

Program EVALPLOT  
(Version 2018-1)

by

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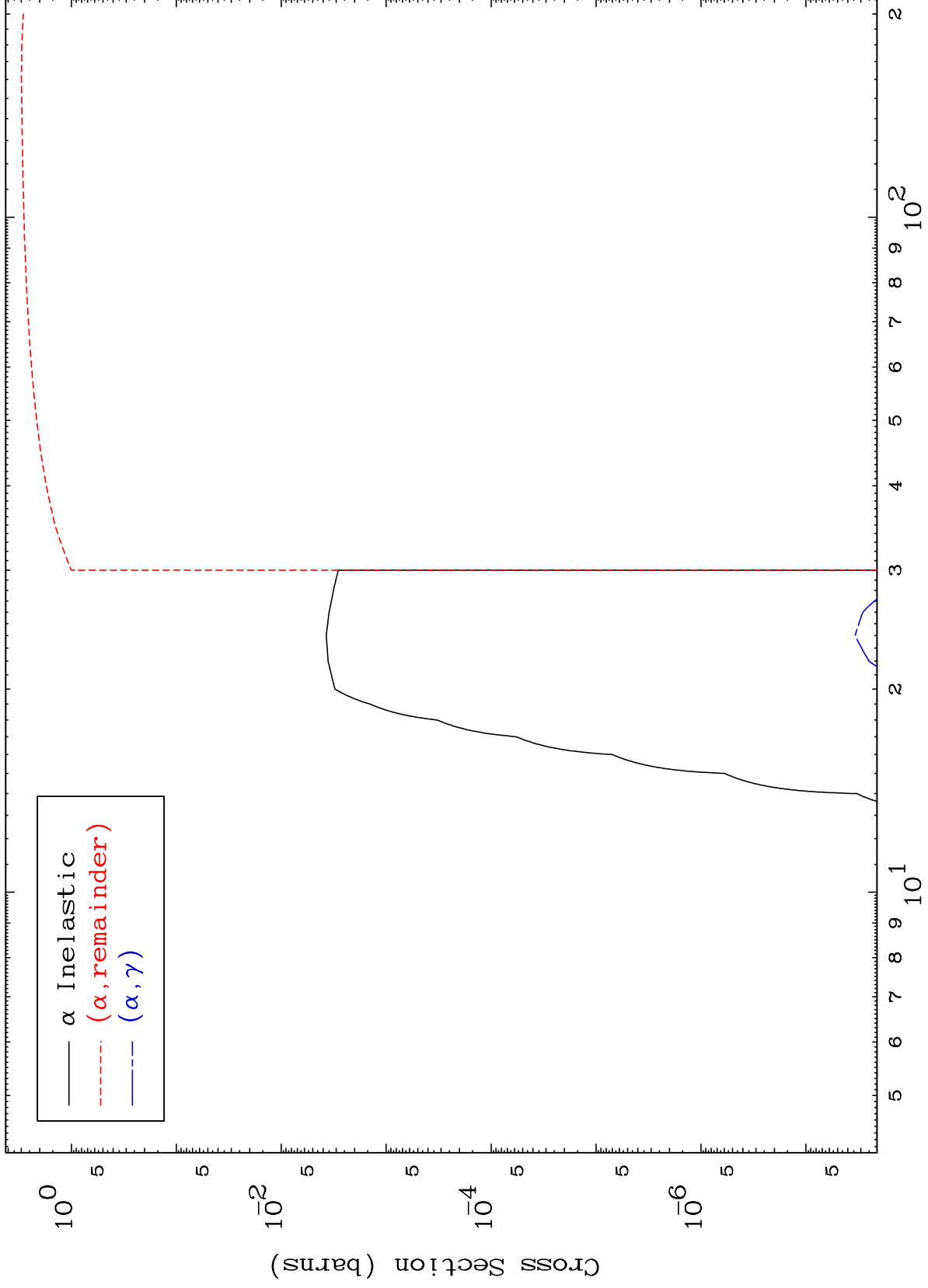
Web: [redcullen1.net/HOMEPAGE.NEW](http://redcullen1.net/HOMEPAGE.NEW)

