

Program EVALPLOT
(Version 2018-1)

by

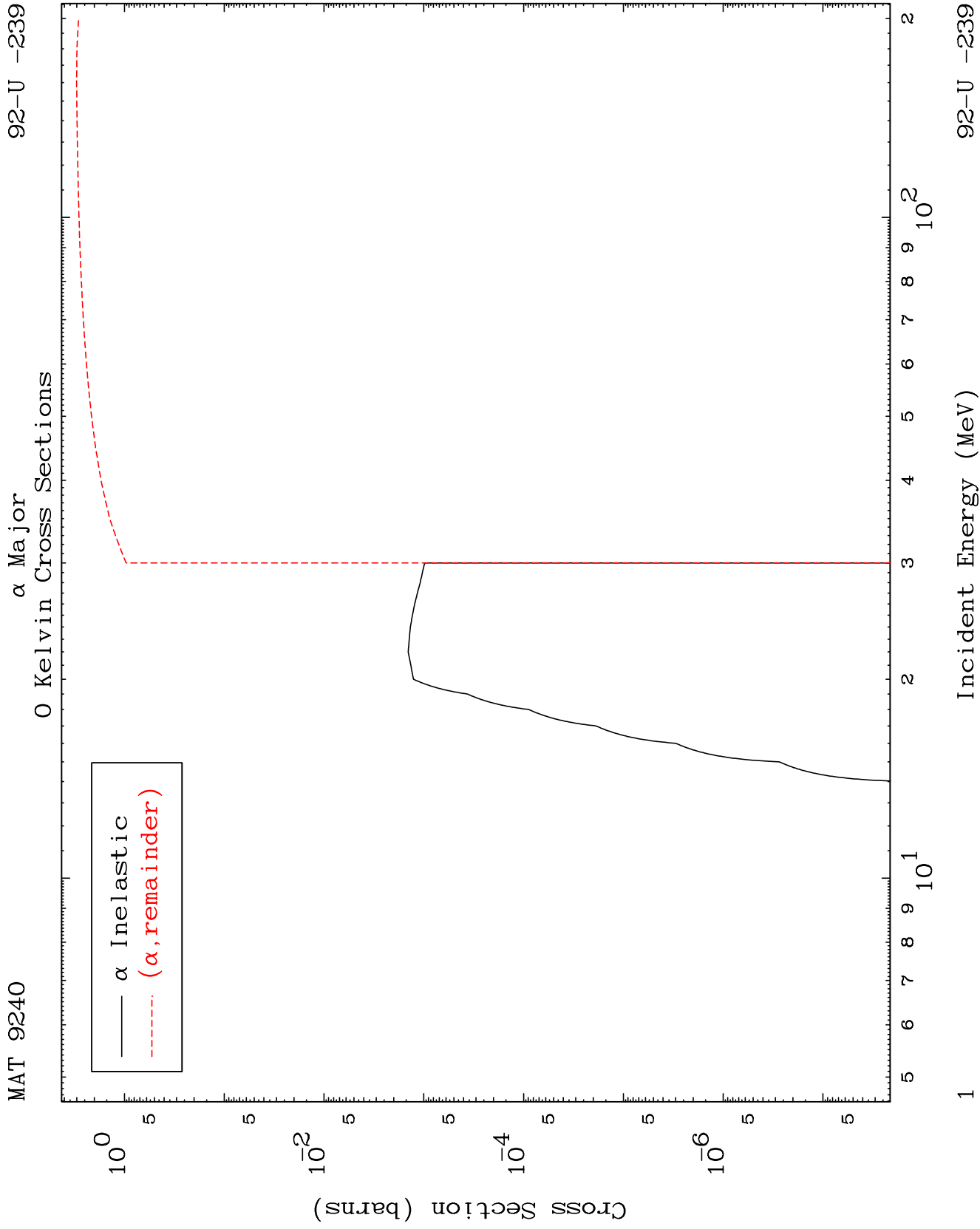
Dermott E. Cullen
(Present Contact Information)

