

Program EVALPLOT  
(Version 2018-1)

by

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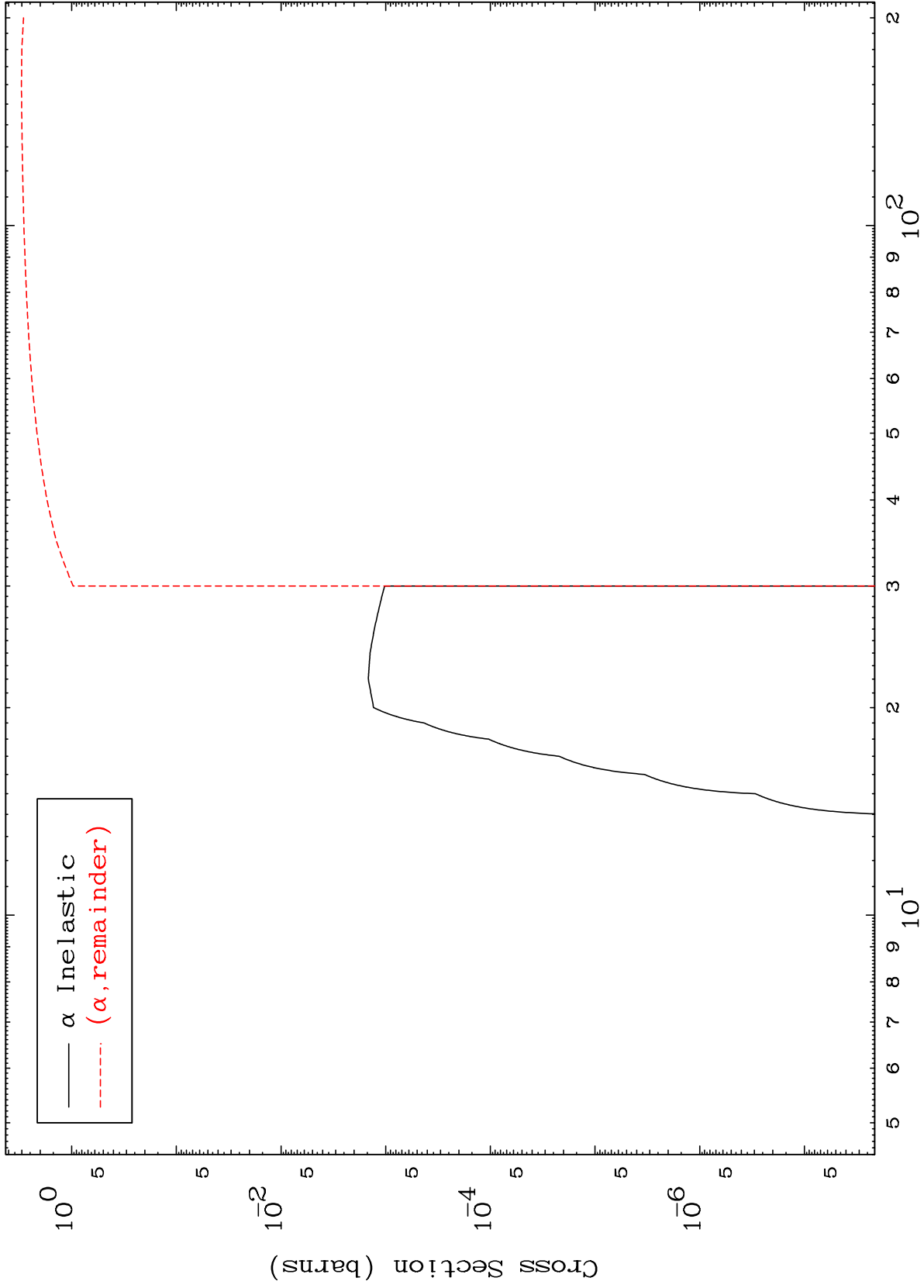
Press Mouse Button to Start

MAT 9243

$\alpha$  Major

92-U -240

0 Kelvin Cross Sections



1

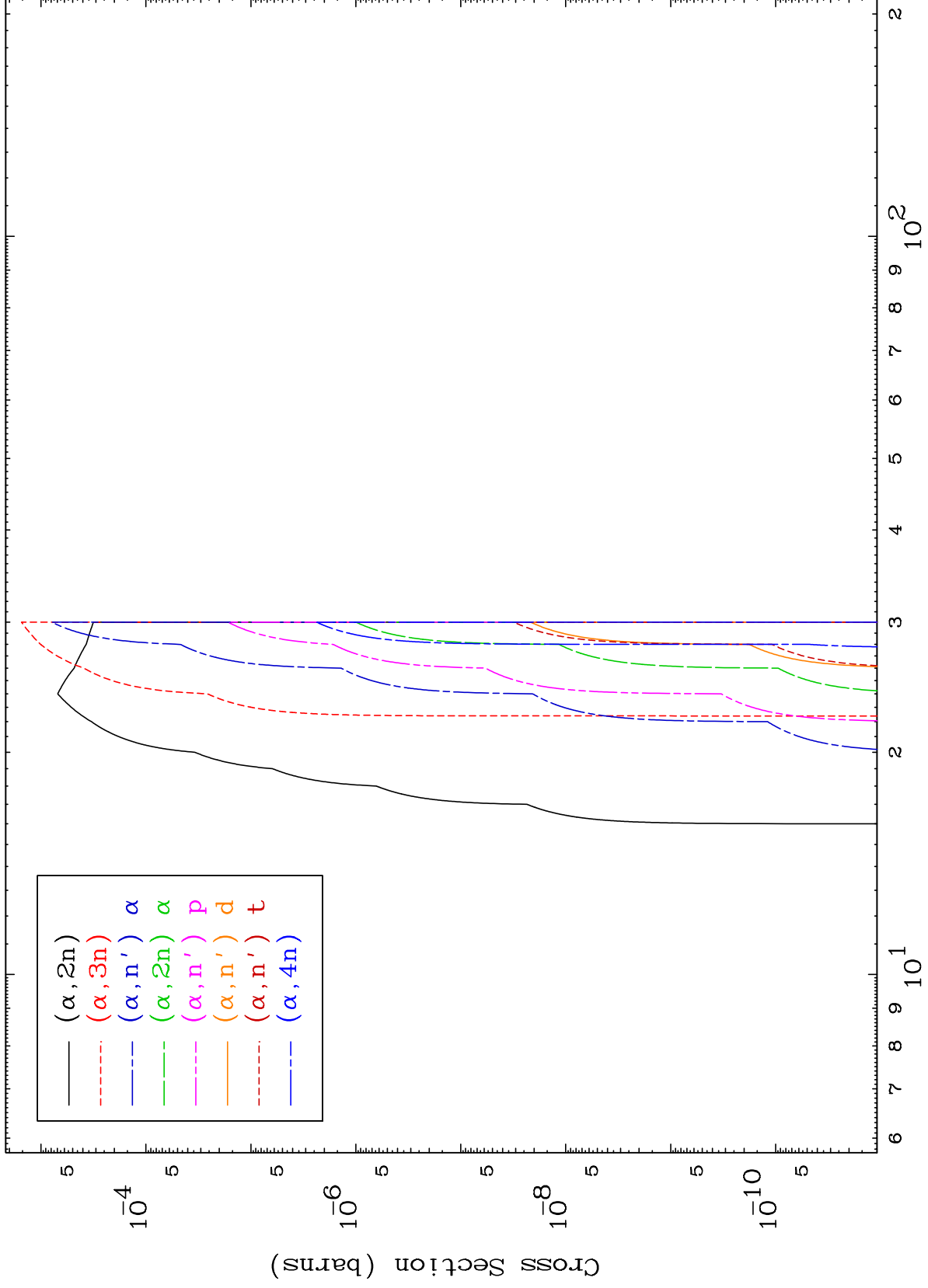
Incident Energy (MeV)

