

Program EVALPLOT  
(Version 2018-1)

by

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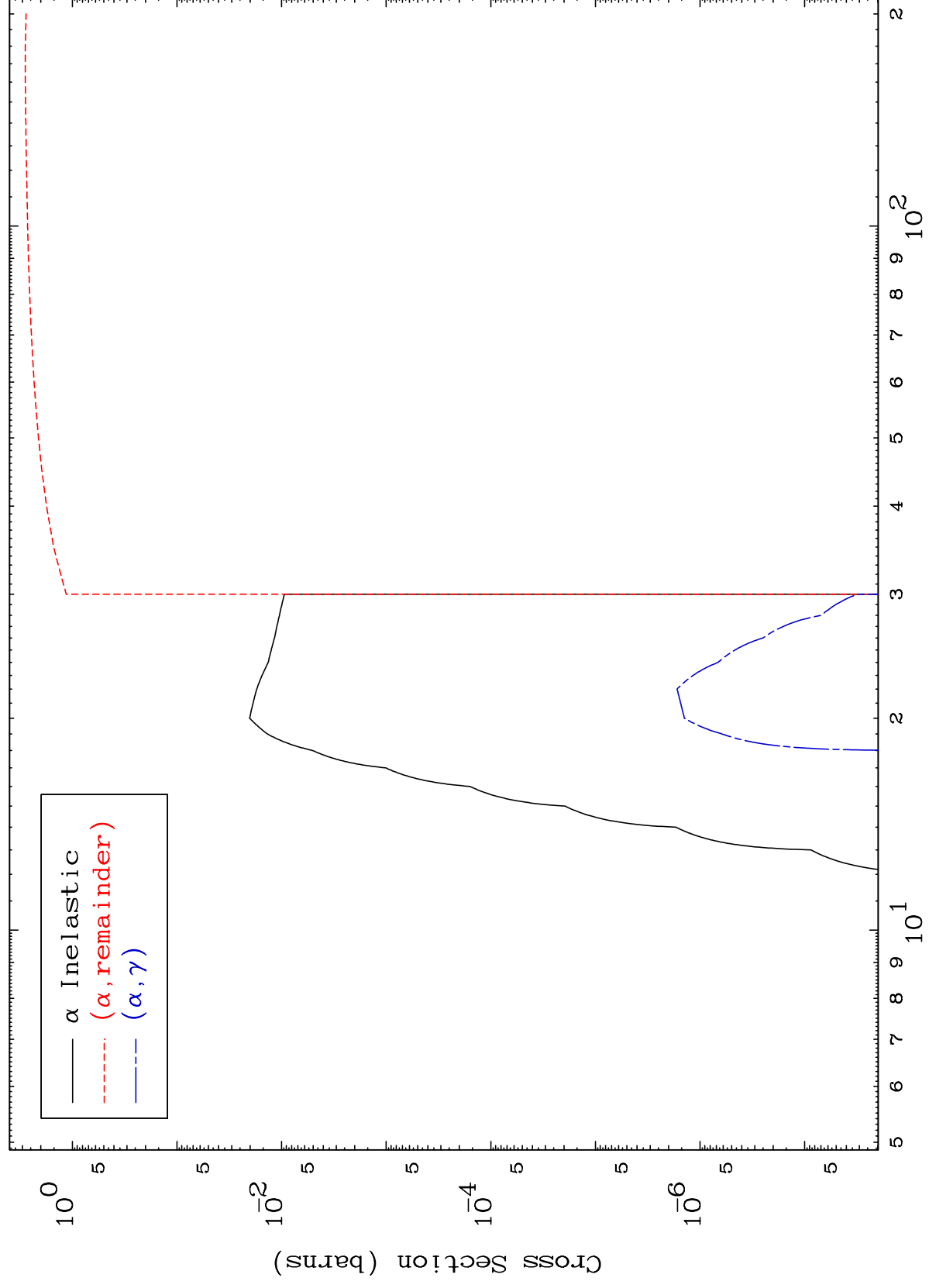
Press Mouse Button to Start