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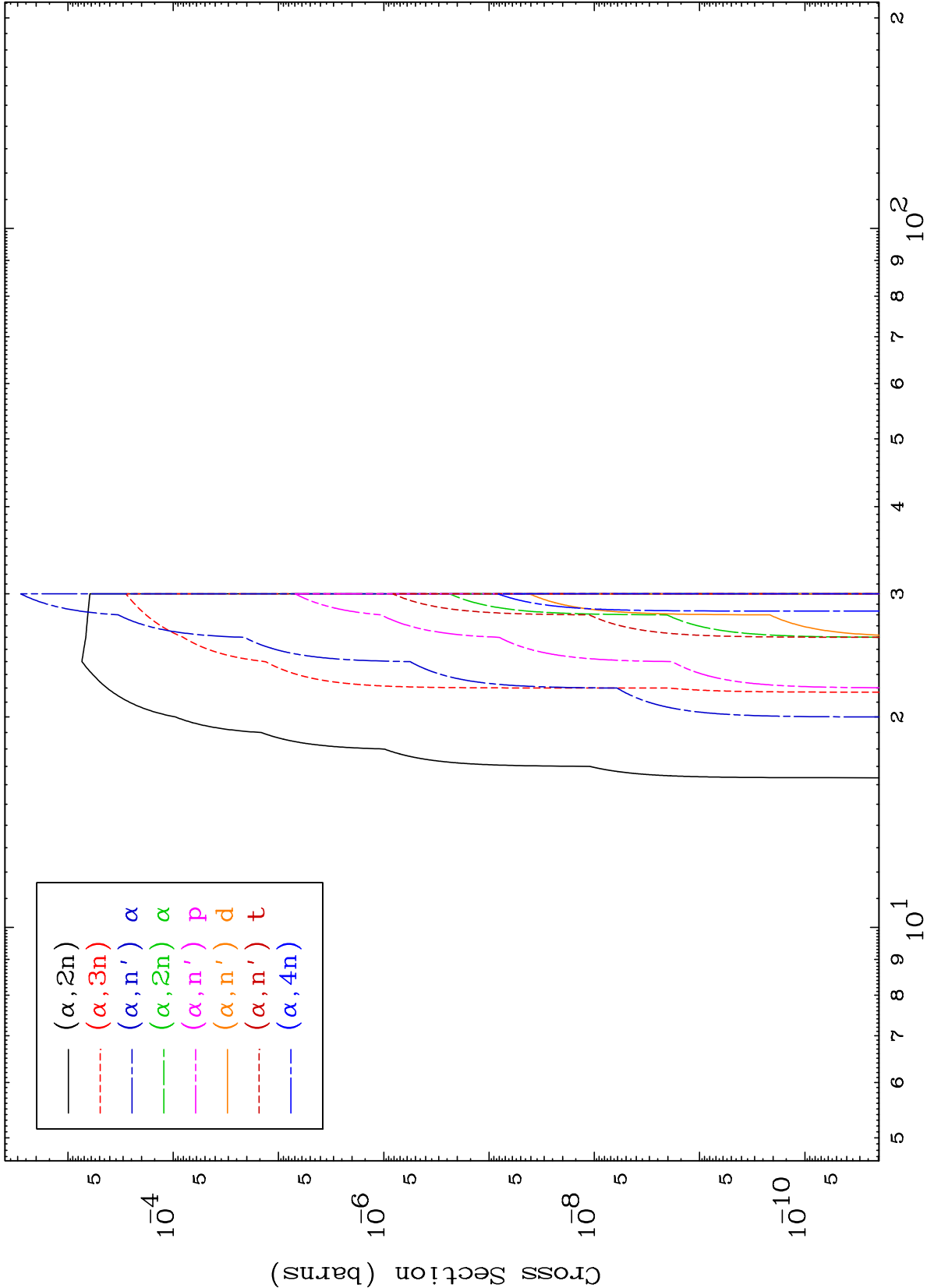
Press Mouse Button to Start