92-U -240

MAT 9243

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

92-U -240



2

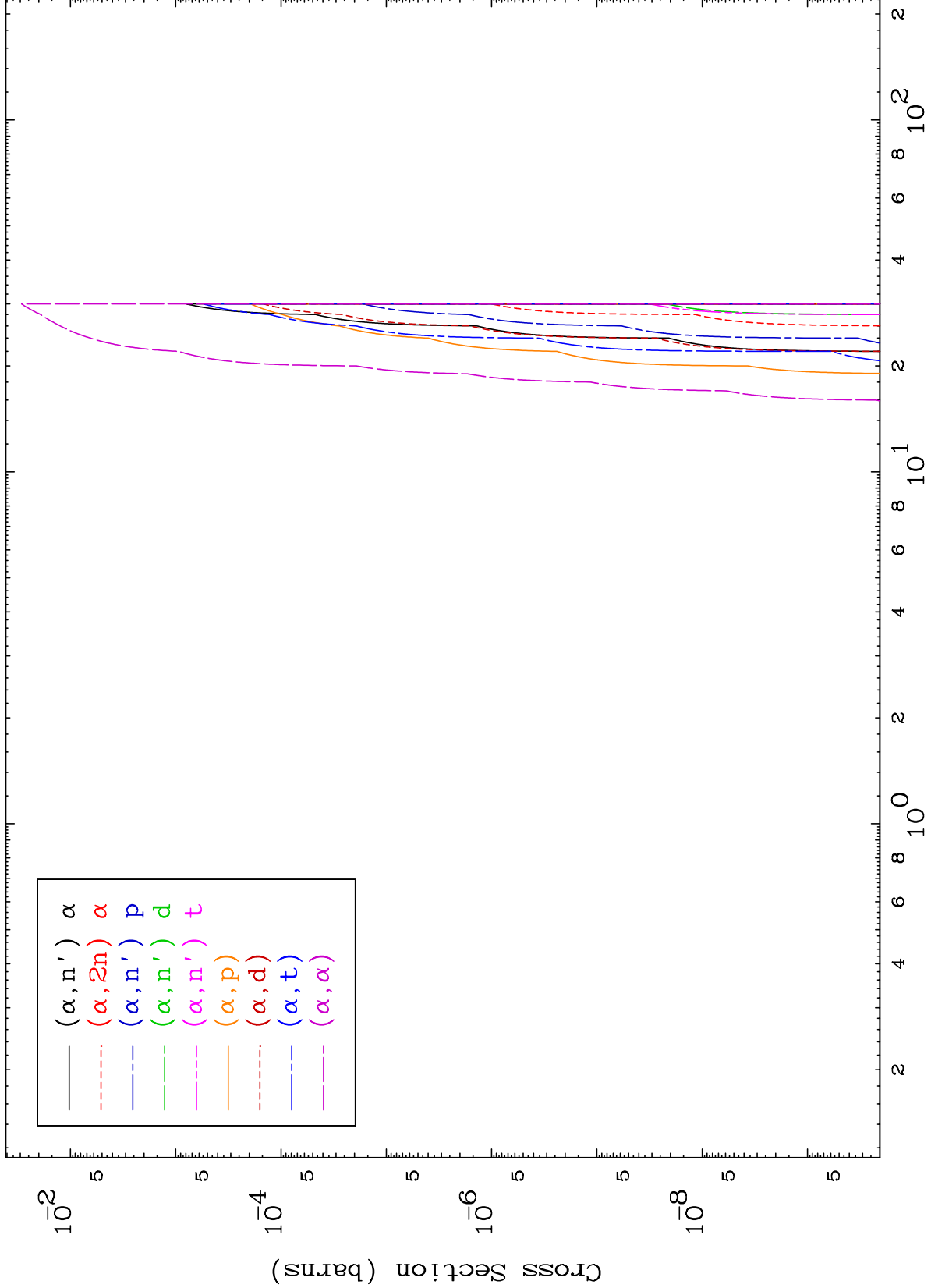
Incident Energy (MeV)

92-U -240

MAT 9243

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

92-U -240



Incident Energy (MeV)

