

Program EVALPLOT
(Version 2018-1)

by

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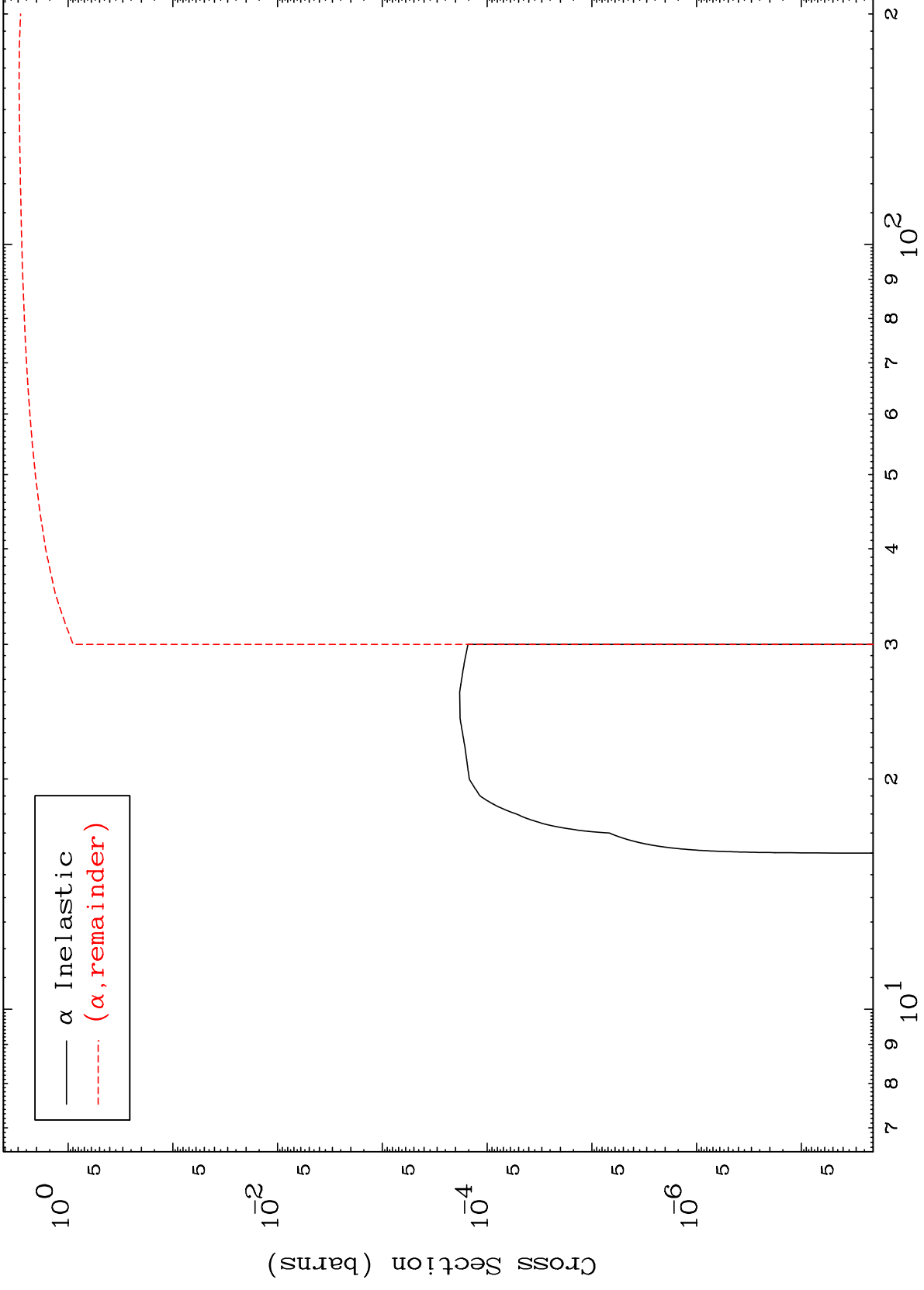
Press Mouse Button to Start