MAT 9240

α Neutron Production
0 Kelvin Cross Sections

92-U -239

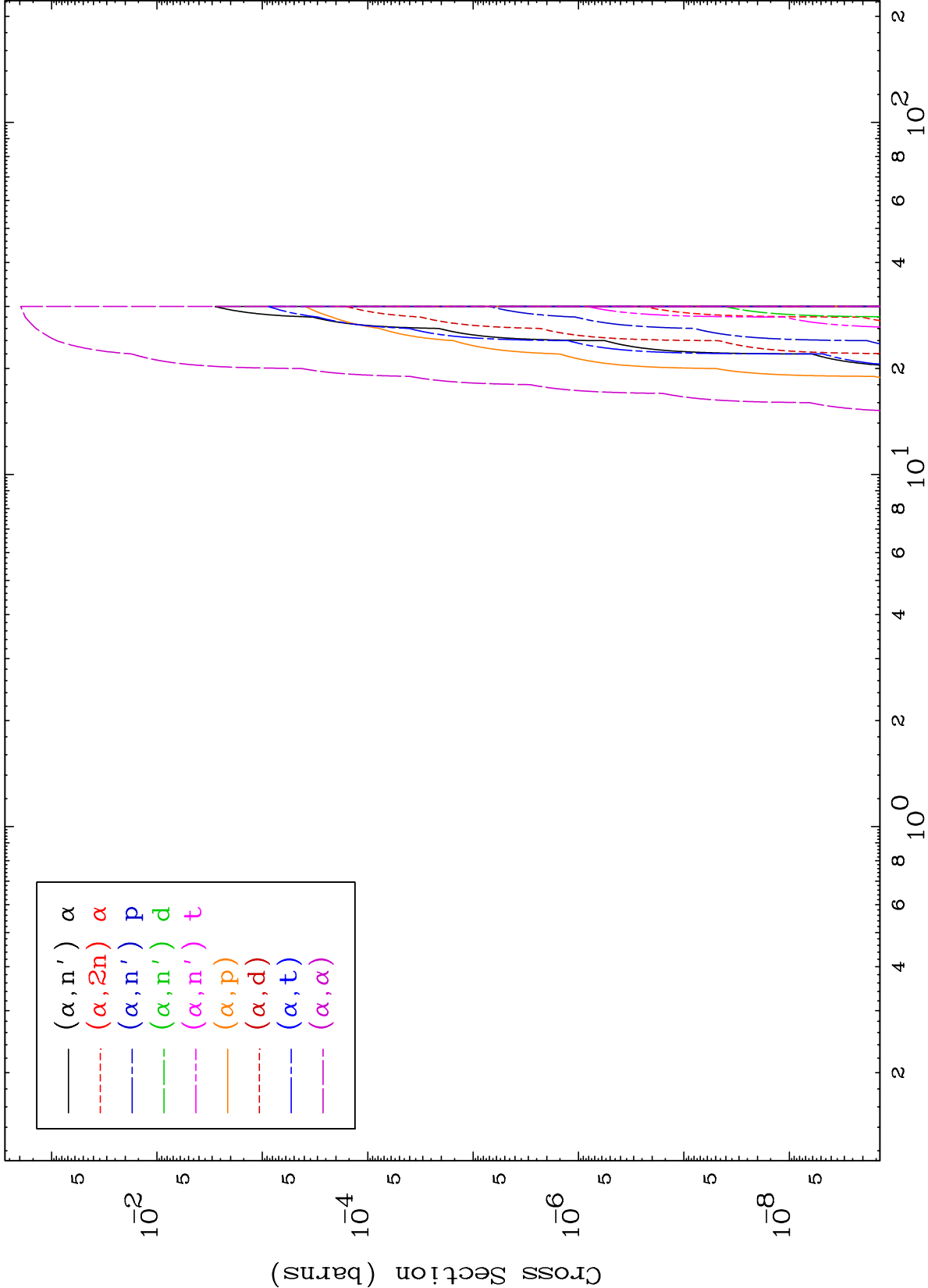


