

Program EVALPLOT  
(Version 2018-1)

by

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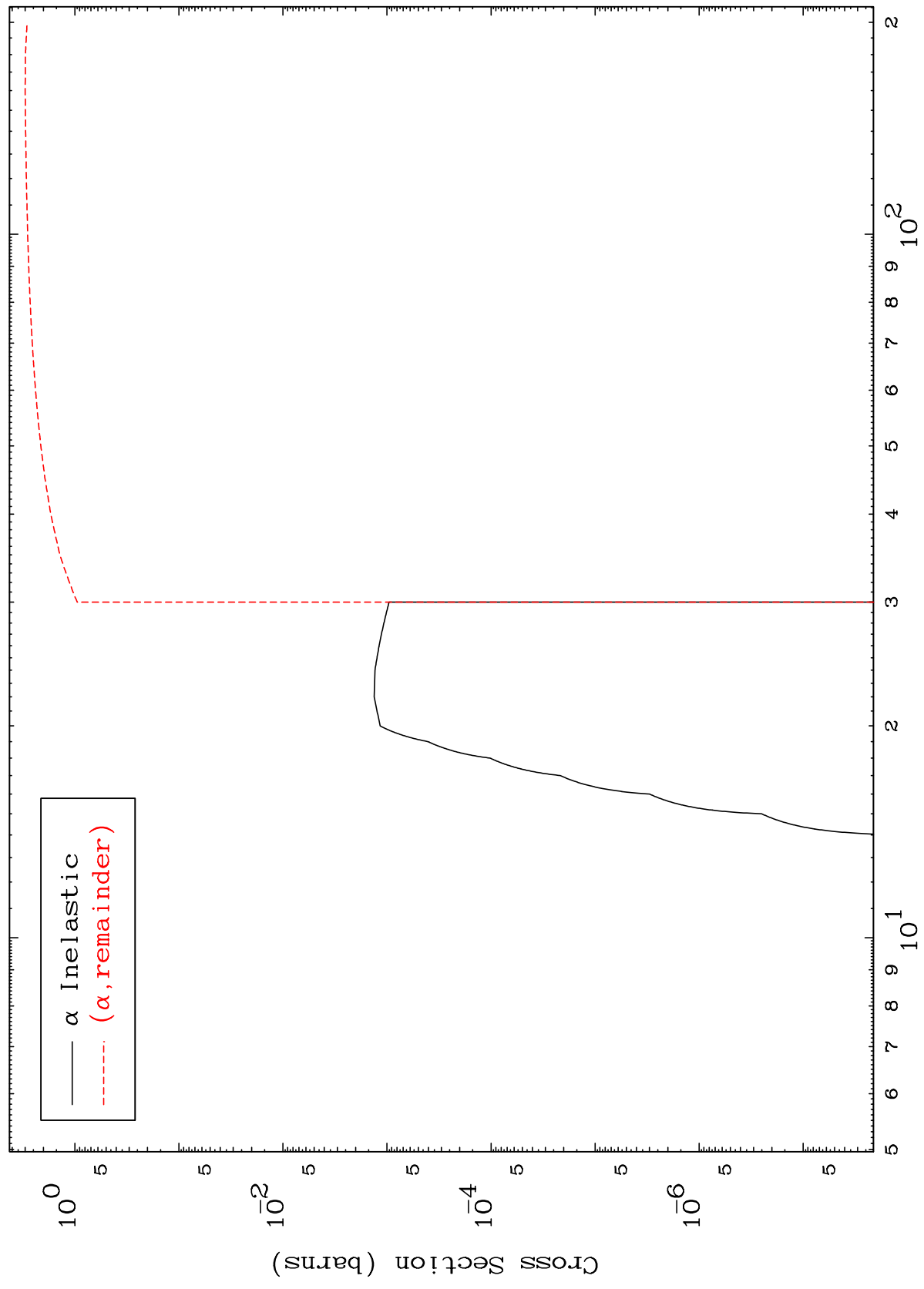
Press Mouse Button to Start

