

Program EVALPLOT
(Version 2018-1)

by

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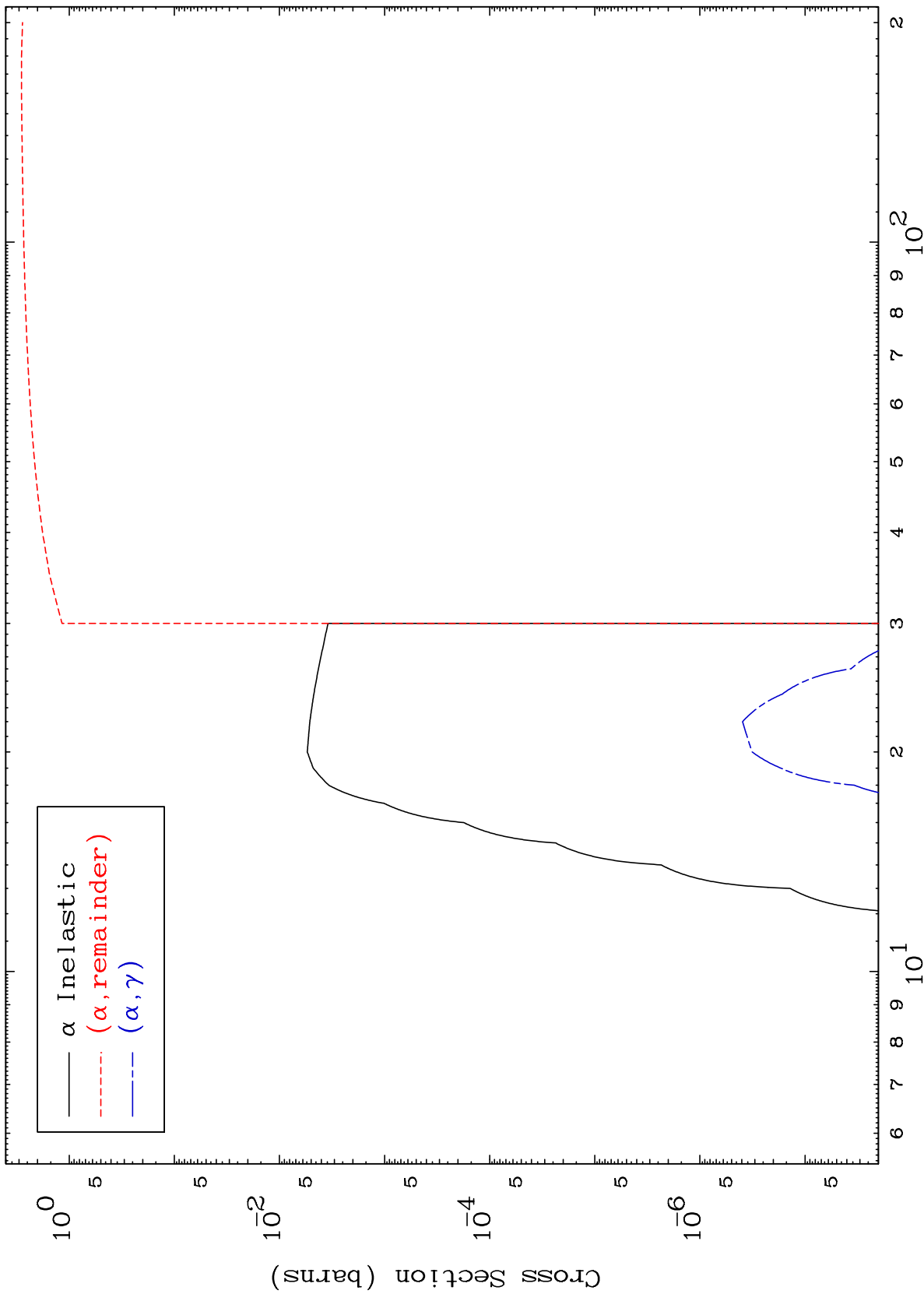
Press Mouse Button to Start

MAT 8146

α Major

81-Tl-210

0 Kelvin Cross Sections



α Inelastic
(α , remainder)
(α, γ)

Incident Energy (MeV)