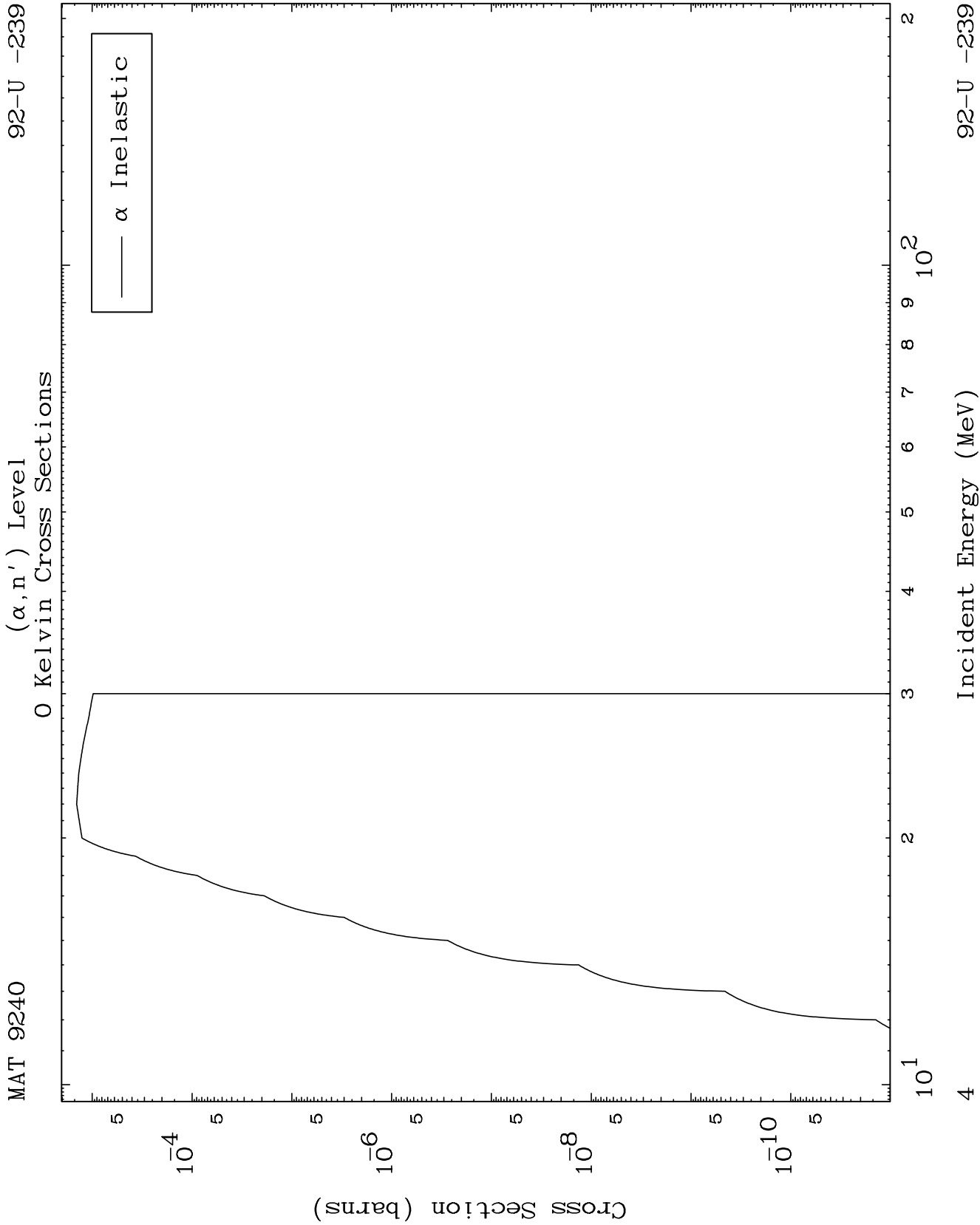
2

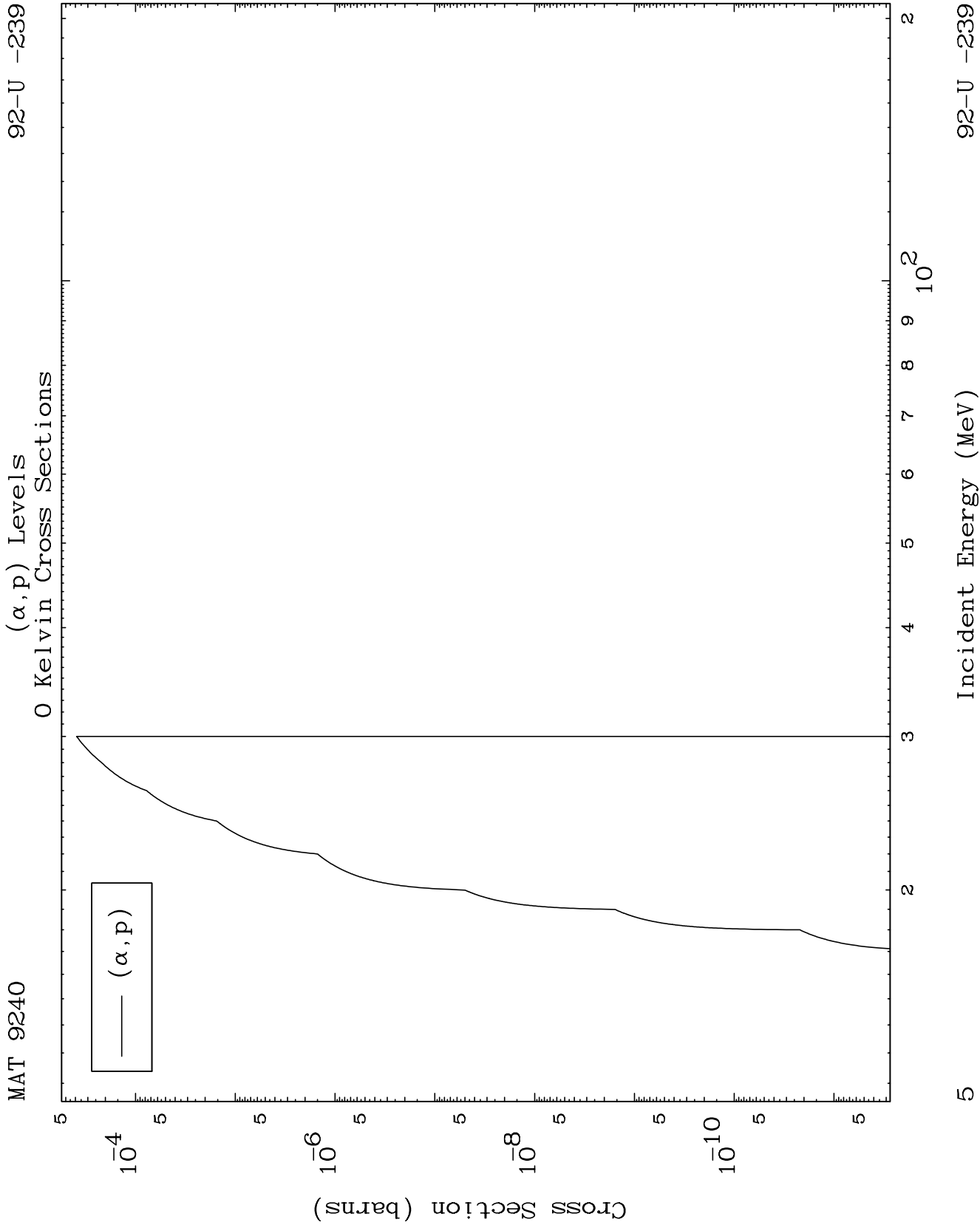
Incident Energy (MeV)