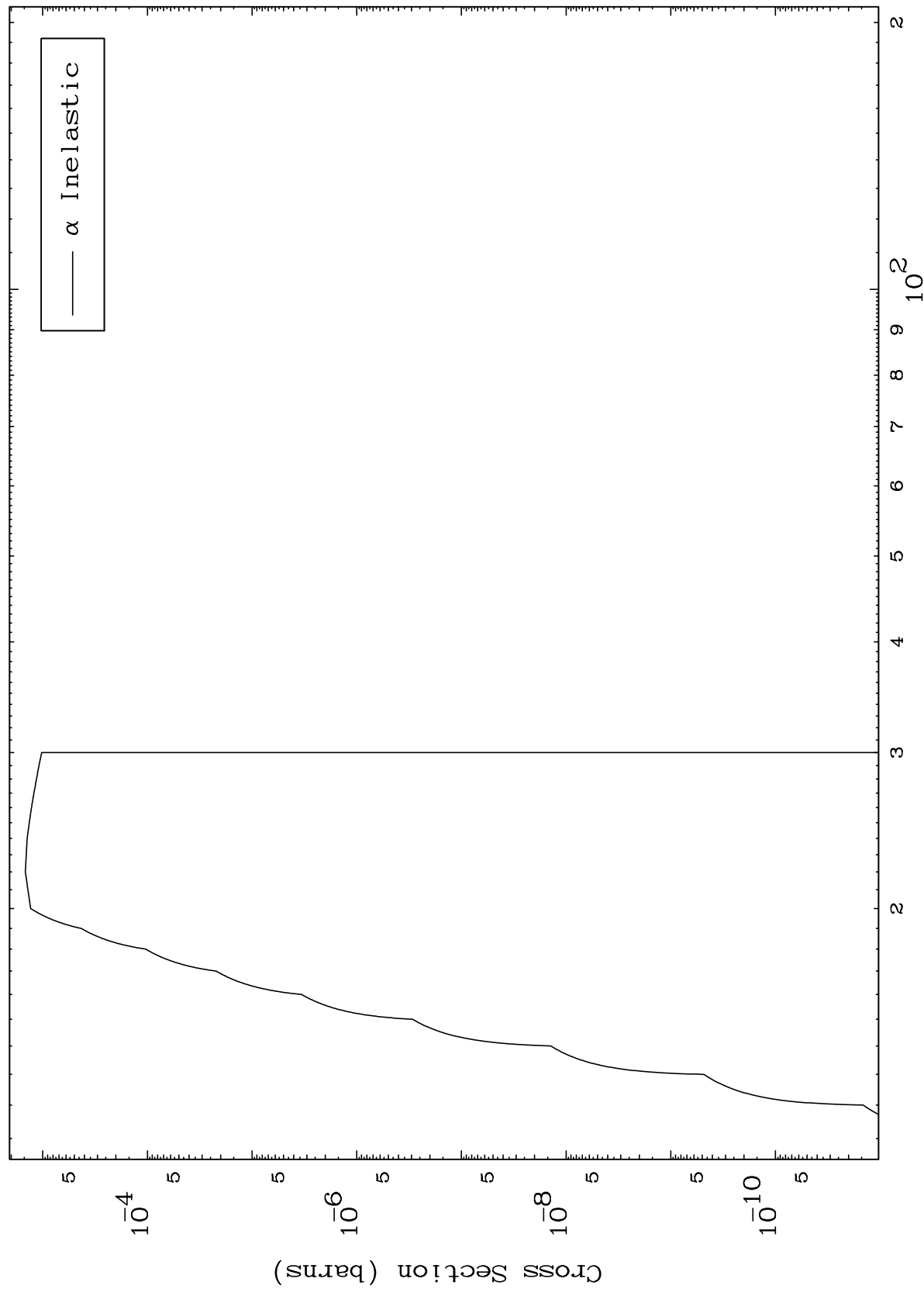
92-U -240

MAT 9243

( $\alpha, n'$ ) Level

92-U -240

0 Kelvin Cross Sections



Incident Energy (MeV)