Press Mouse Button to Start

MAT 9046

0 Kelvin Major  $\alpha$  Cross Sections

90-Th-234

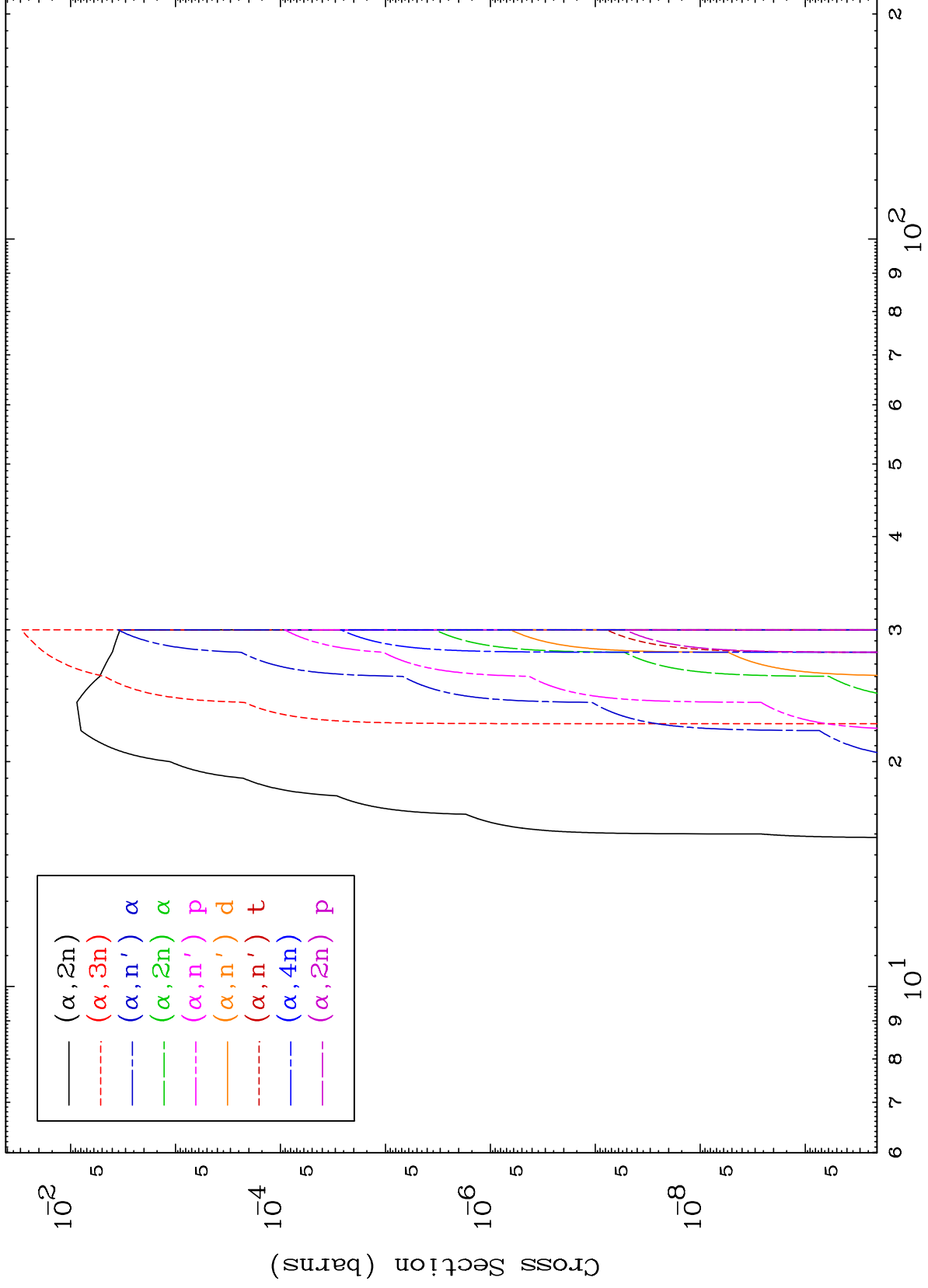


$\alpha$  Inelastic  
 $(\alpha, \text{remainder})$   