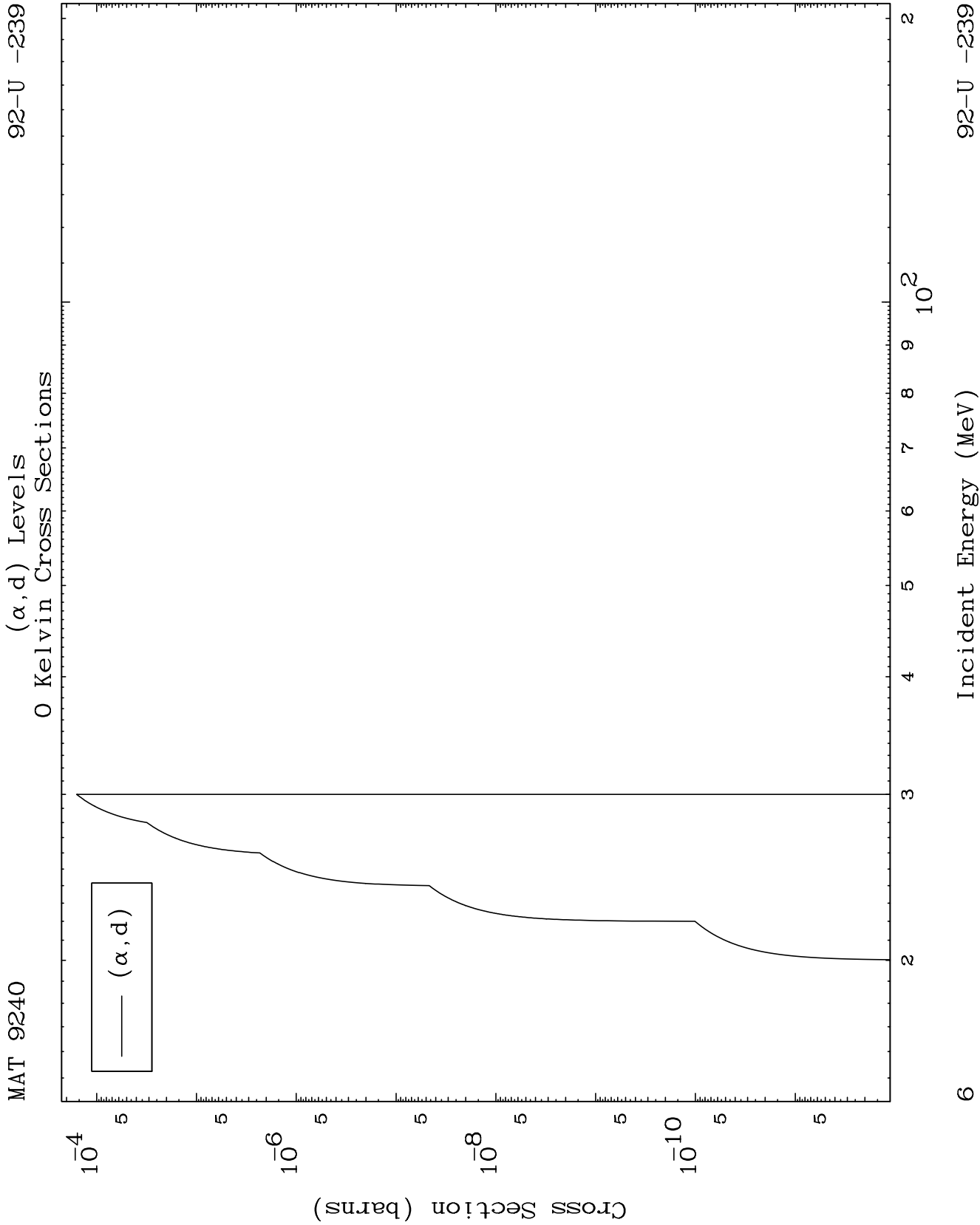
92-U -239

0 Kelvin Cross Sections





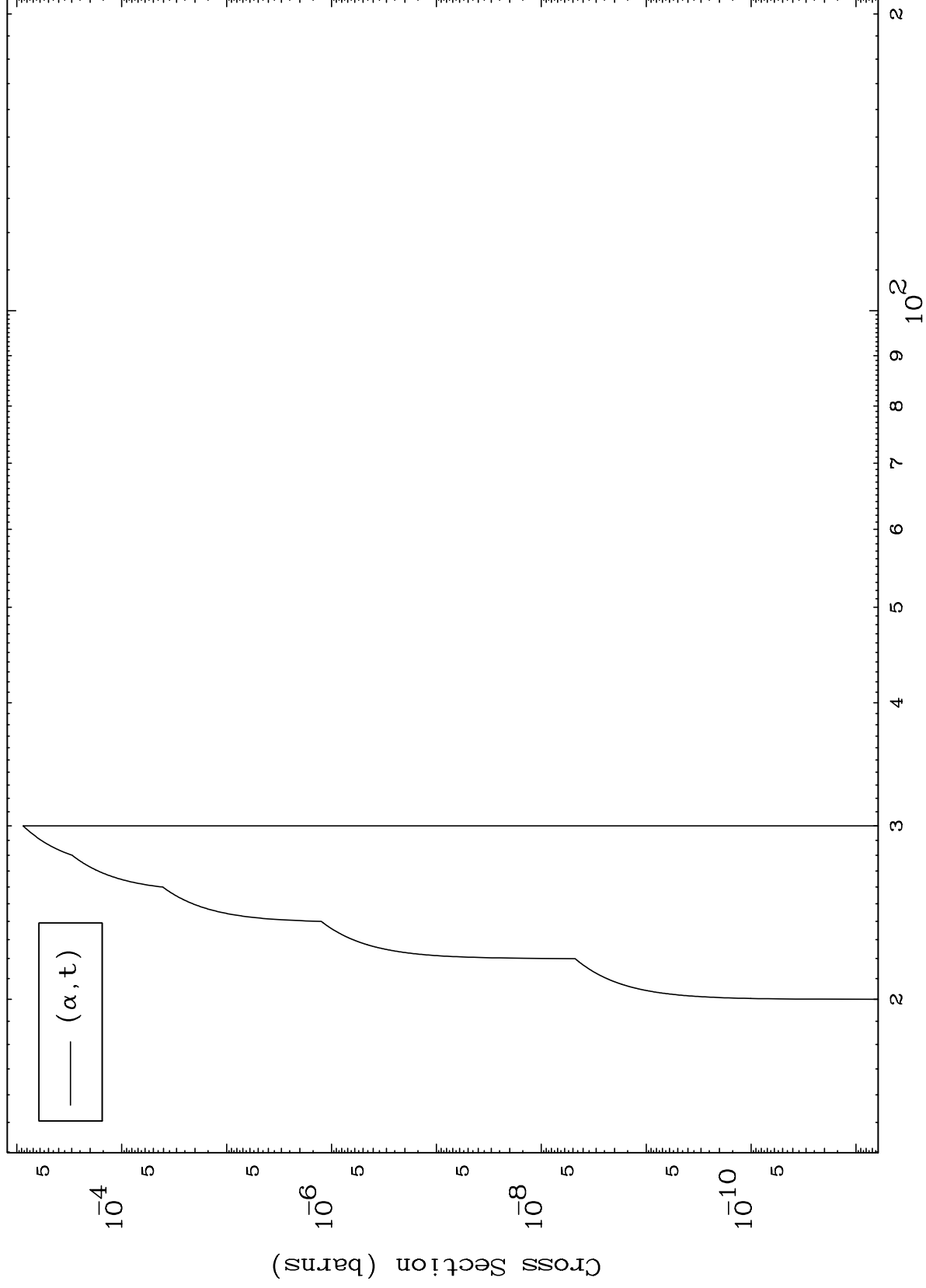




MAT 9240

