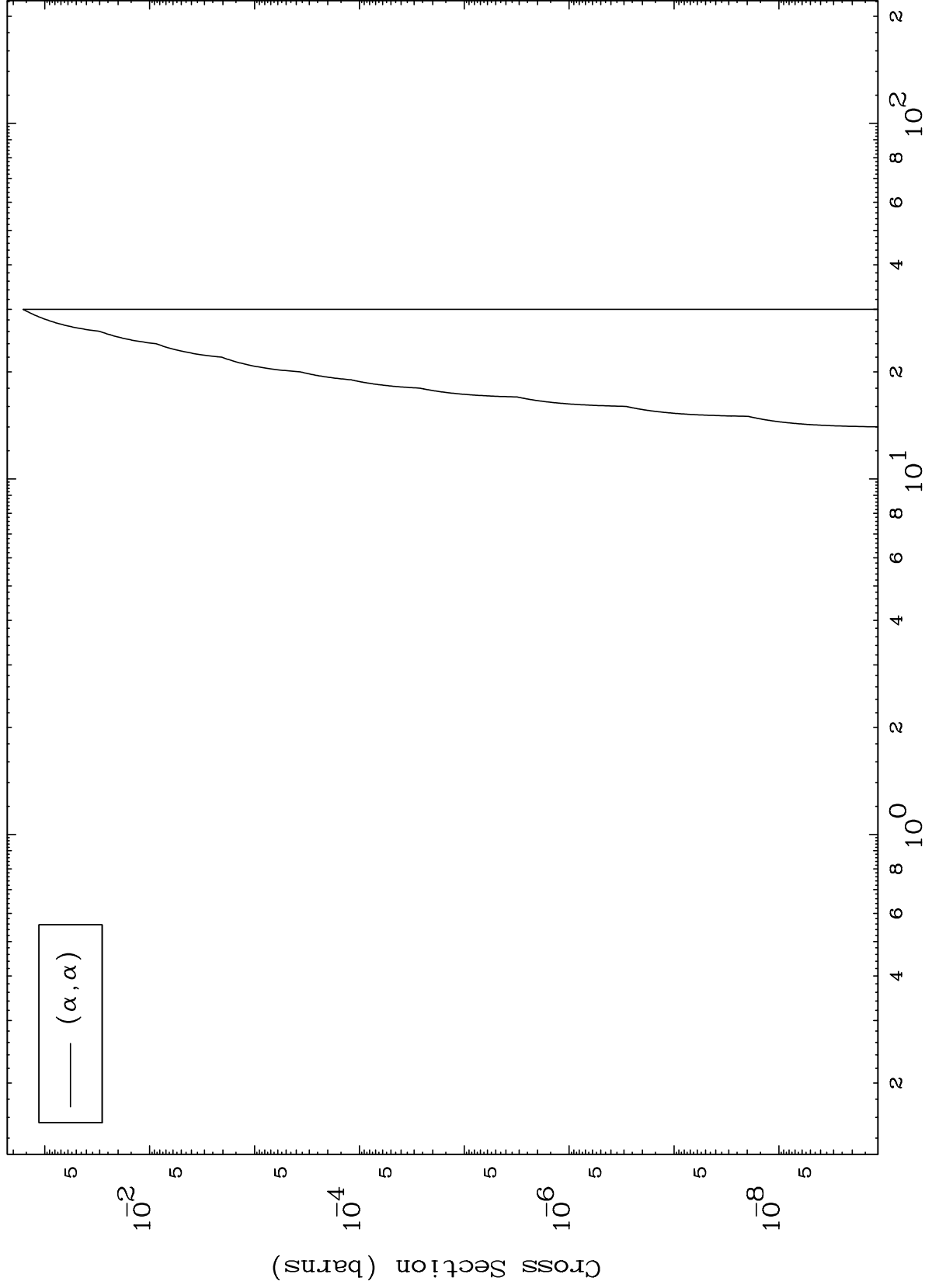


MAT 7355

( $\alpha, \alpha$ ) Levels

73-Ta-190

0 Kelvin Cross Sections



10

Incident Energy (MeV)