 $(\alpha, \gamma)$

MAT 9046

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

90-Th-234



2

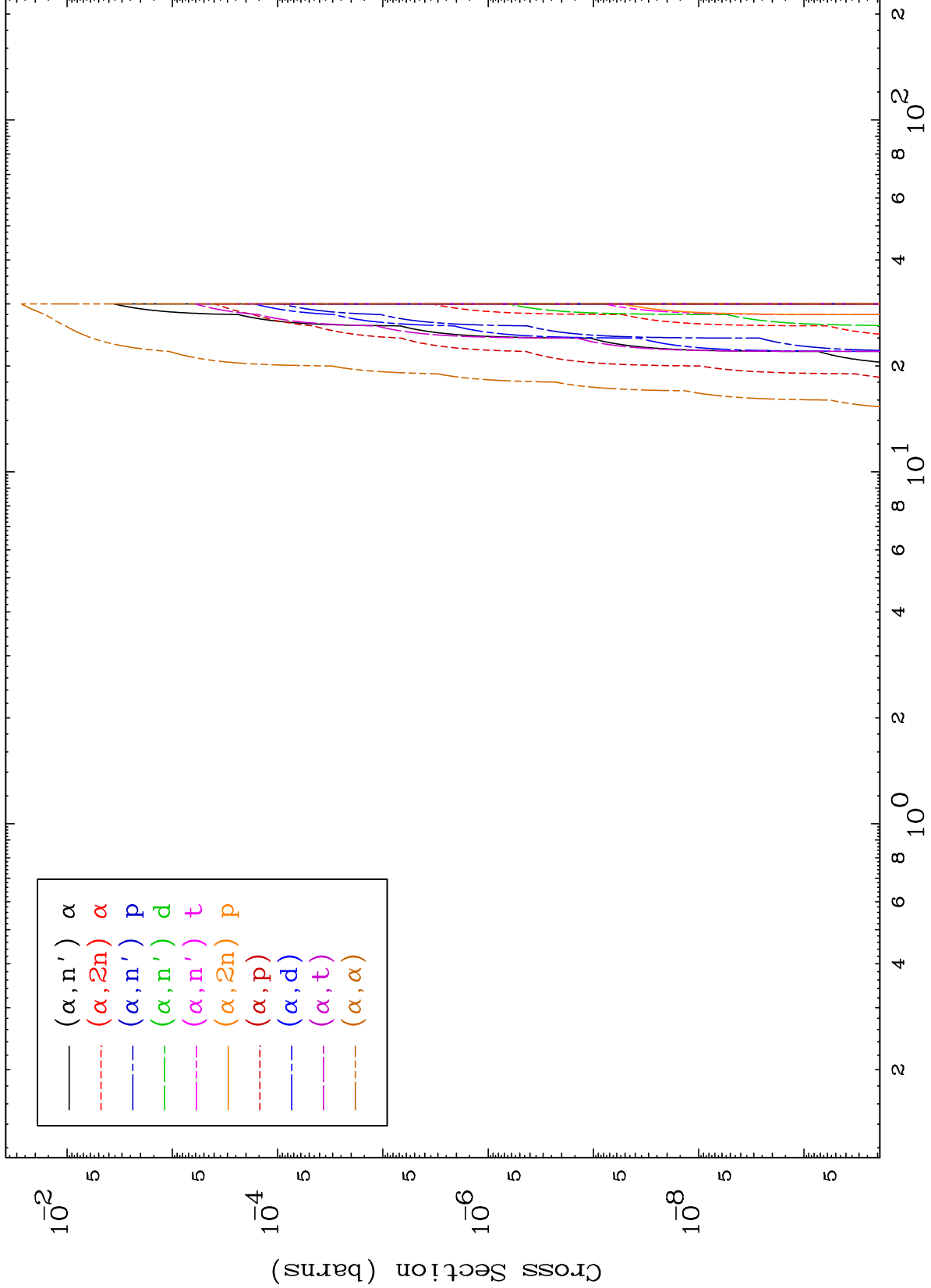
Incident Energy (MeV)

