

Program EVALPLOT  
(Version 2018-1)

by

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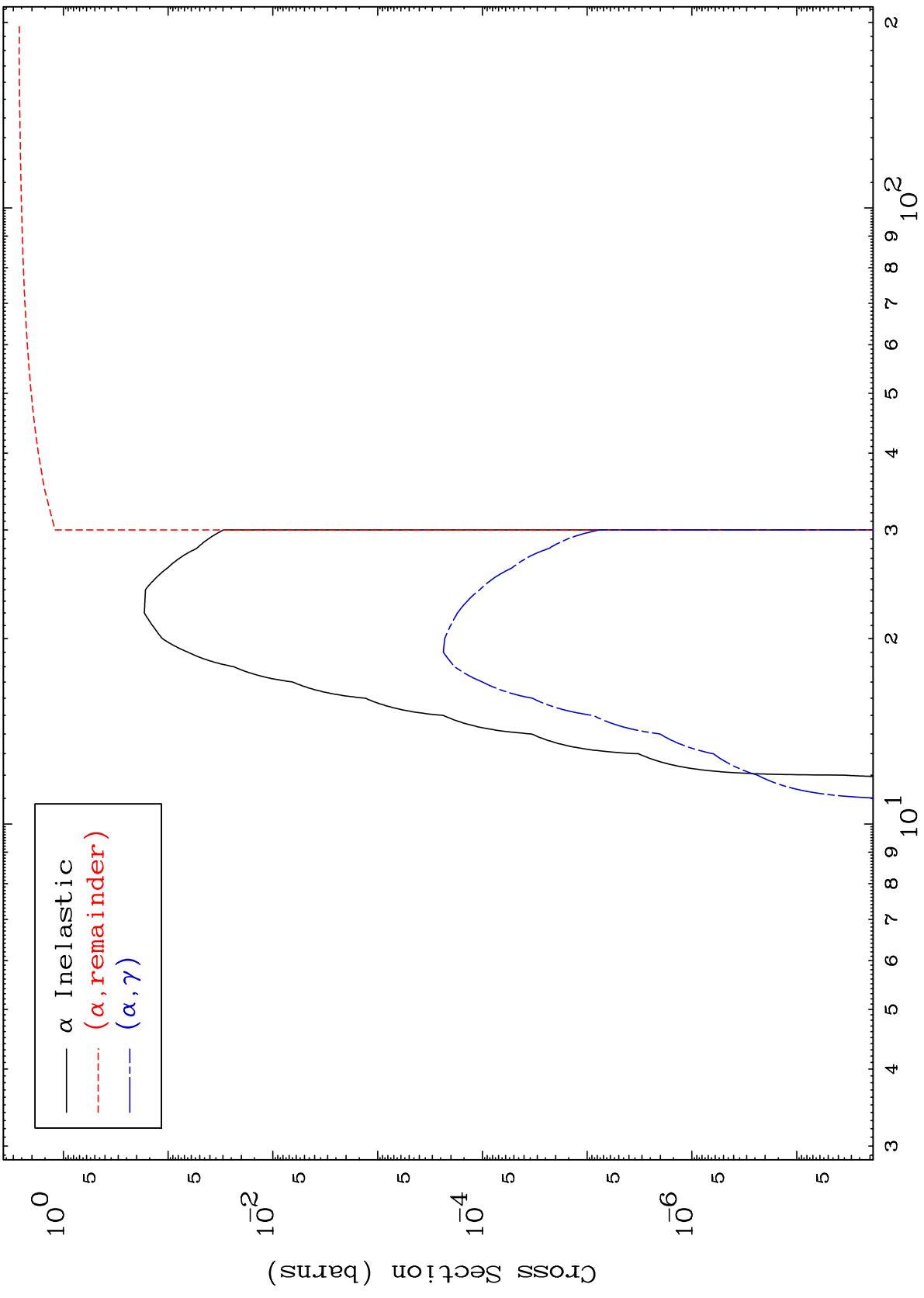
Press Mouse Button to Start

MAT 7425

$\alpha$  Major

74-W -180

0 Kelvin Cross Sections



$\alpha$  Inelastic  
 $(\alpha, \text{remainder})$   
 $(\alpha, \gamma)$

Incident Energy (MeV)