92-U -239

(α, t) Levels
0 Kelvin Cross Sections



7

92-U -239

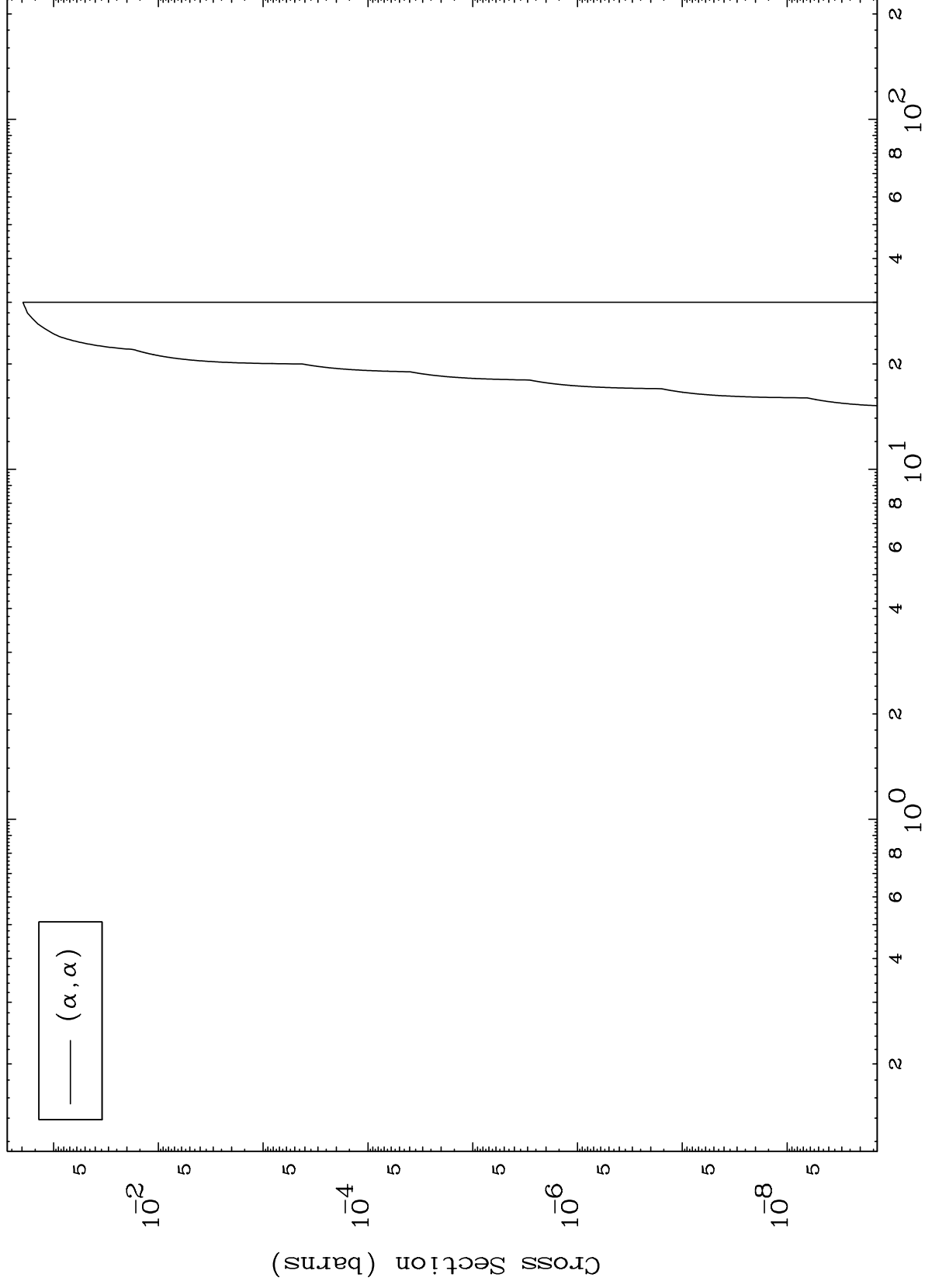
Incident Energy (MeV)