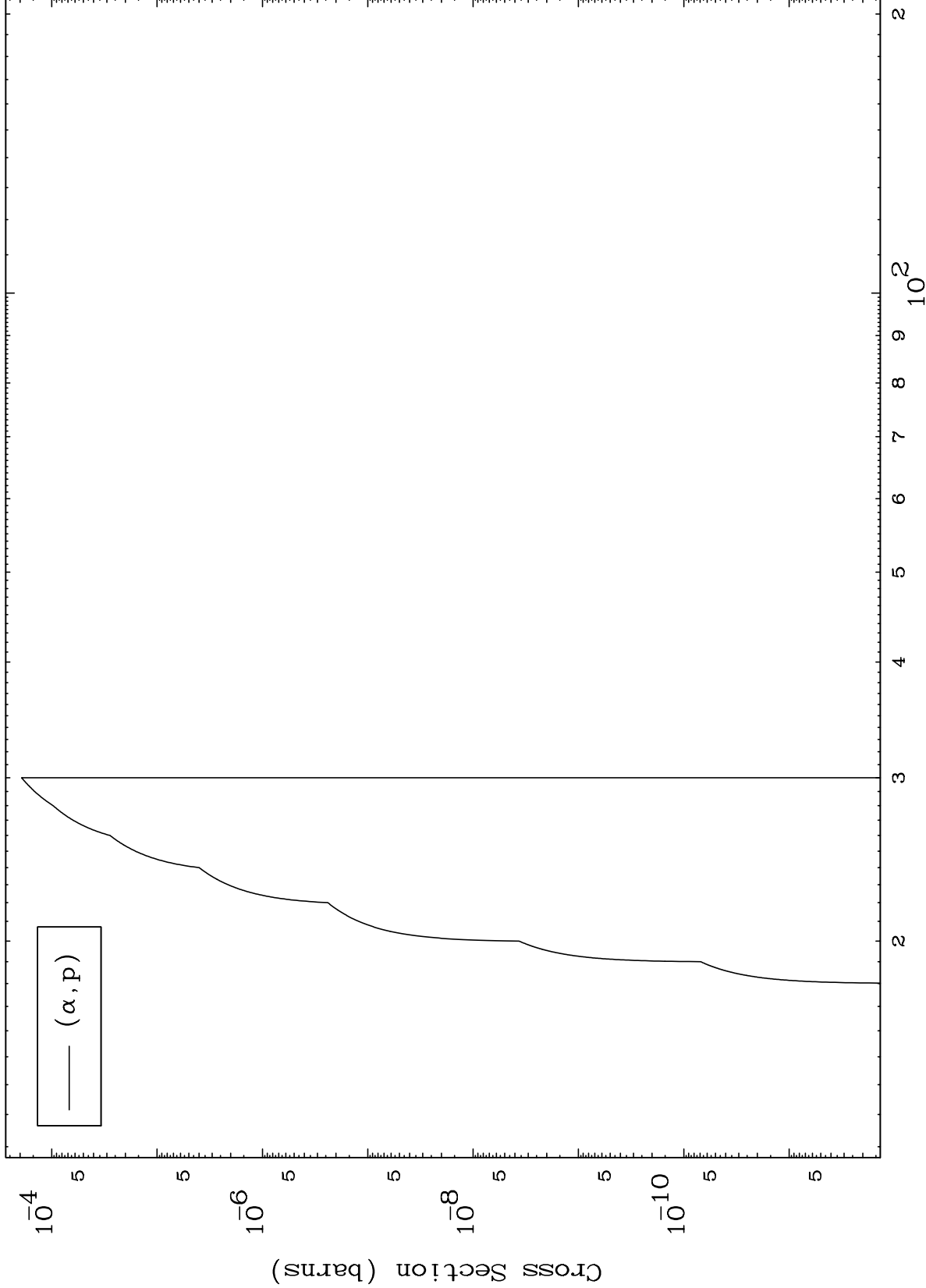
92-U -240

4

MAT 9243

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

92-U -240



5

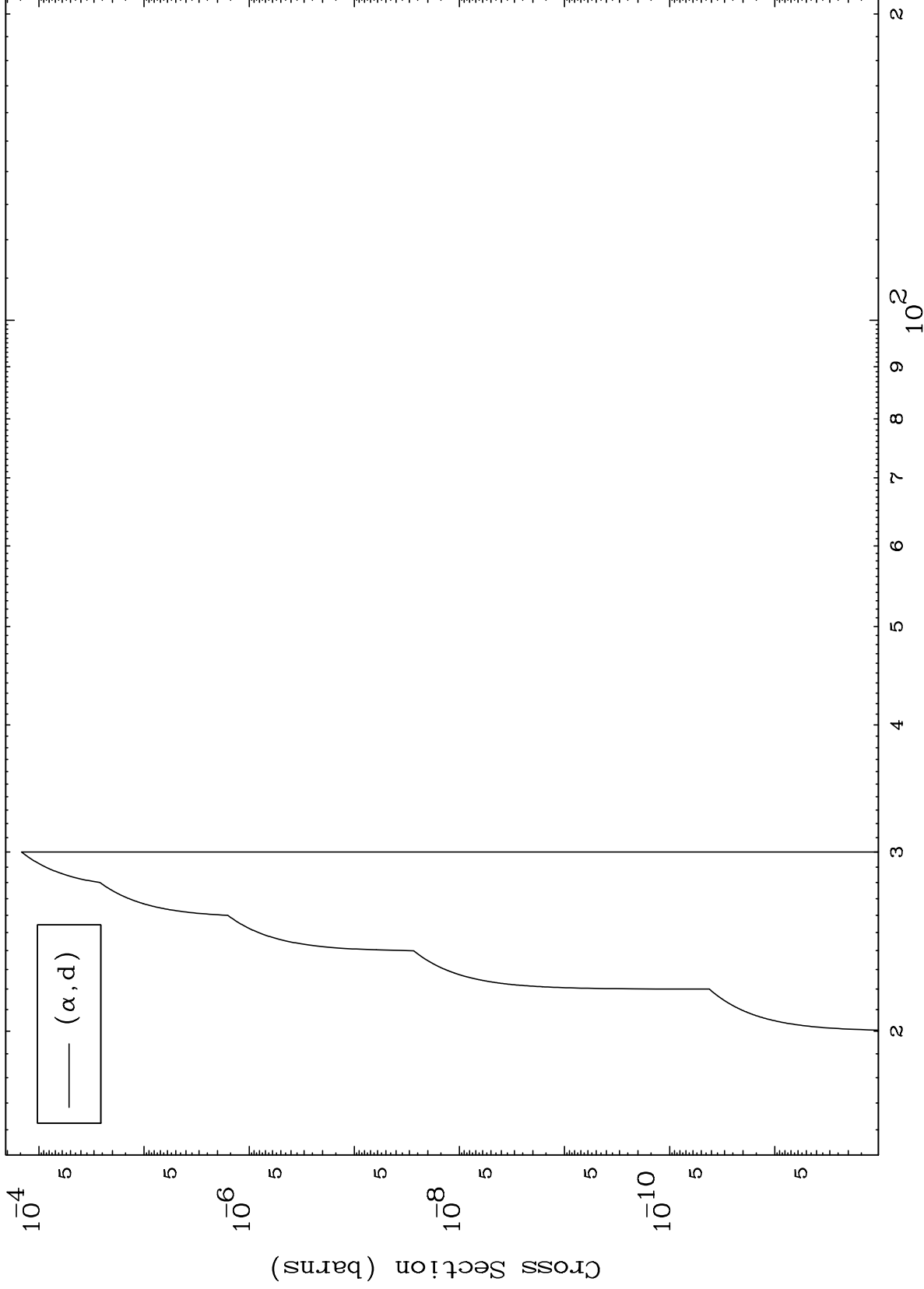
Incident Energy (MeV)