MAT 9207

α Major

92-U -228

0 Kelvin Cross Sections



1

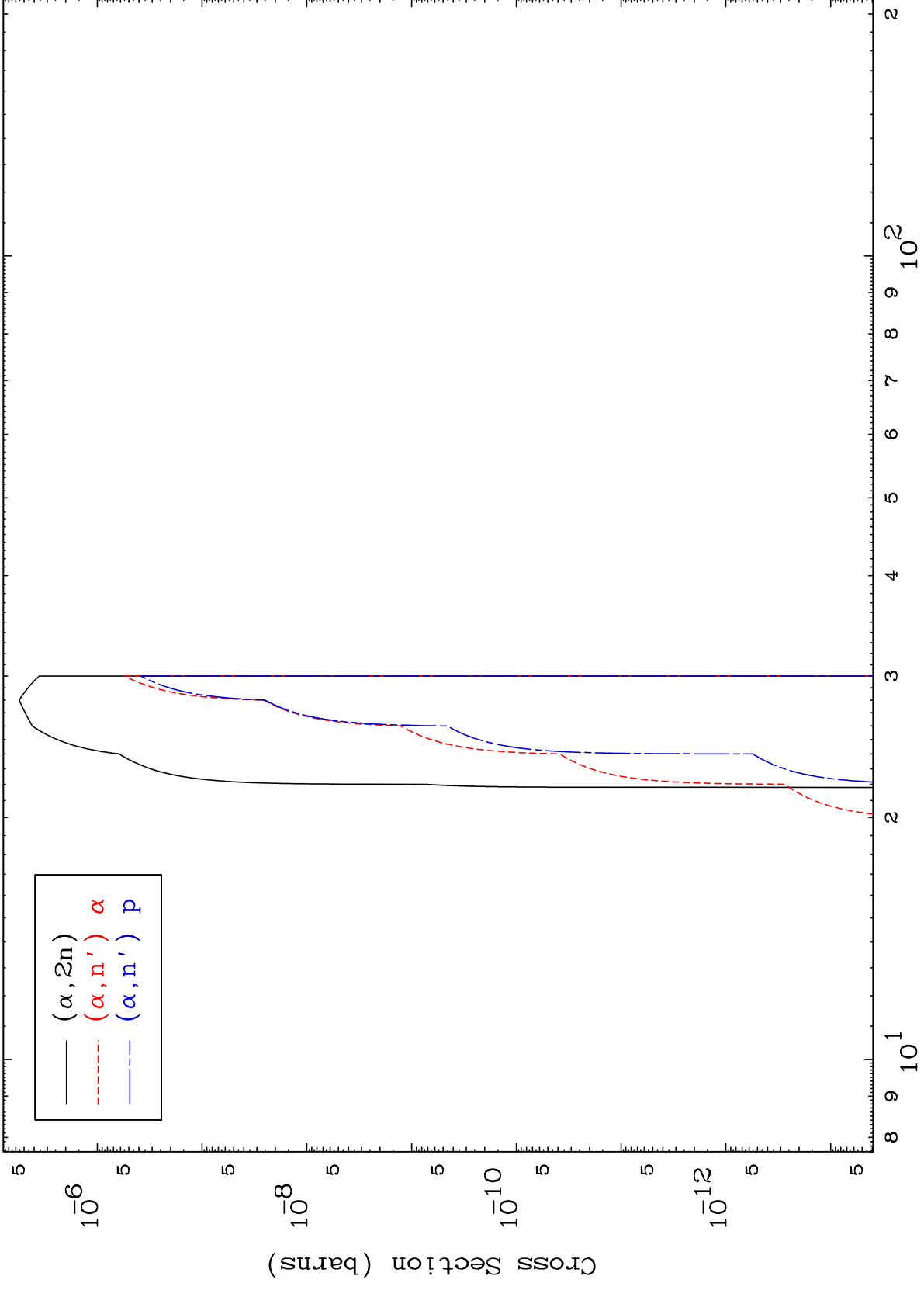
Incident Energy (MeV)