MAT 9240

(α, α) Levels

92-U -239

0 Kelvin Cross Sections



8

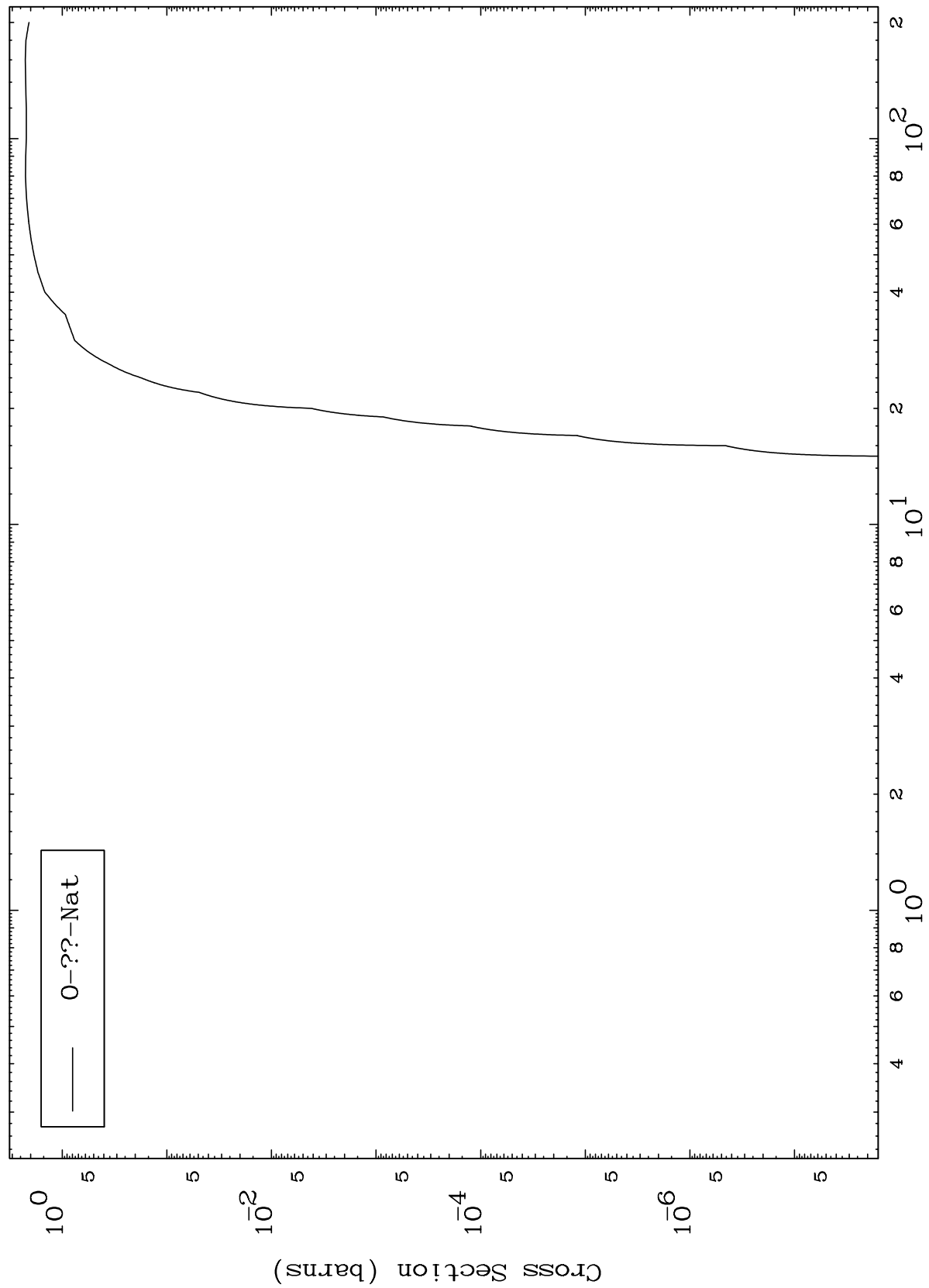
Incident Energy (MeV)

