

Program EVALPLOT  
(Version 2018-1)

by

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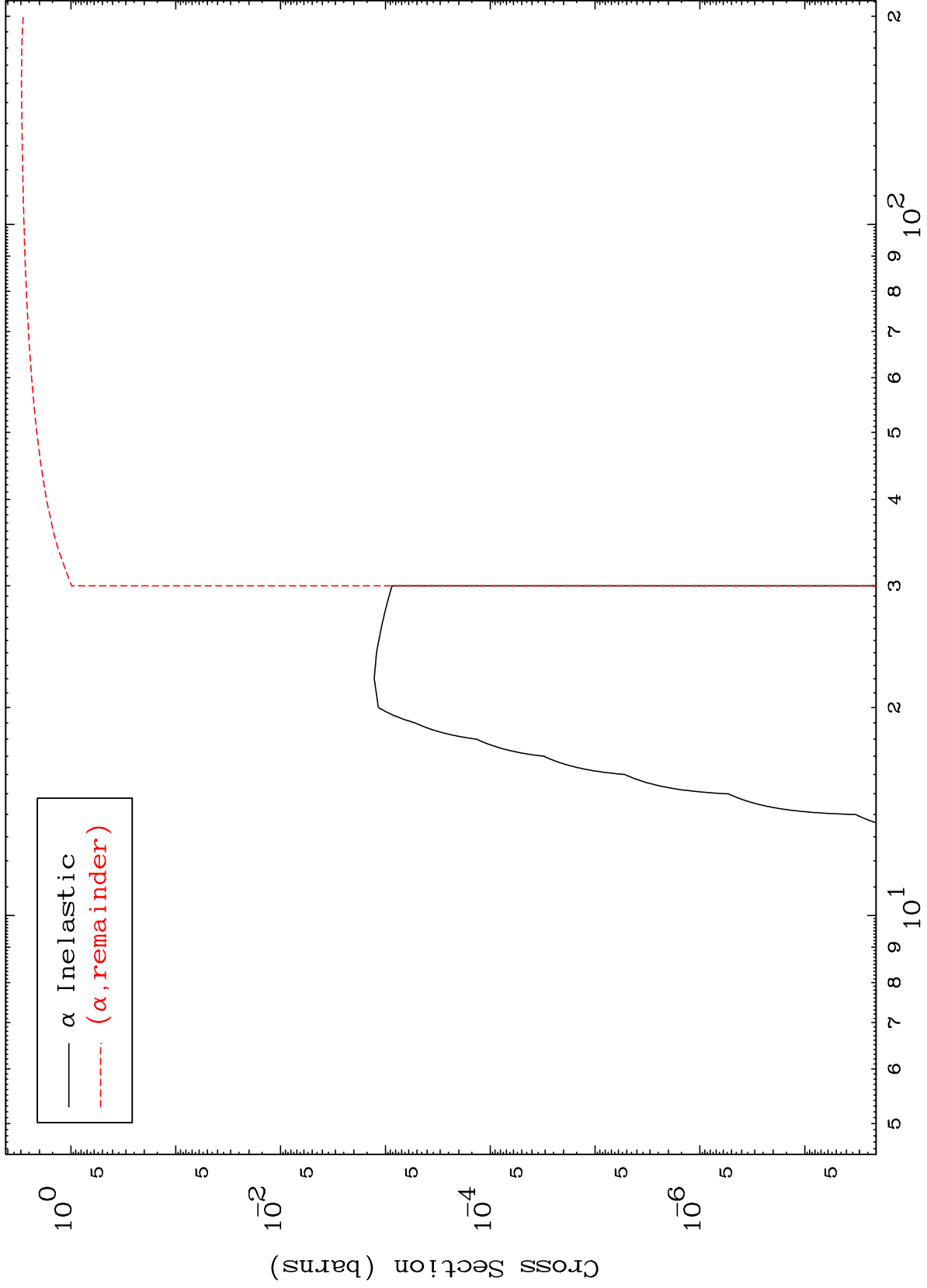
Press Mouse Button to Start