90-Th-234

MAT 9046

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

90-Th-234

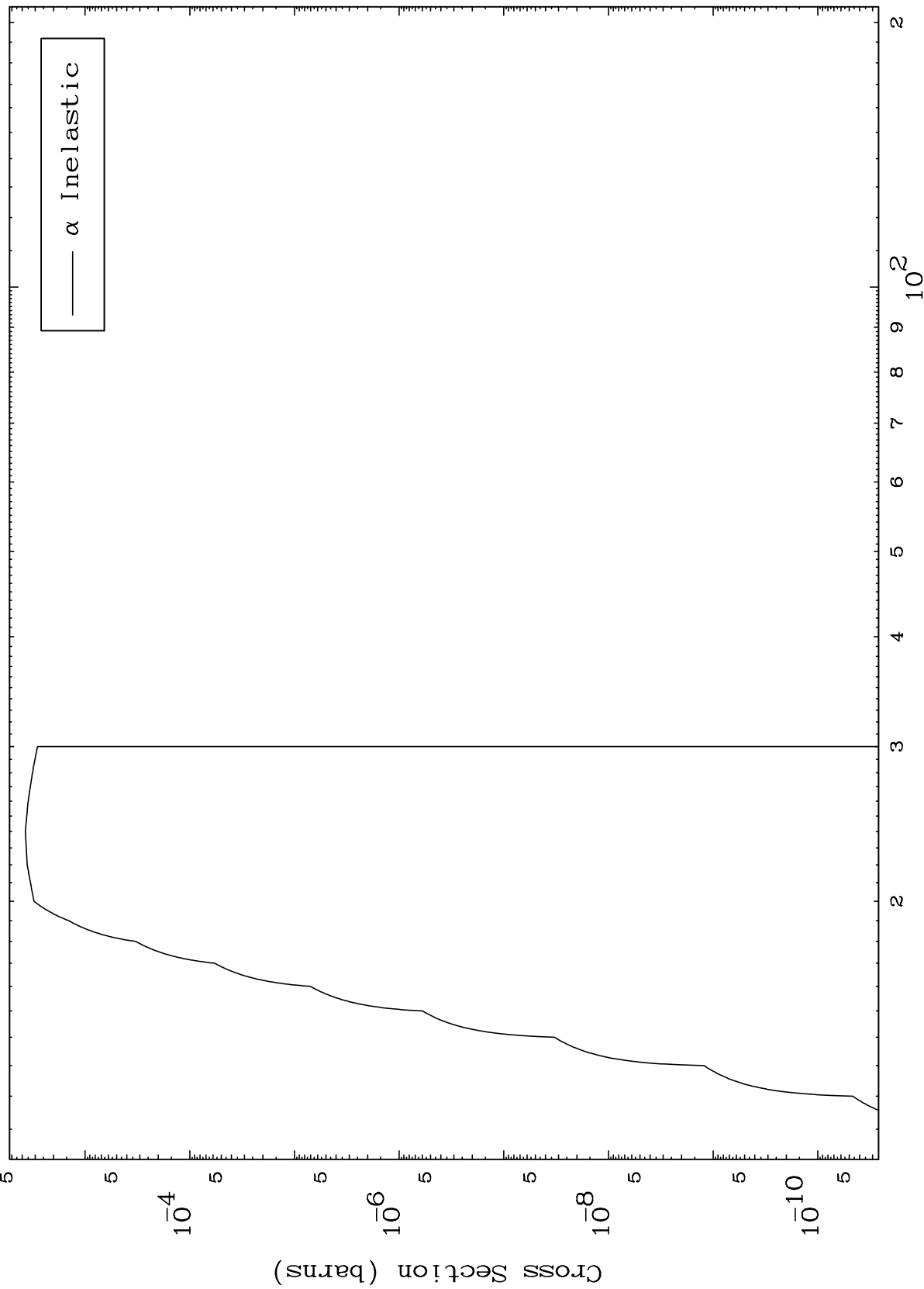


MAT 9046

( $\alpha, n'$ ) Level

90-Th-234

0 Kelvin Cross Sections



Incident Energy (MeV)