MAT 8134

0 Kelvin

$\alpha$  Major Cross Sections

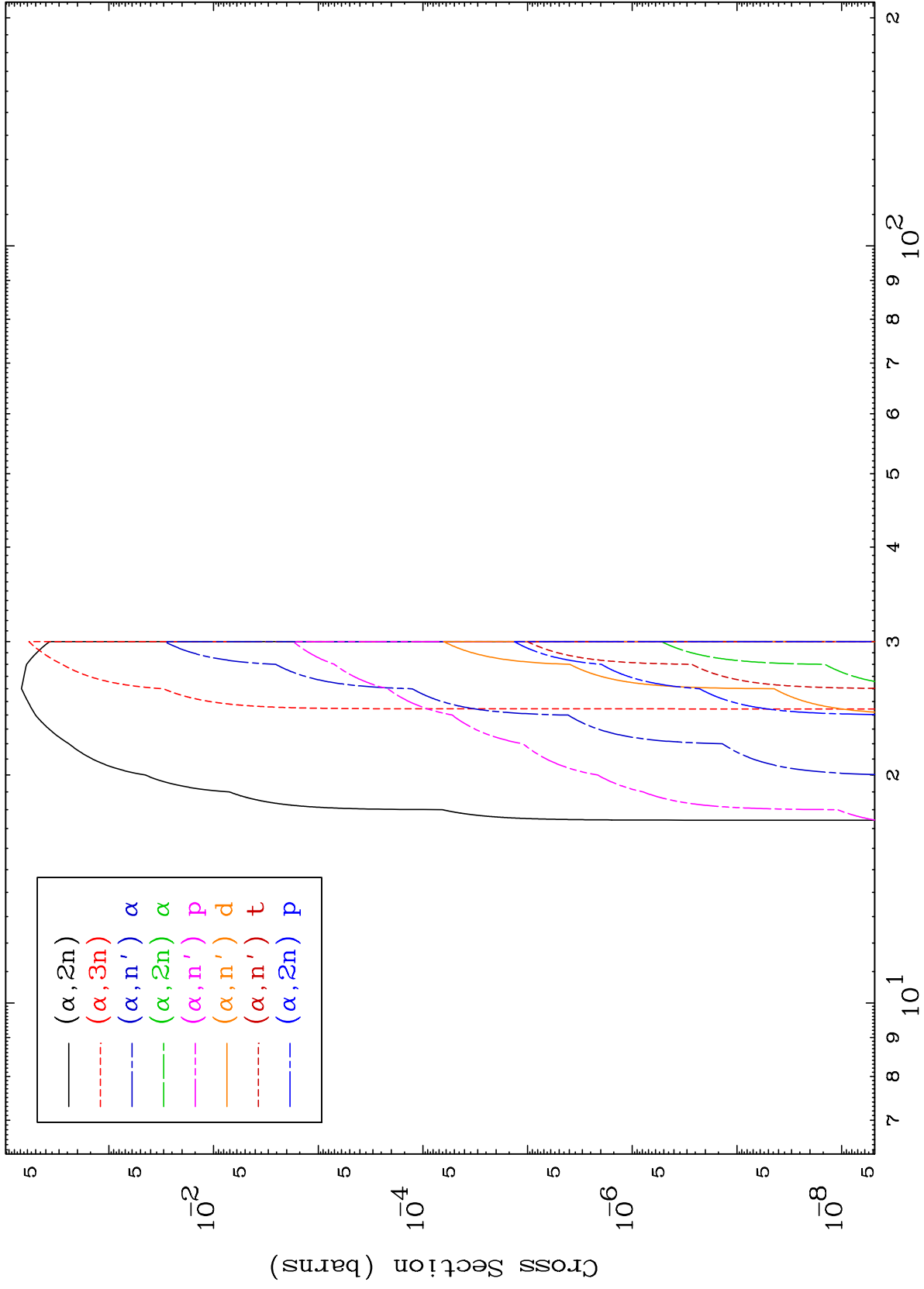
81-Tl-206

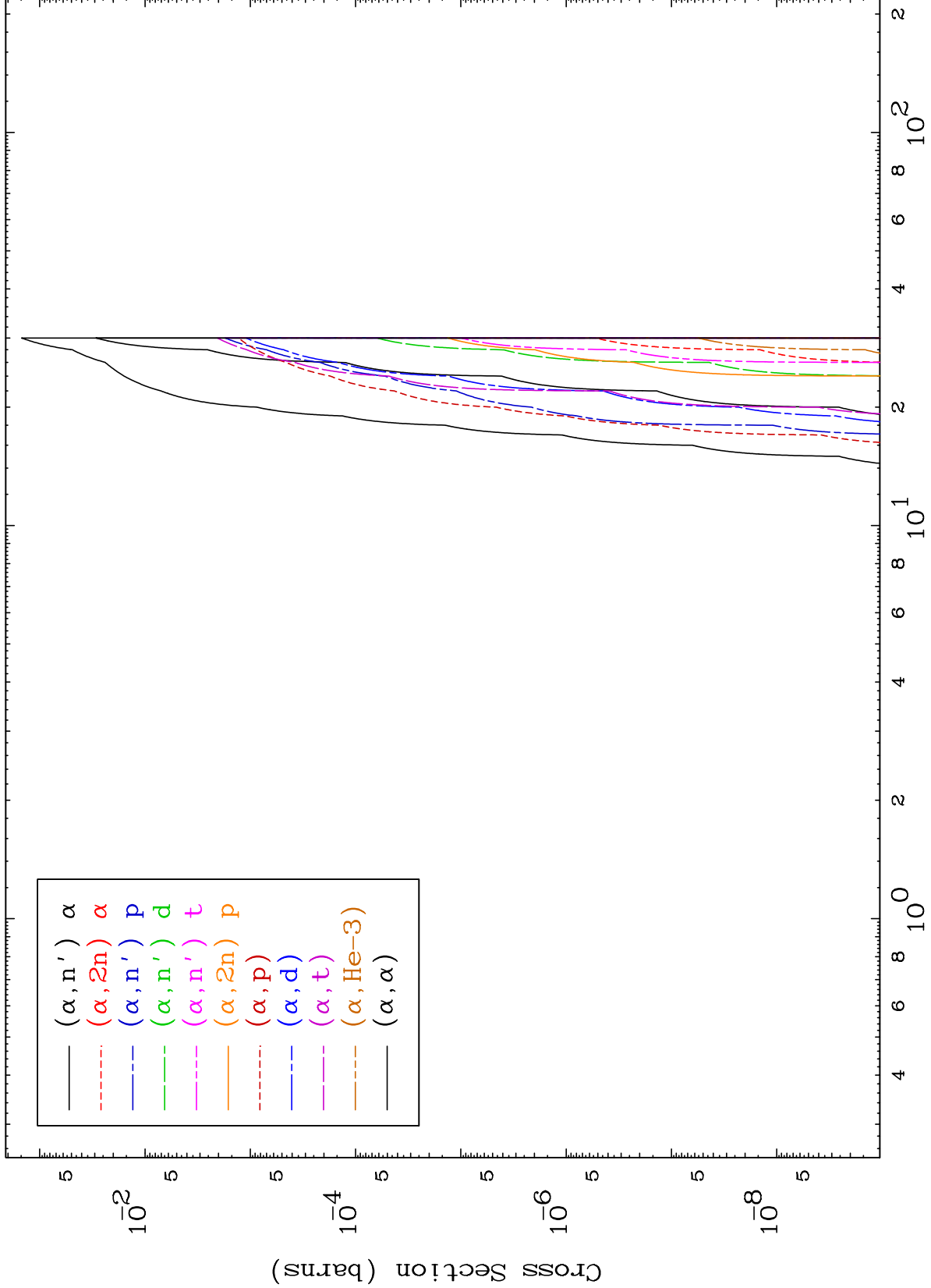


1

Incident Energy (MeV)