MAT 9037

$\alpha$  Major

90-Th-231

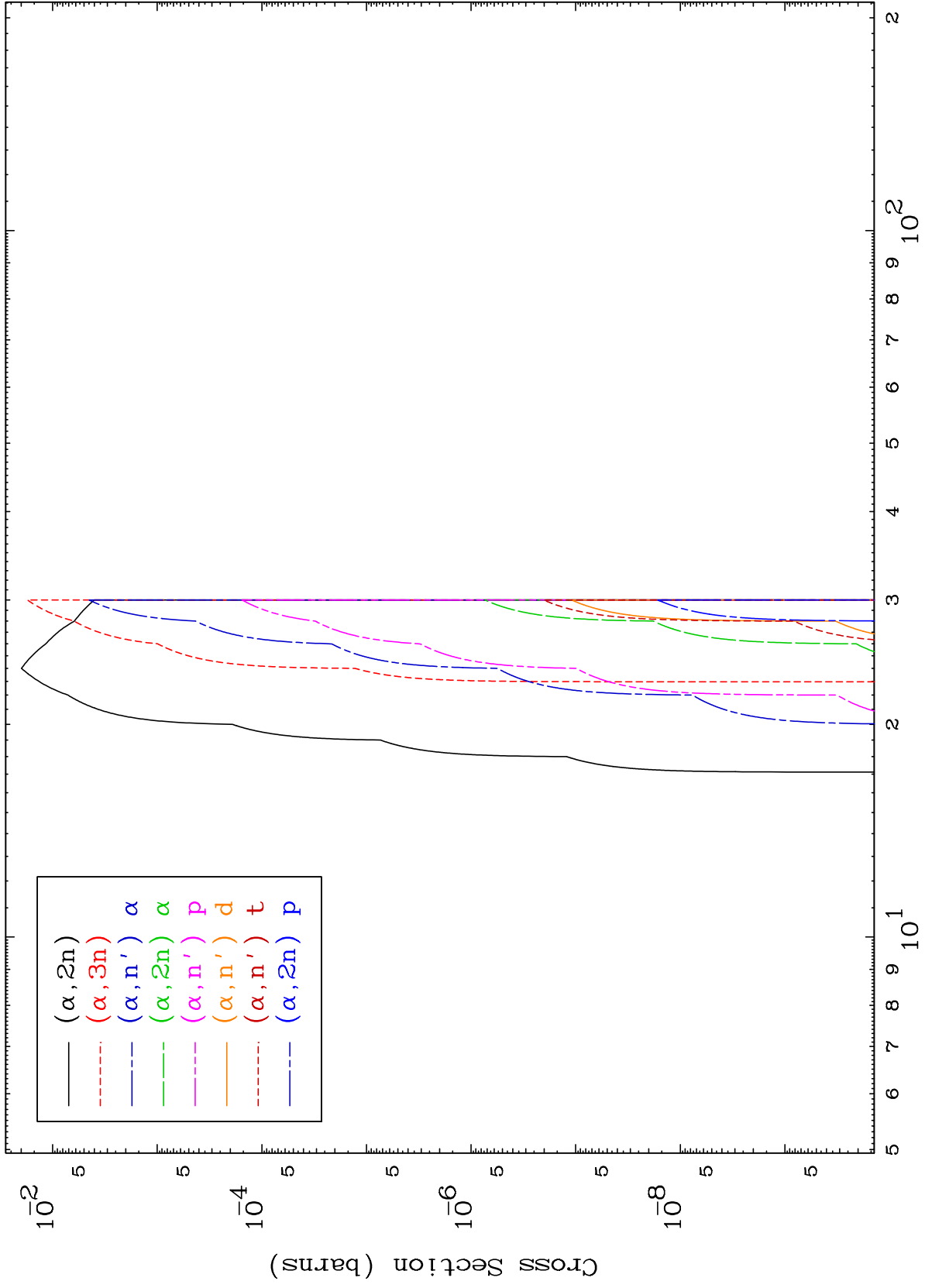
0 Kelvin Cross Sections



MAT 9037