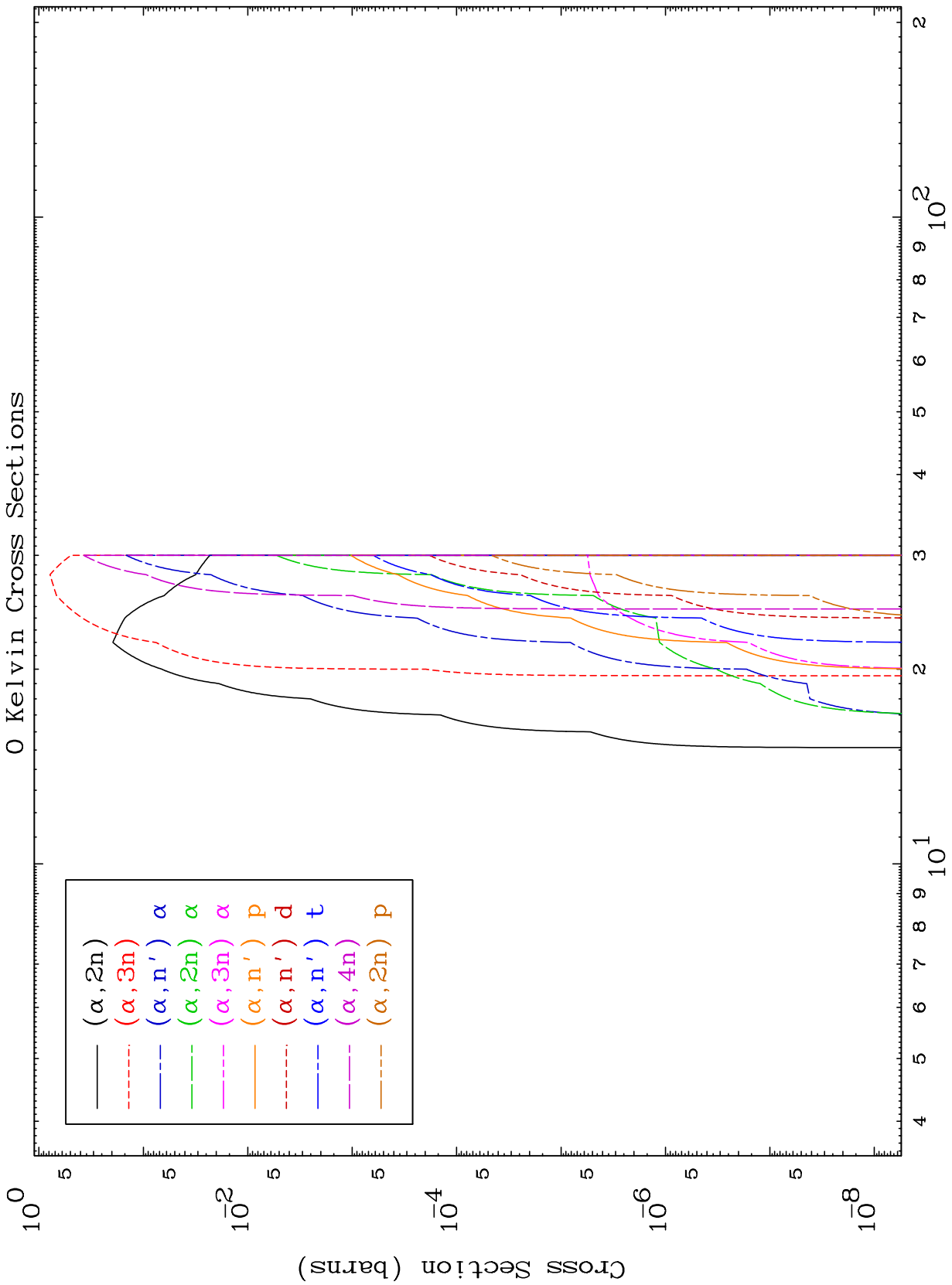
81-Tl-210

1

MAT 8146

α Neutron Production
0 Kelvin Cross Sections

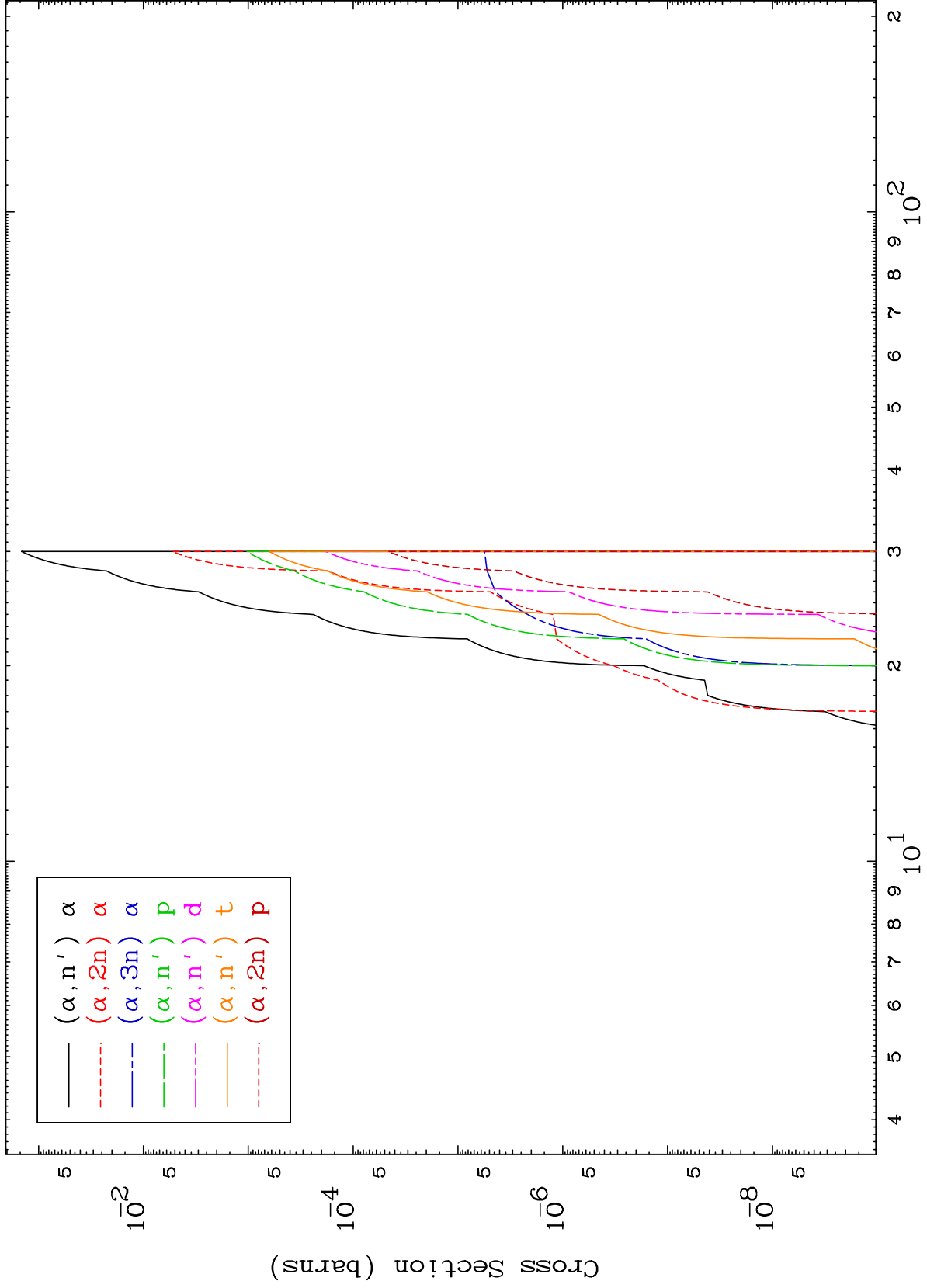
81-Tl-210

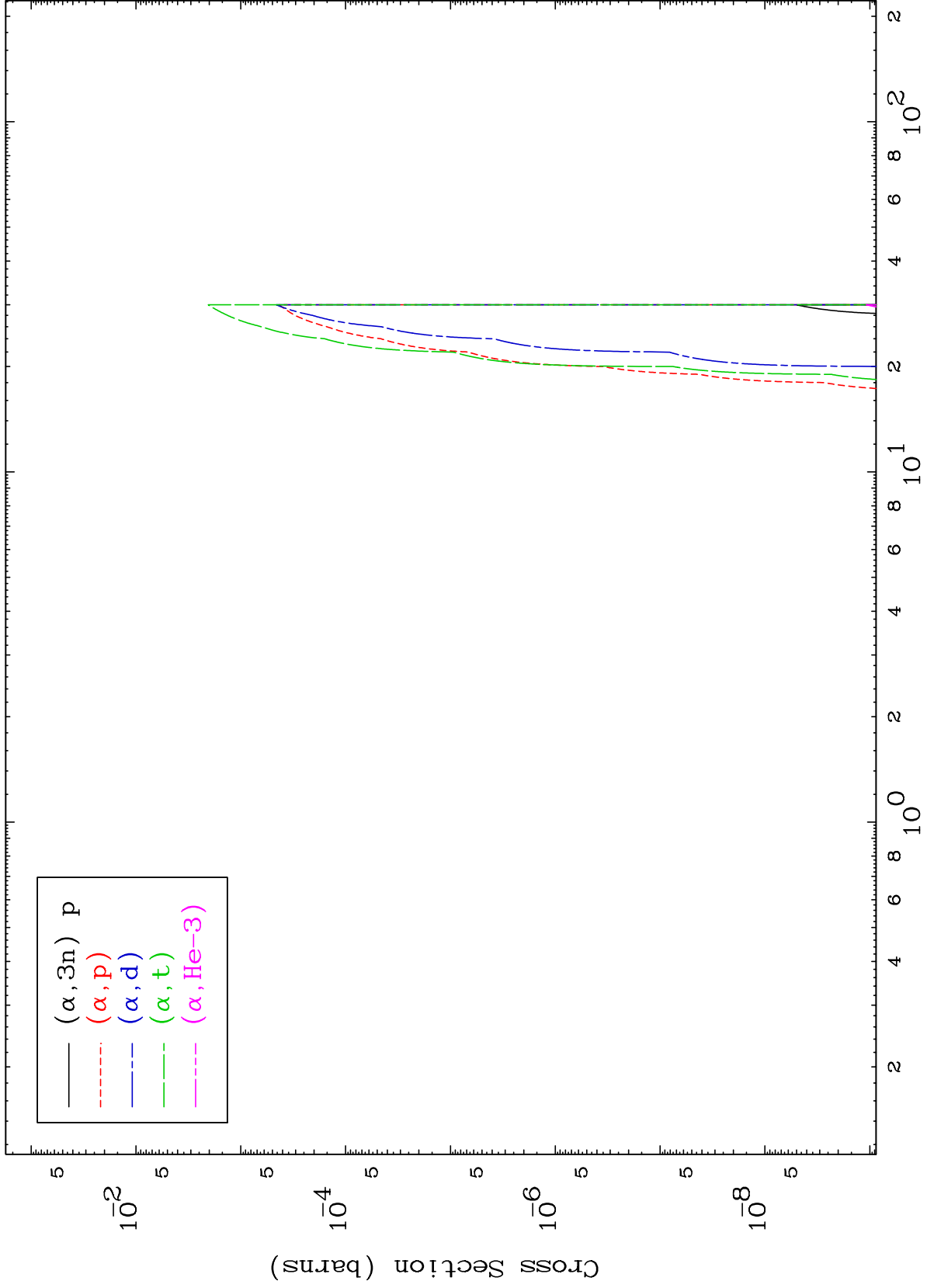


Incident Energy (MeV)