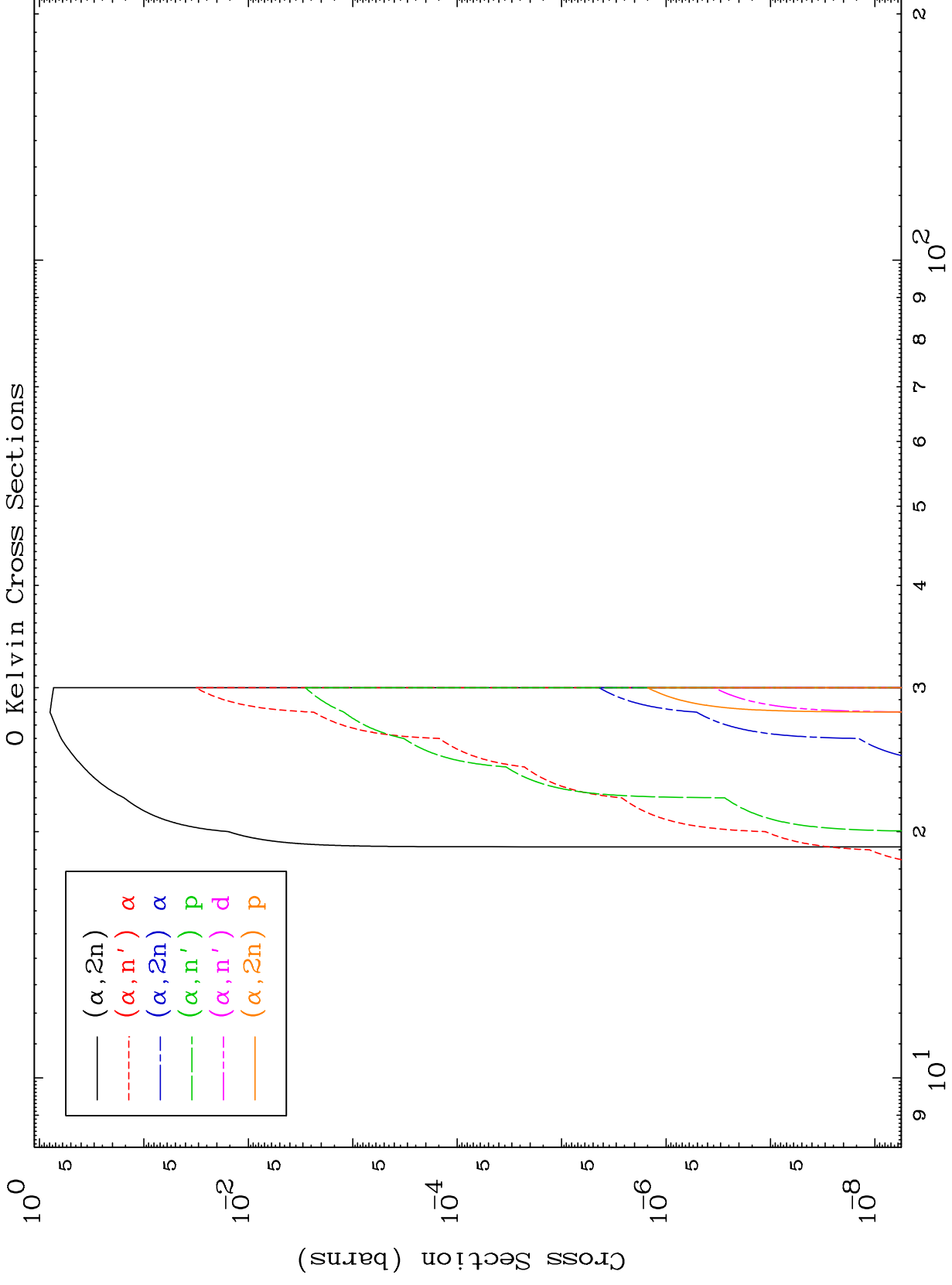
74-W -180

1

MAT 7425

$\alpha$  Neutron Production  
0 Kelvin Cross Sections