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

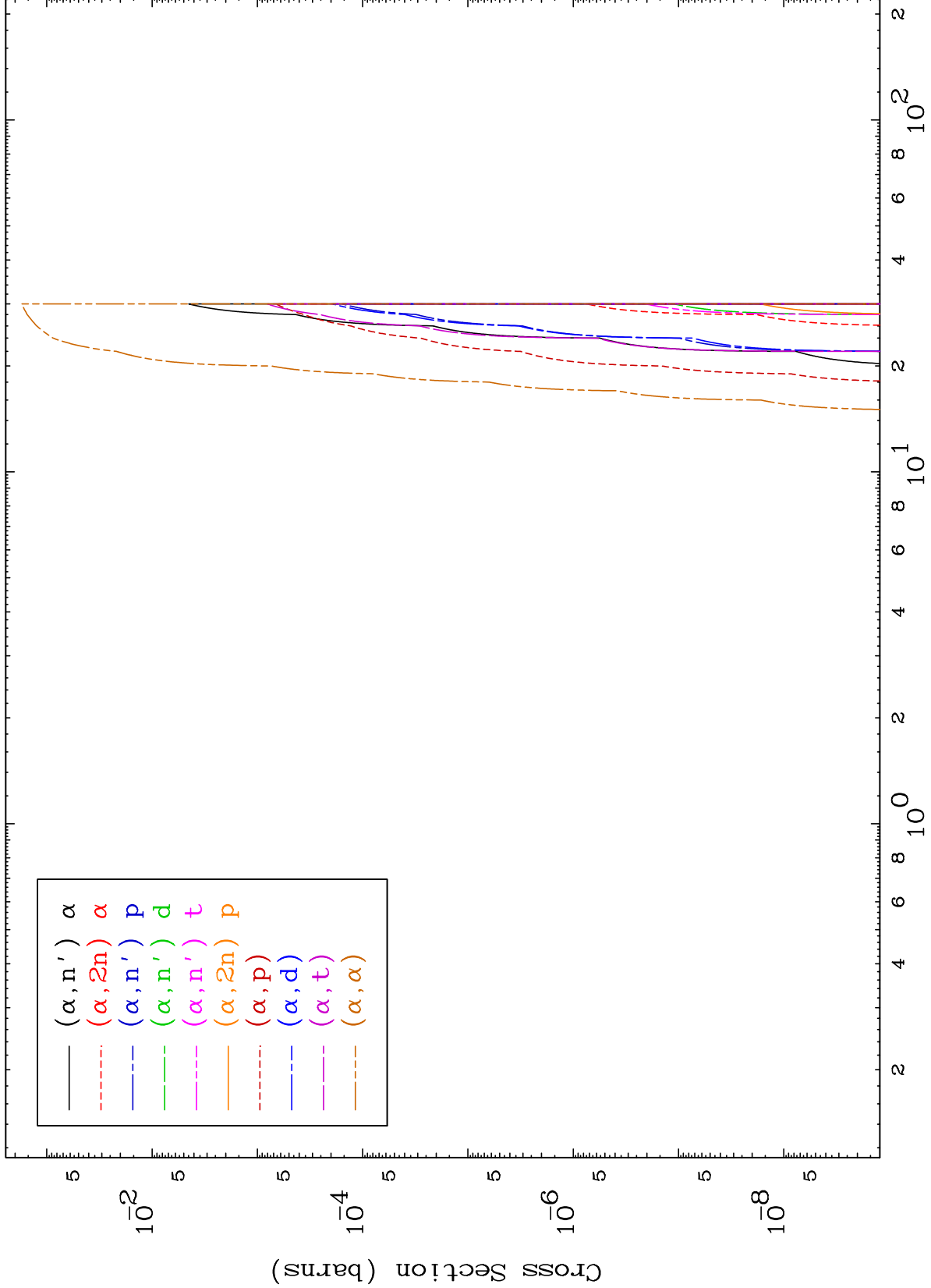
90-Th-231



2

Incident Energy (MeV)