92-U -228

MAT 9207

α Neutron Production
0 Kelvin Cross Sections

92-U -228

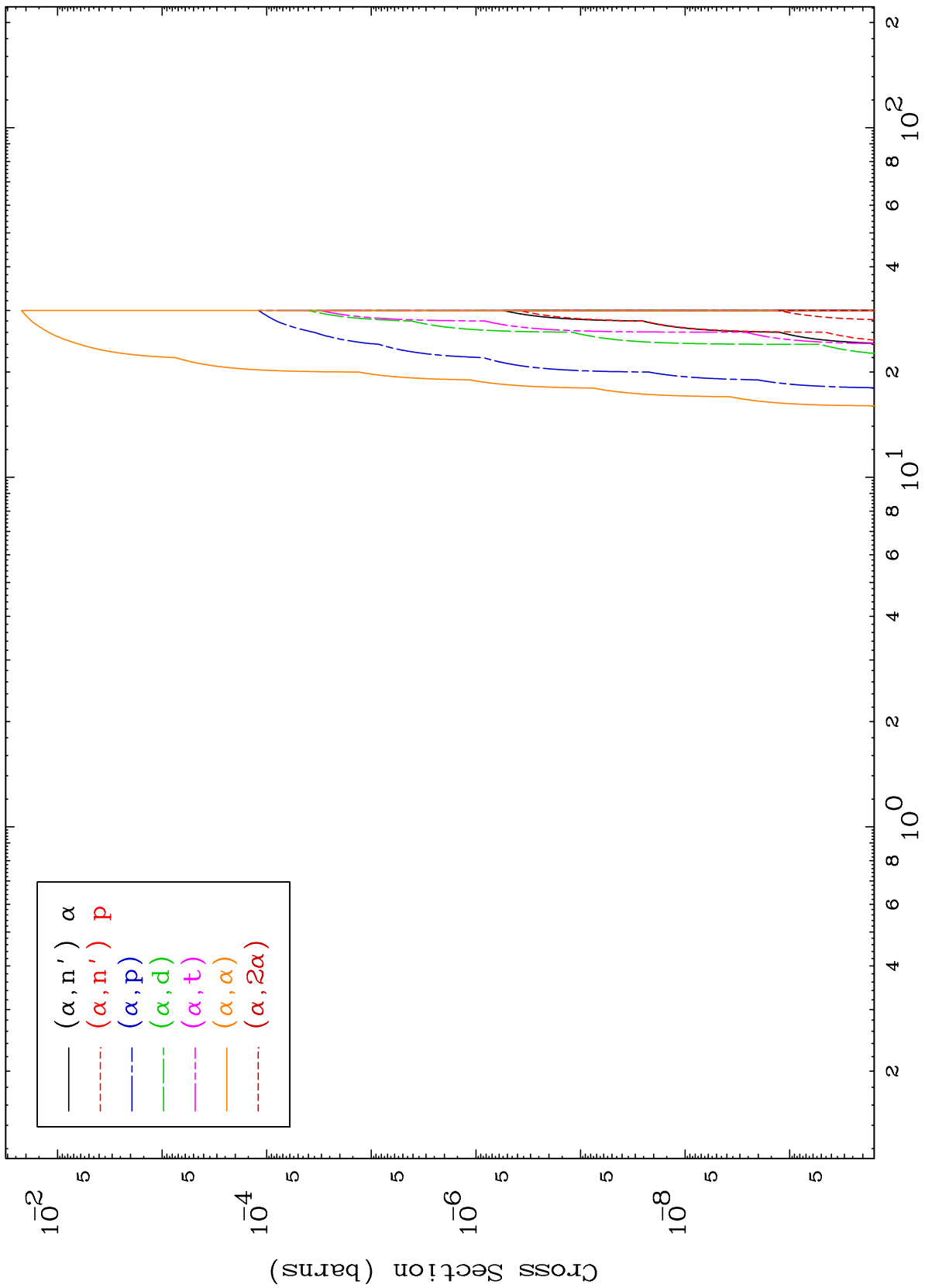


($\alpha, 2n$)
(α, n') α
(α, n') p

2

Incident Energy (MeV)