92-U -240

MAT 9243

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

92-U -240



6

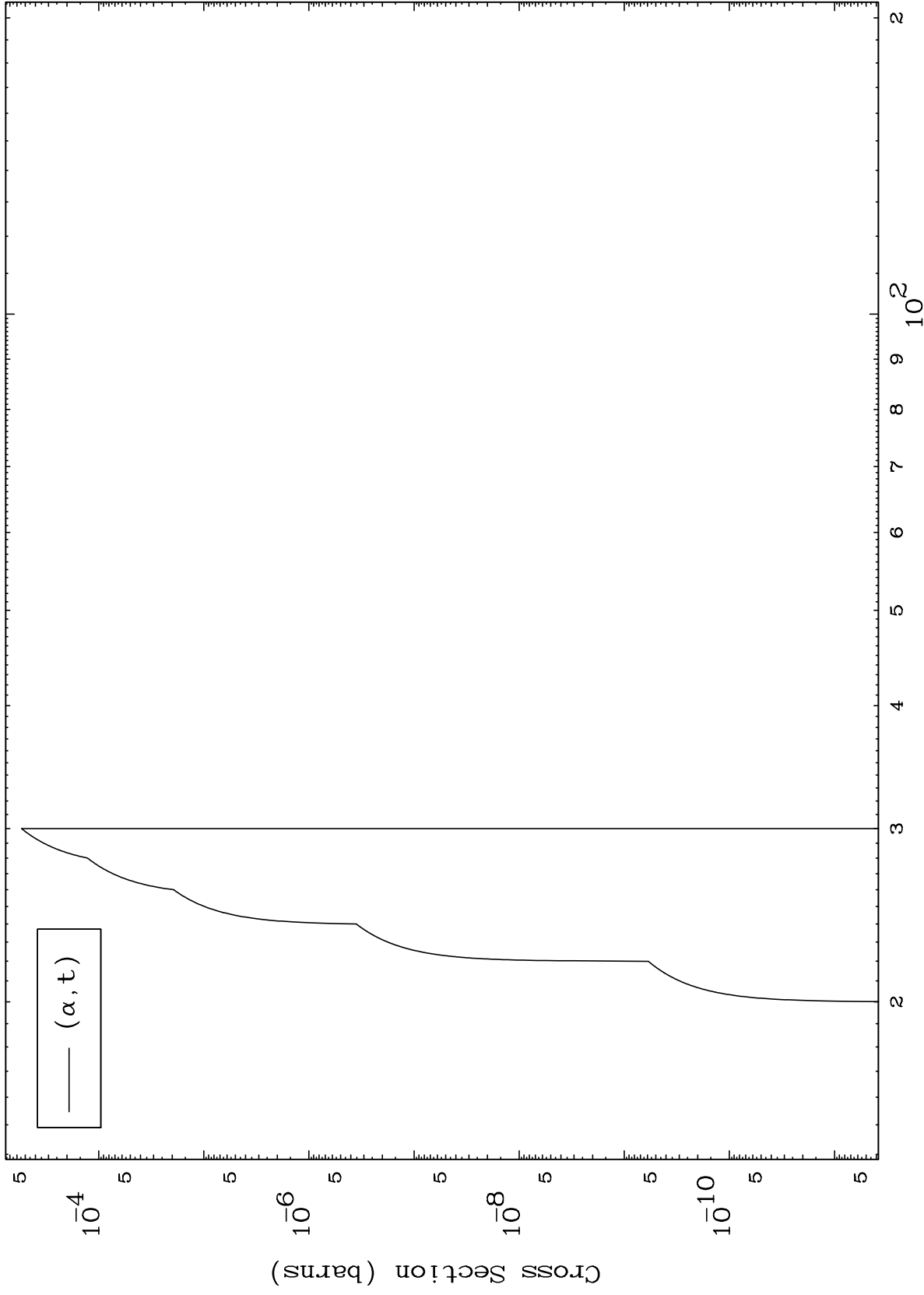
Incident Energy (MeV)

92-U -240

MAT 9243

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

92-U -240



7

Incident Energy (MeV)