90-Th-231

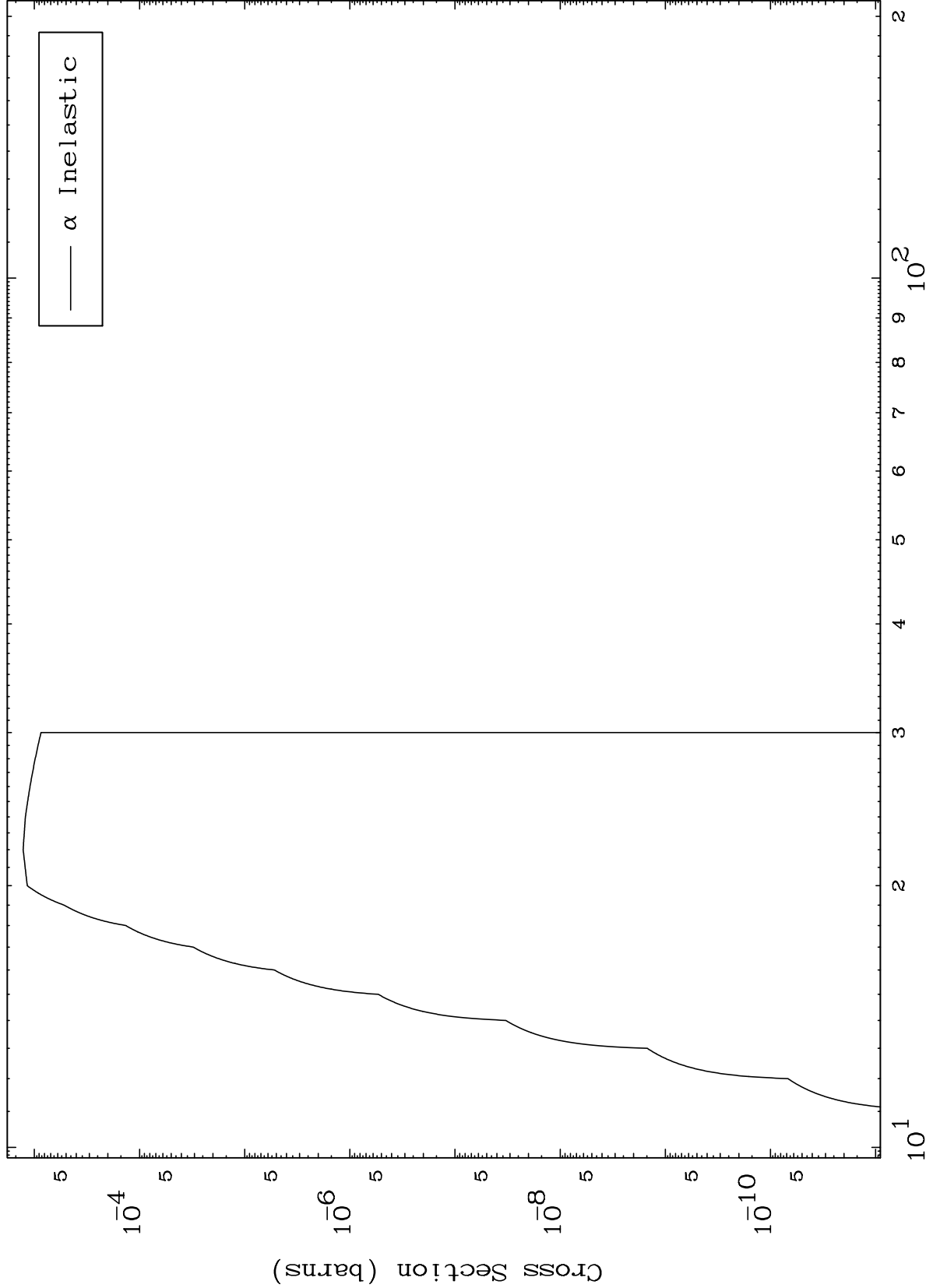


MAT 9037

( $\alpha, n'$ ) Level

