

Program EVALPLOT  
(Version 2018-1)

by

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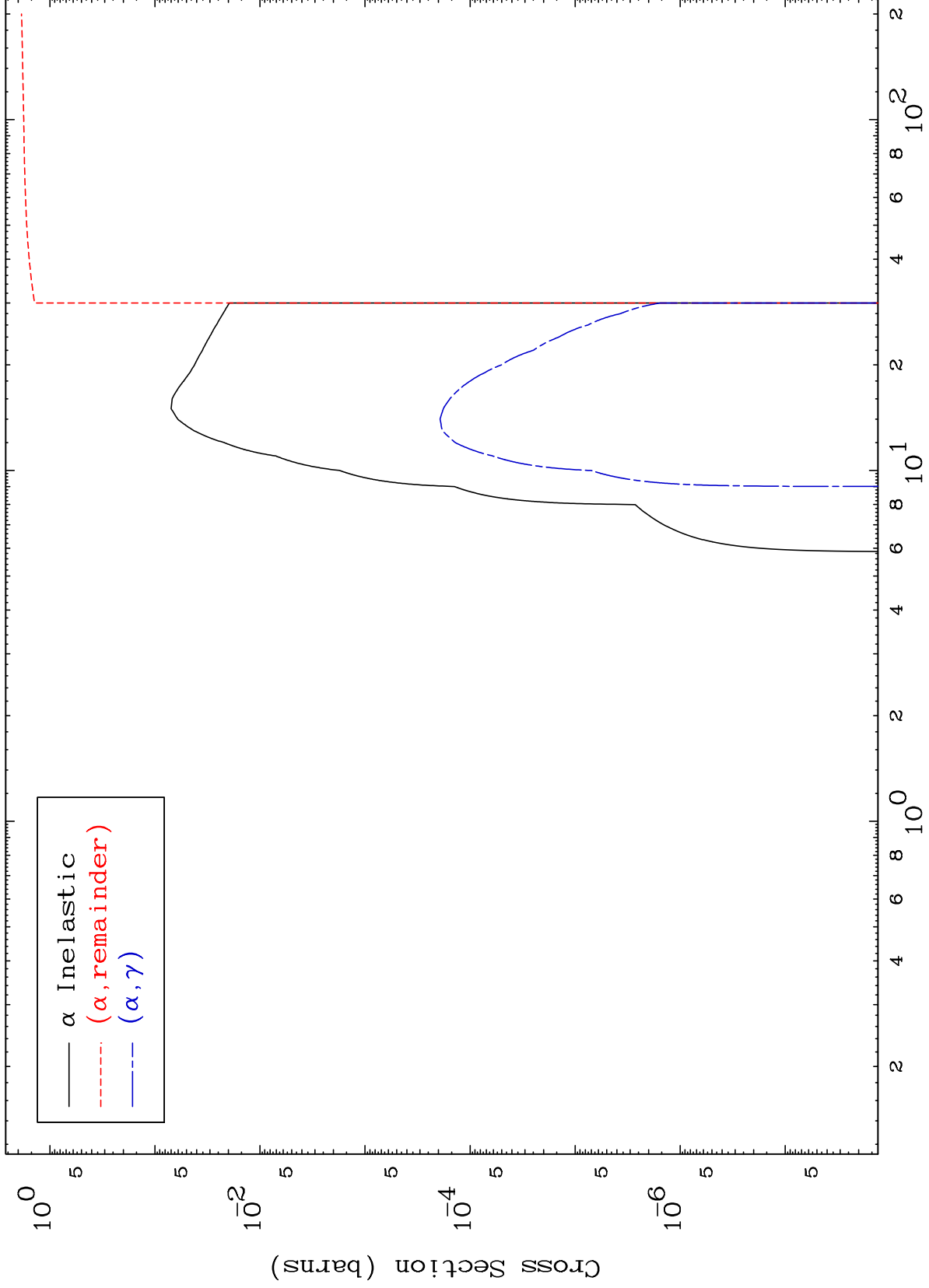
Press Mouse Button to Start