81-Tl-206



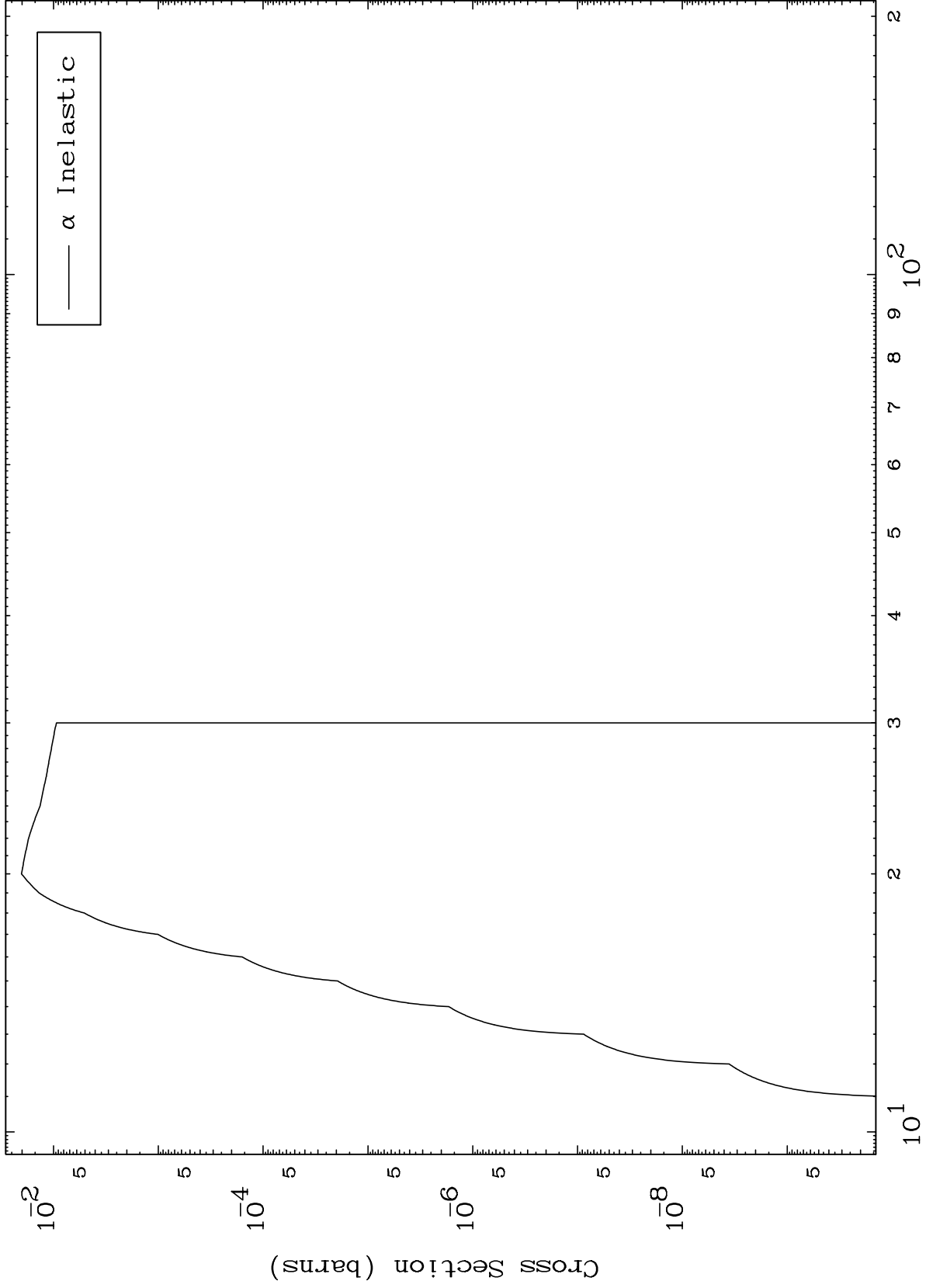


MAT 8134

( $\alpha, n'$ ) Level

81-Tl-206

0 Kelvin Cross Sections



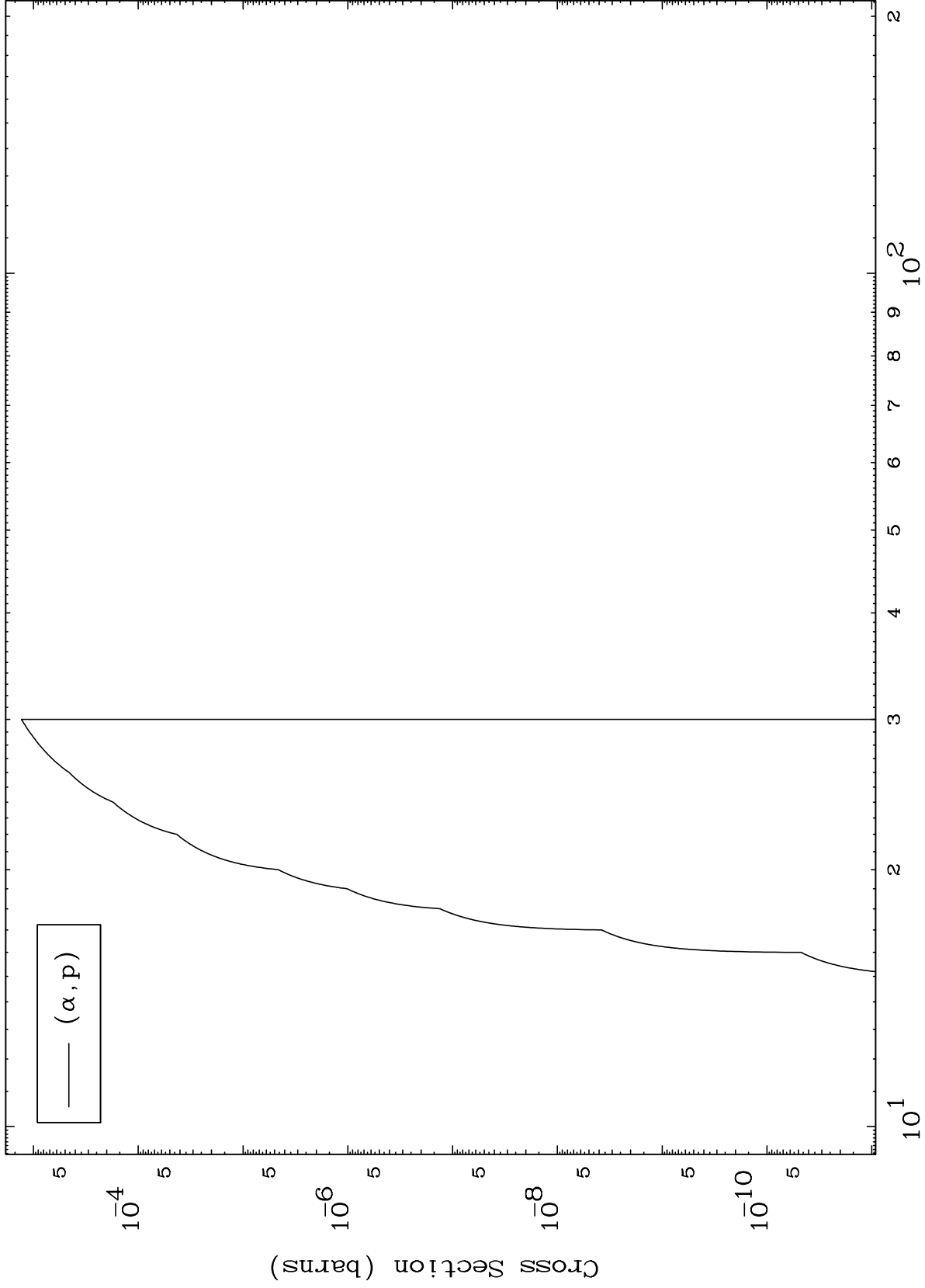
Incident Energy (MeV)

81-Tl-206

MAT 8134

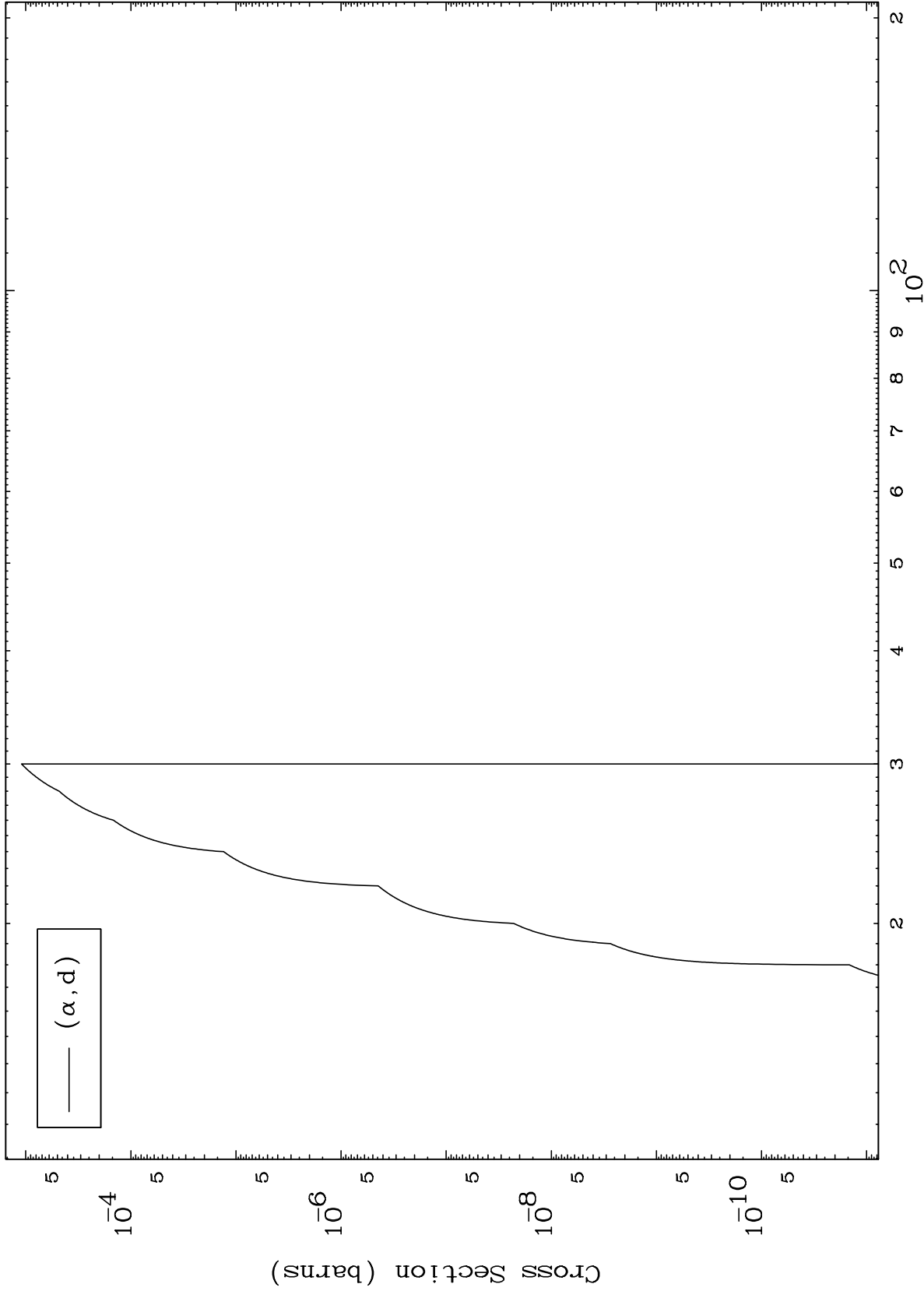
( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