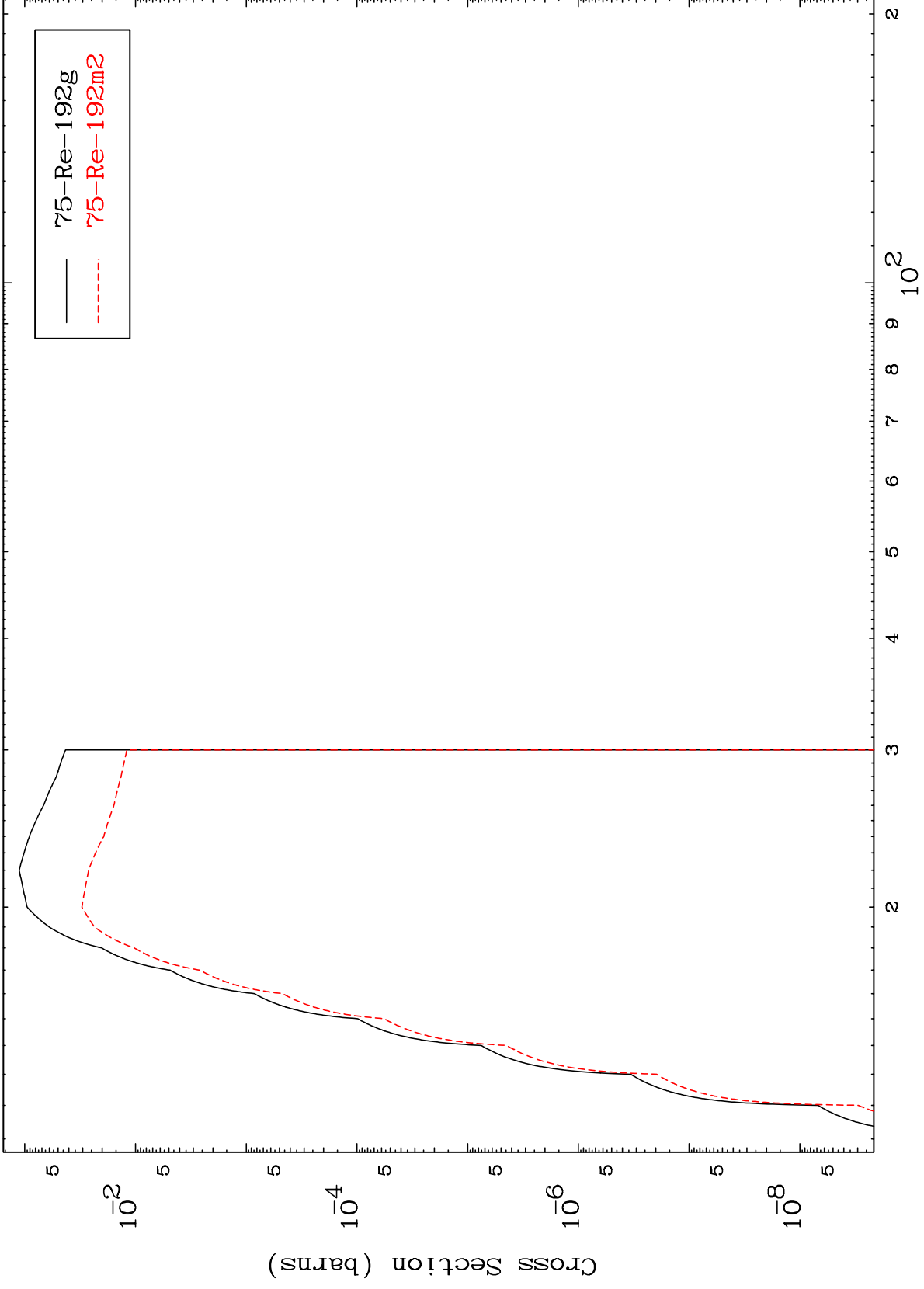
73-Ta-190

MAT 7355

( $\alpha, 2n$ )