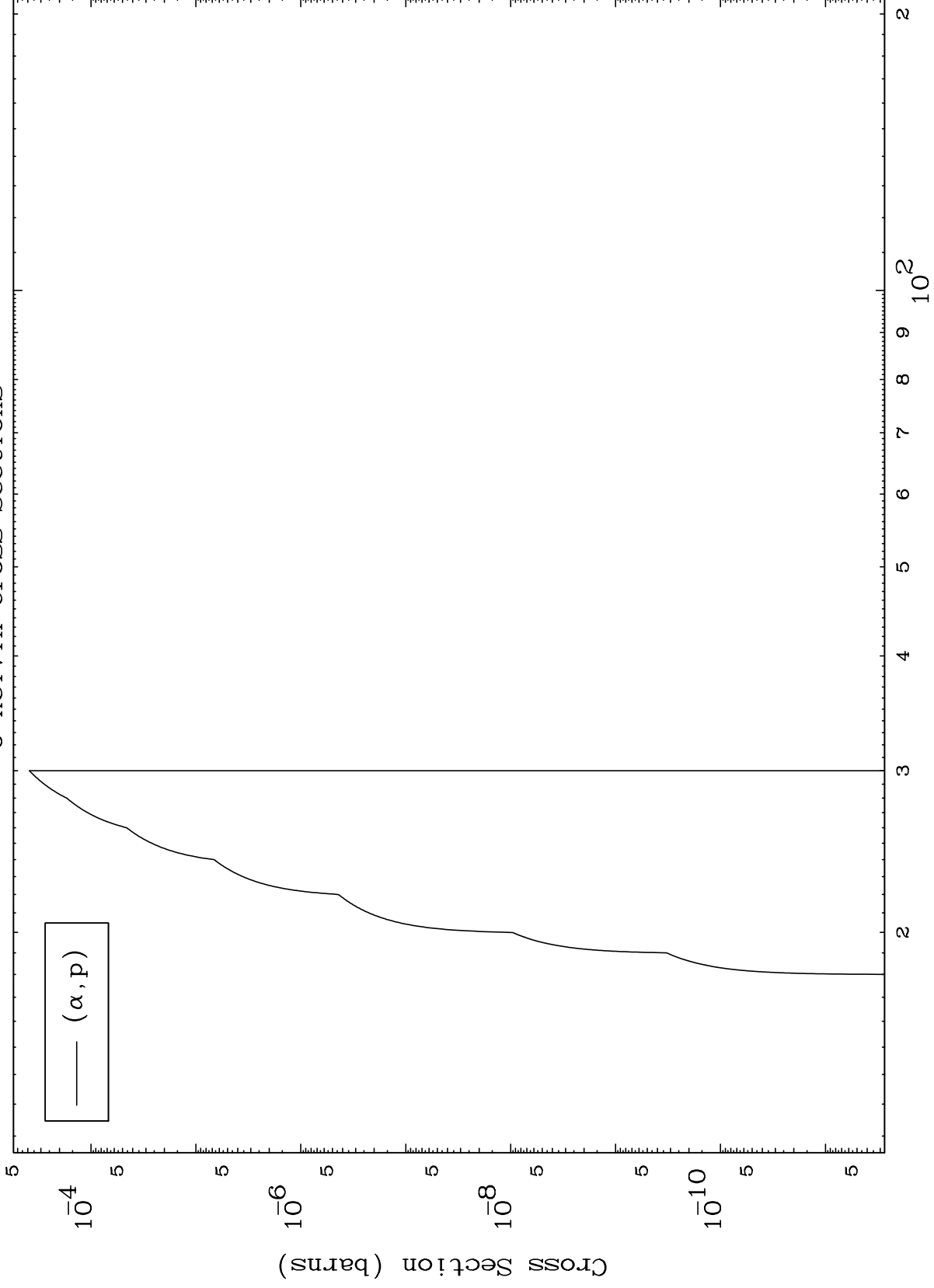
90-Th-234

4

MAT 9046

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

90-Th-234



5

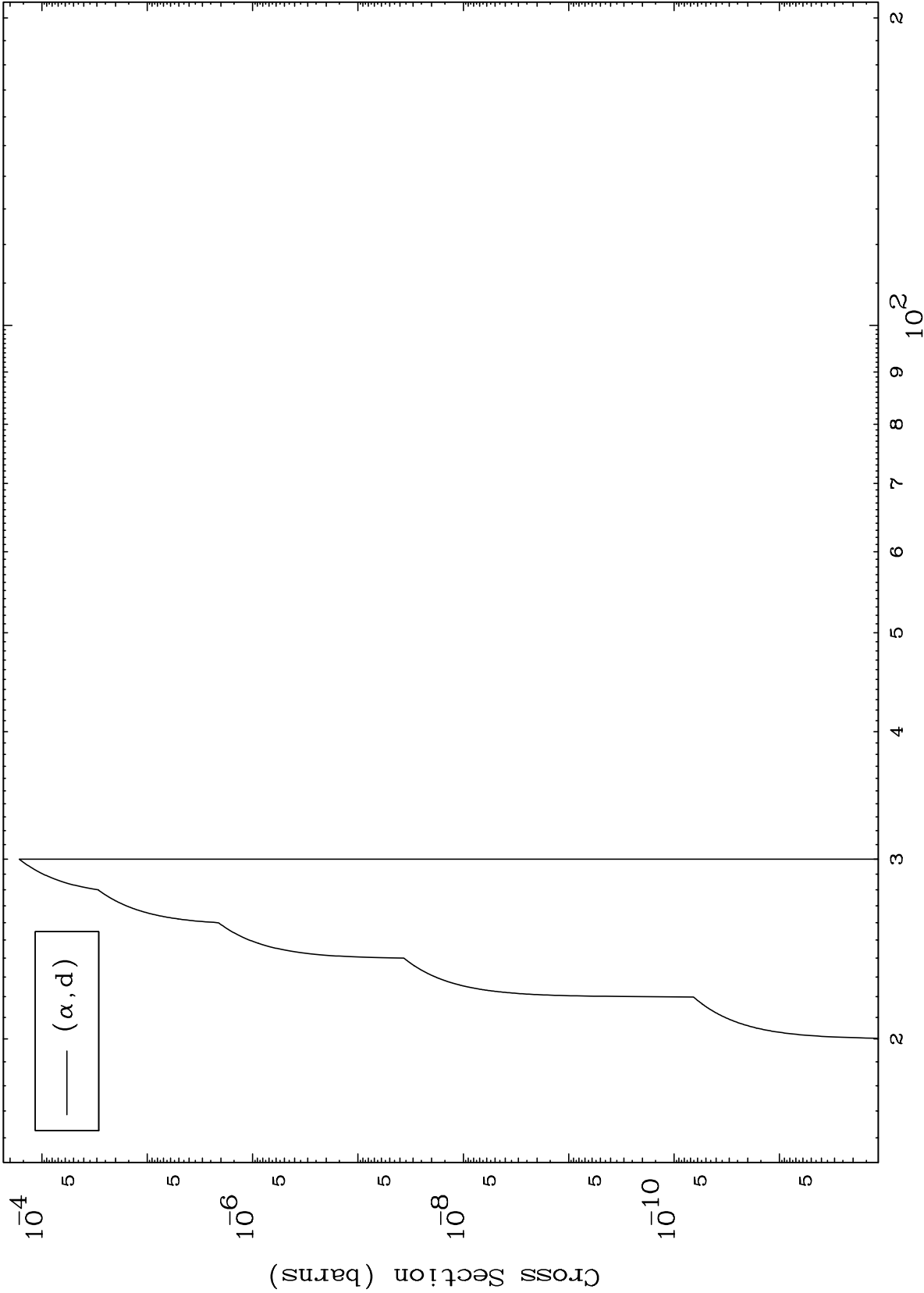
Incident Energy (MeV)