MAT 9234

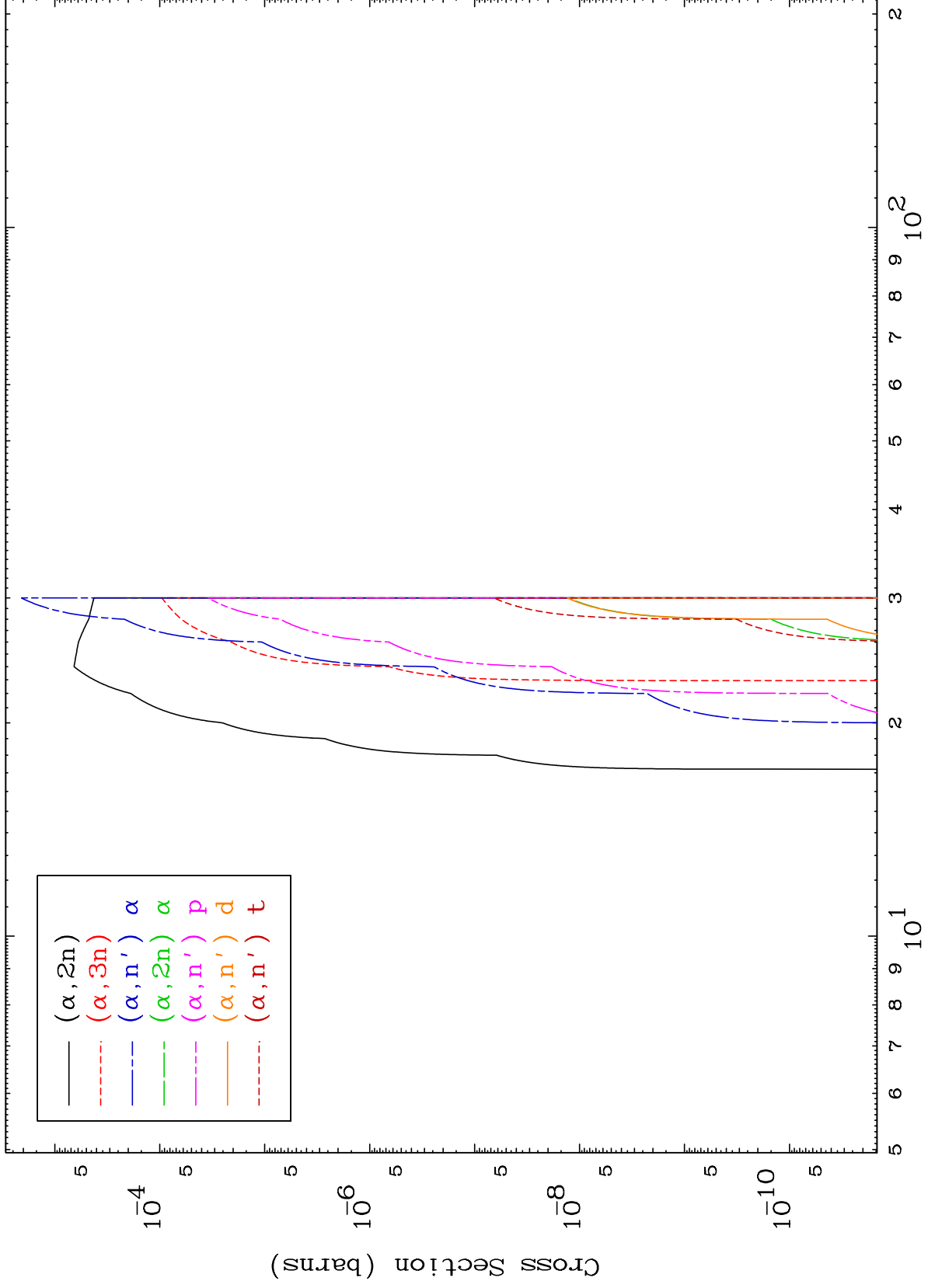
0 Kelvin

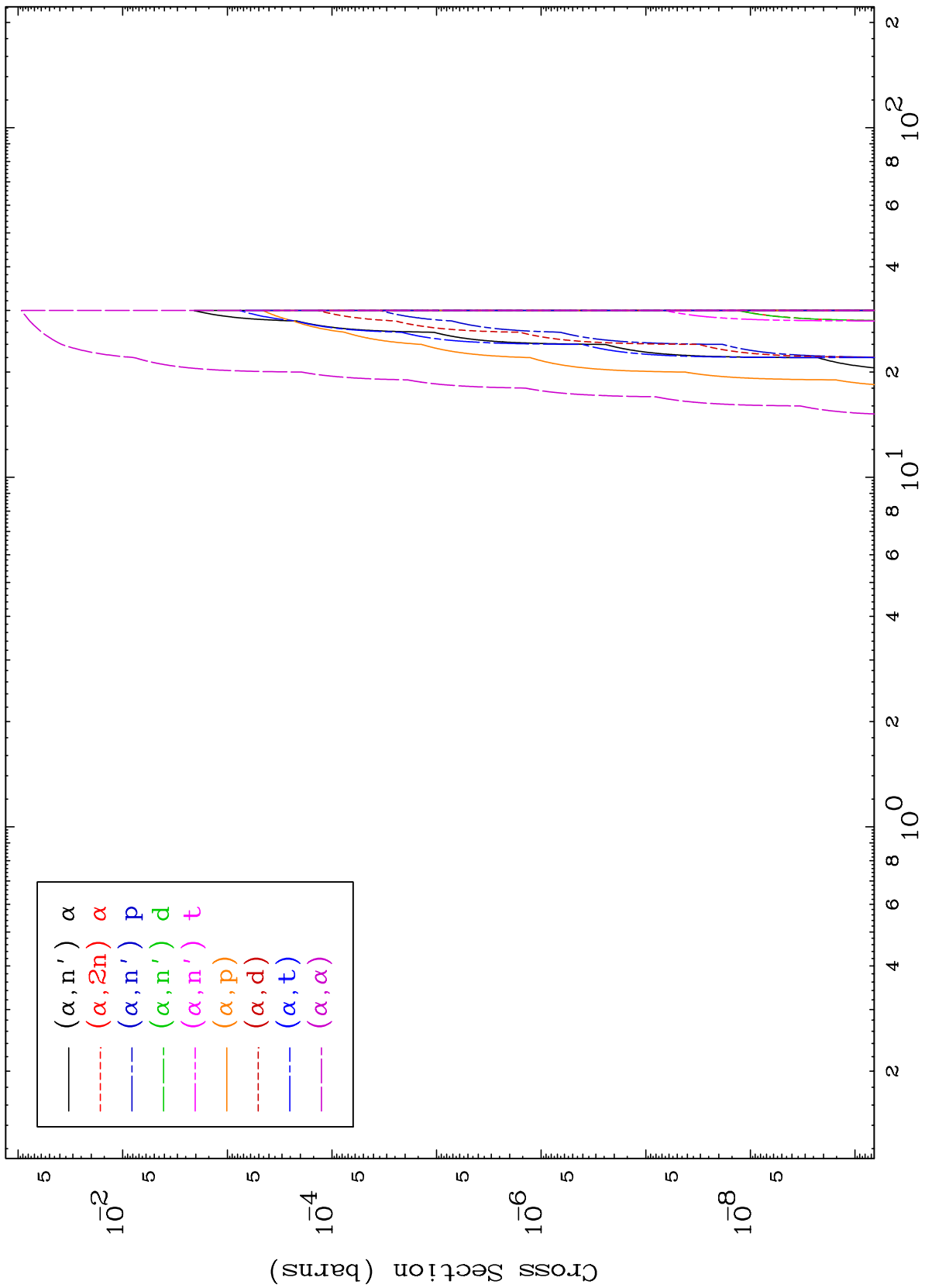
$\alpha$  Major Cross Sections