92-U -240

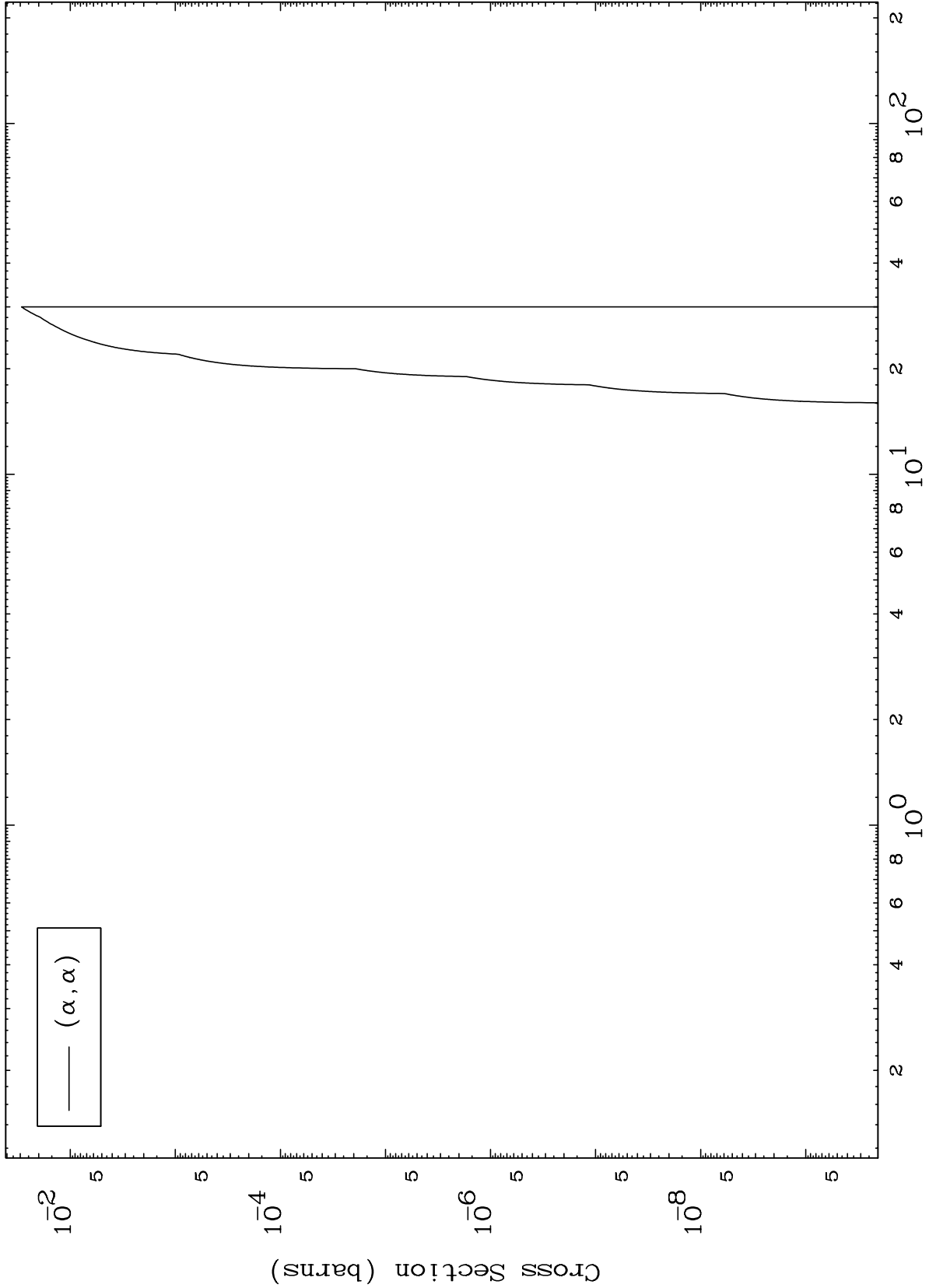


MAT 9243

( $\alpha, \alpha$ ) Levels

92-U -240

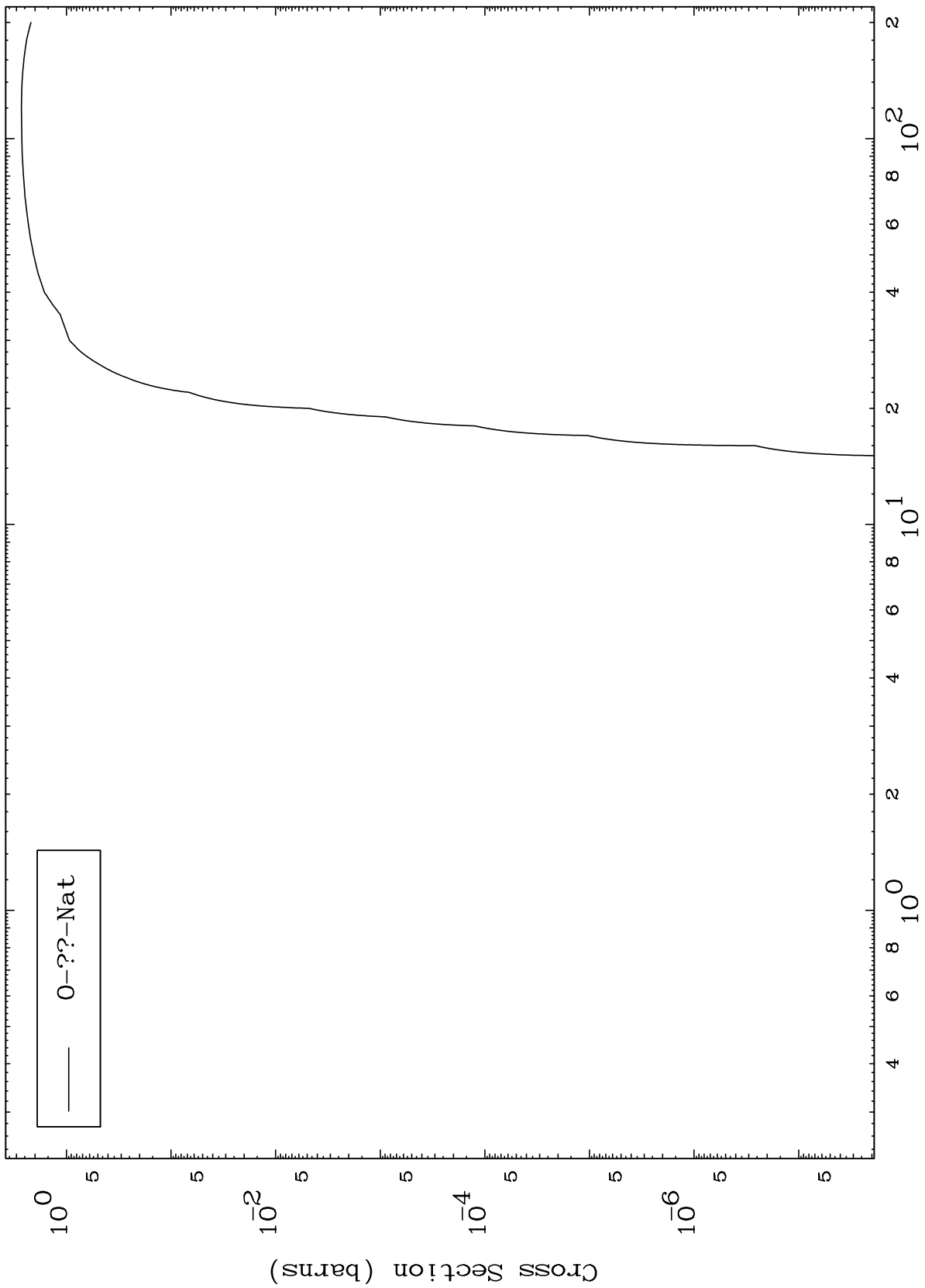
0 Kelvin Cross Sections



MAT 9243

92-U -240