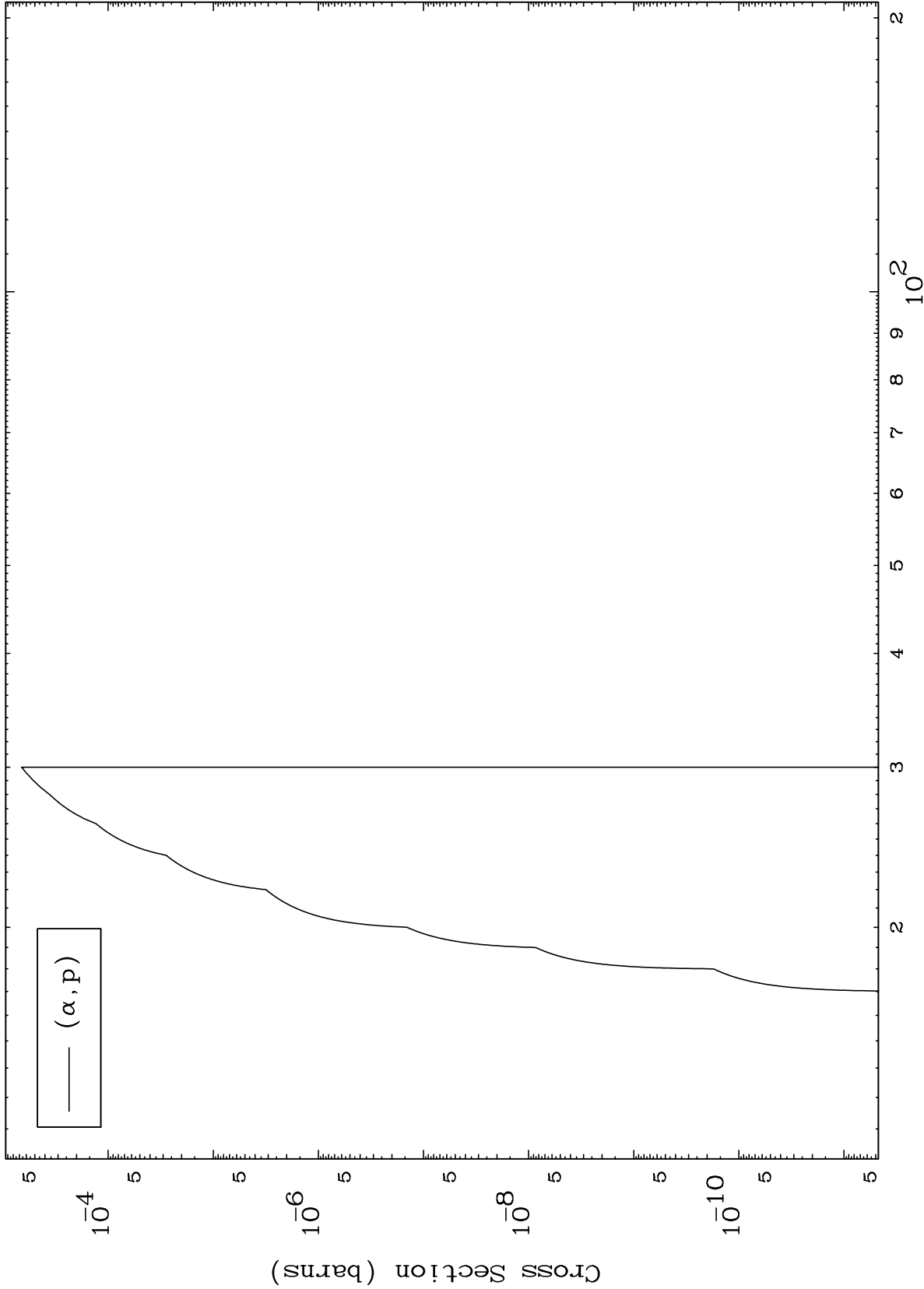
90-Th-231

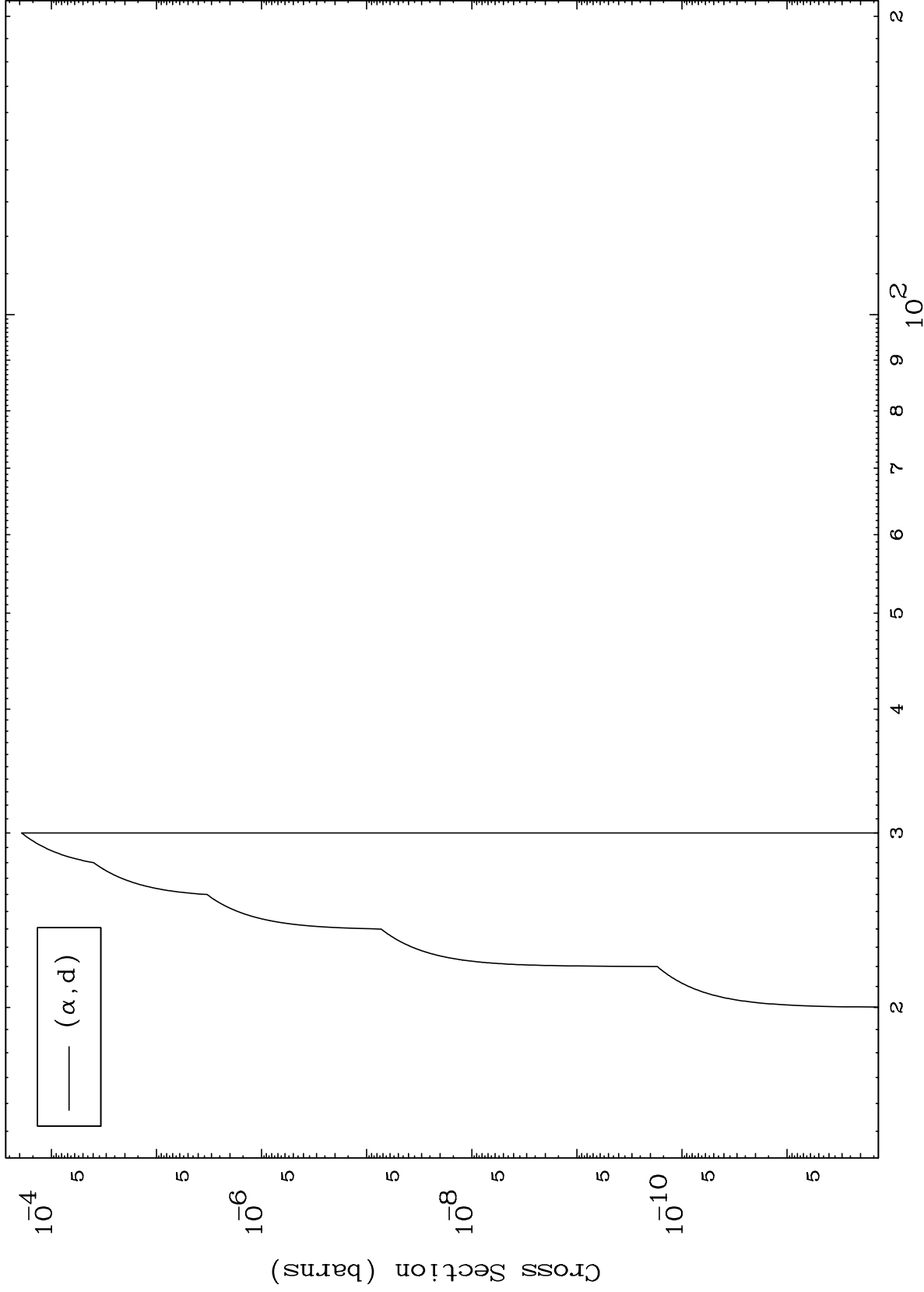