92-U -228

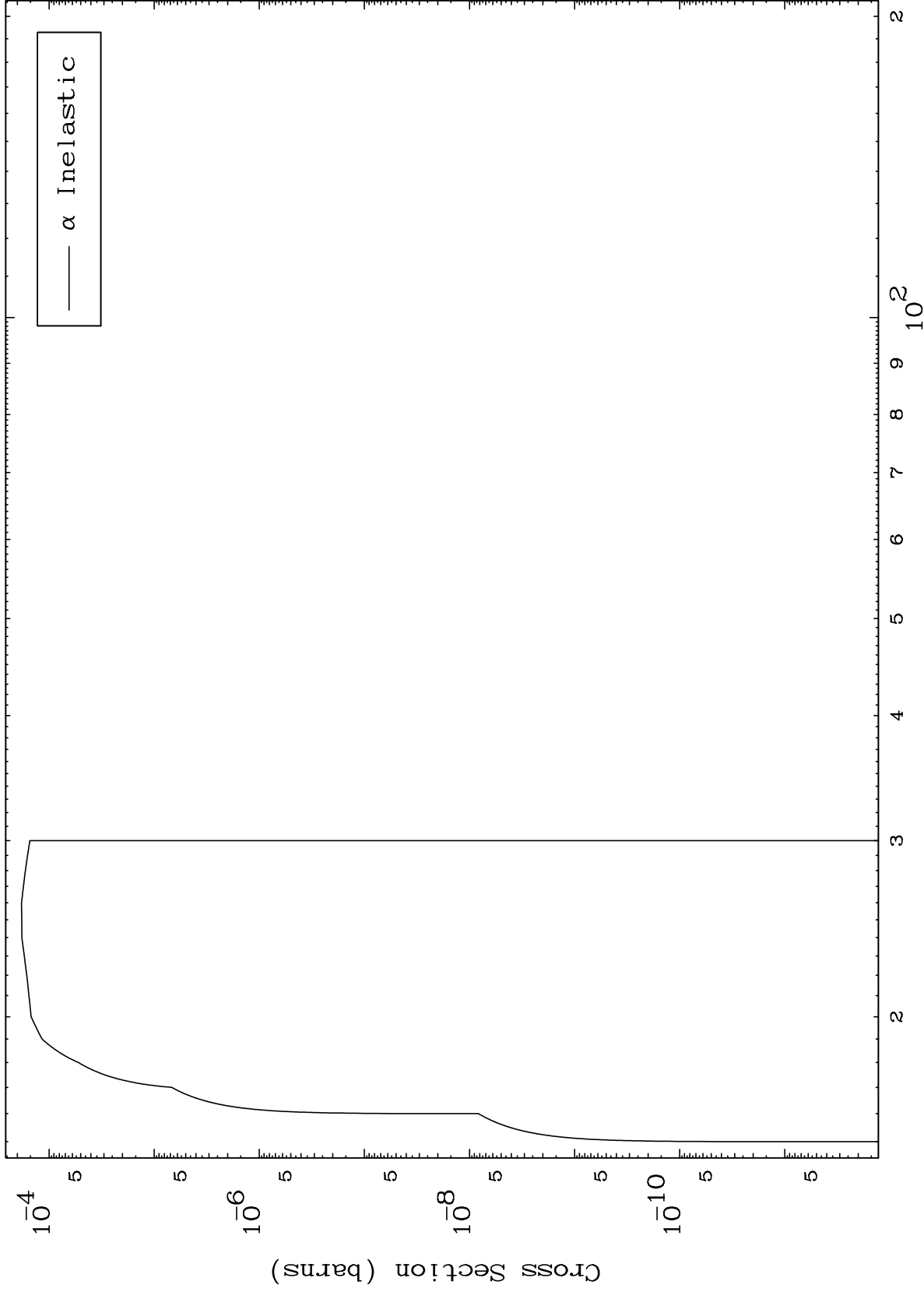


MAT 9207

(α, n') Level

92-U -228