92-U -237



—  $\alpha$  Inelastic  
- - -  $\alpha$  remainder



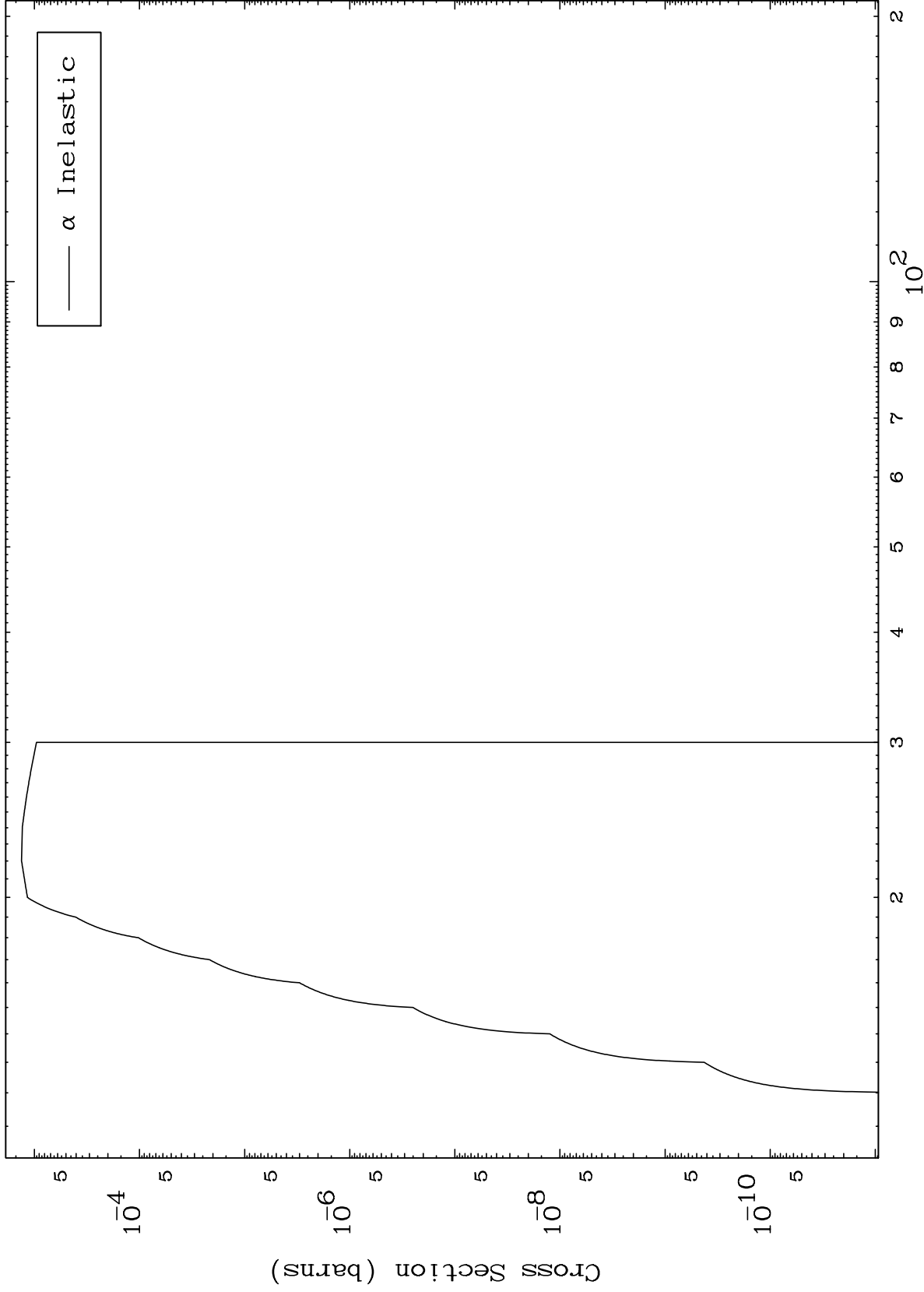


MAT 9234