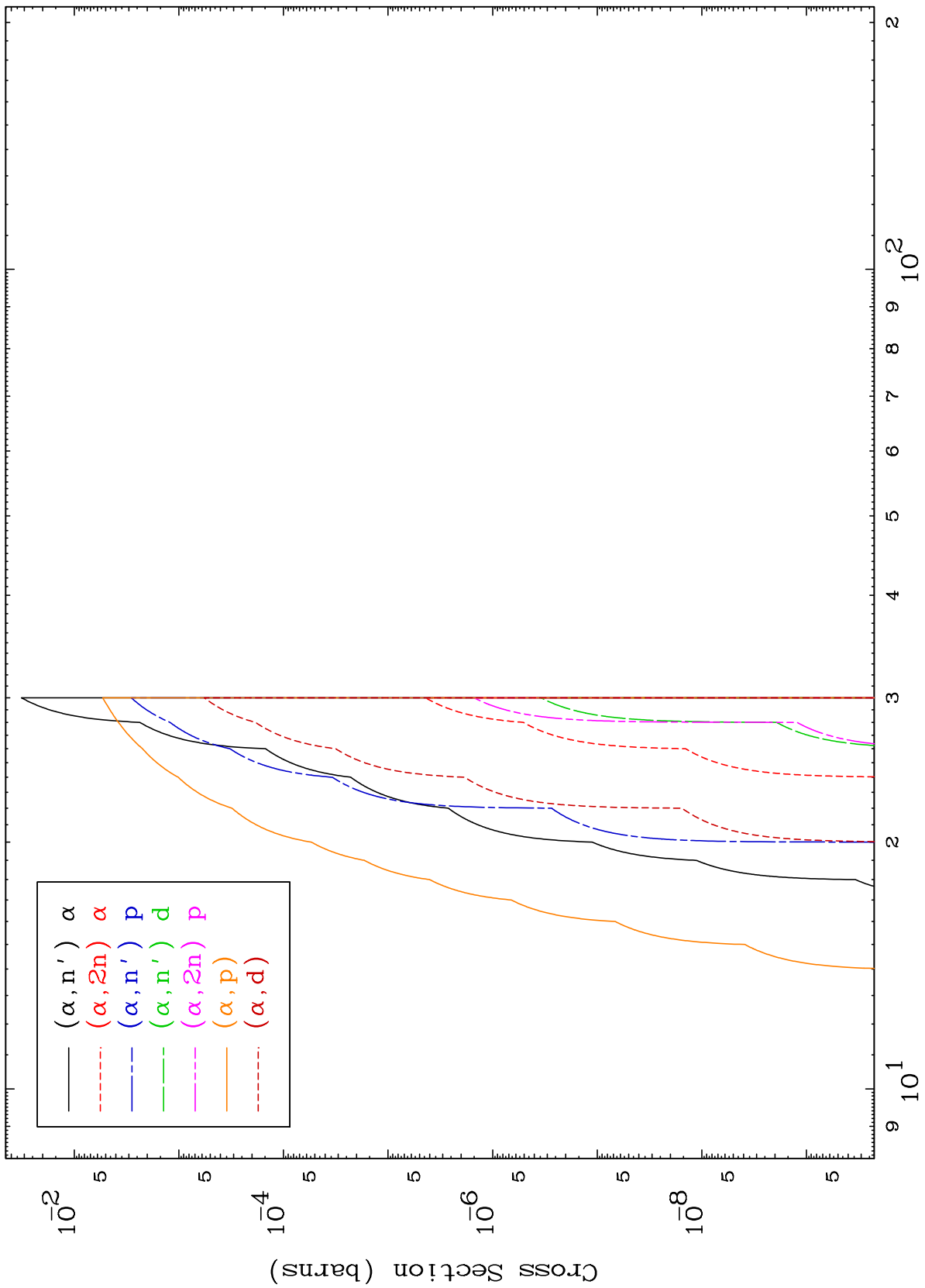
74-W -180

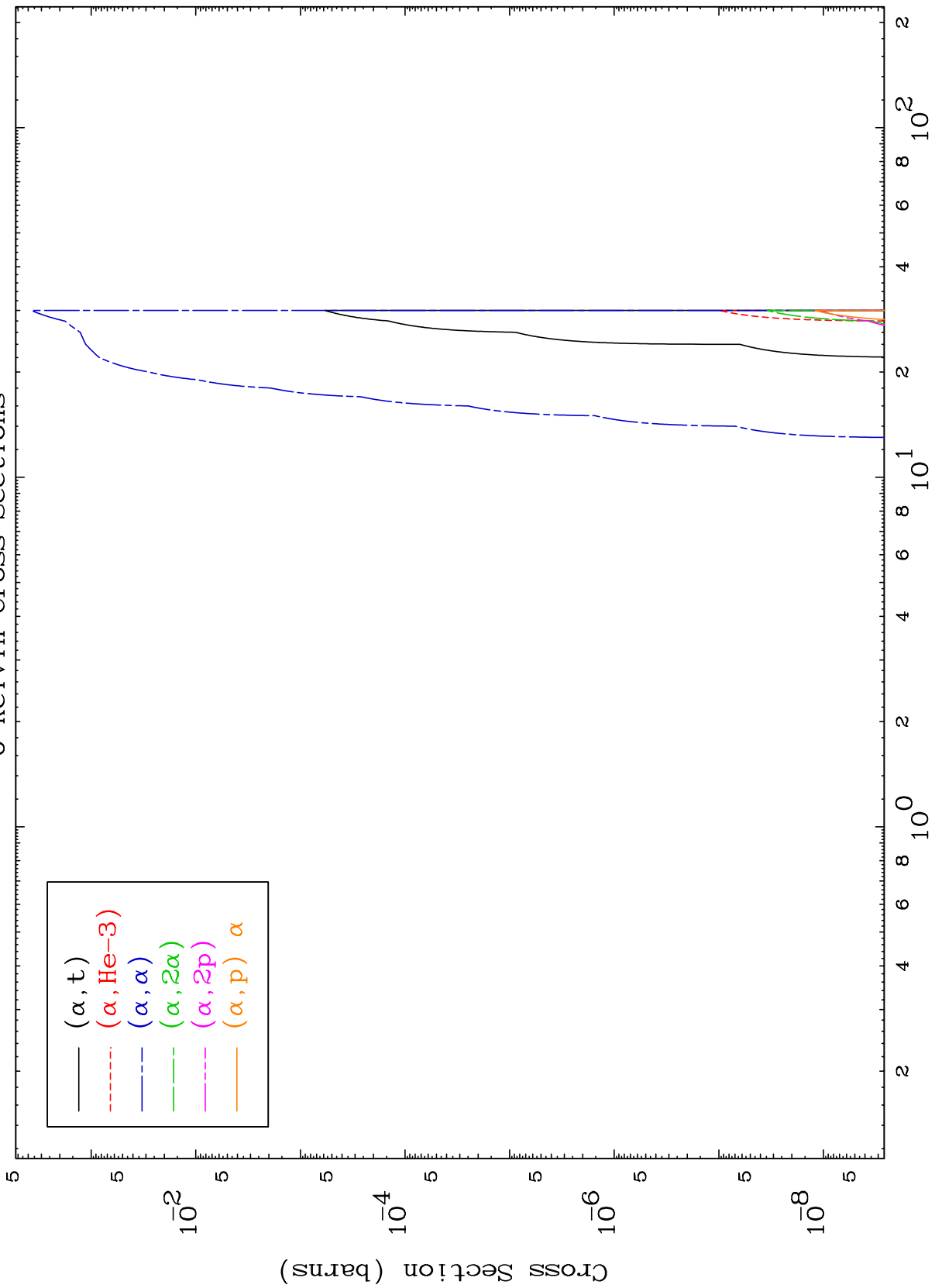


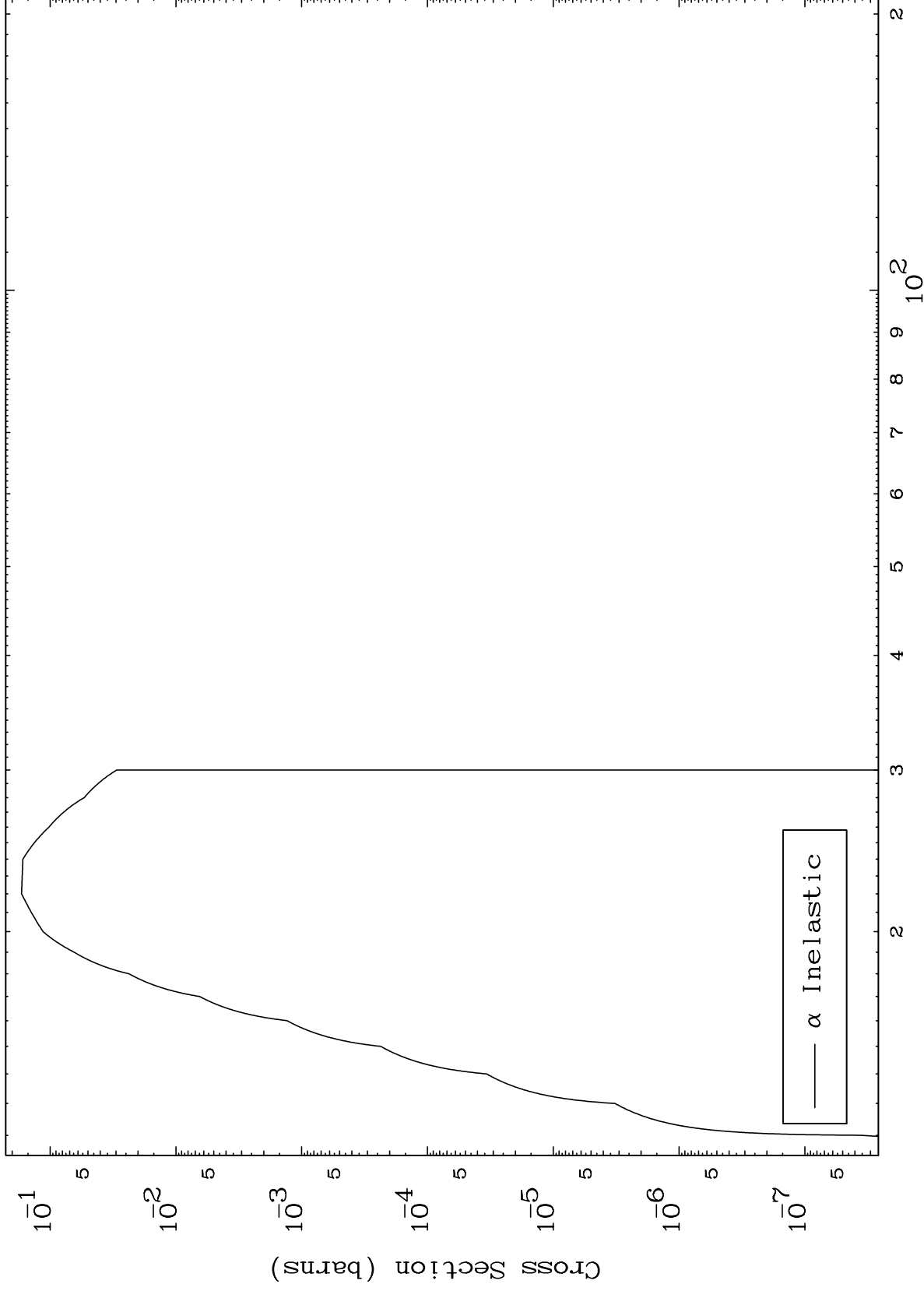
Incident Energy (MeV)