90-Th-234

MAT 9046

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

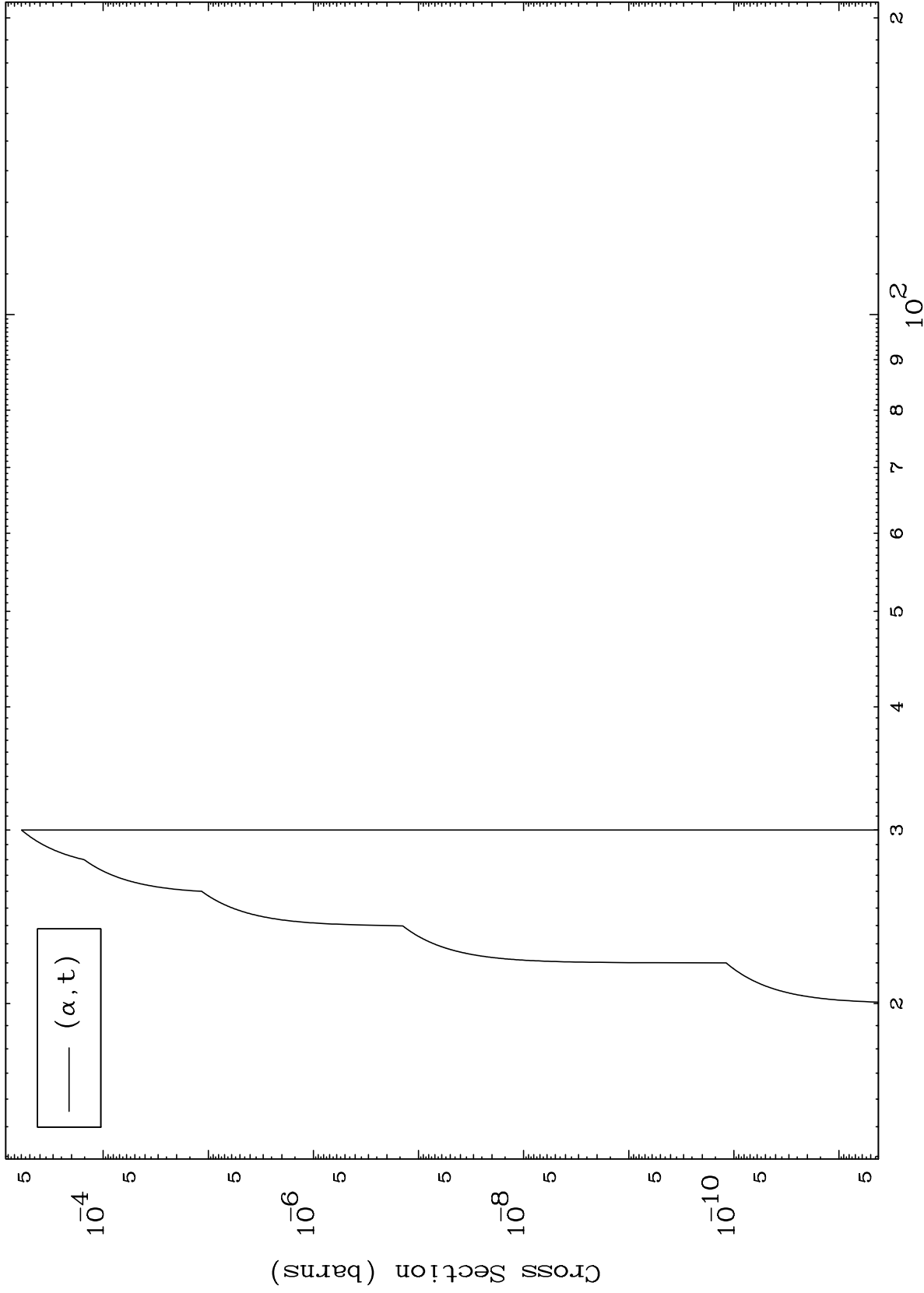
90-Th-234



6

Incident Energy (MeV)