81-Tl-210

2



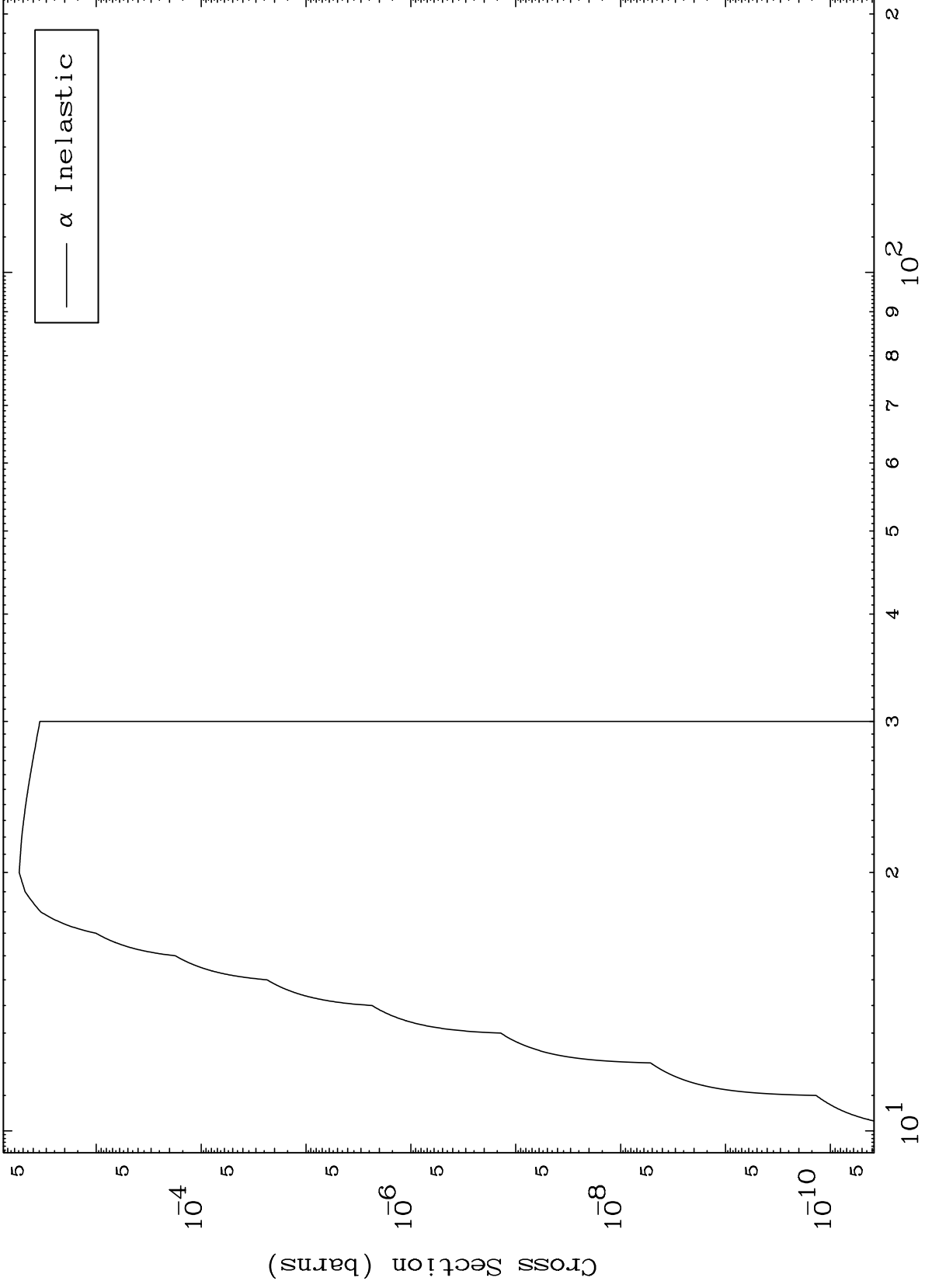


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(α, n') Level

81-Tl-210

0 Kelvin Cross Sections