74-W -180

MAT 7425



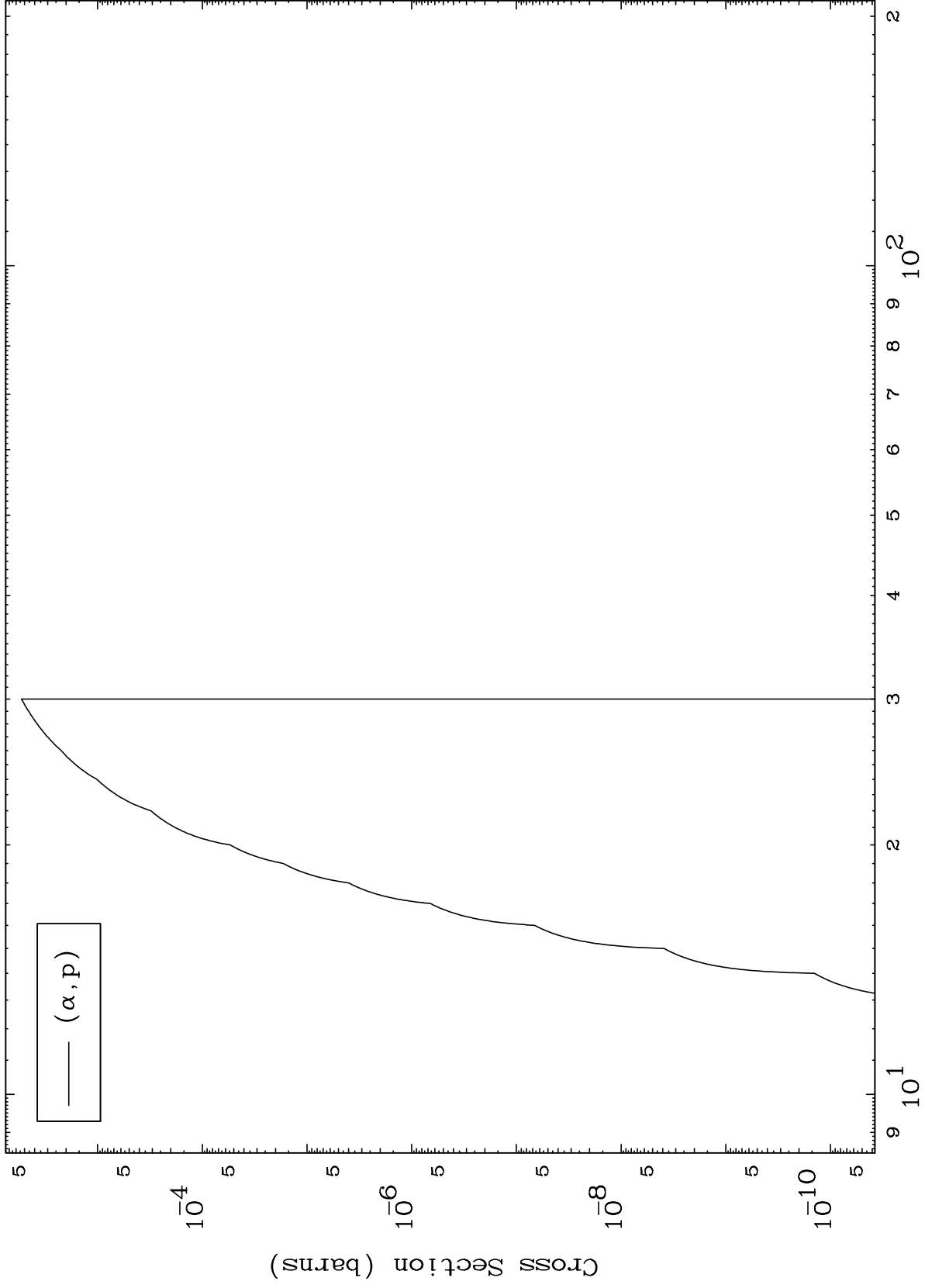




MAT 7425

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

74-W -180



Incident Energy (MeV)