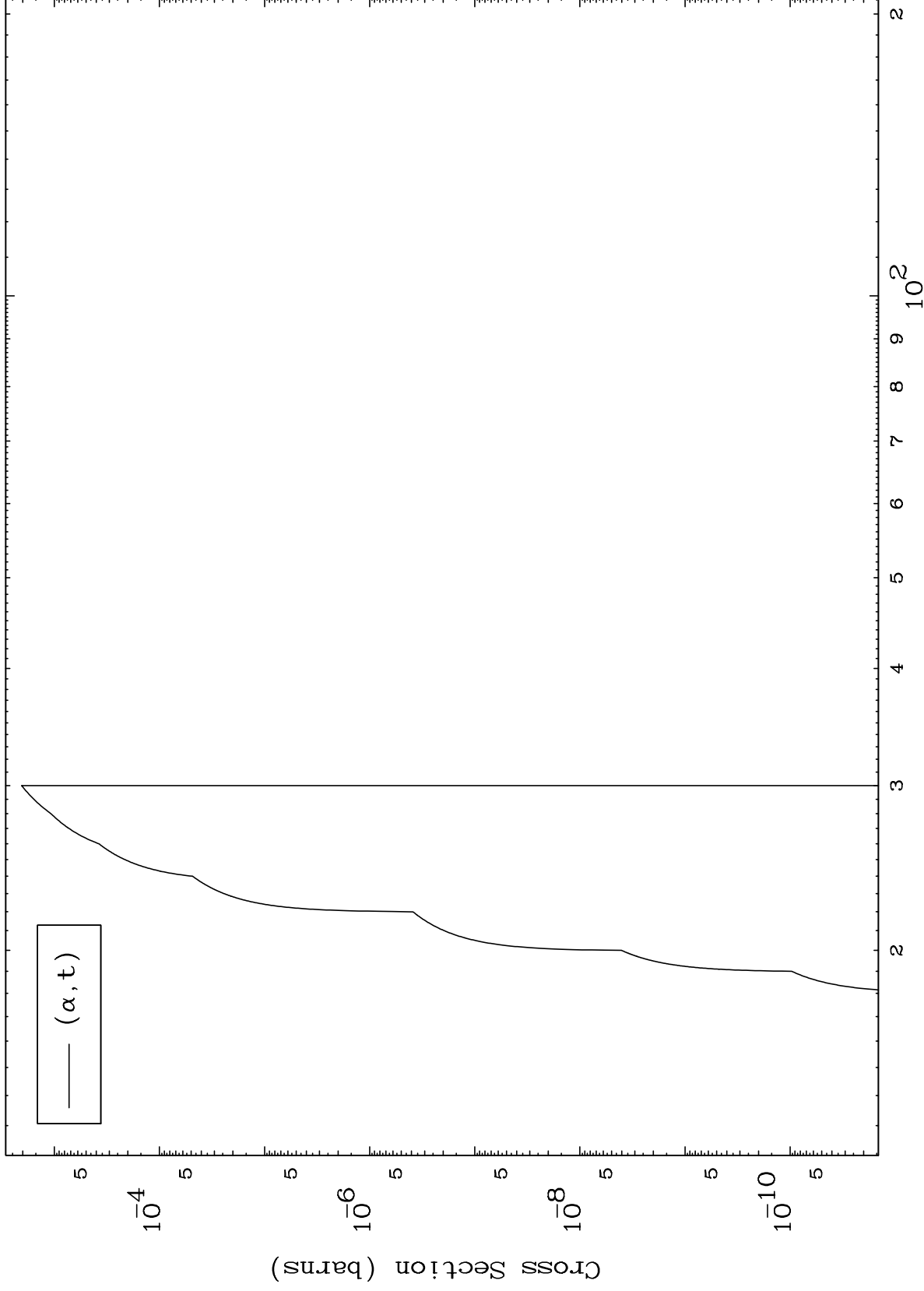
81-T1-206



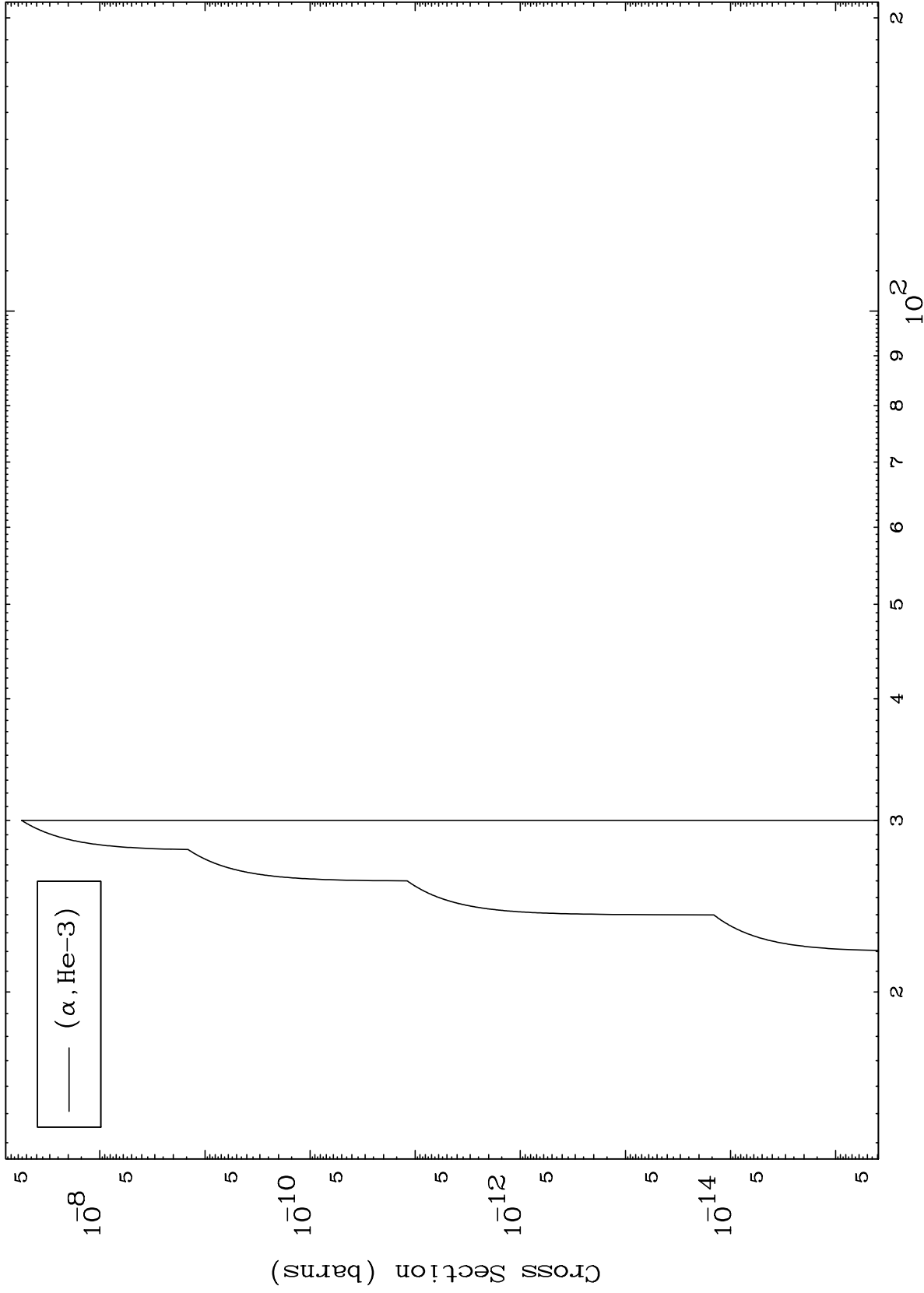
81-T1-206

81-T1-206

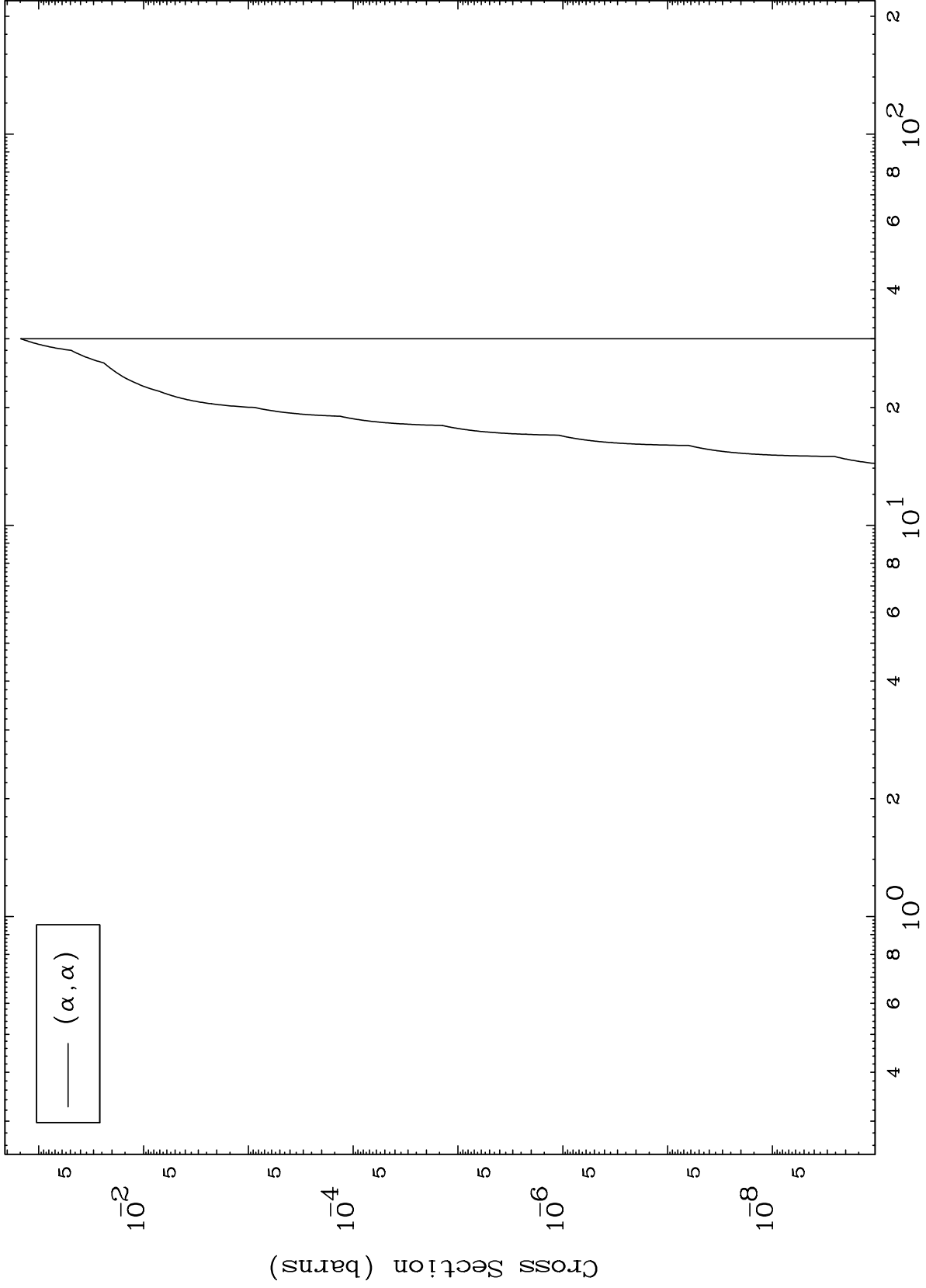