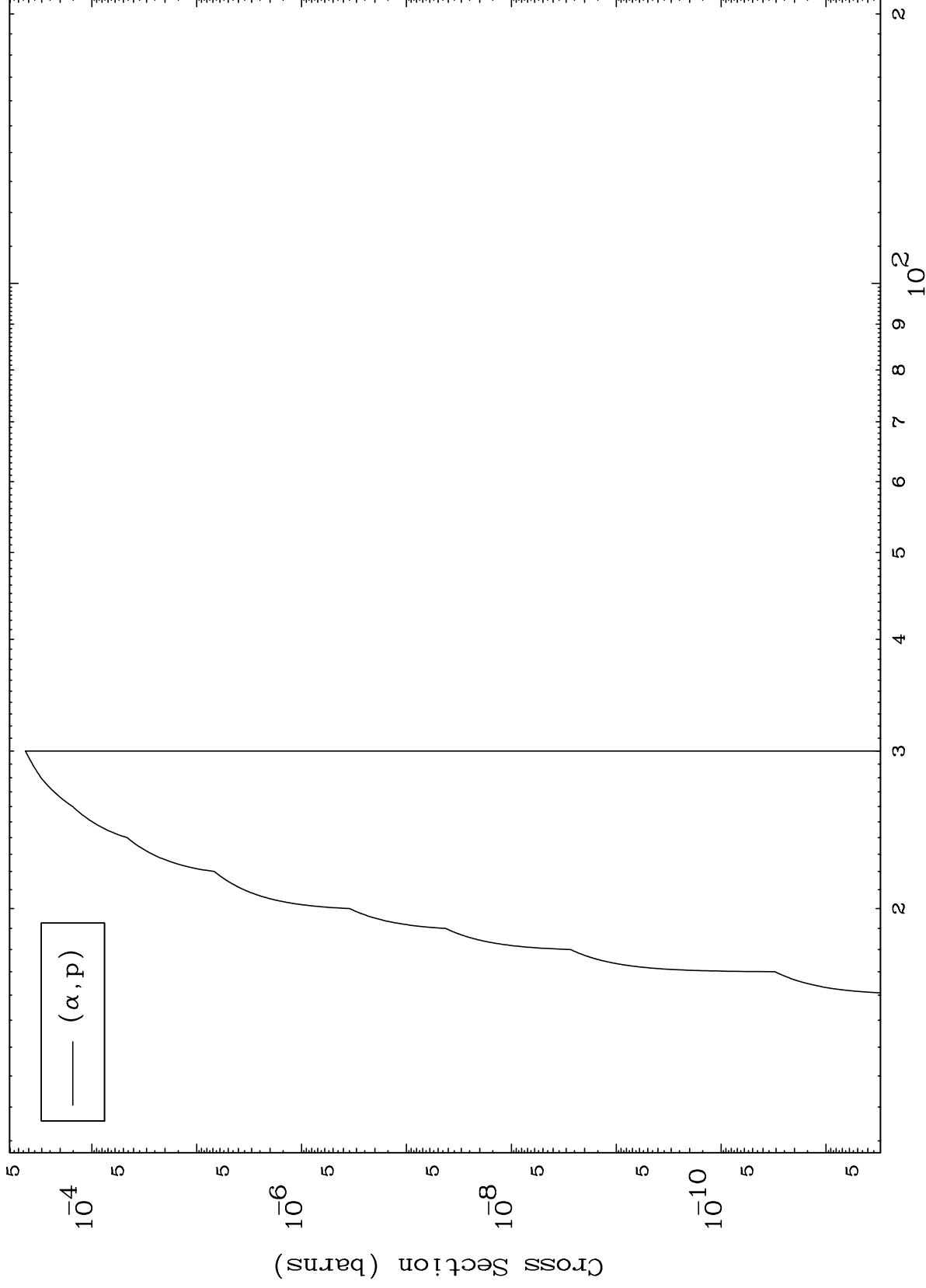
Incident Energy (MeV)

81-Tl-210

MAT 8146

(α, p) Levels
0 Kelvin Cross Sections

81-Tl-210



6

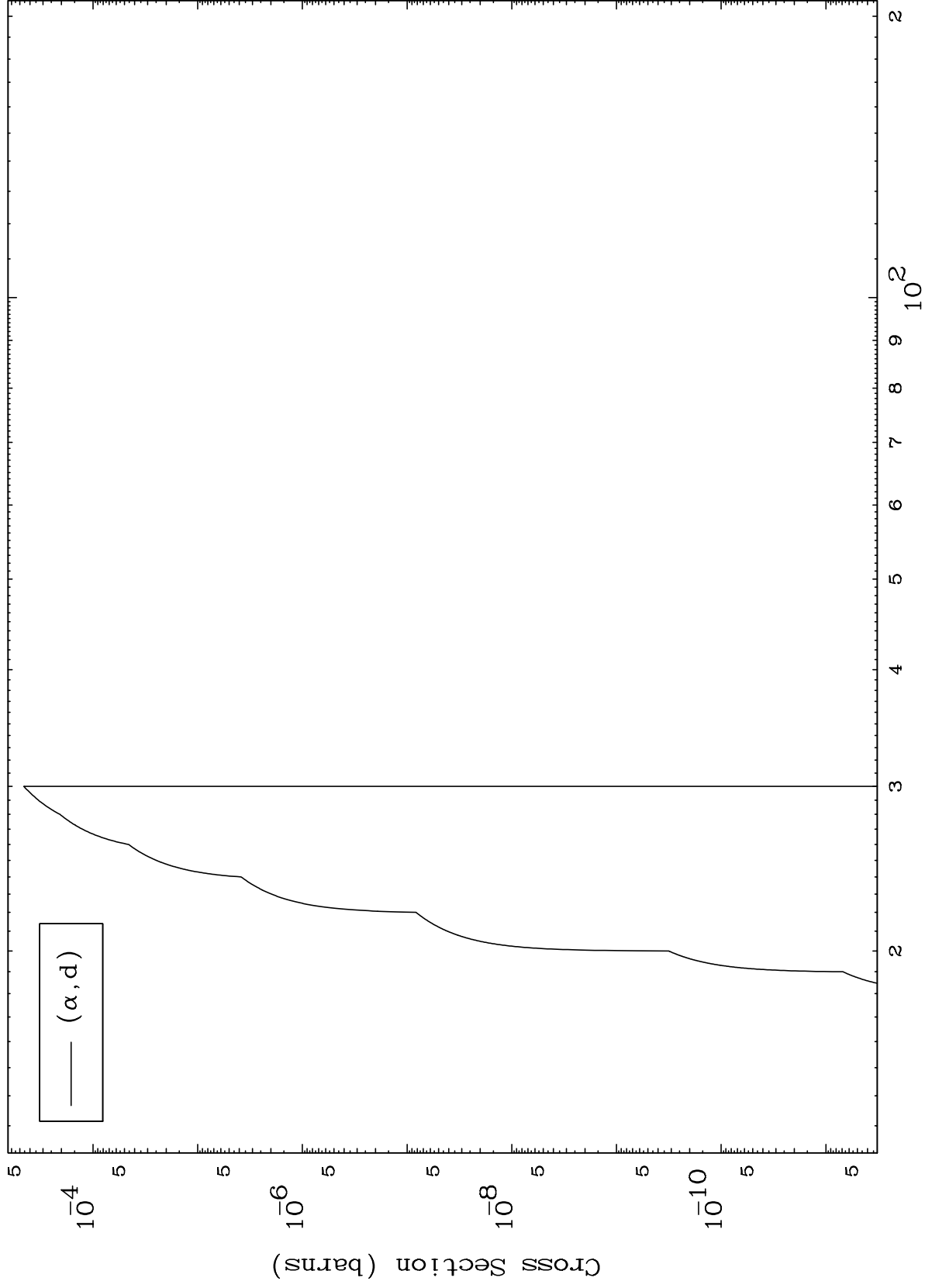
Incident Energy (MeV)