0 Kelvin Cross Sections



4

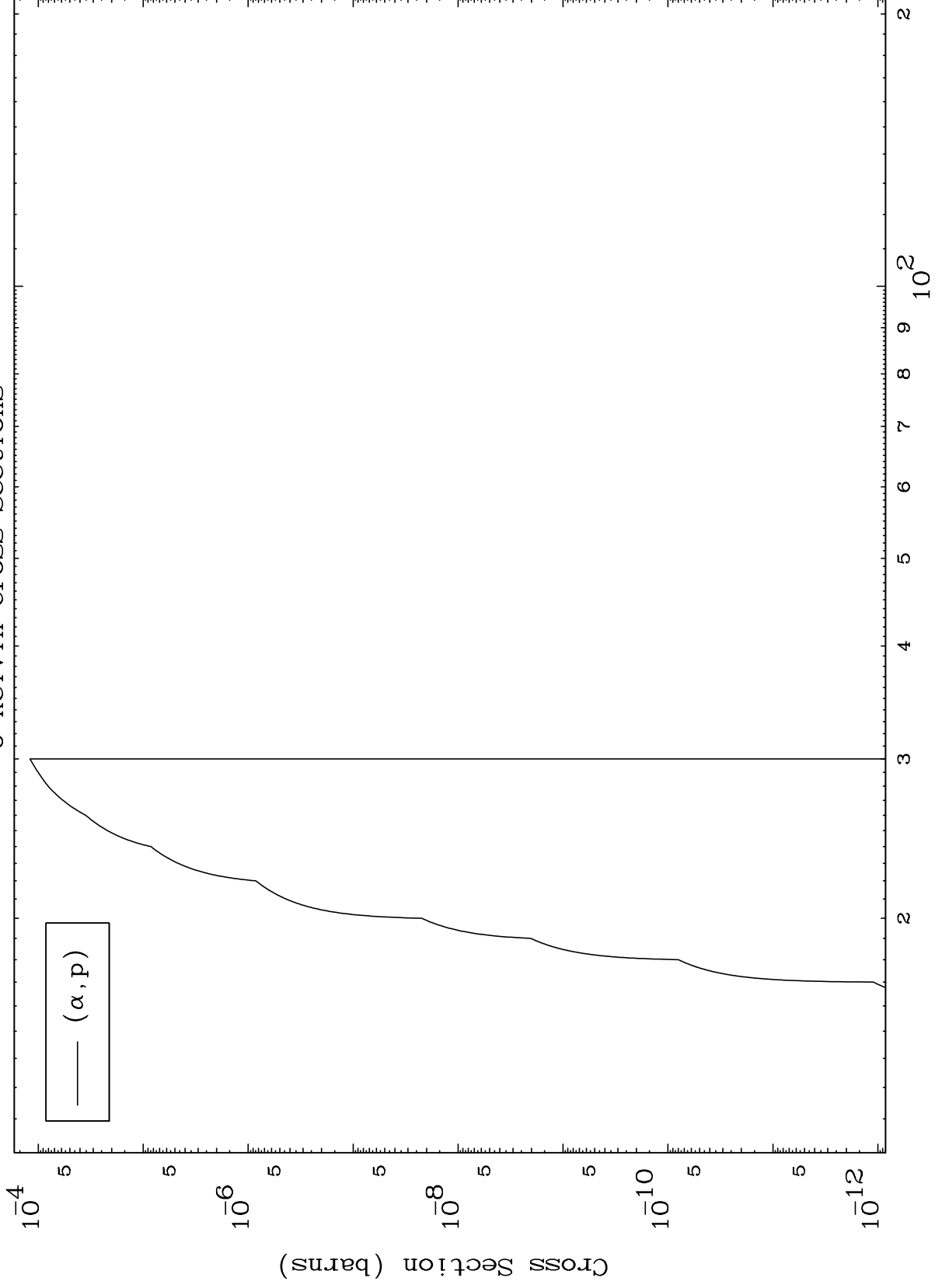
Incident Energy (MeV)