( $\alpha, n'$ ) Level

92-U -237

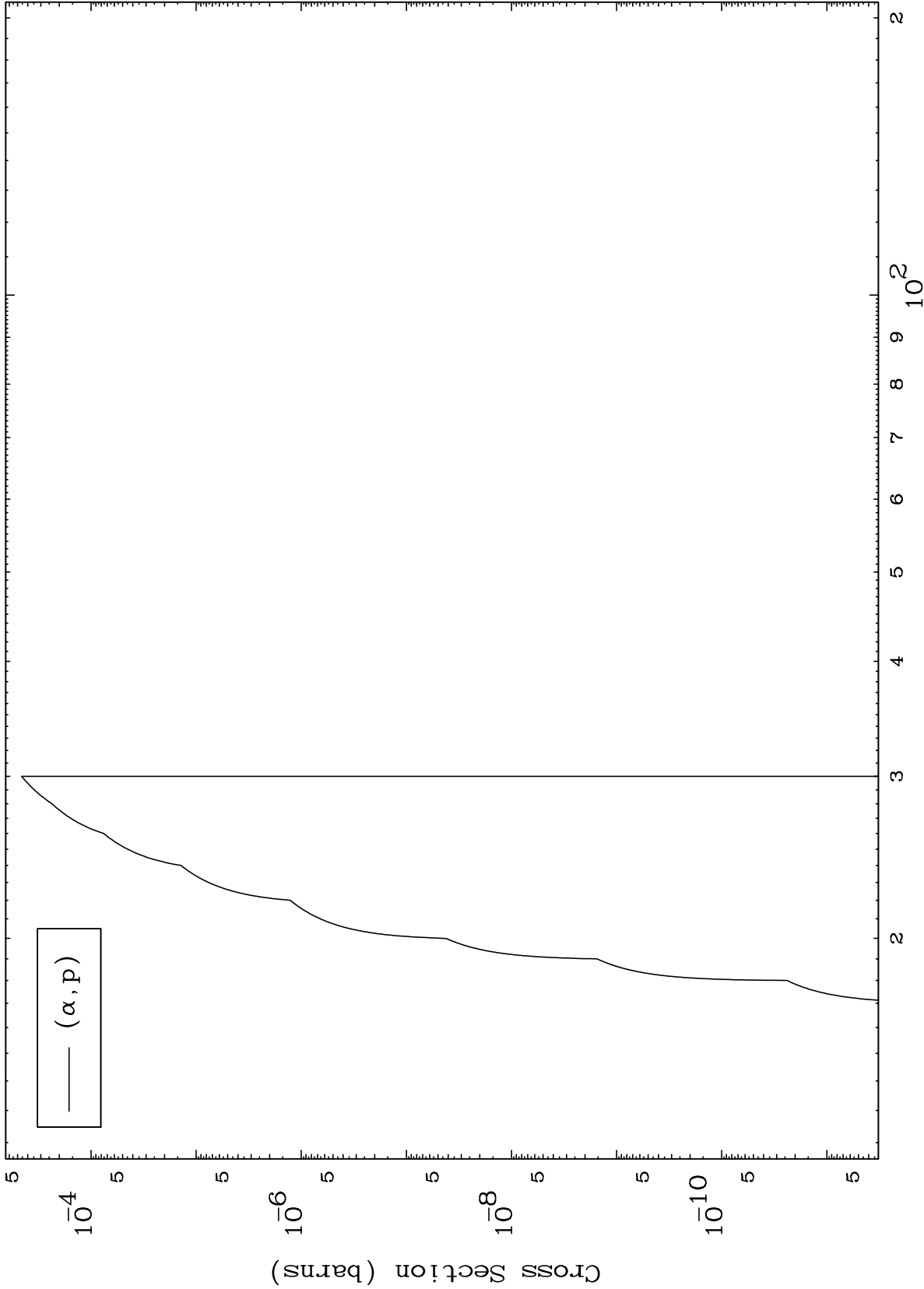
0 Kelvin Cross Sections



4

Incident Energy (MeV)