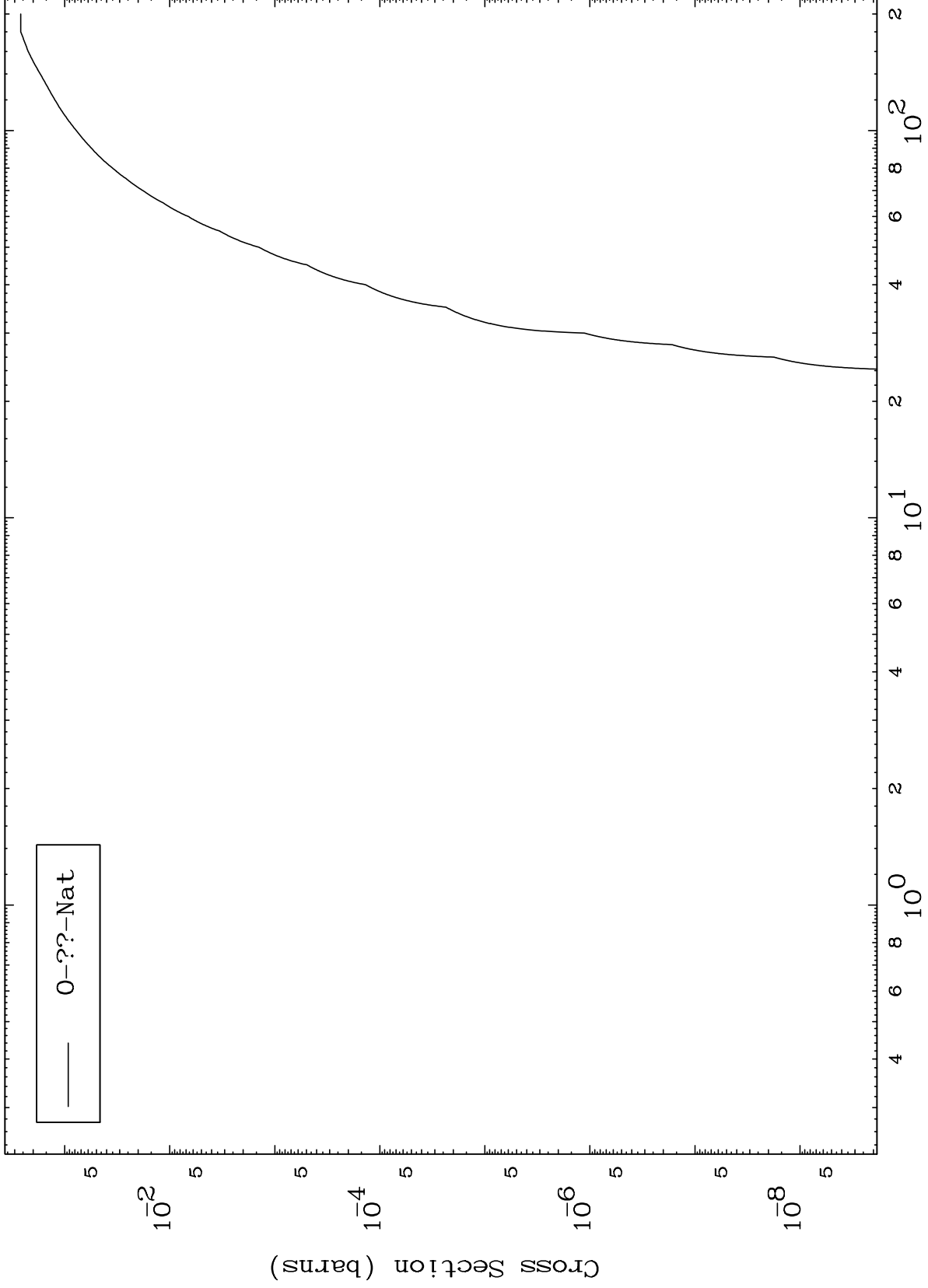
0 Kelvin Cross Sections



MAT 8134

81-Tl-206

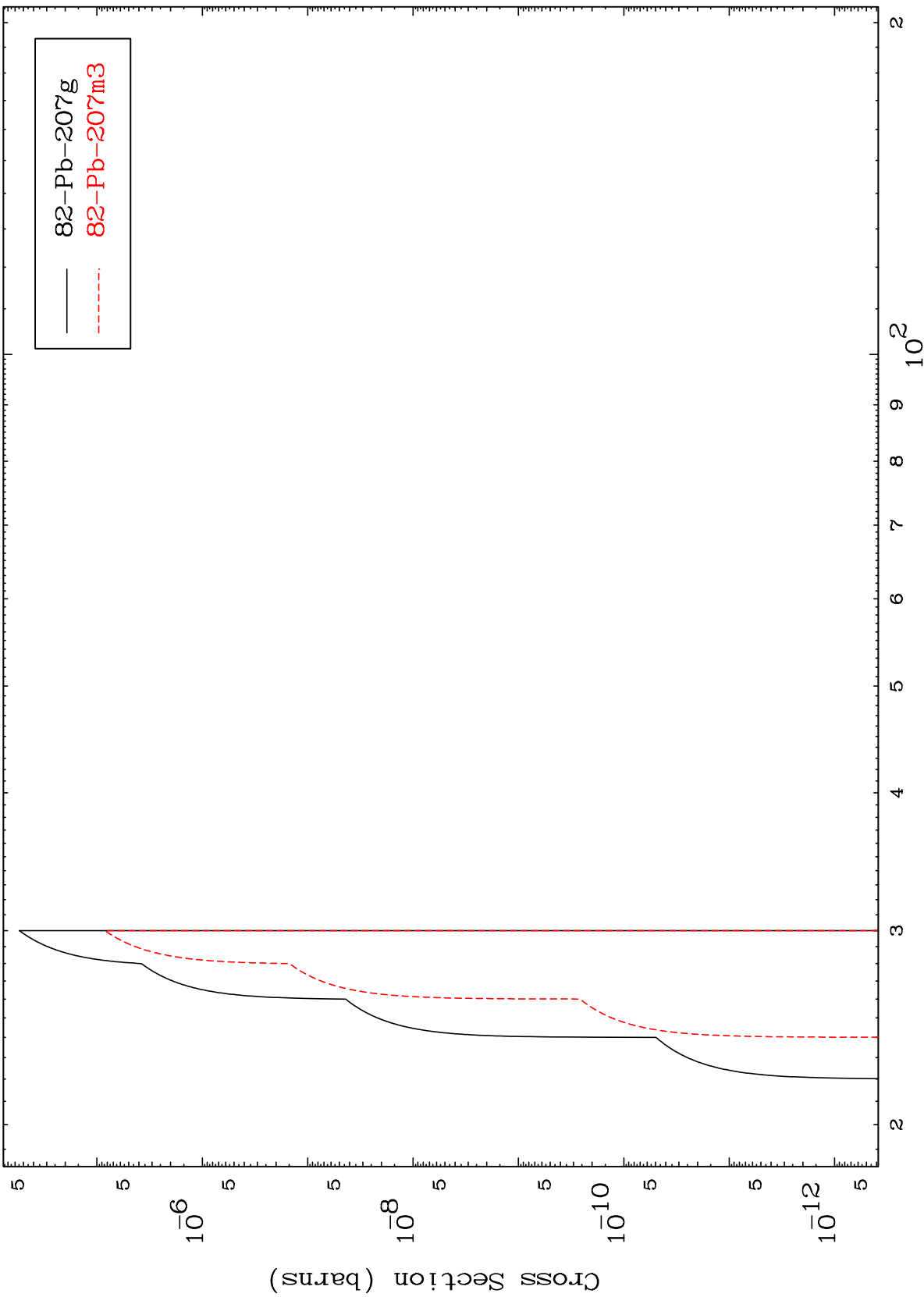
$\alpha$  Fission  
Radionuclide Production Cross Section