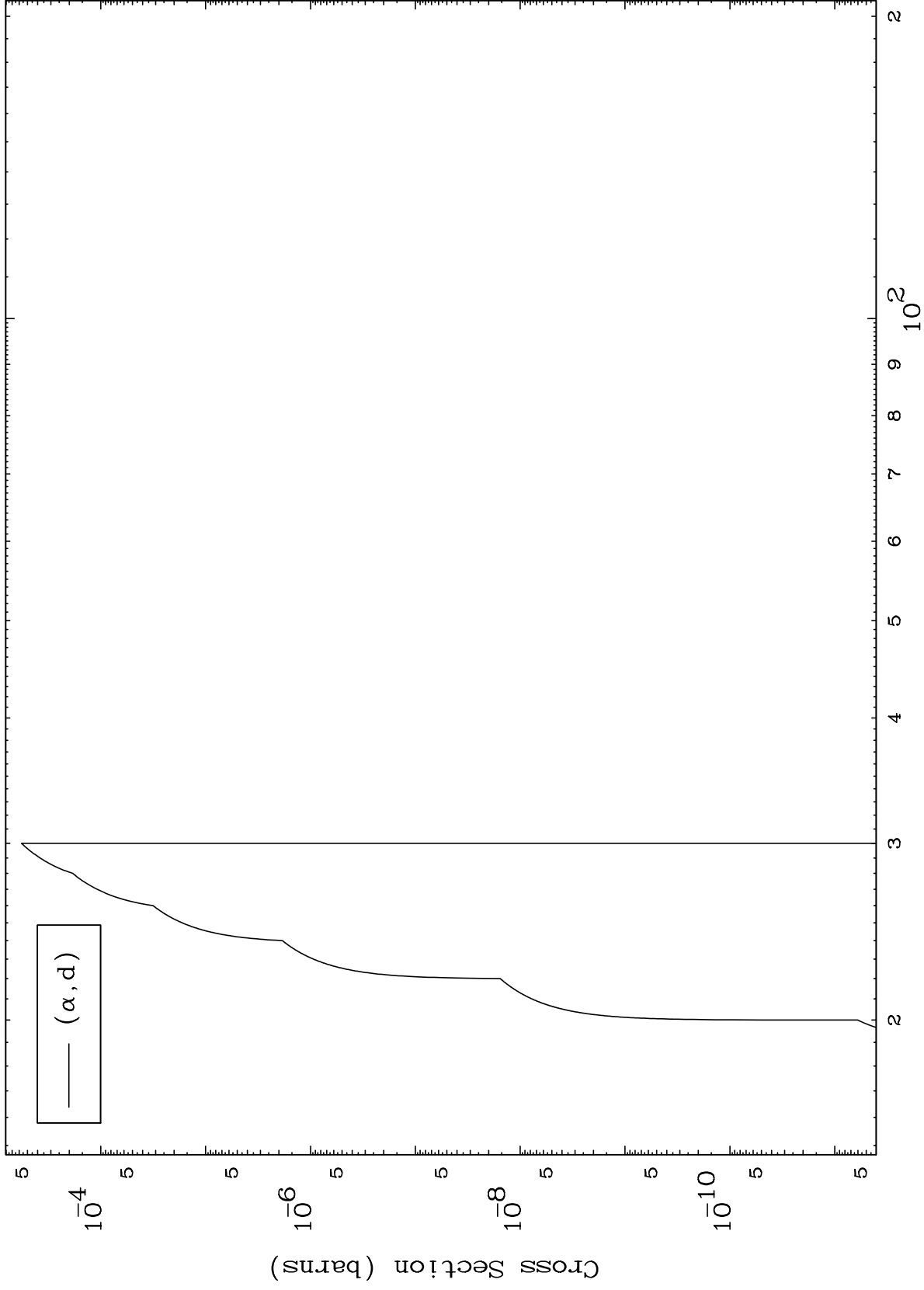
74-W -180

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MAT 7425

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

74-W -180

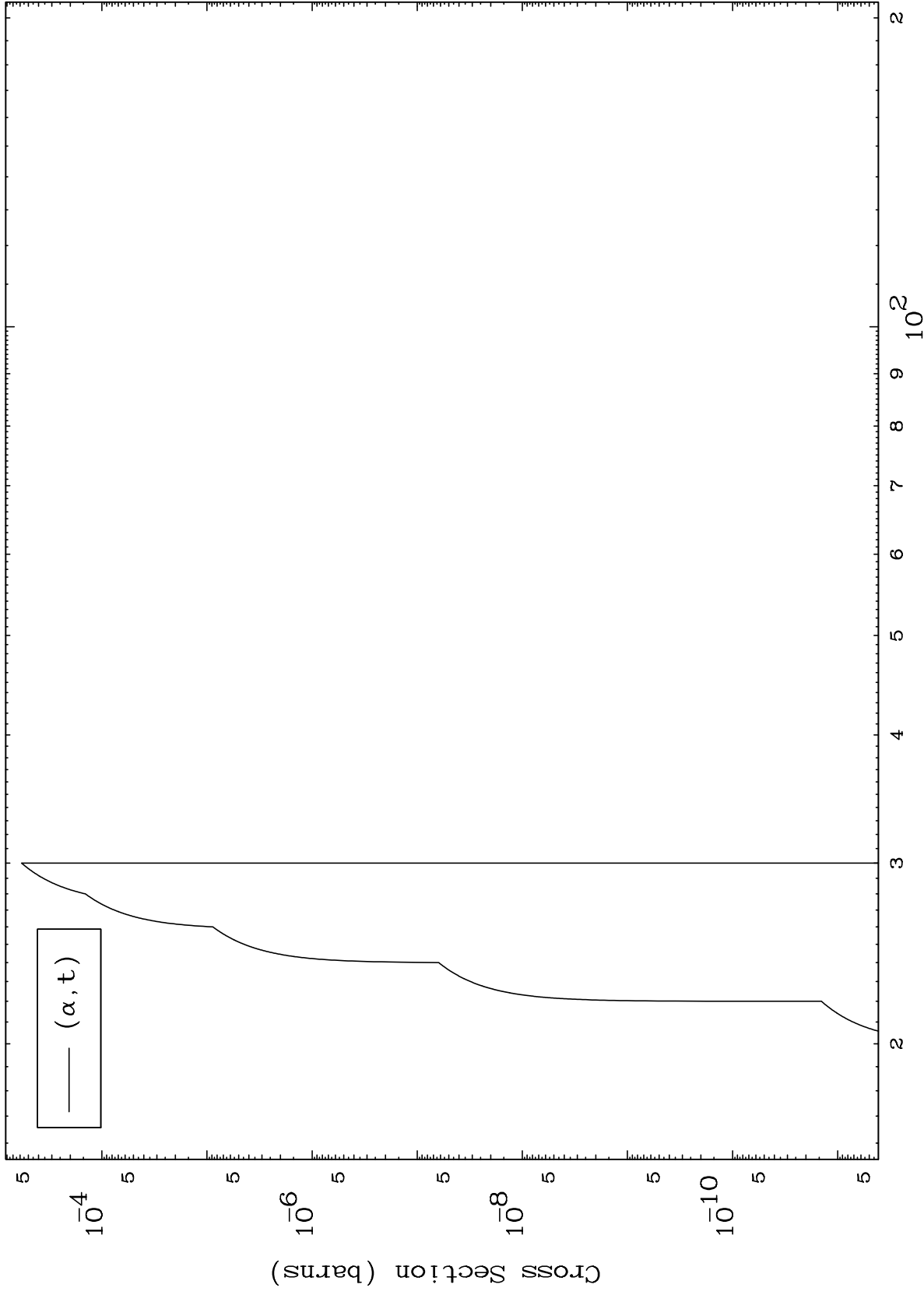


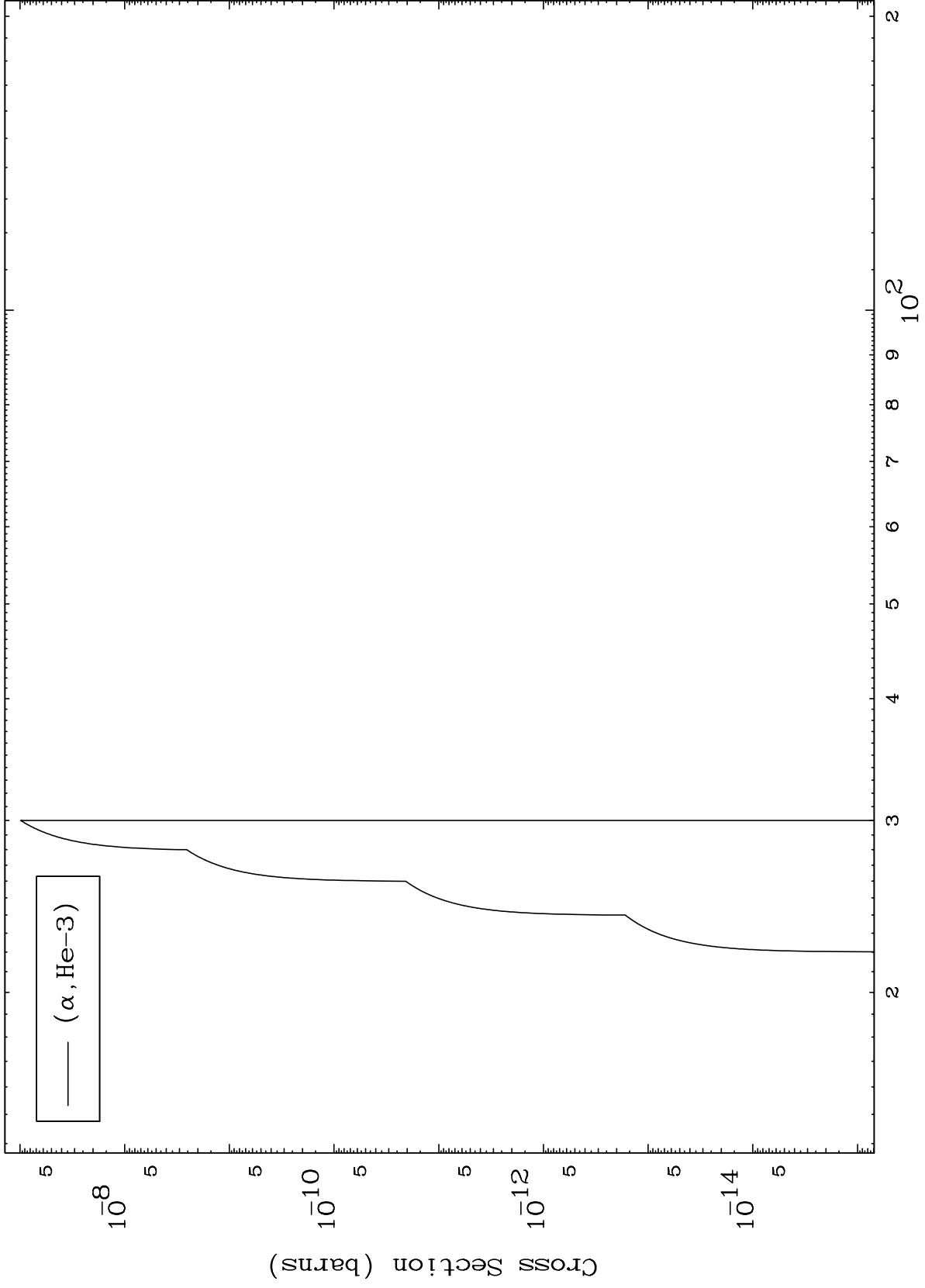
7

Incident Energy (MeV)