$\alpha$  Fission  
Radionuclide Production Cross Section

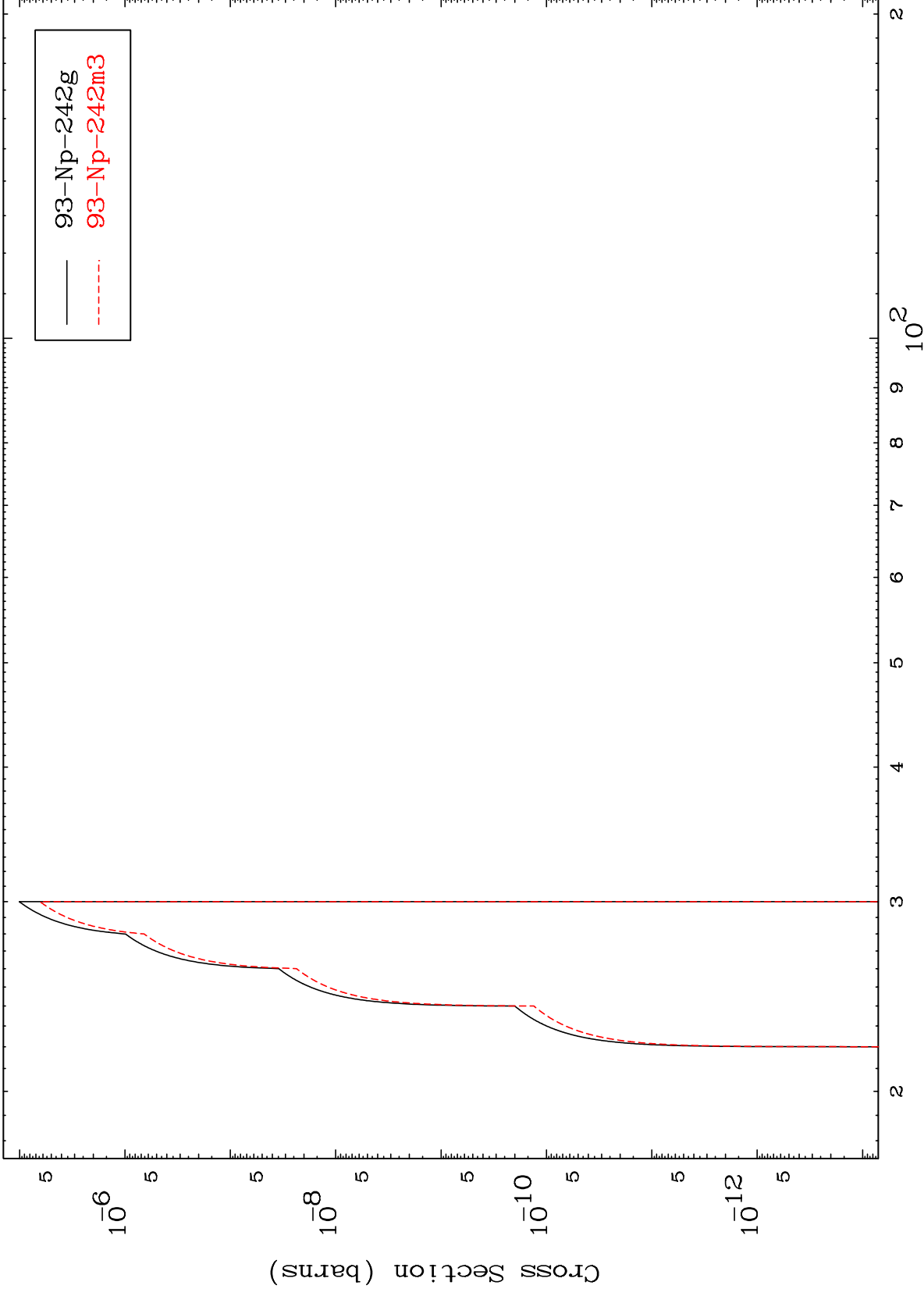


MAT 9243

( $\alpha, n'$ ) p

92-U -240

Radionuclide Production Cross Section



10

Incident Energy (MeV)