10

81-Tl-206

Radionuclide Production Cross Section

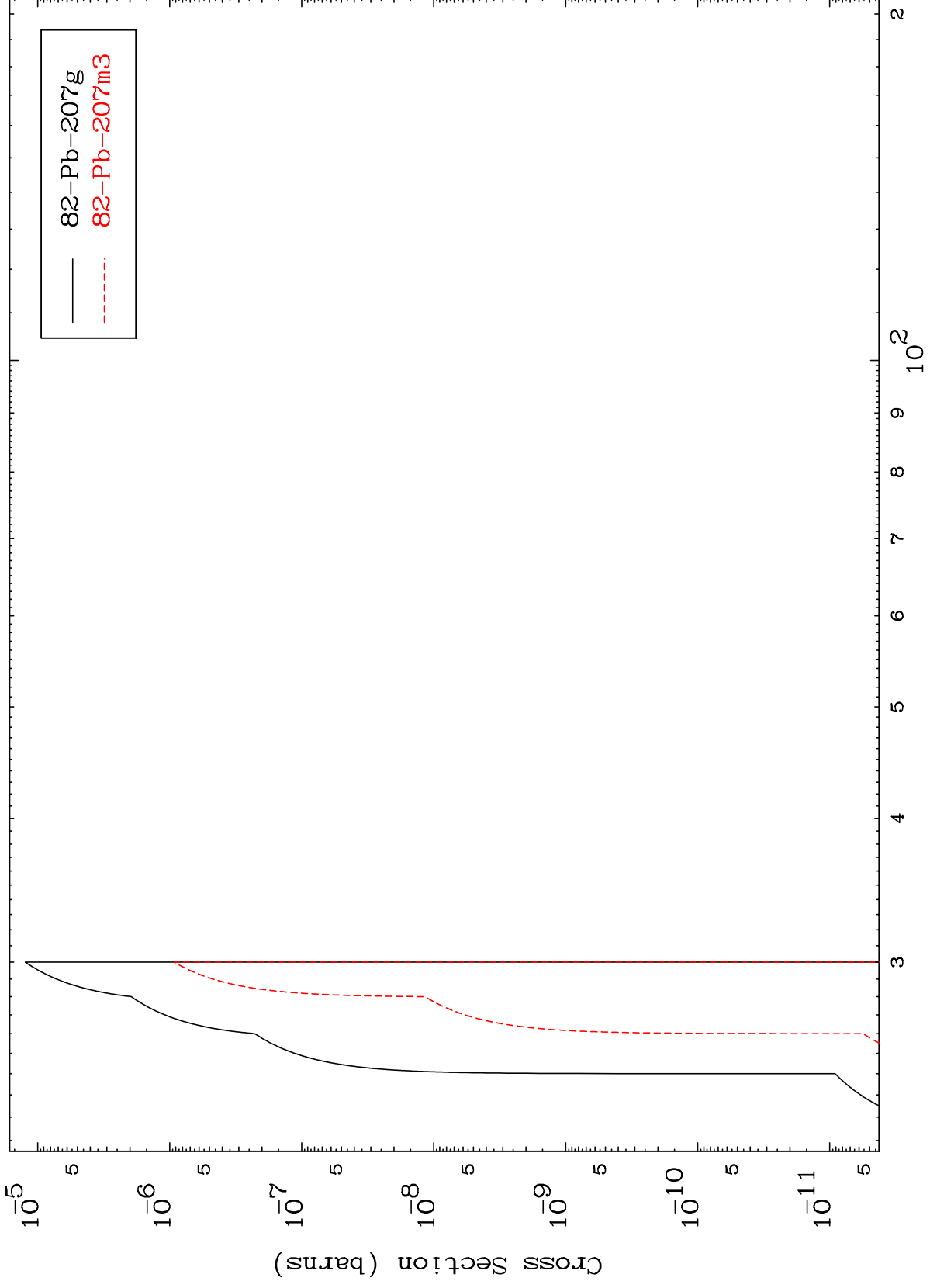


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( $\alpha, 2n$ ) p

81-T1-206

Radionuclide Production Cross Section



12