0 Kelvin Cross Sections

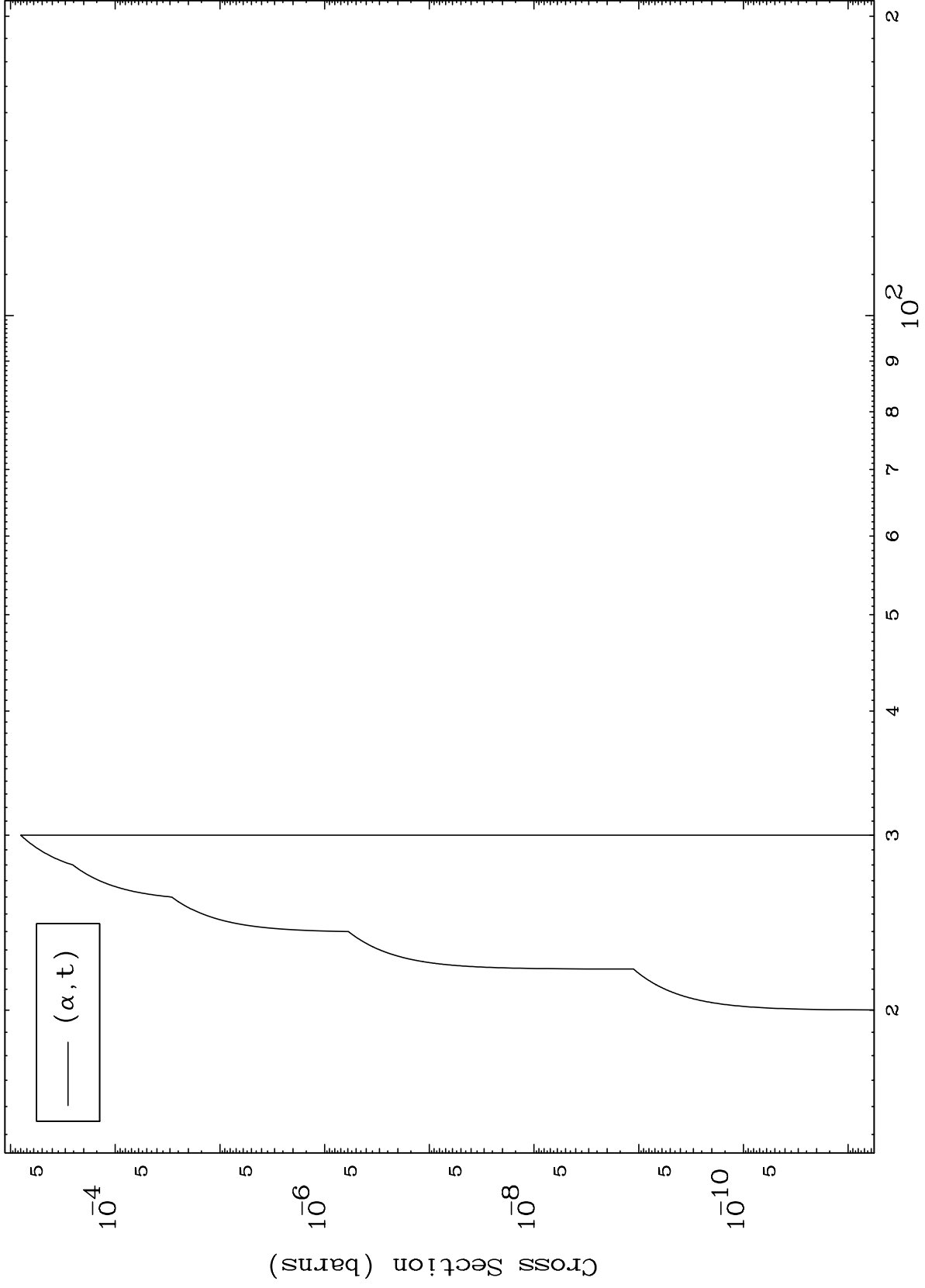


Incident Energy (MeV)