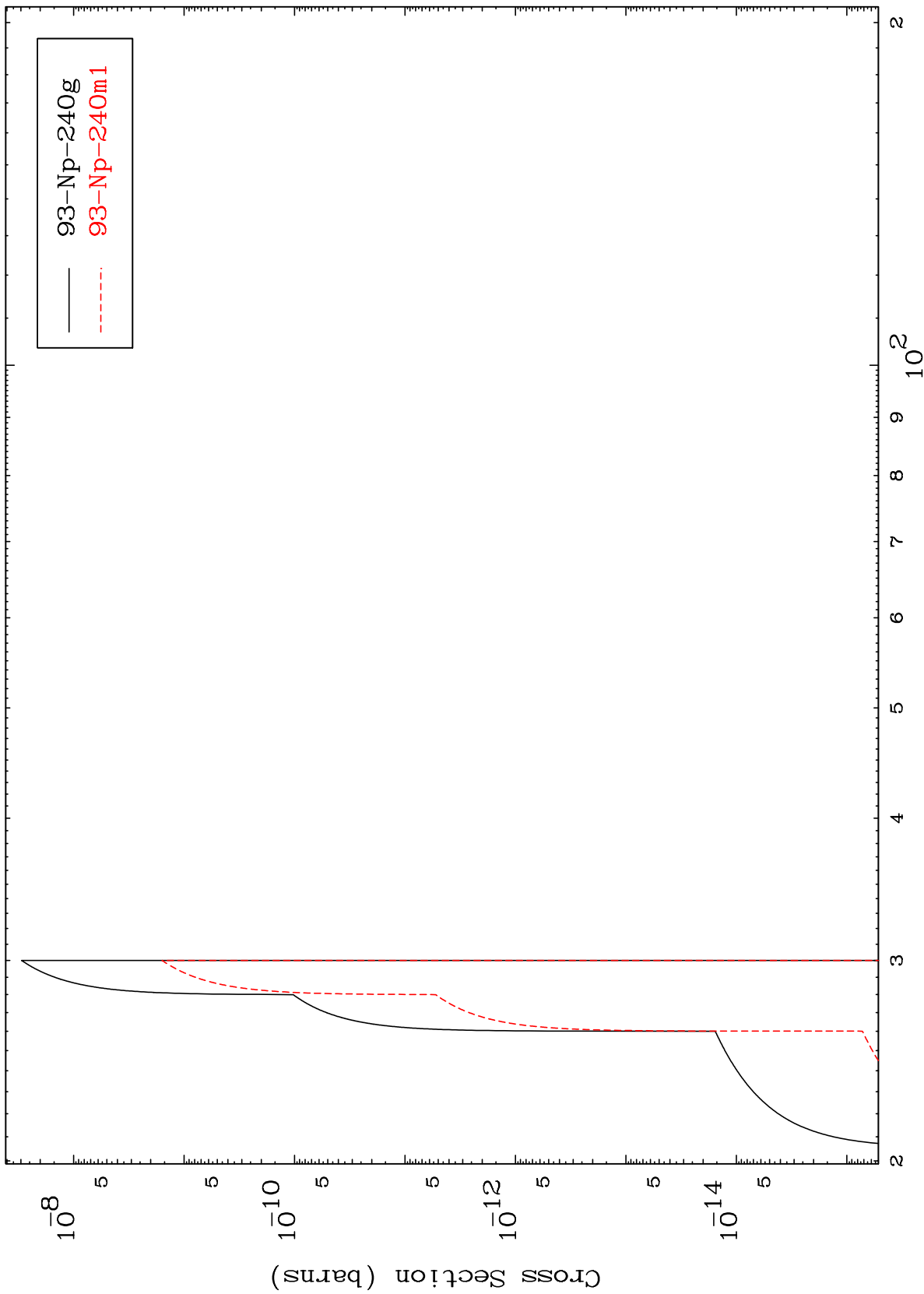
92-U -240

MAT 9243

( $\alpha, n'$ ) t

92-U -240

Radionuclide Production Cross Section



11

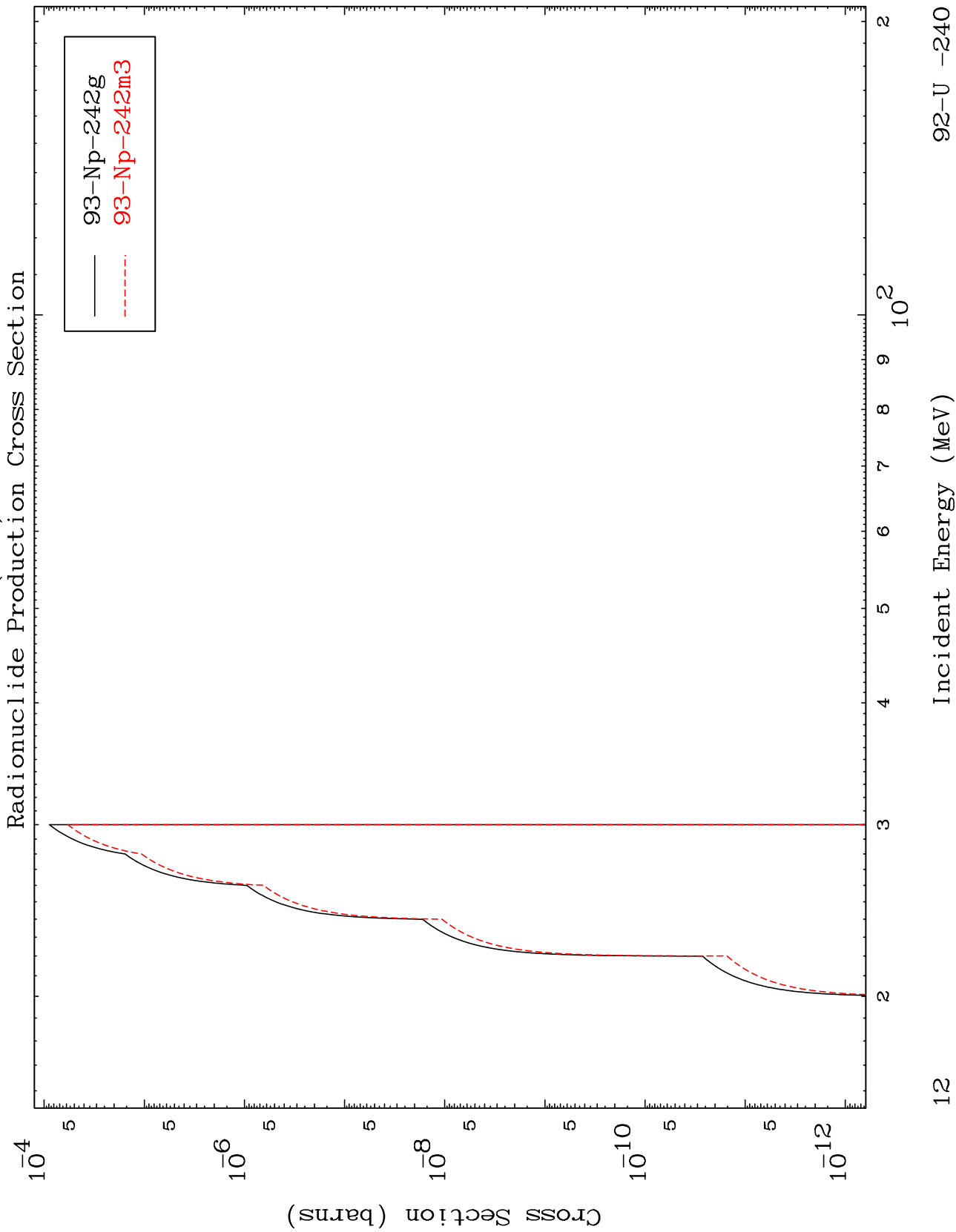
Incident Energy (MeV)

92-U -240

MAT 9243

( $\alpha, d$ )

92-U -240



12

92-U -240