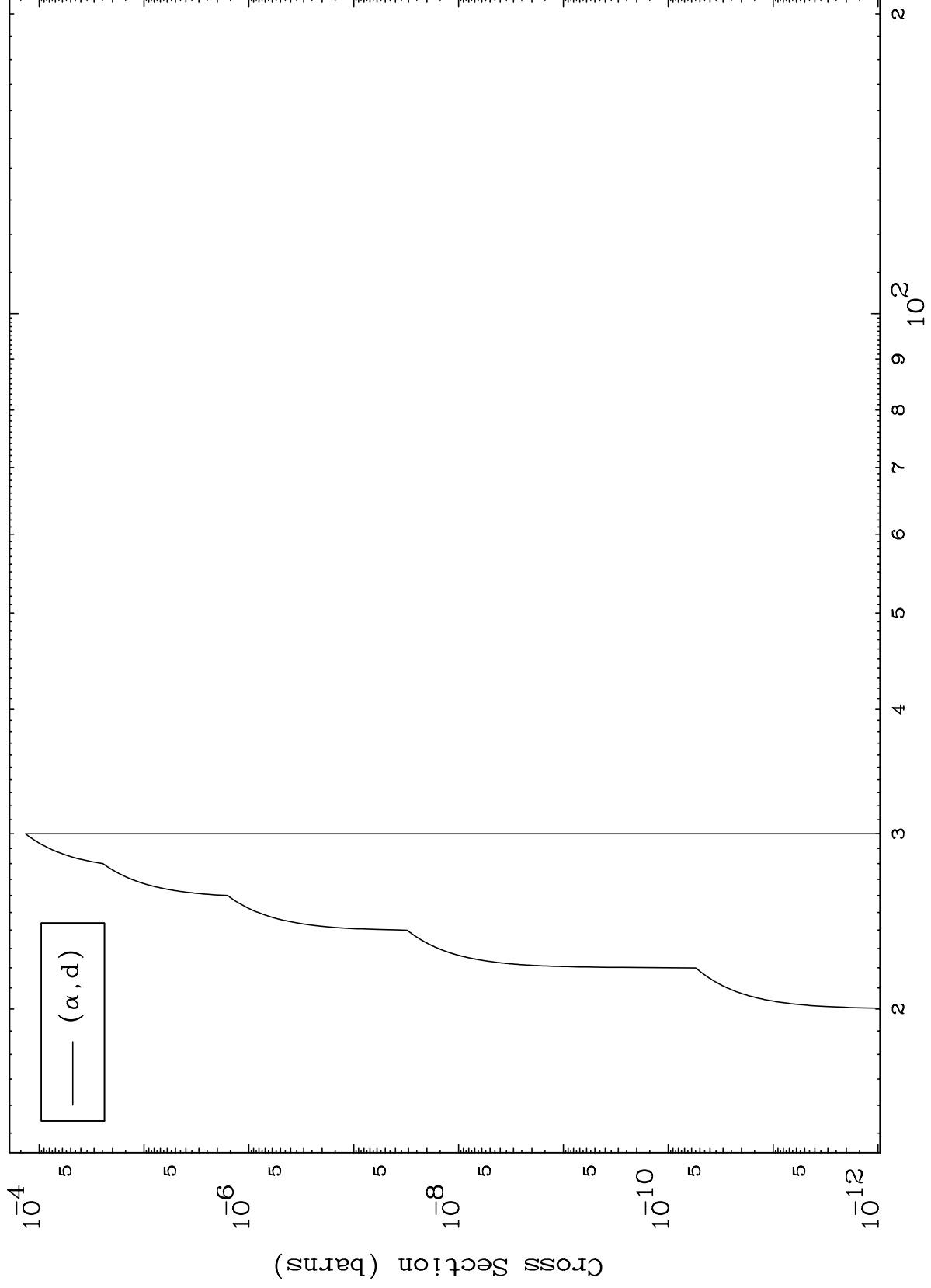
92-U -237



MAT 92334

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

92-U -237



6

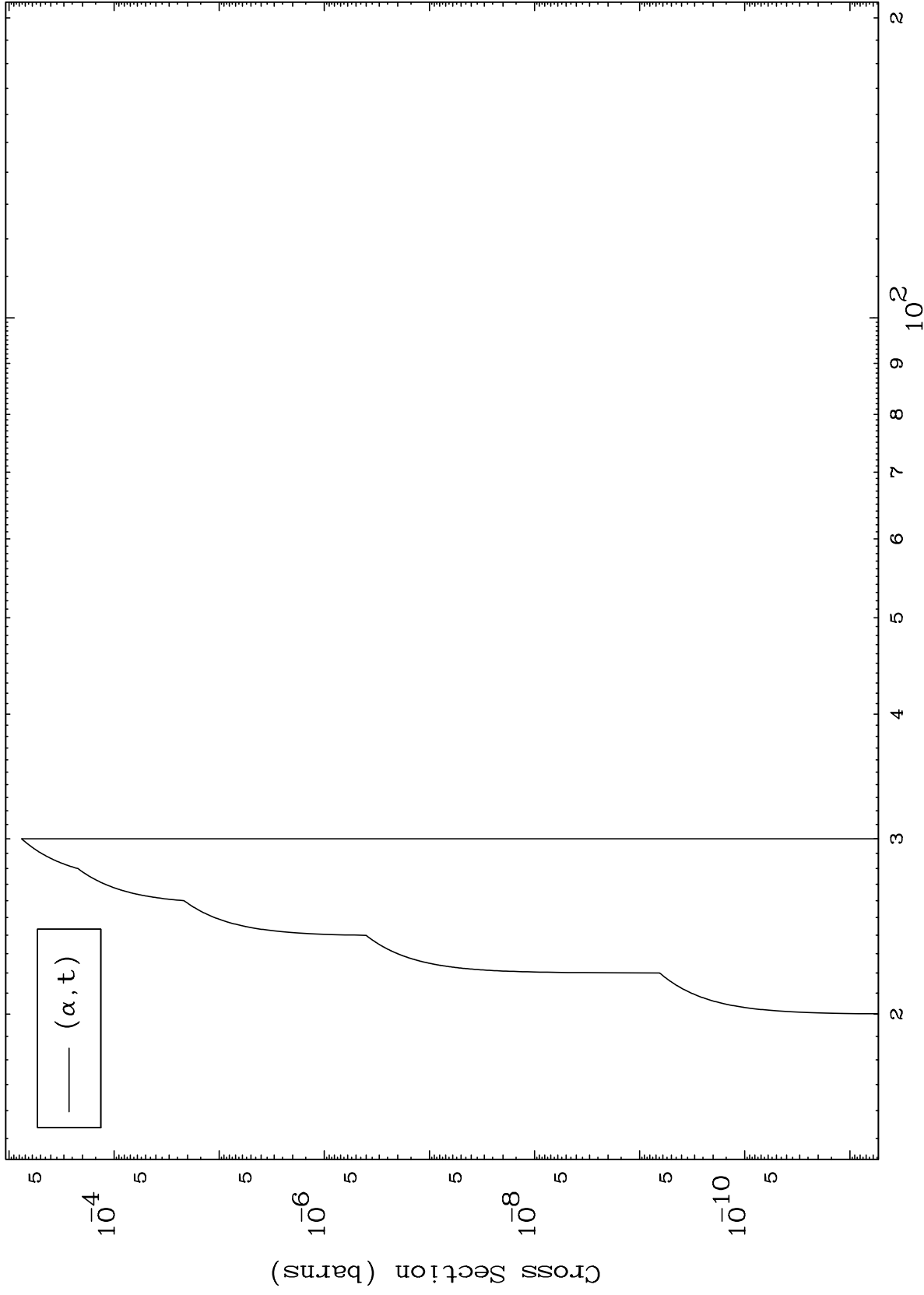
Incident Energy (MeV)

92-U -237

MAT 9234

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

92-U -237



7

Incident Energy (MeV)