81-Tl-210

MAT 8146

(α, d) Levels
0 Kelvin Cross Sections

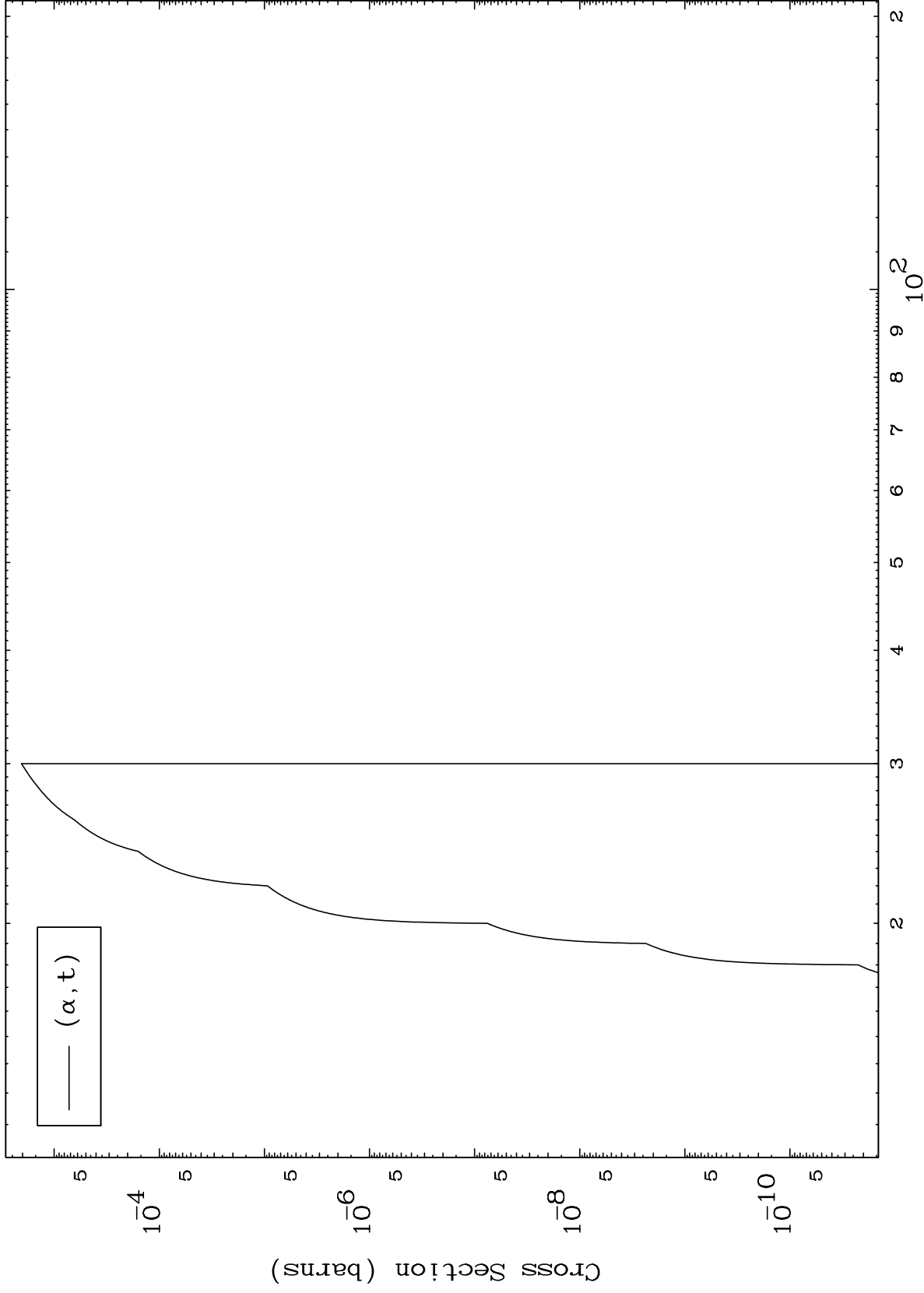
81-Tl-210



7

Incident Energy (MeV)

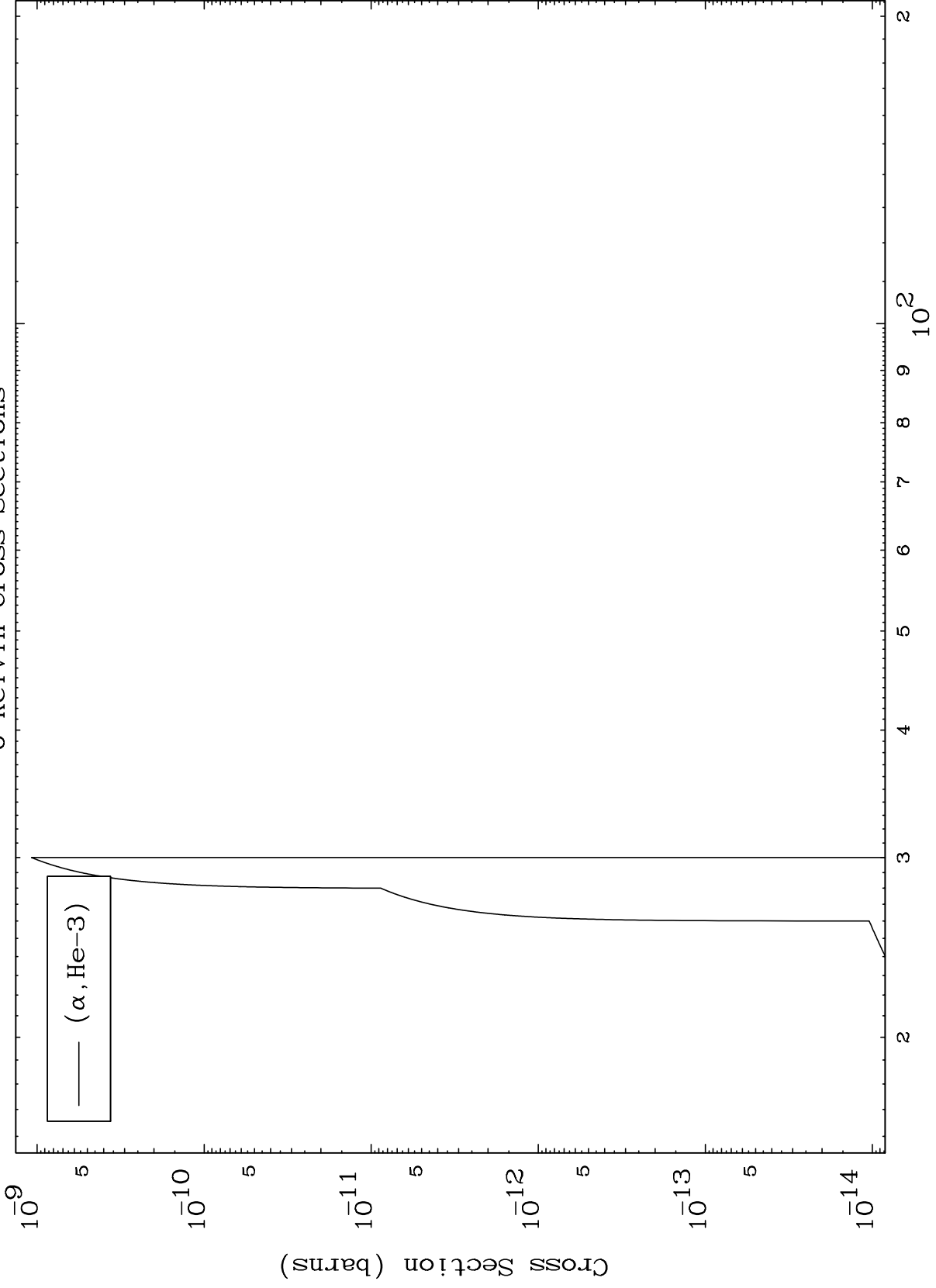
81-Tl-210



MAT 8146

(α ,He3) Levels
0 Kelvin Cross Sections

81-T1-210



9

Incident Energy (MeV)