74-W -180





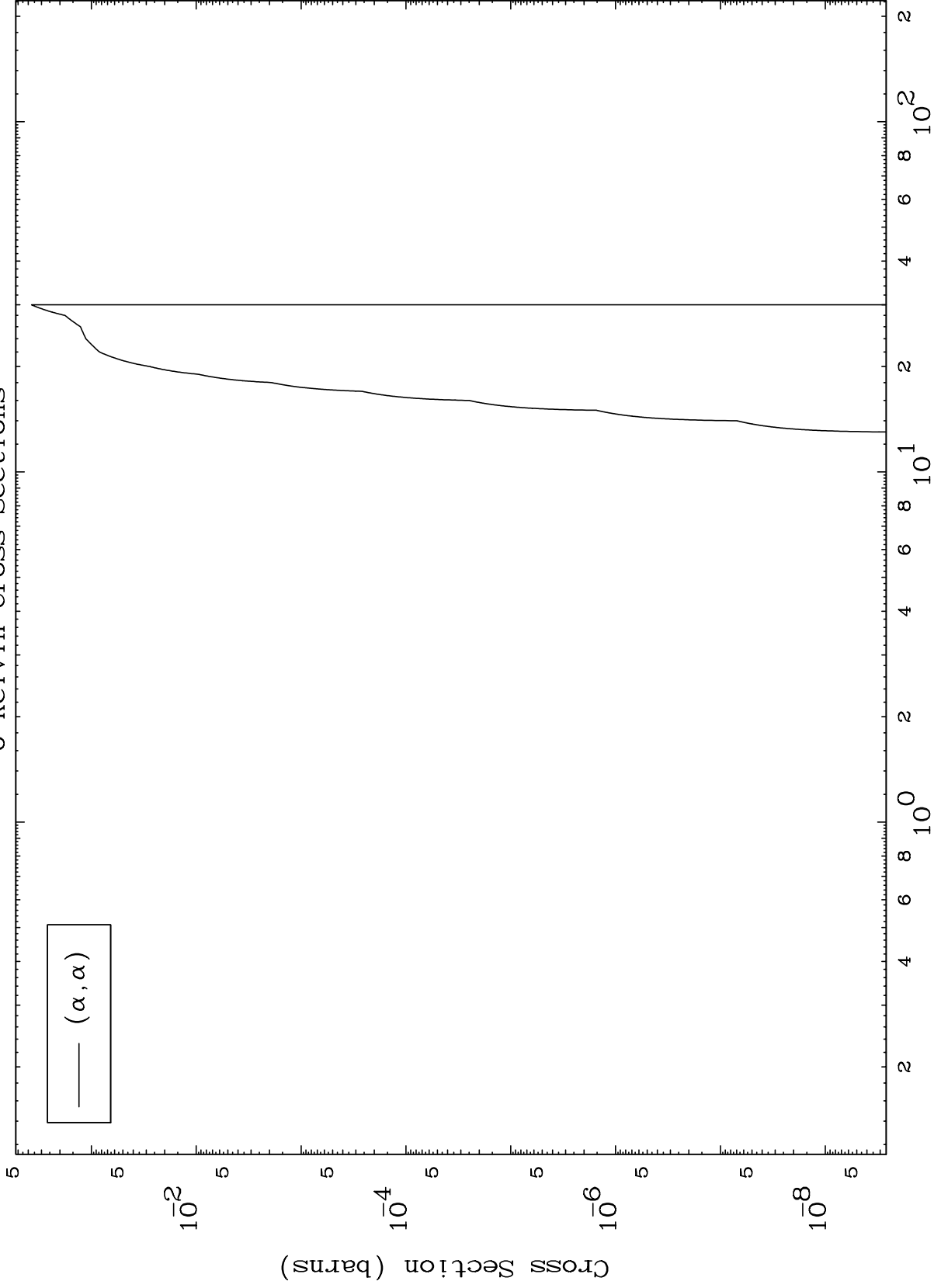


MAT 7425

( $\alpha, \alpha$ ) Levels

74-W -180

0 Kelvin Cross Sections

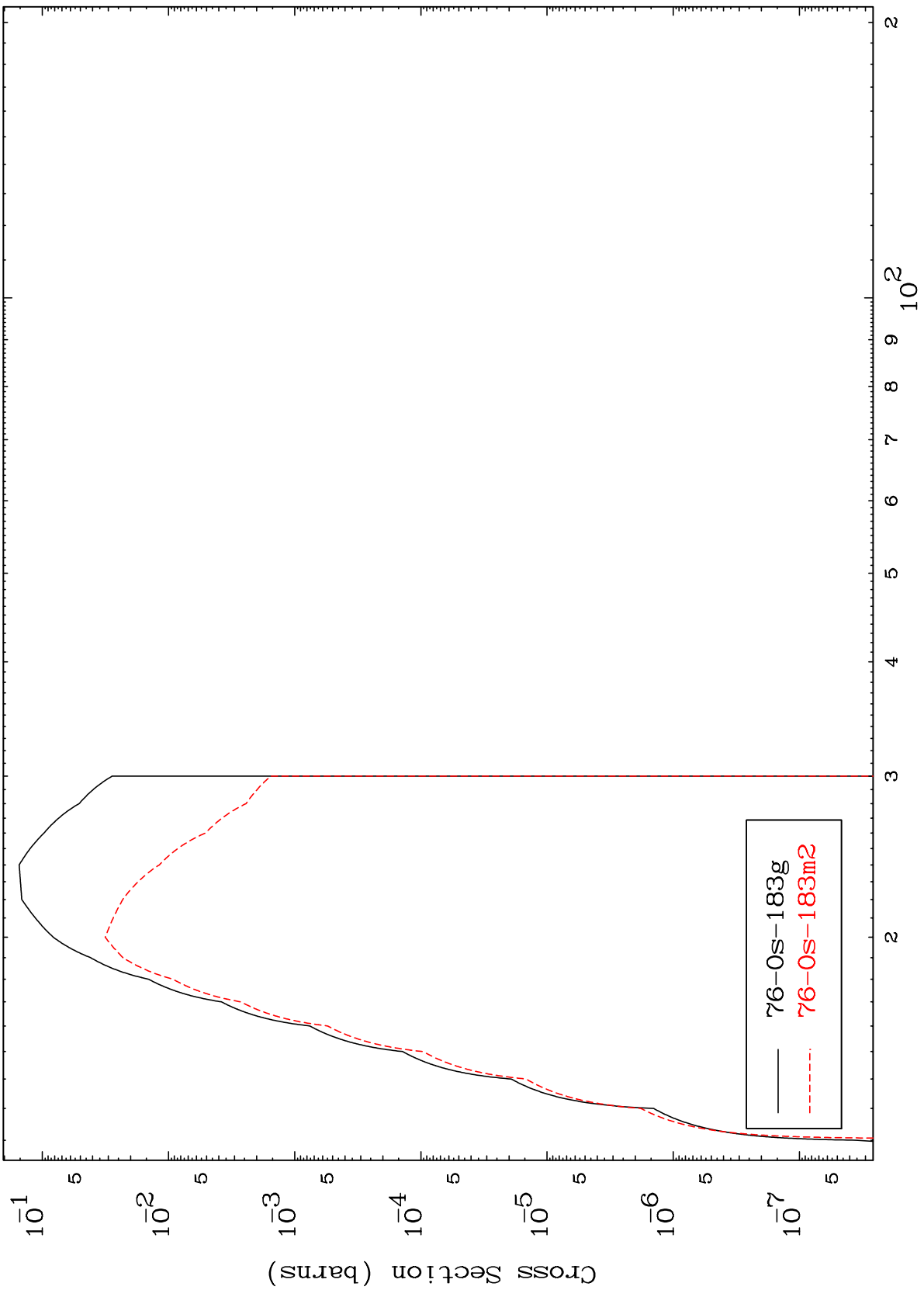


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Incident Energy (MeV)