92-U -237

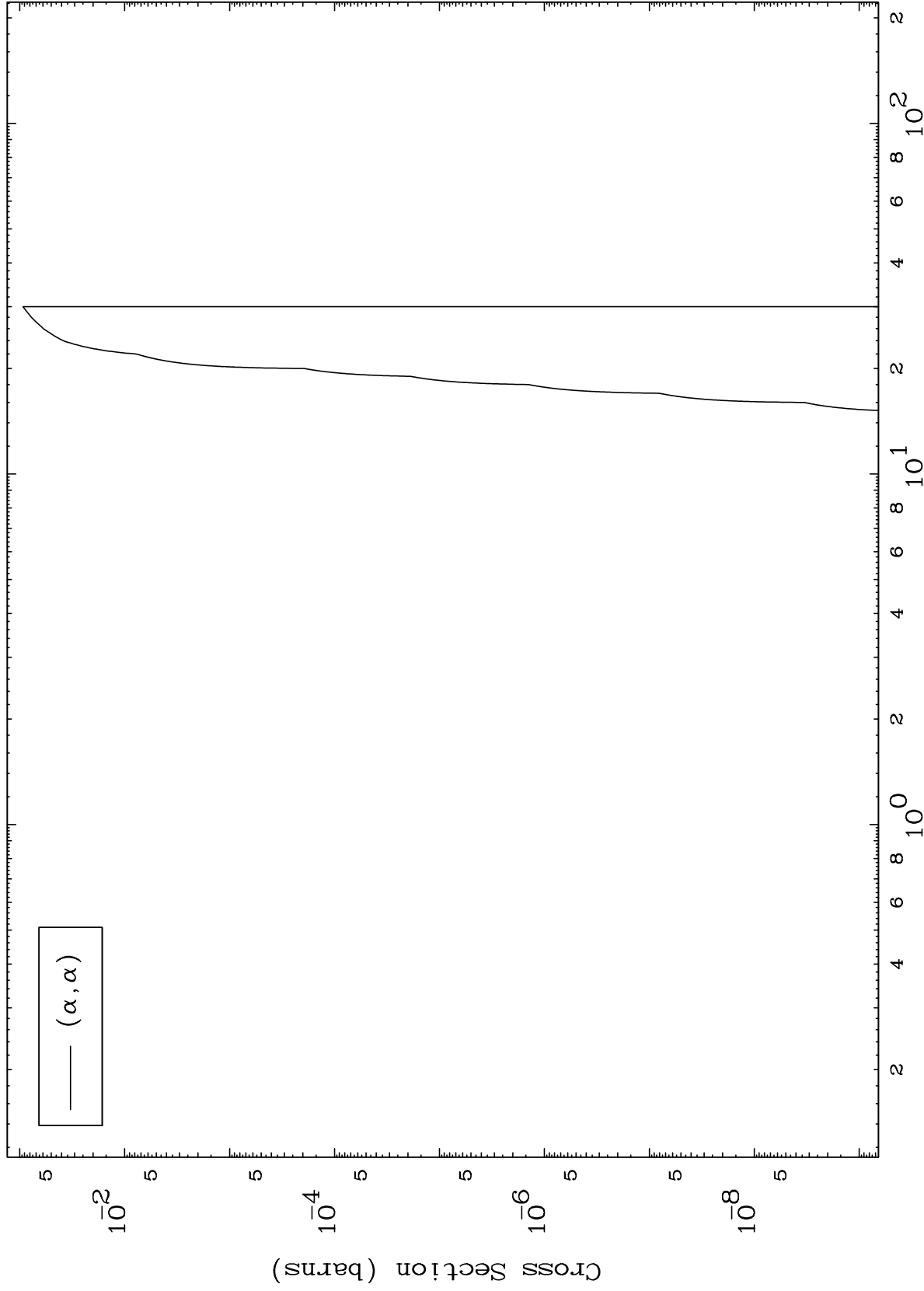


MAT 9234

( $\alpha, \alpha$ ) Levels

92-U -237

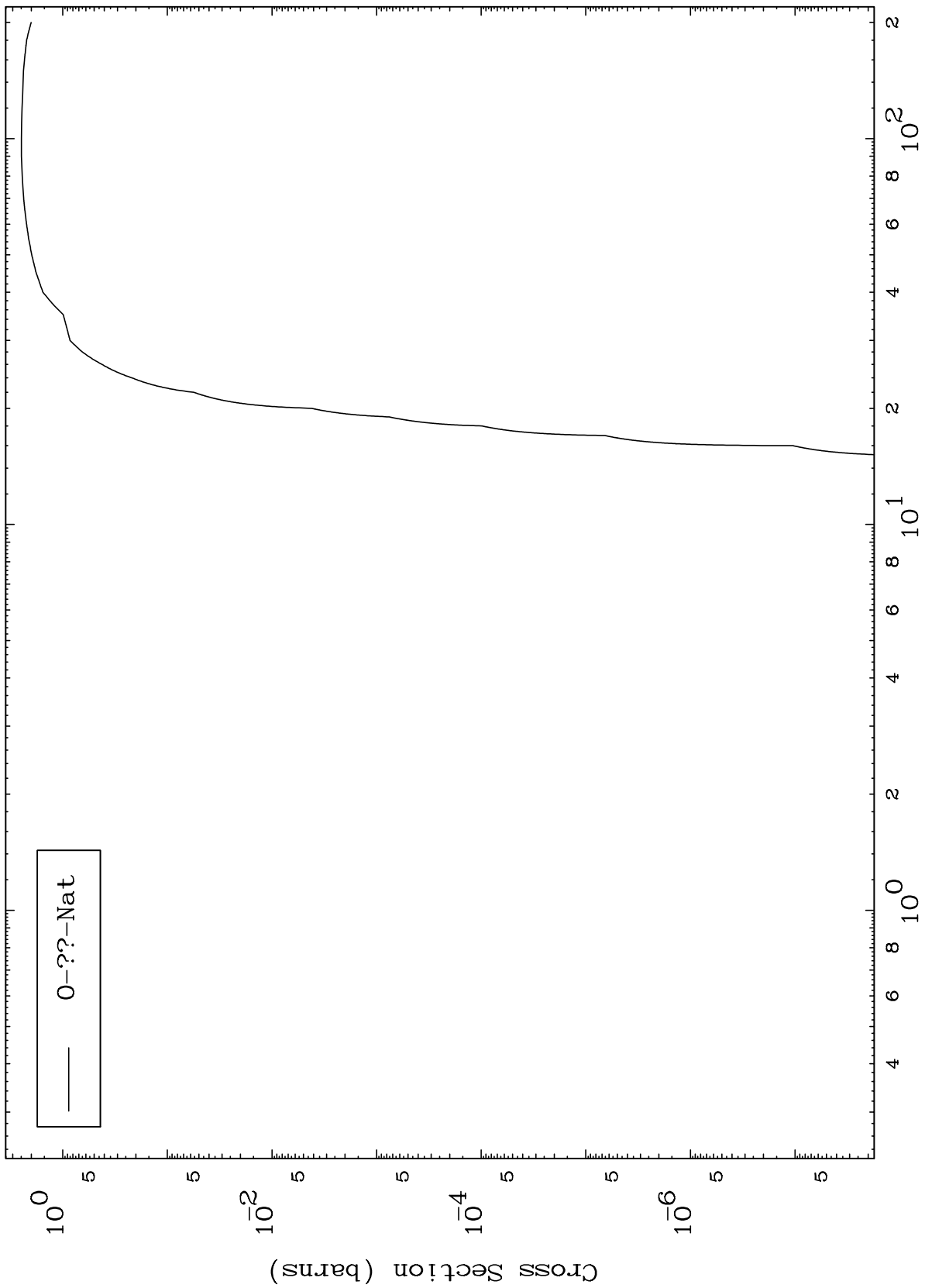
0 Kelvin Cross Sections