92-U -228

MAT 9207

(α, p) Levels
0 Kelvin Cross Sections

92-U -228



5

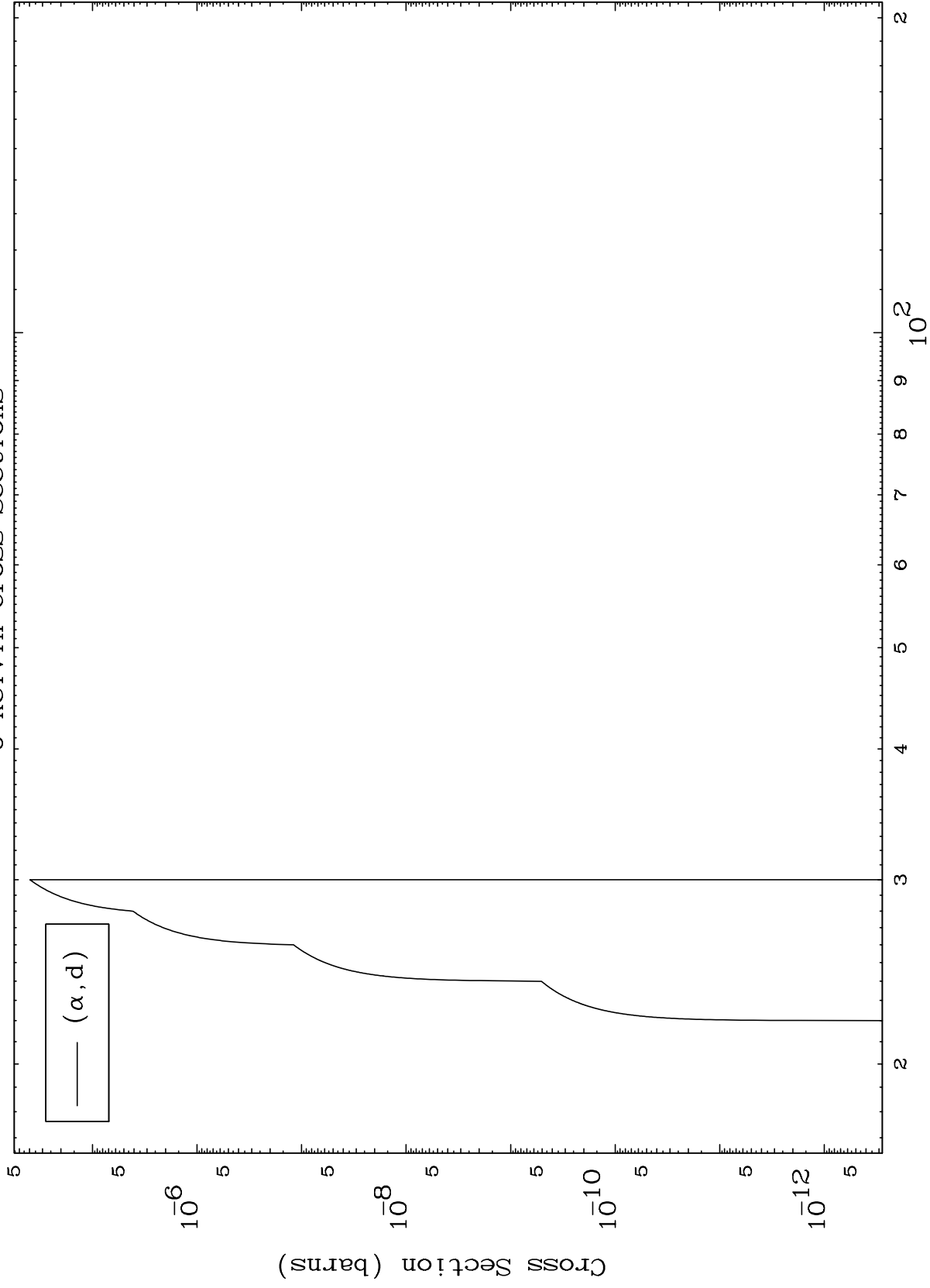
Incident Energy (MeV)