90-Th-234

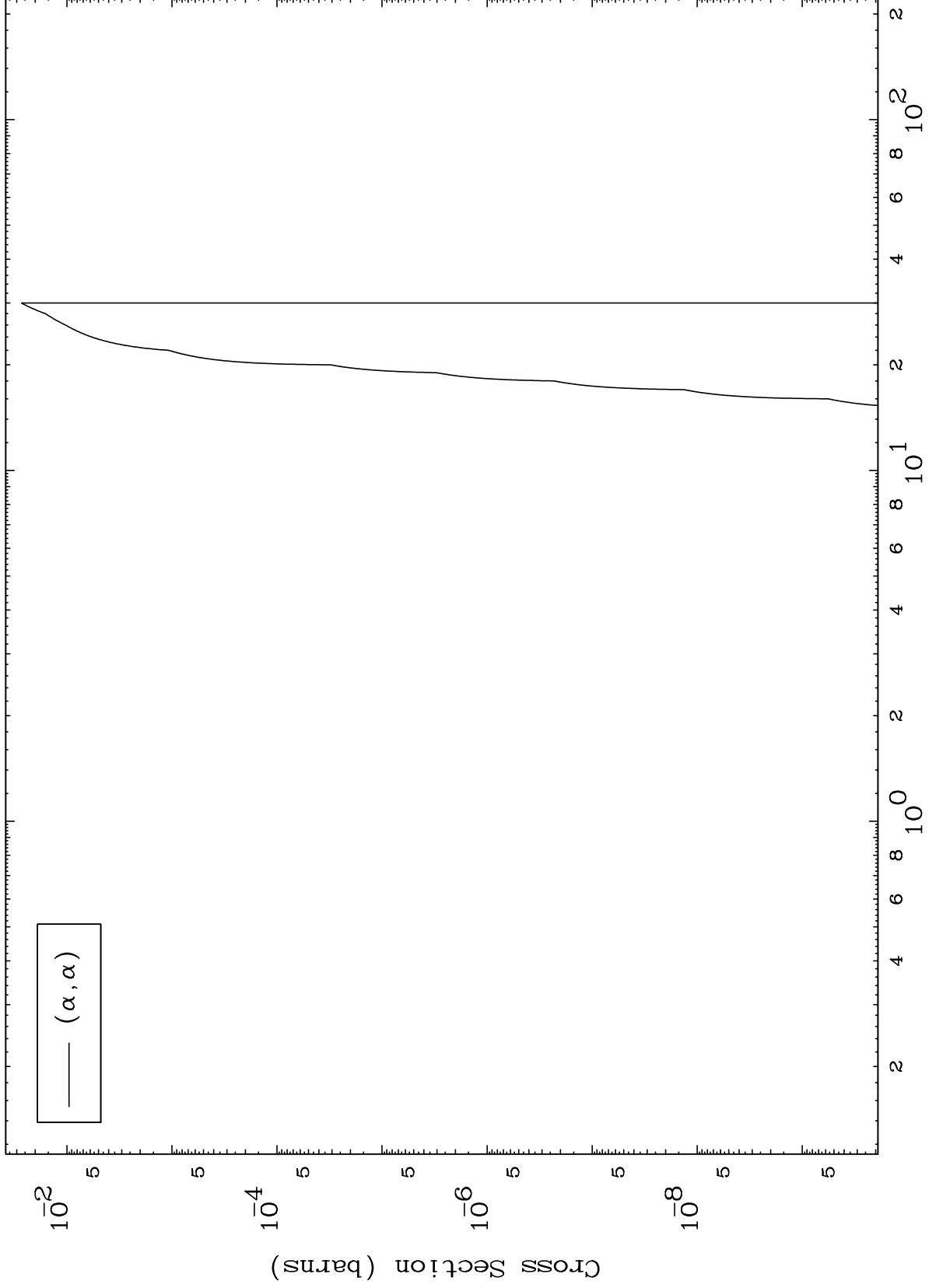




MAT 9046

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

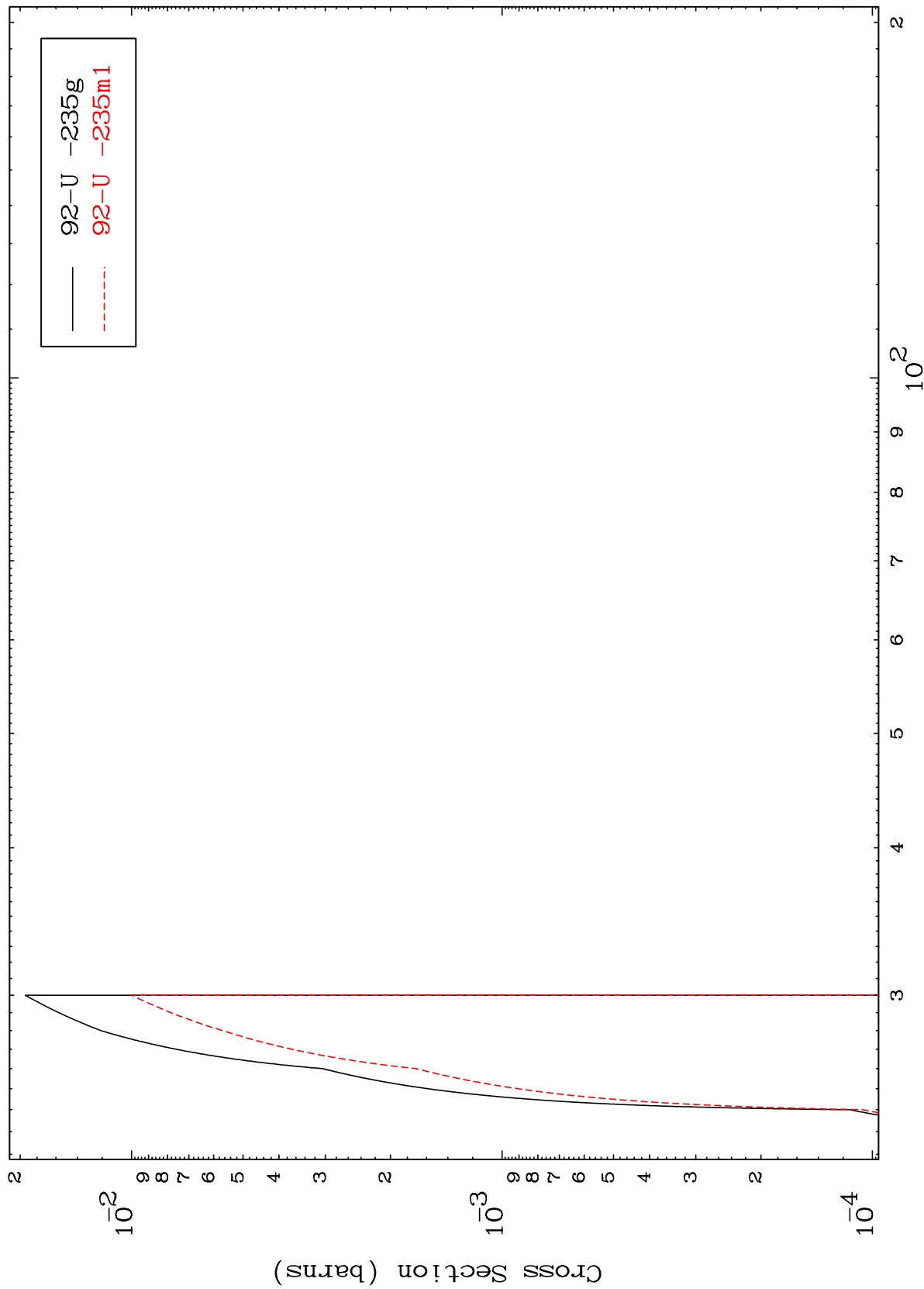
90-Th-234



MAT 9046

90-Th-234

( $\alpha, 3n$ )  
Radionuclide Production Cross Section



92-U -235g  