90-Th-231







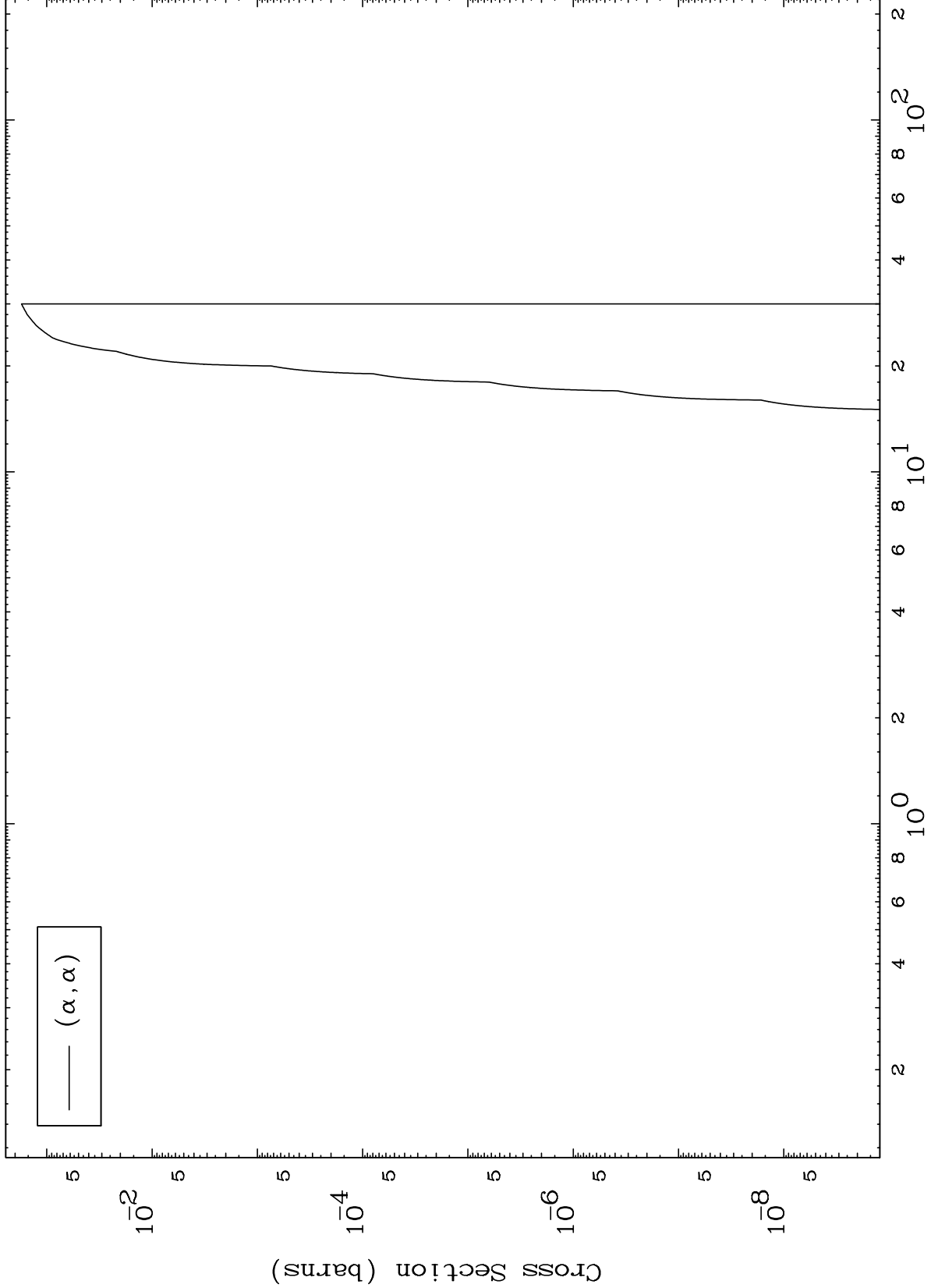


MAT 9037

( $\alpha, \alpha$ ) Levels

90-Th-231

0 Kelvin Cross Sections

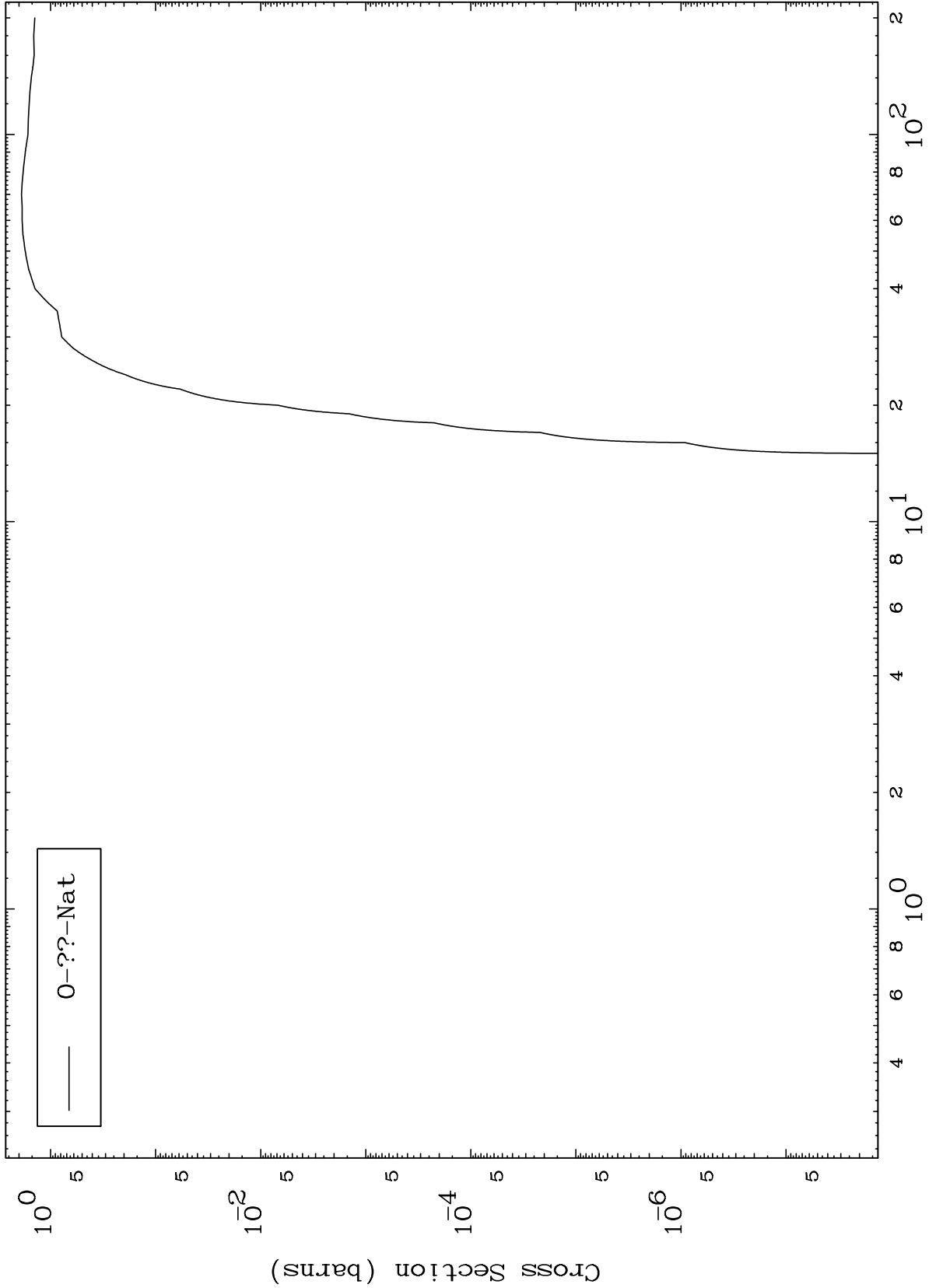


( $\alpha, \alpha$ )

MAT 9037

90-Th-231

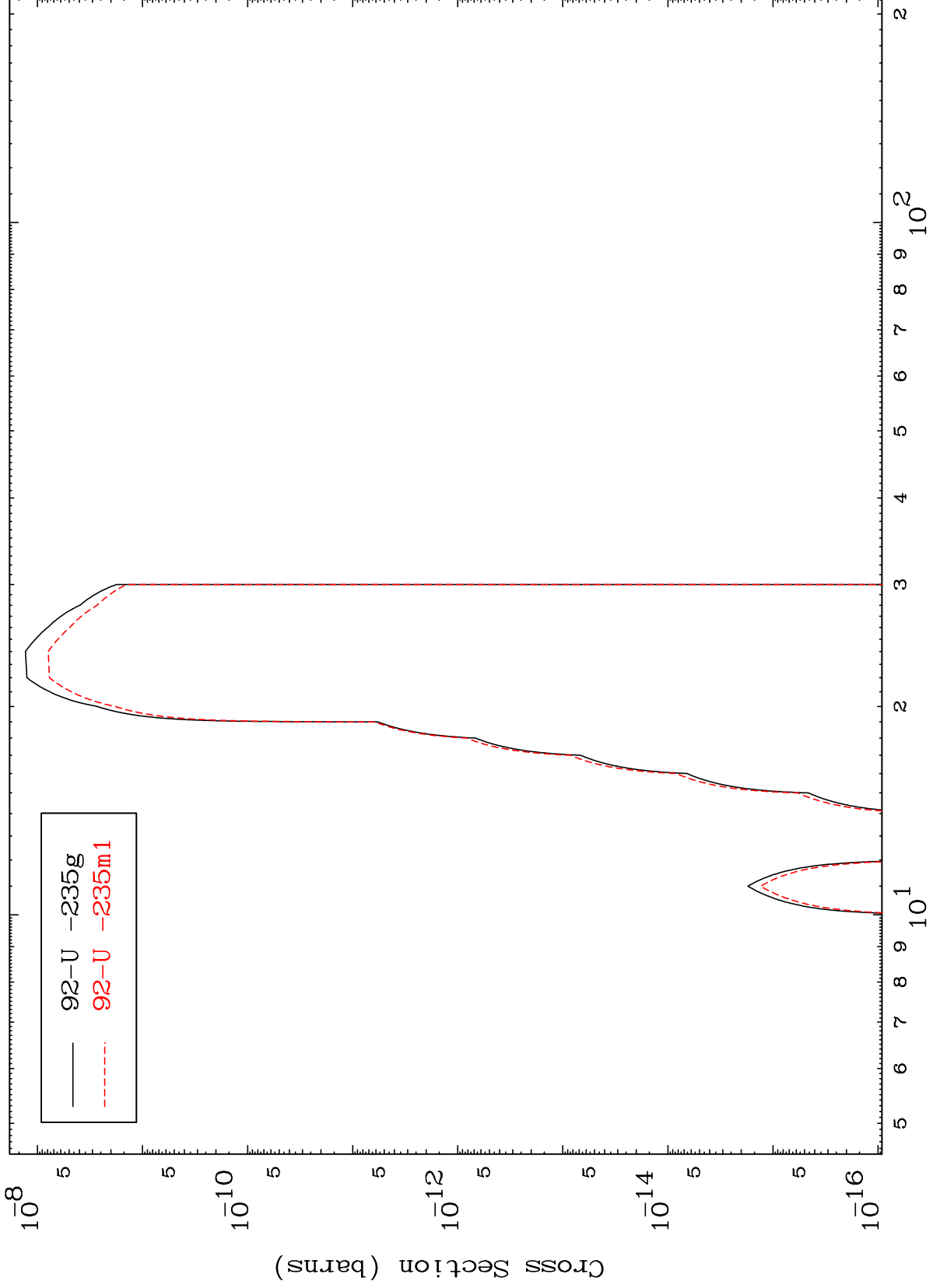
$\alpha$  Fission  
Radionuclide Production Cross Section



MAT 9037

90-Th-231

Radionuclide Production Cross Section



10

Incident Energy (MeV)

90-Th-231

MAT 9037

( $\alpha, p$ )

90-Th-231

Radionuclide Production Cross Section