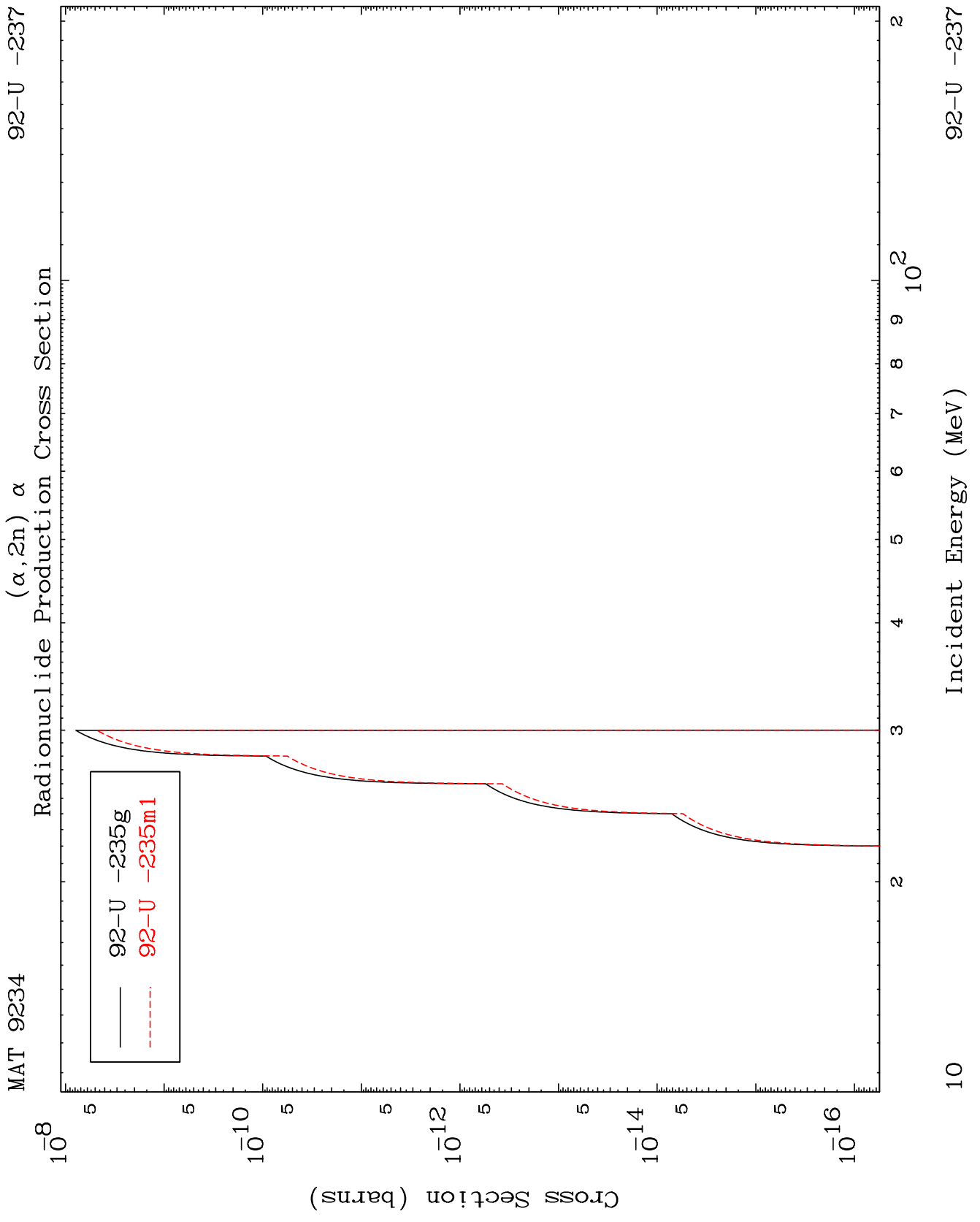


MAT 9234

92-U -237

$\alpha$  Fission  
Radionuclide Production Cross Section





( $\alpha, p$ )  
Radionuclide Production Cross Section