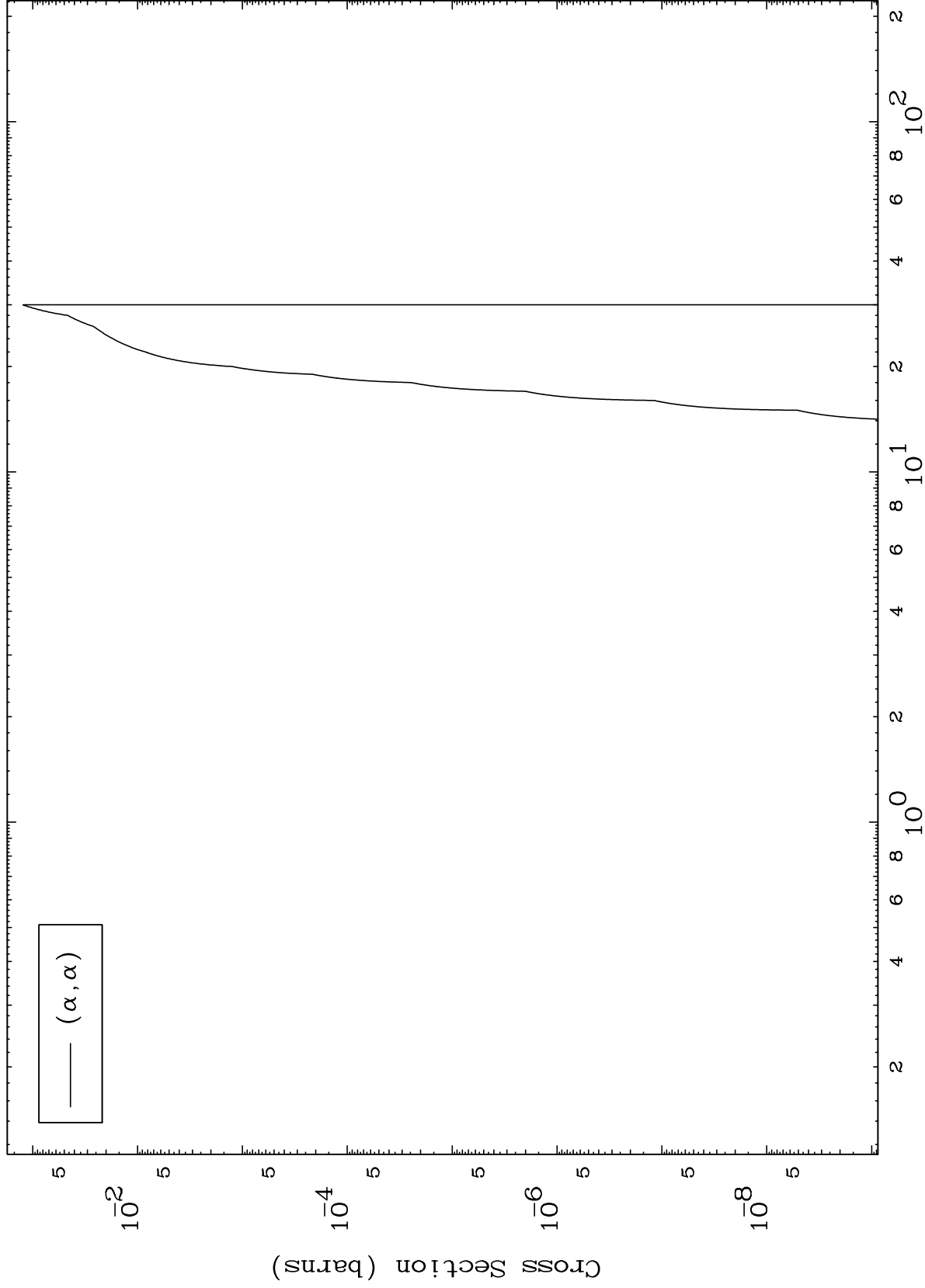
81-T1-210

MAT 8146

(α, α) Levels

81-Tl-210

0 Kelvin Cross Sections



10

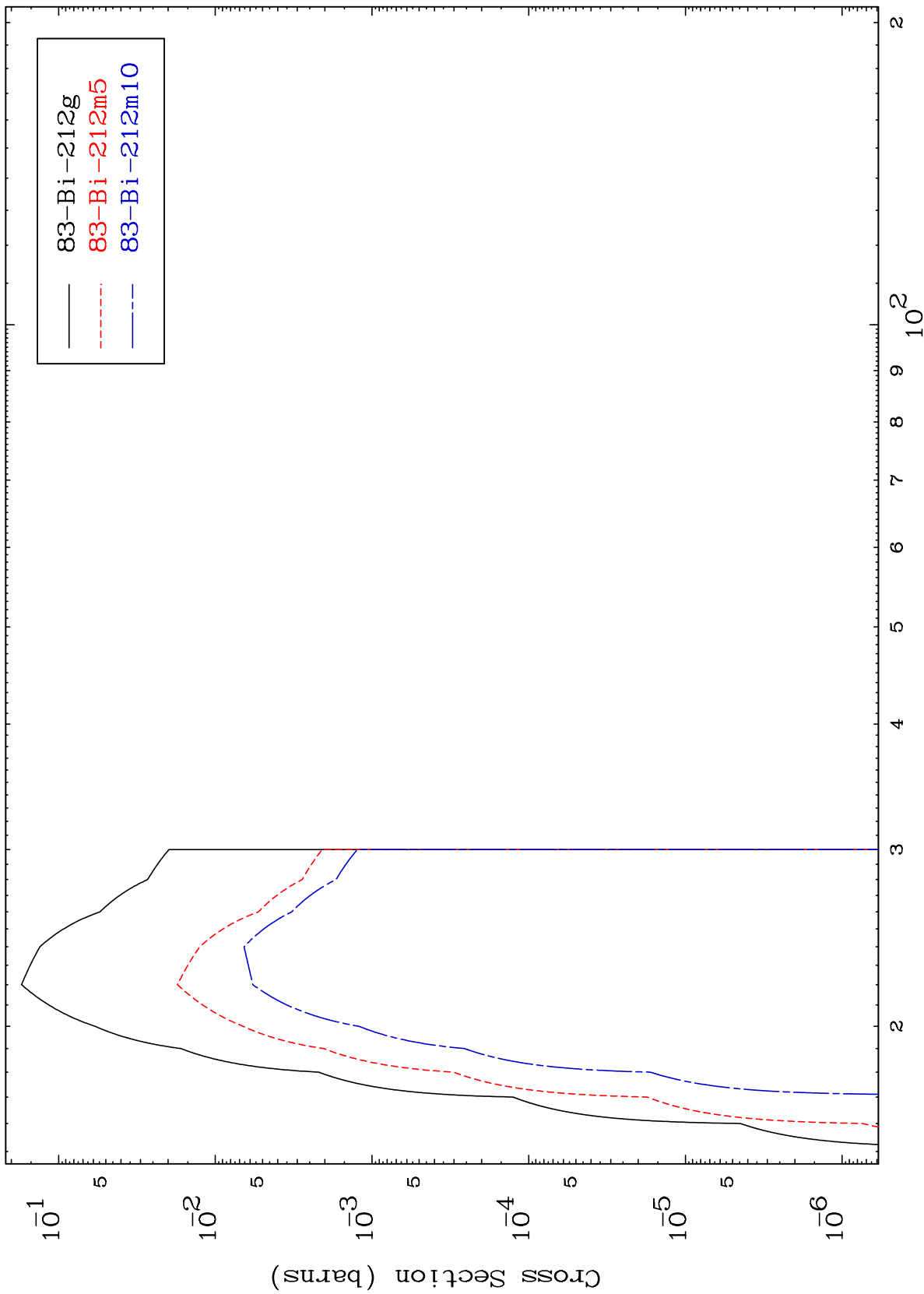
Incident Energy (MeV)