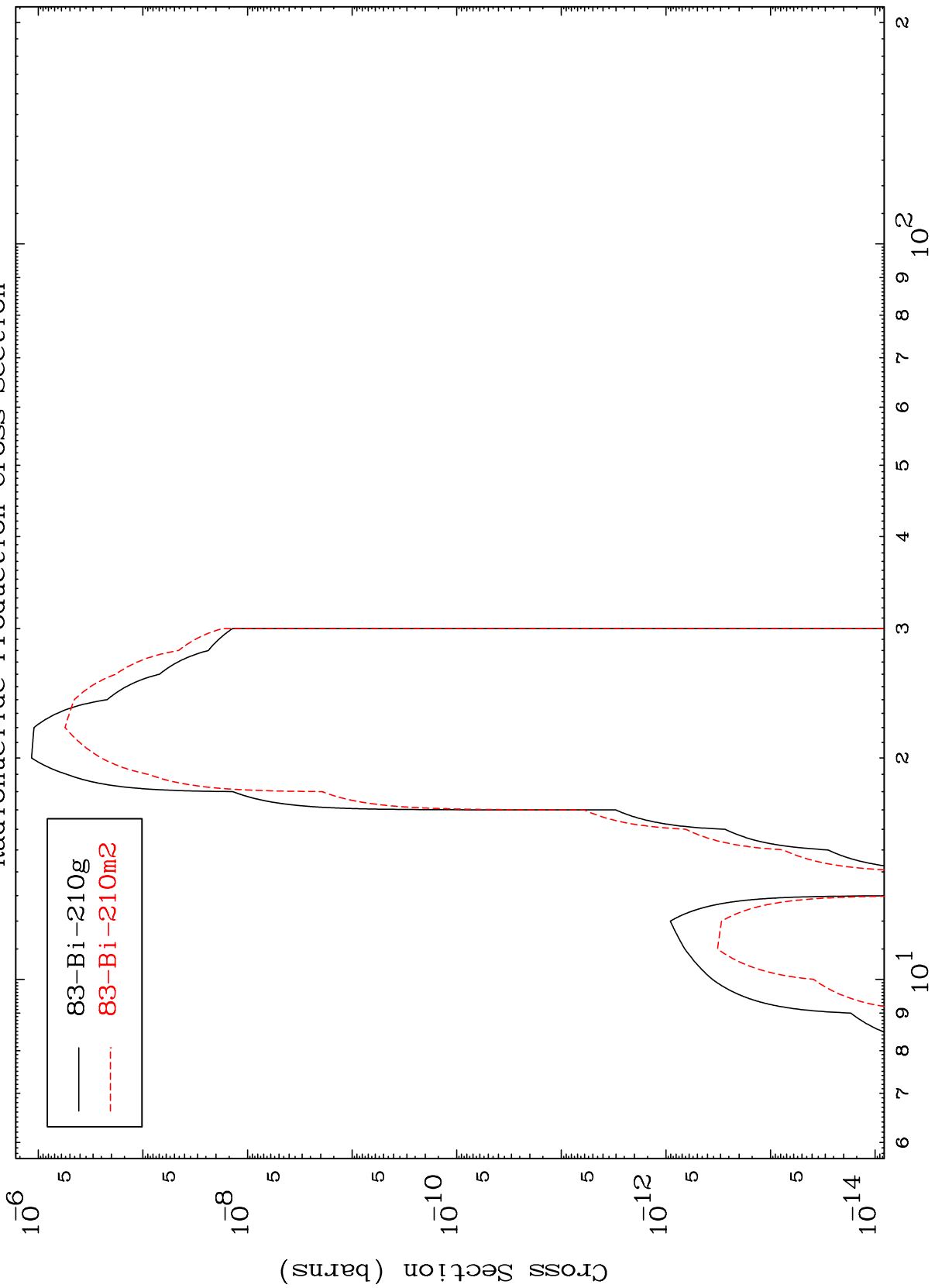
Incident Energy (MeV)

81-T1-206

MAT 8134

81-Tl-206

Radionuclide Production Cross Section  
( $\alpha, \gamma$ )

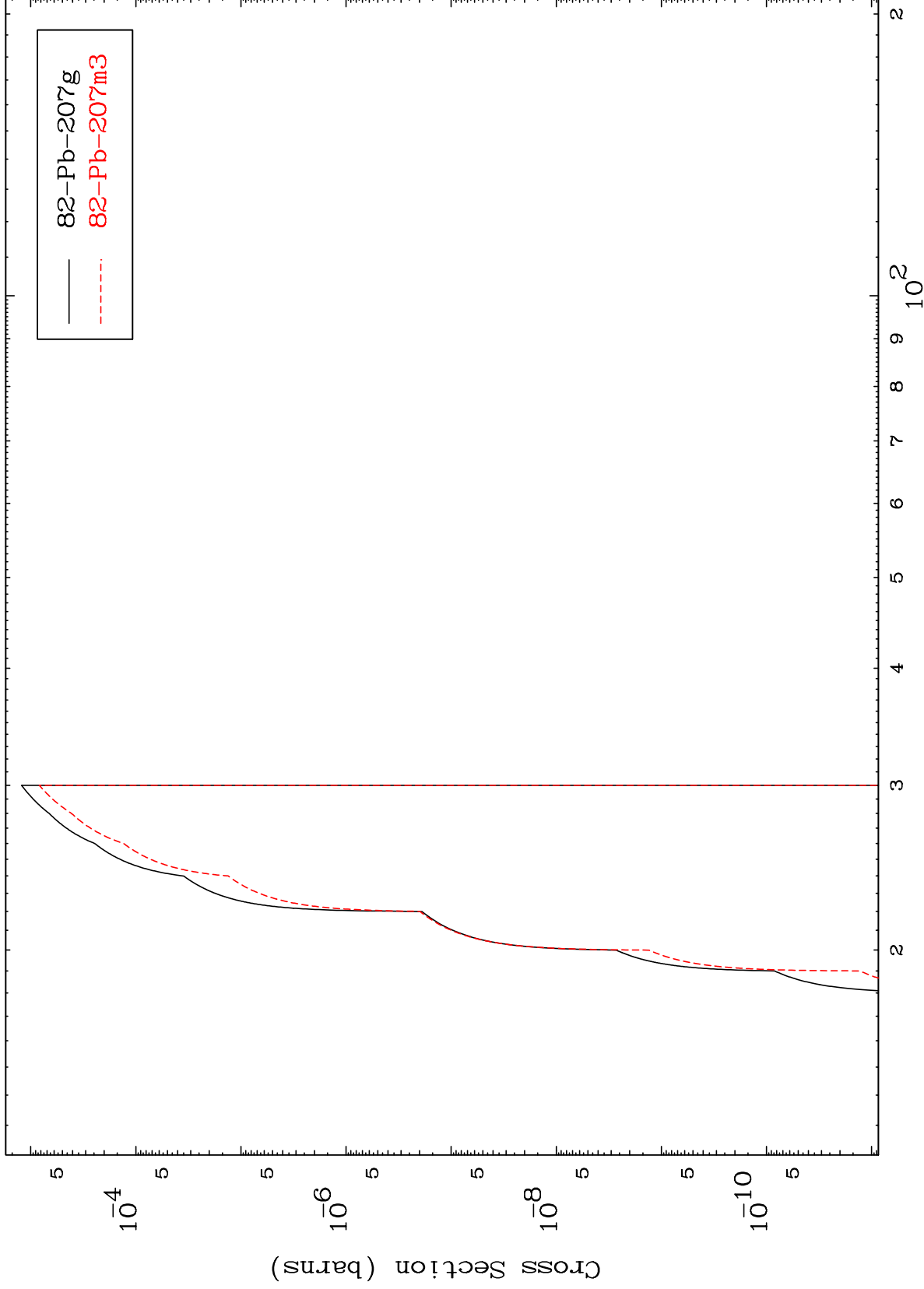


81-Tl-206

Incident Energy (MeV)