92-U -239

MAT 9240

92-U -239

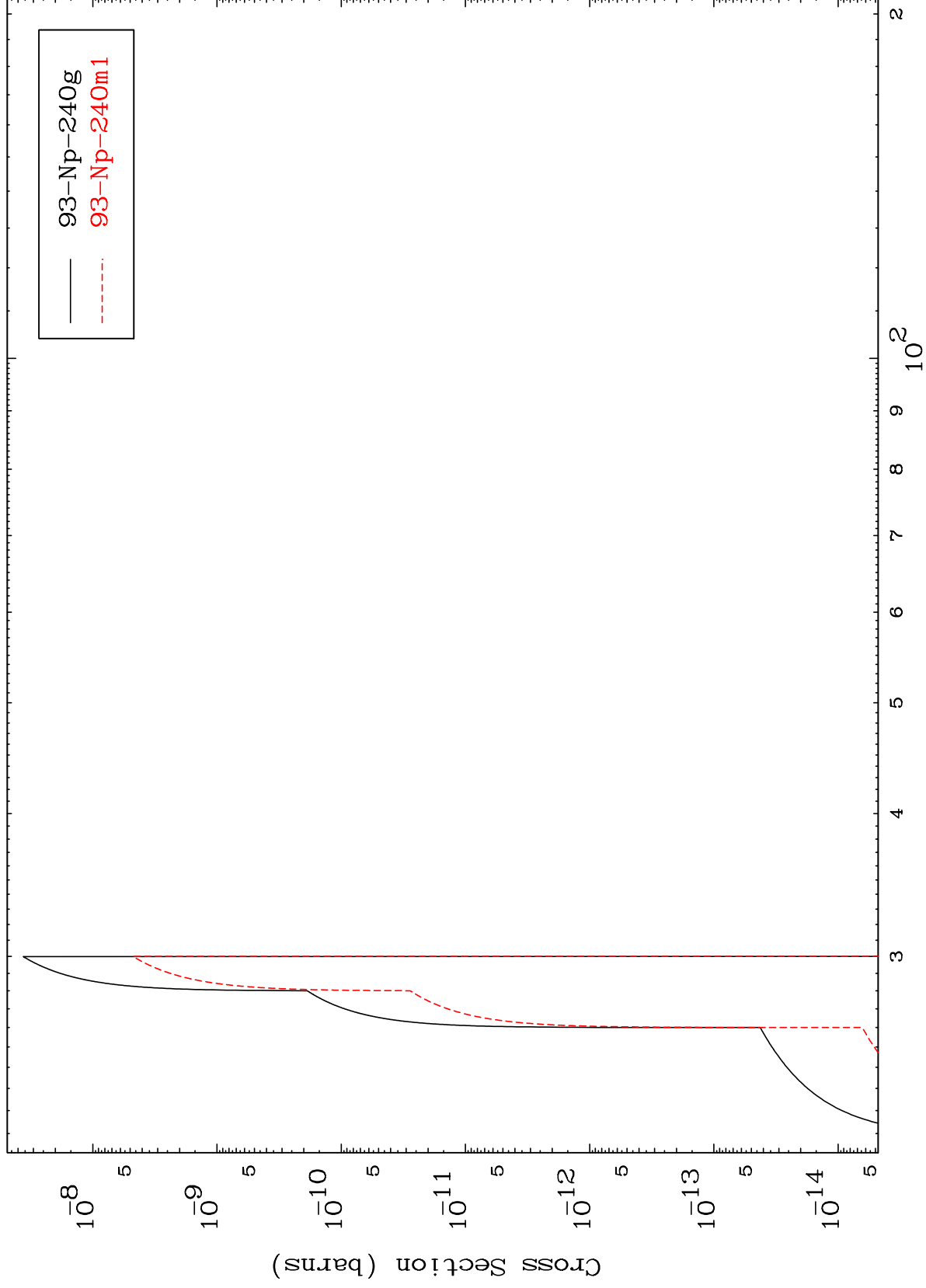
α Fission
Radionuclide Production Cross Section



MAT 9240

92-U -239

(α, n') d
Radionuclide Production Cross Section



92-U -239

Incident Energy (MeV)

10