74-W -180

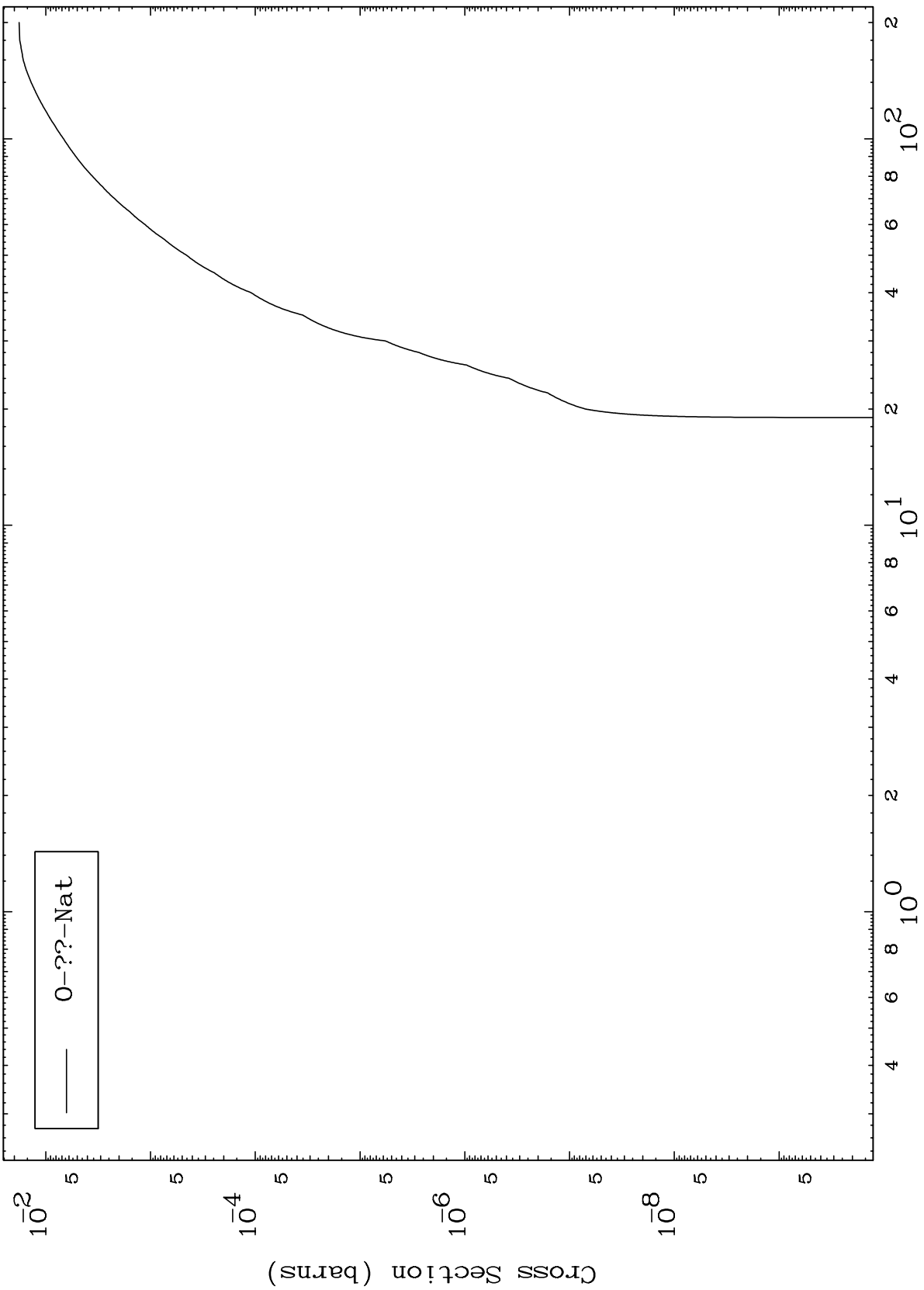
Radionuclide Production Cross Section  
 $\alpha$  Inelastic



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74-W -180

$\alpha$  Fission  
Radionuclide Production Cross Section



74-W -180

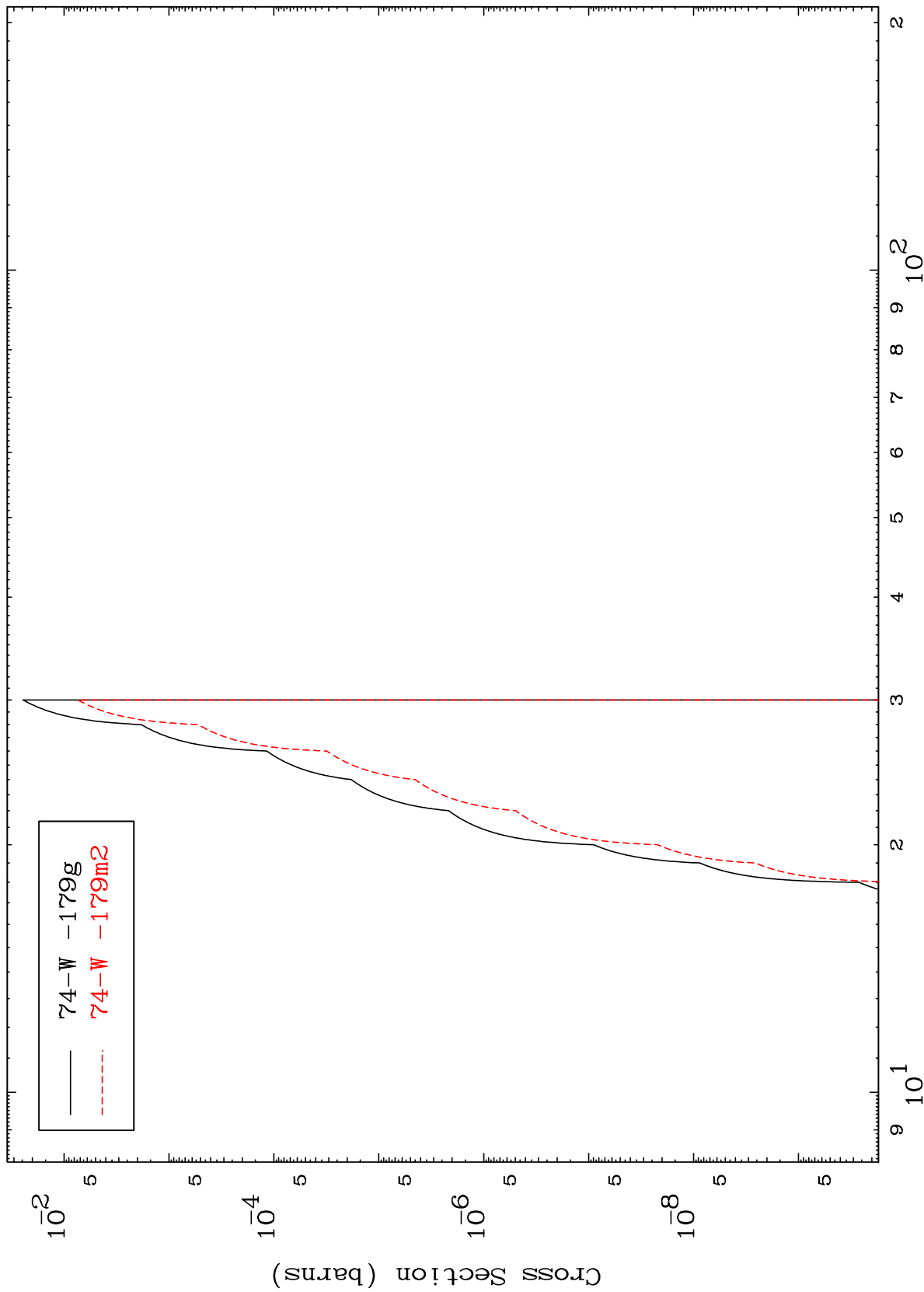
Incident Energy (MeV)