92-U -228

MAT 9207

(α, d) Levels
0 Kelvin Cross Sections

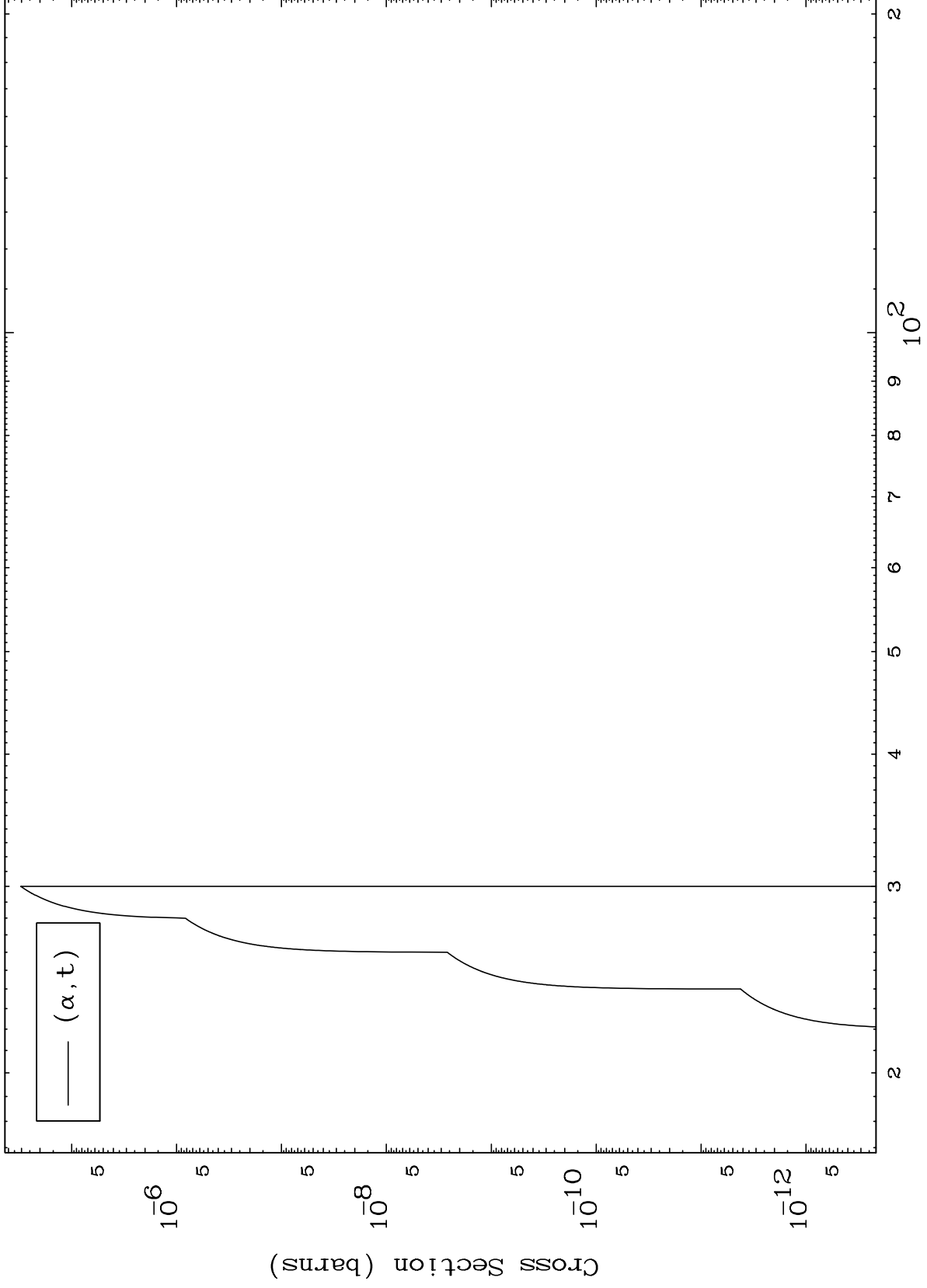
92-U -228



6

Incident Energy (MeV)