<sup>73</sup>Ta-190

Radionuclide Production Cross Section

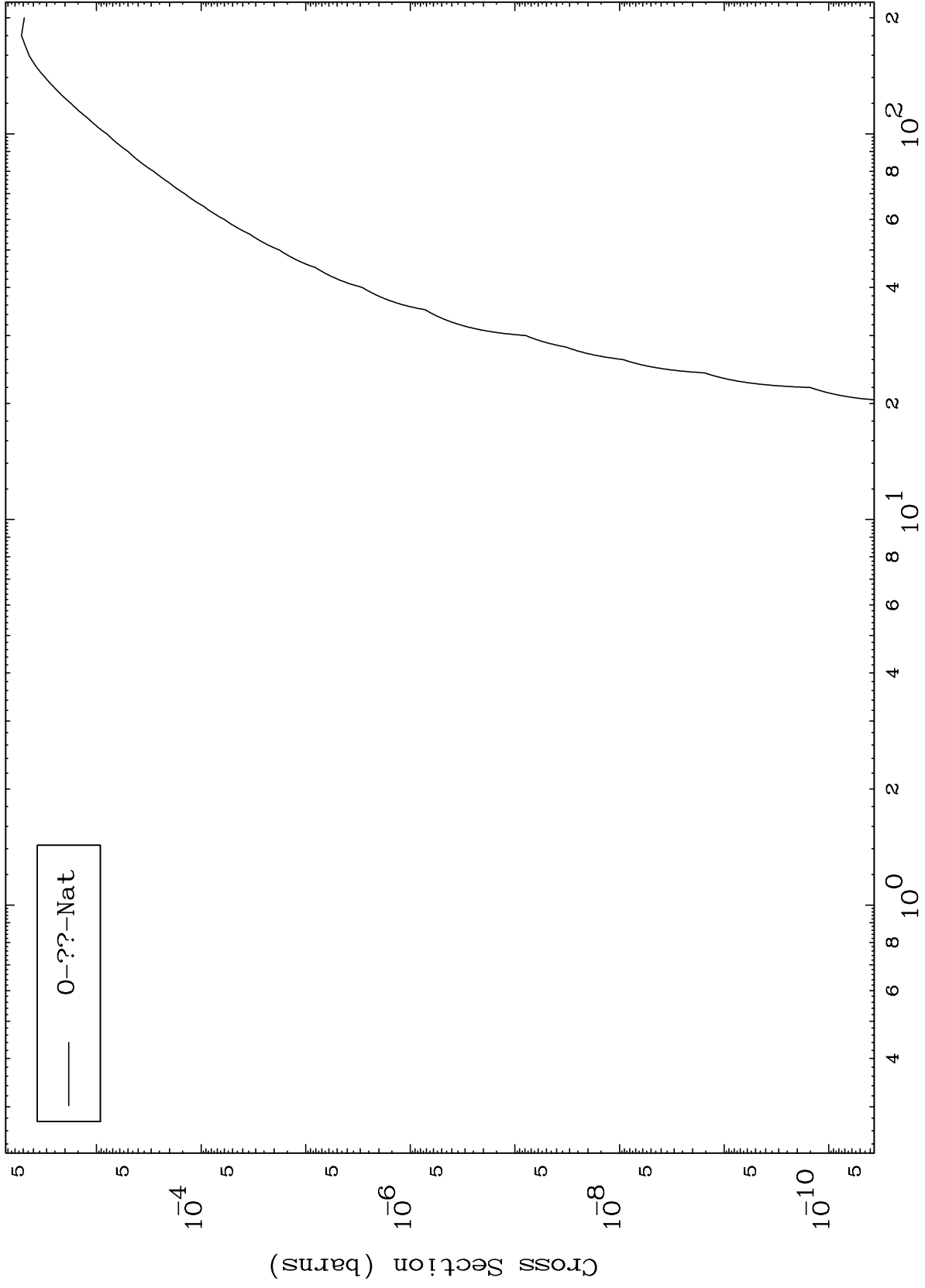


MAT 7355

$\alpha$  Fission

<sup>73</sup>Ta-190

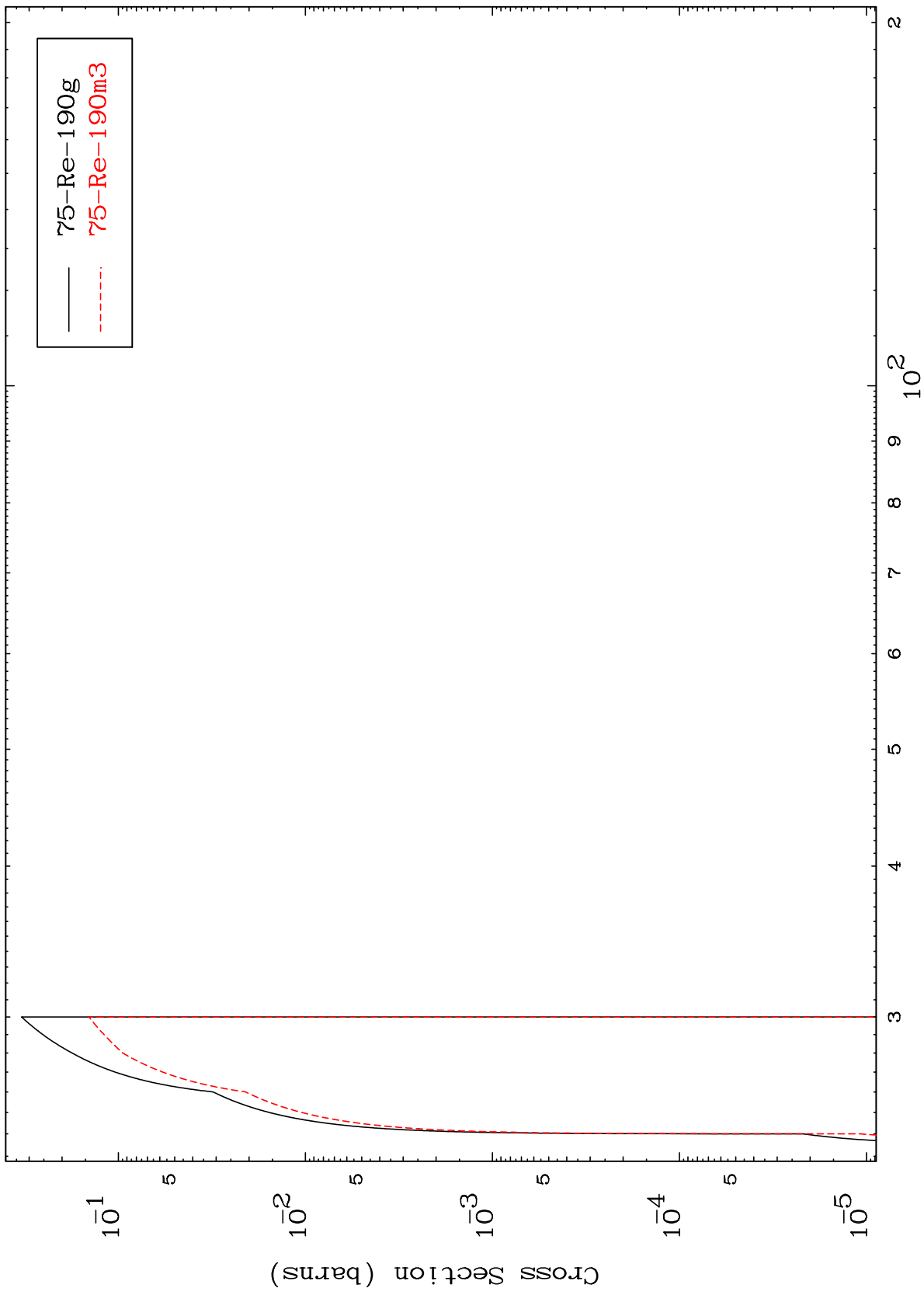
Radionuclide Production Cross Section



MAT 7355

73-Ta-190

( $\alpha, 4n$ )  
Radionuclide Production Cross Section



13

73-Ta-190

Incident Energy (MeV)