92-U -235m1

9

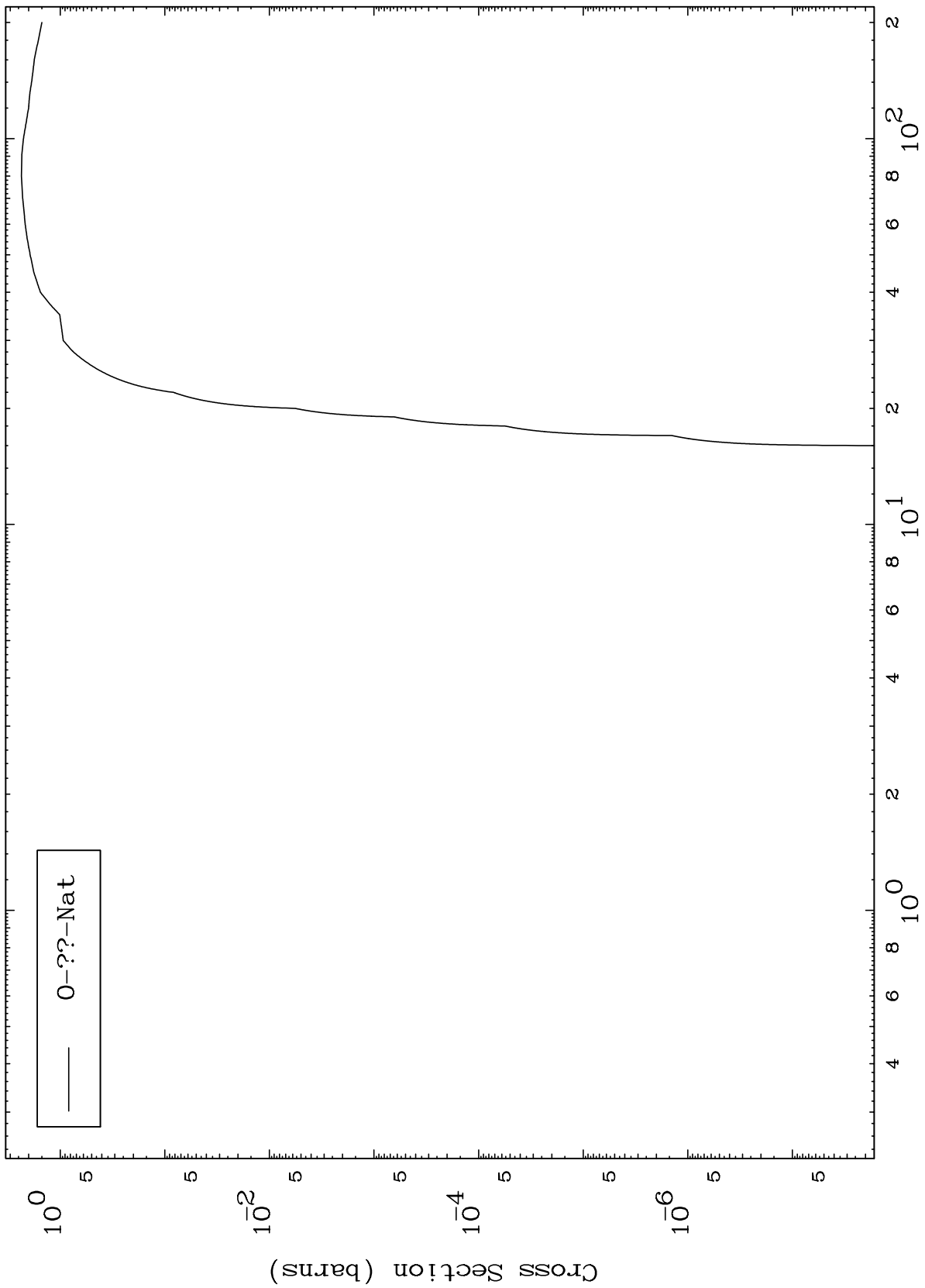
Incident Energy (MeV)

90-Th-234

MAT 9046

90-Th-234

$\alpha$  Fission  
Radionuclide Production Cross Section



0-??-Nat

90-Th-234

Incident Energy (MeV)

10

Radionuclide Production Cross Section