81-Tl-210

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81-Tl-210

Radionuclide Production Cross Section
($\alpha, 2n$)



11

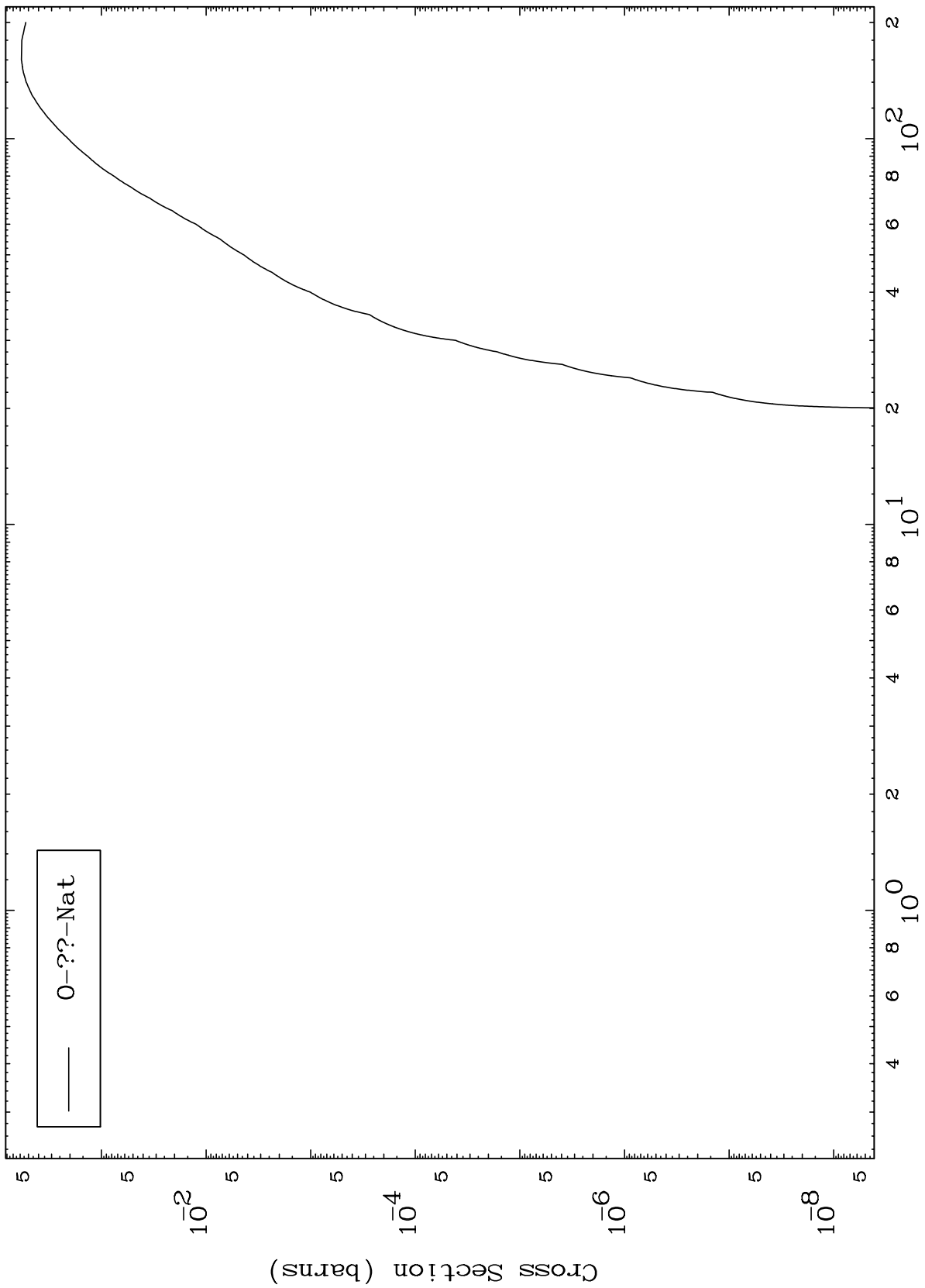
81-Tl-210

Incident Energy (MeV)