13

Radionuclide Production Cross Section

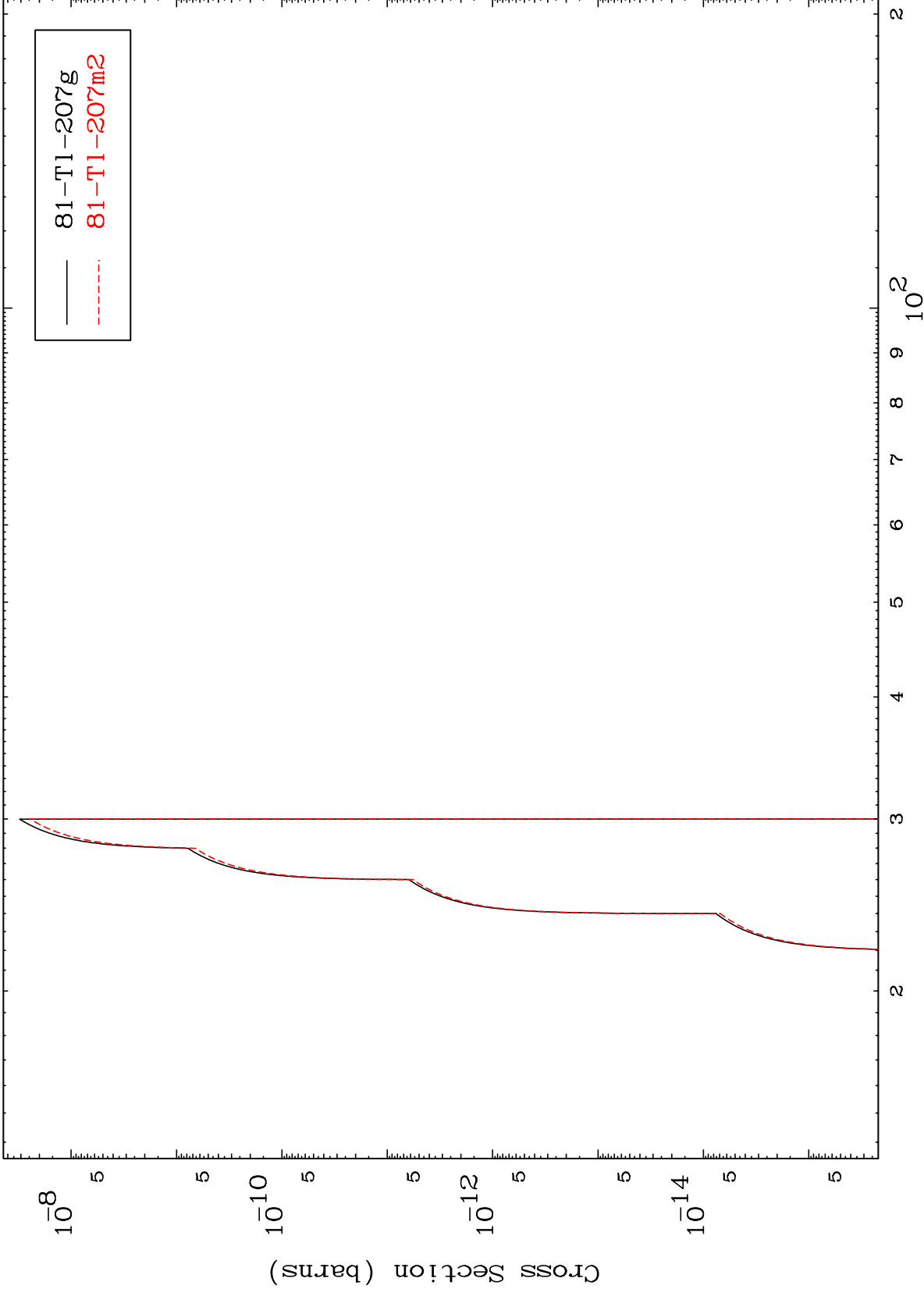


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( $\alpha, \text{He-3}$ )

81-Tl-206

Radionuclide Production Cross Section



15

Incident Energy (MeV)

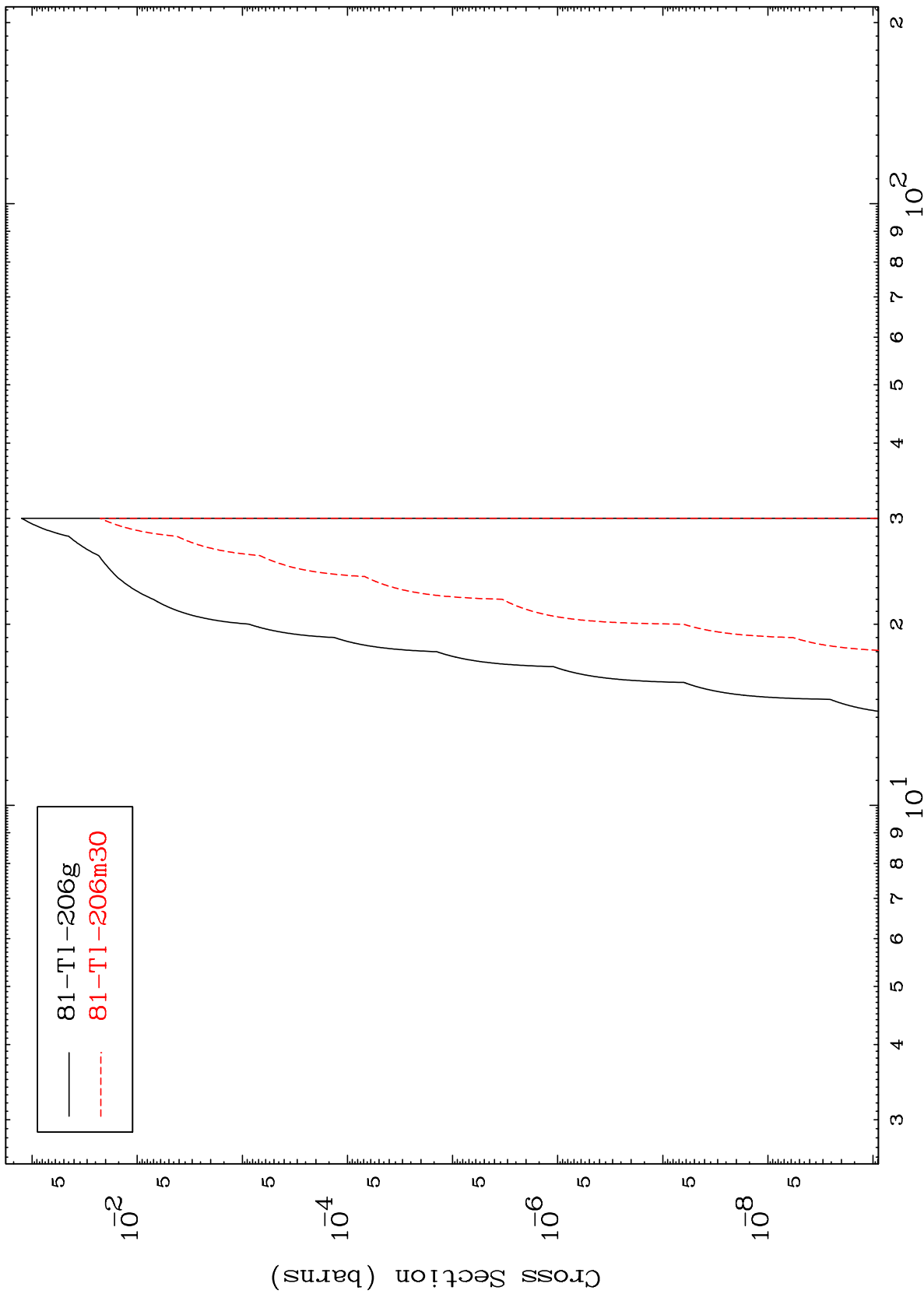
81-Tl-206



MAT 8134

81-Tl-206

( $\alpha, \alpha$ )  
Radionuclide Production Cross Section



16

81-Tl-206

Incident Energy (MeV)