92-U -228

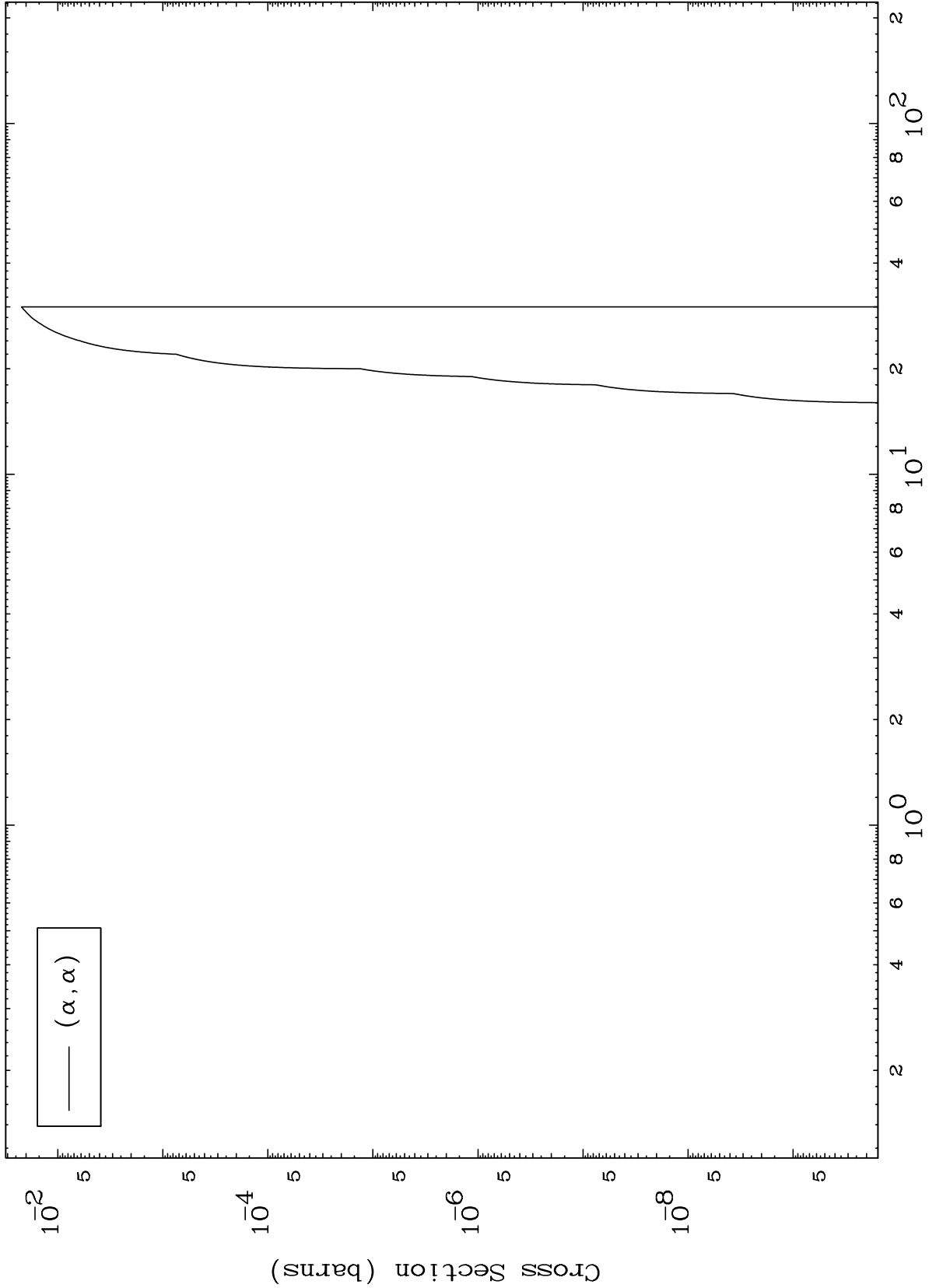


MAT 9207

(α, α) Levels

92-U -228

0 Kelvin Cross Sections



MAT 9207

92-U -228

α Fission
Radionuclide Production Cross Section