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81-Tl-210

α Fission
Radionuclide Production Cross Section



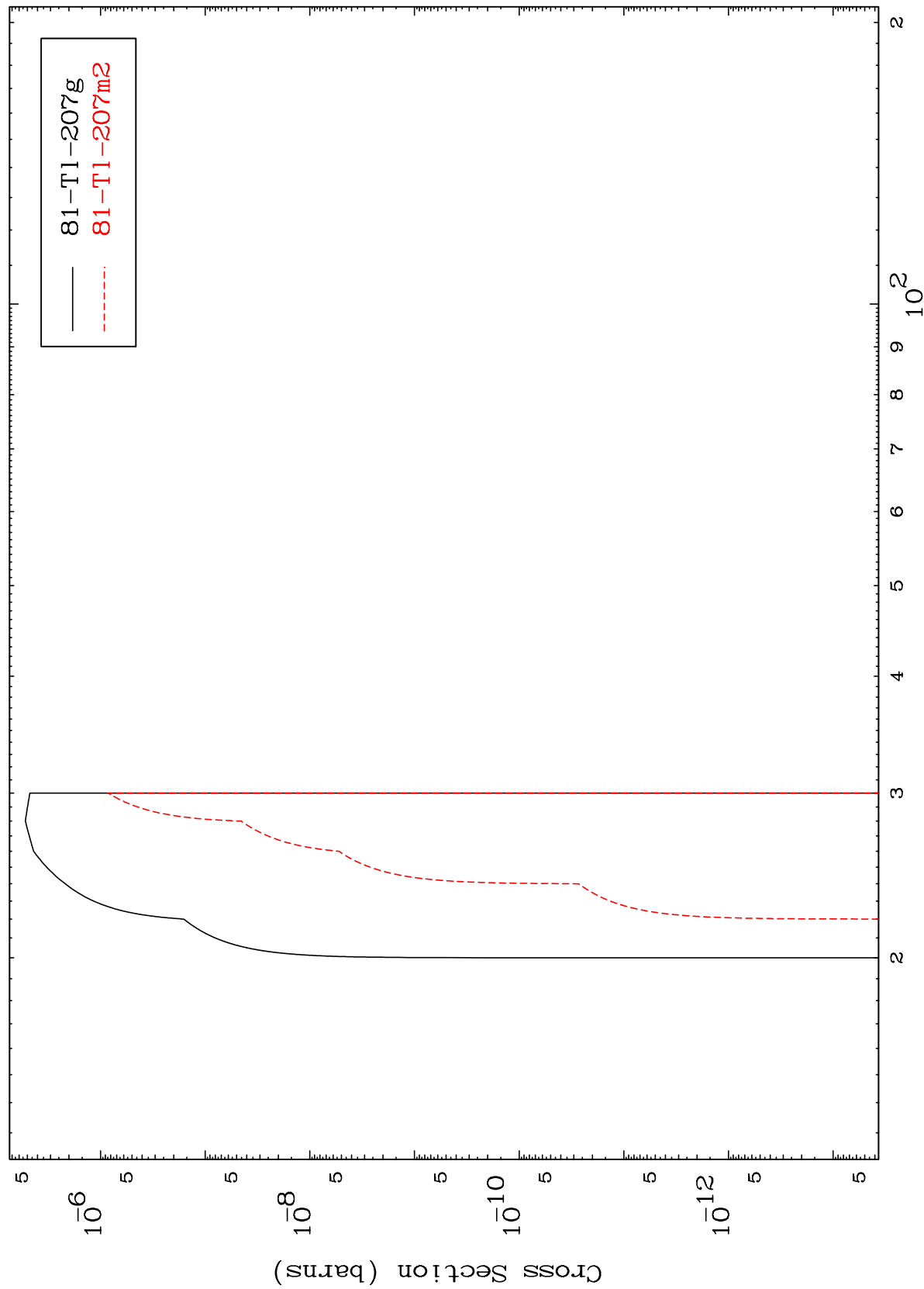
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$(\alpha, 3n) \alpha$

81-Tl-210

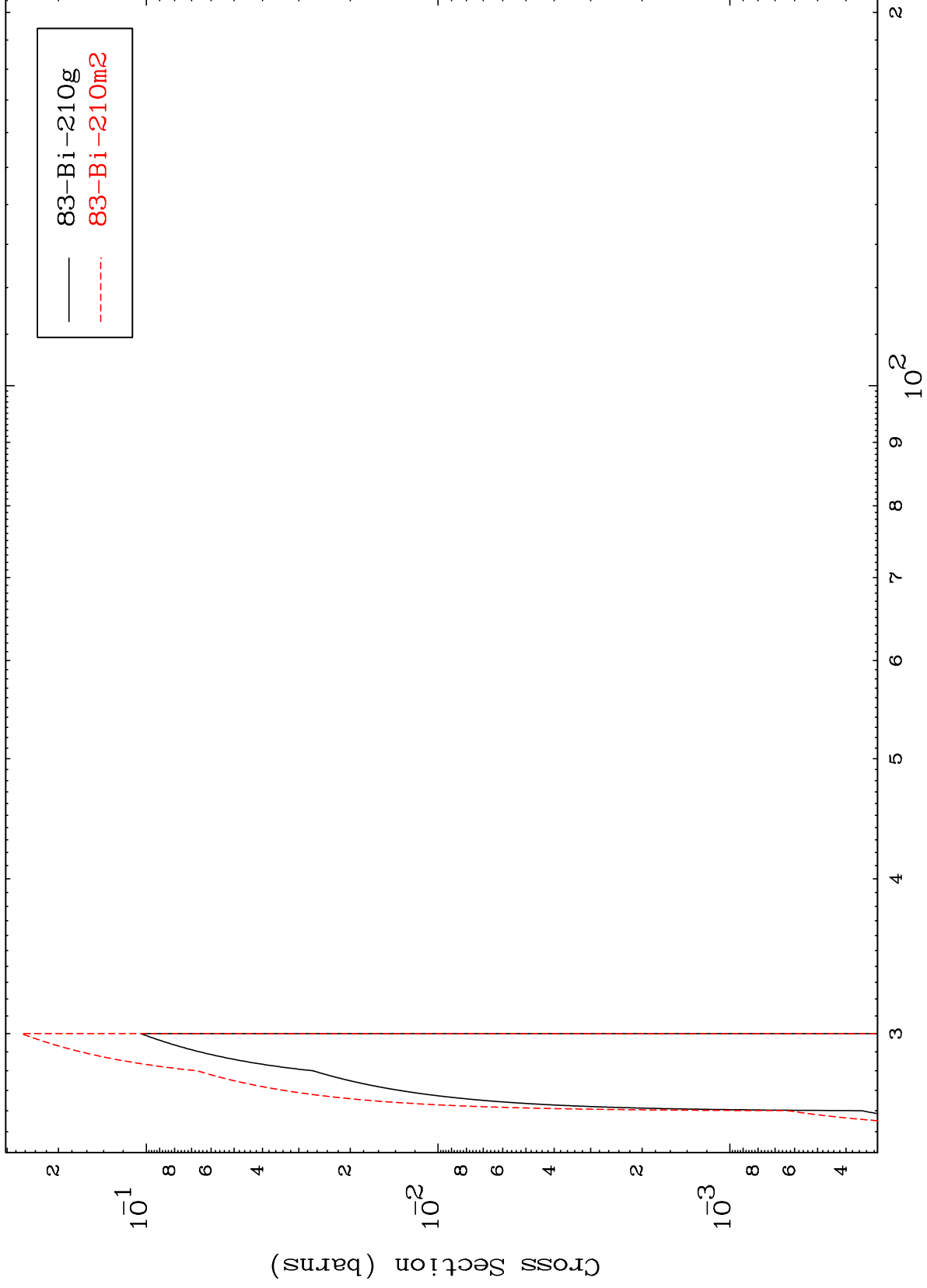
Radionuclide Production Cross Section



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81-Tl-210

$(\alpha, 4n)$
Radionuclide Production Cross Section



14

Incident Energy (MeV)

81-Tl-210