MAT 7425

( $\alpha, n'$ )  $\alpha$

74-W -180

Radionuclide Production Cross Section

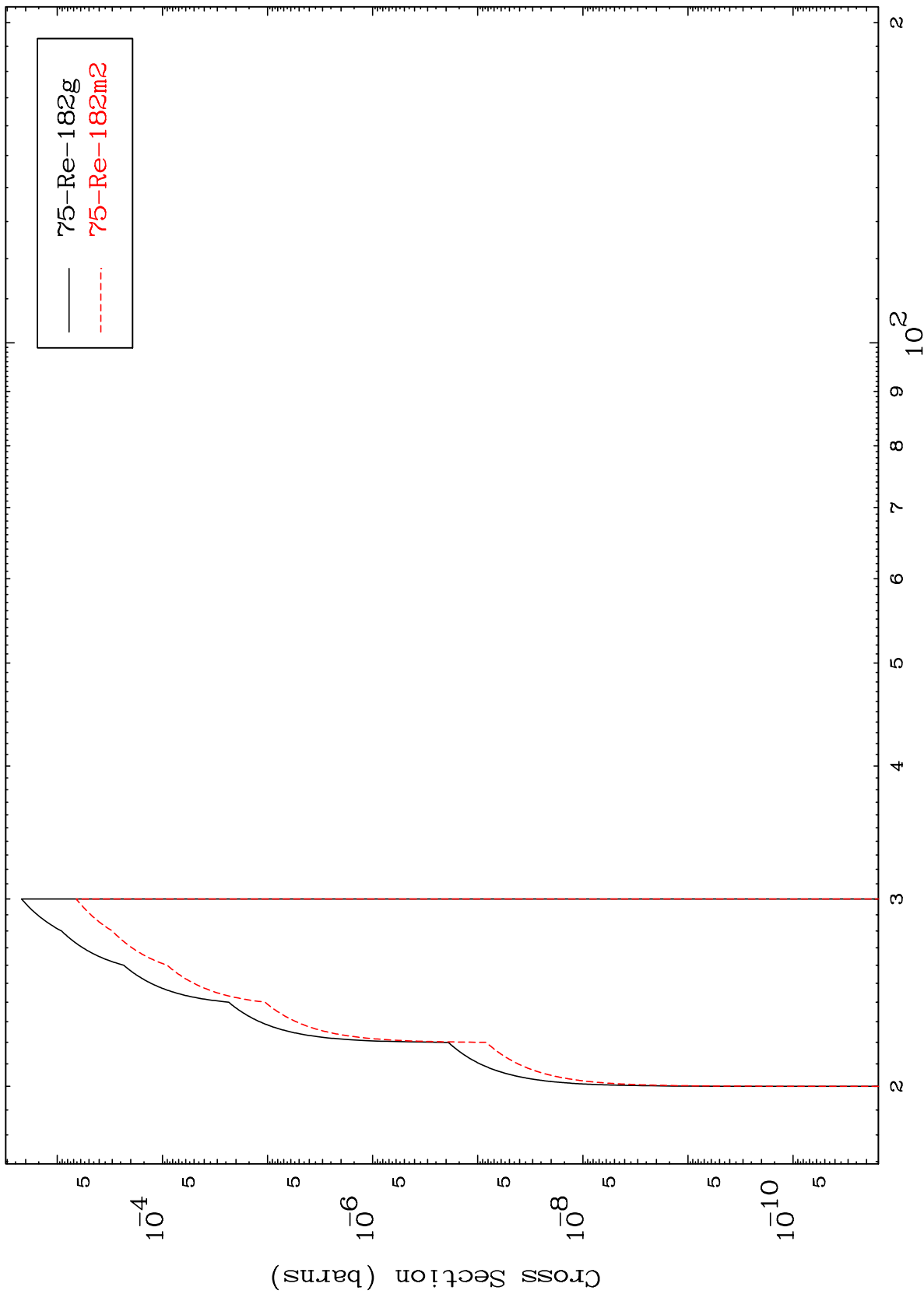


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Incident Energy (MeV)

74-W -180

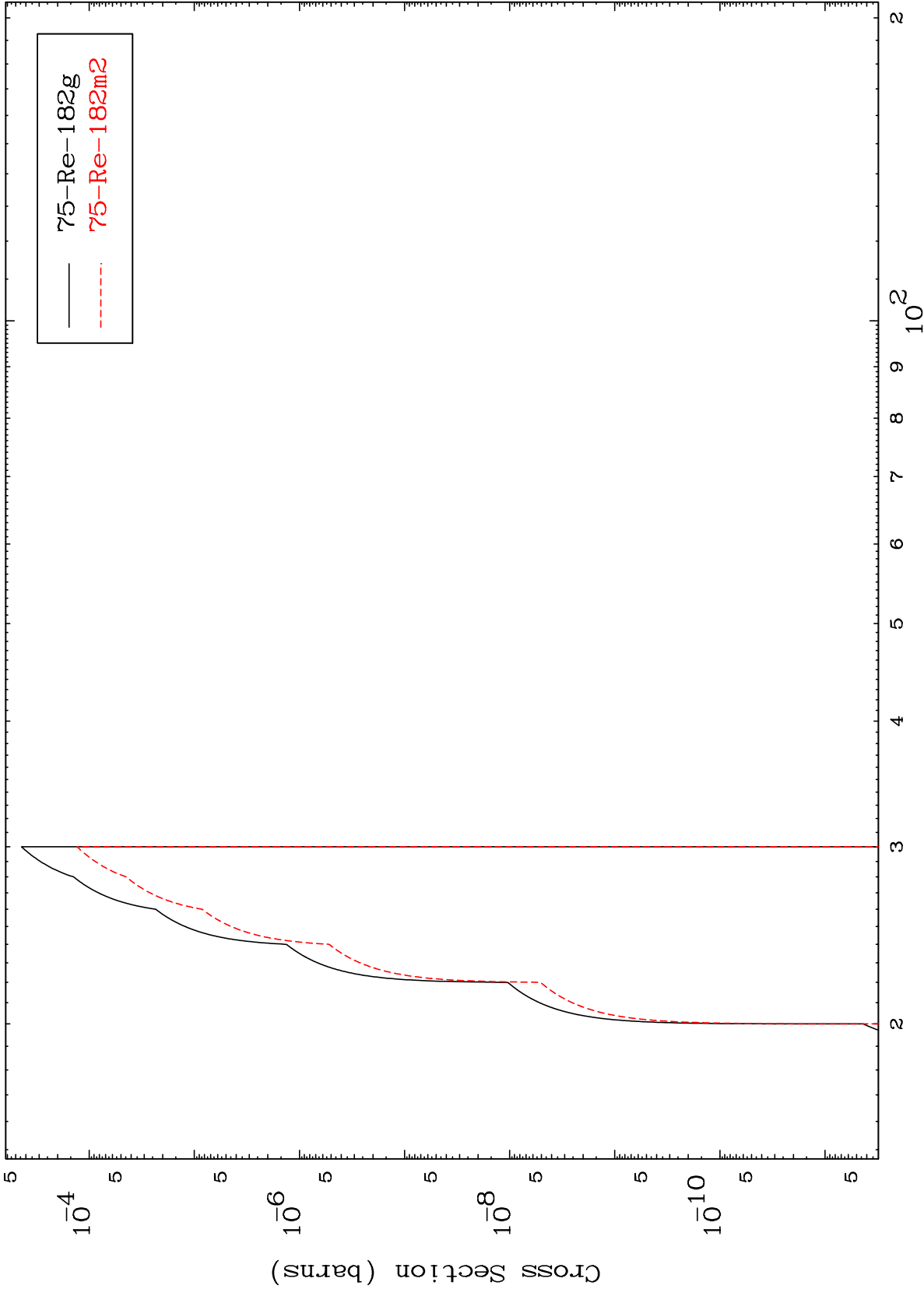
$(\alpha, n')$  p  
Radionuclide Production Cross Section



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74-W -180

( $\alpha, d$ )  
Radionuclide Production Cross Section



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74-W -180

Incident Energy (MeV)