MAT 3911

$\alpha$  Major

39-Y -84

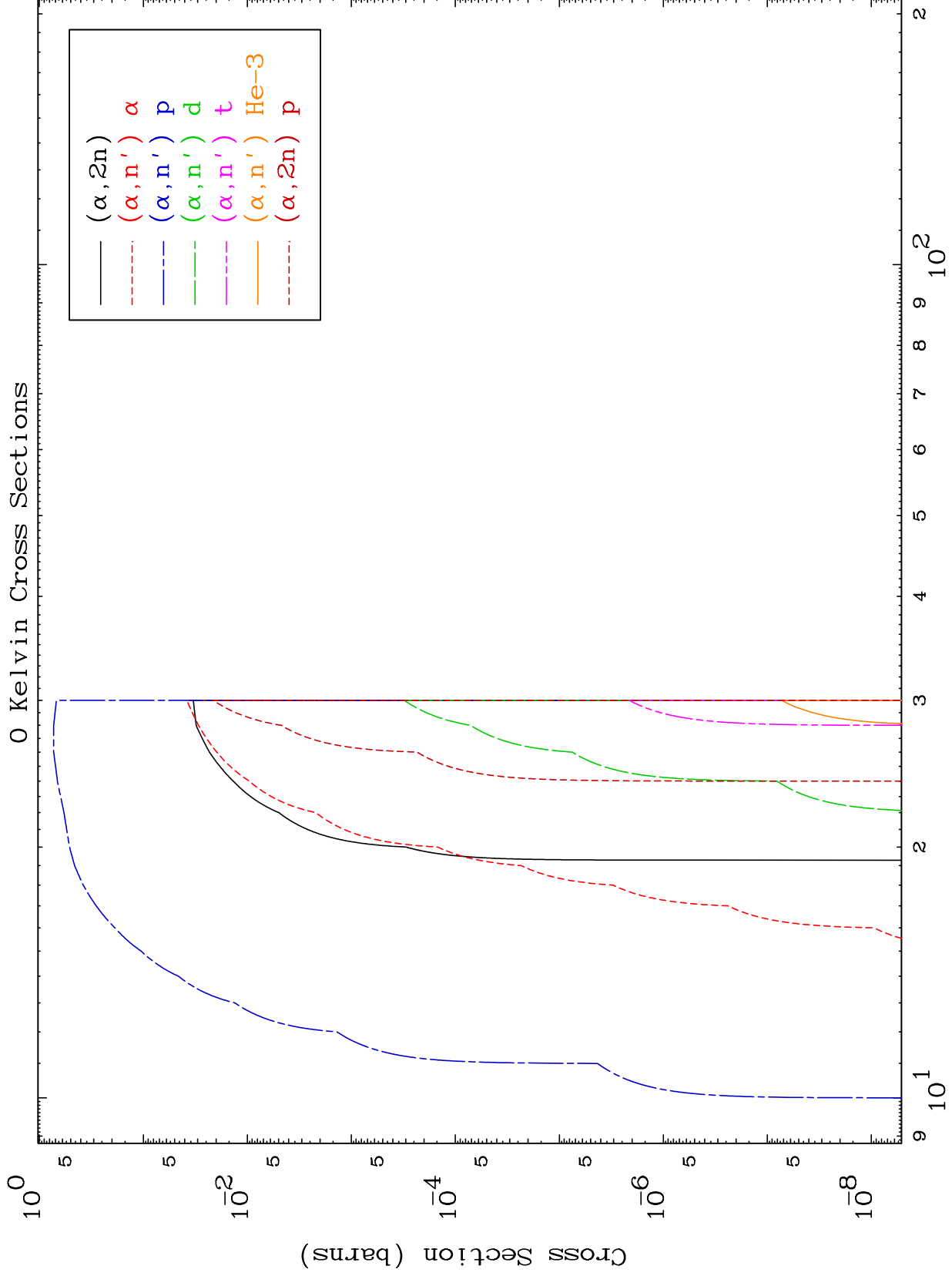
0 Kelvin Cross Sections



MAT 3911

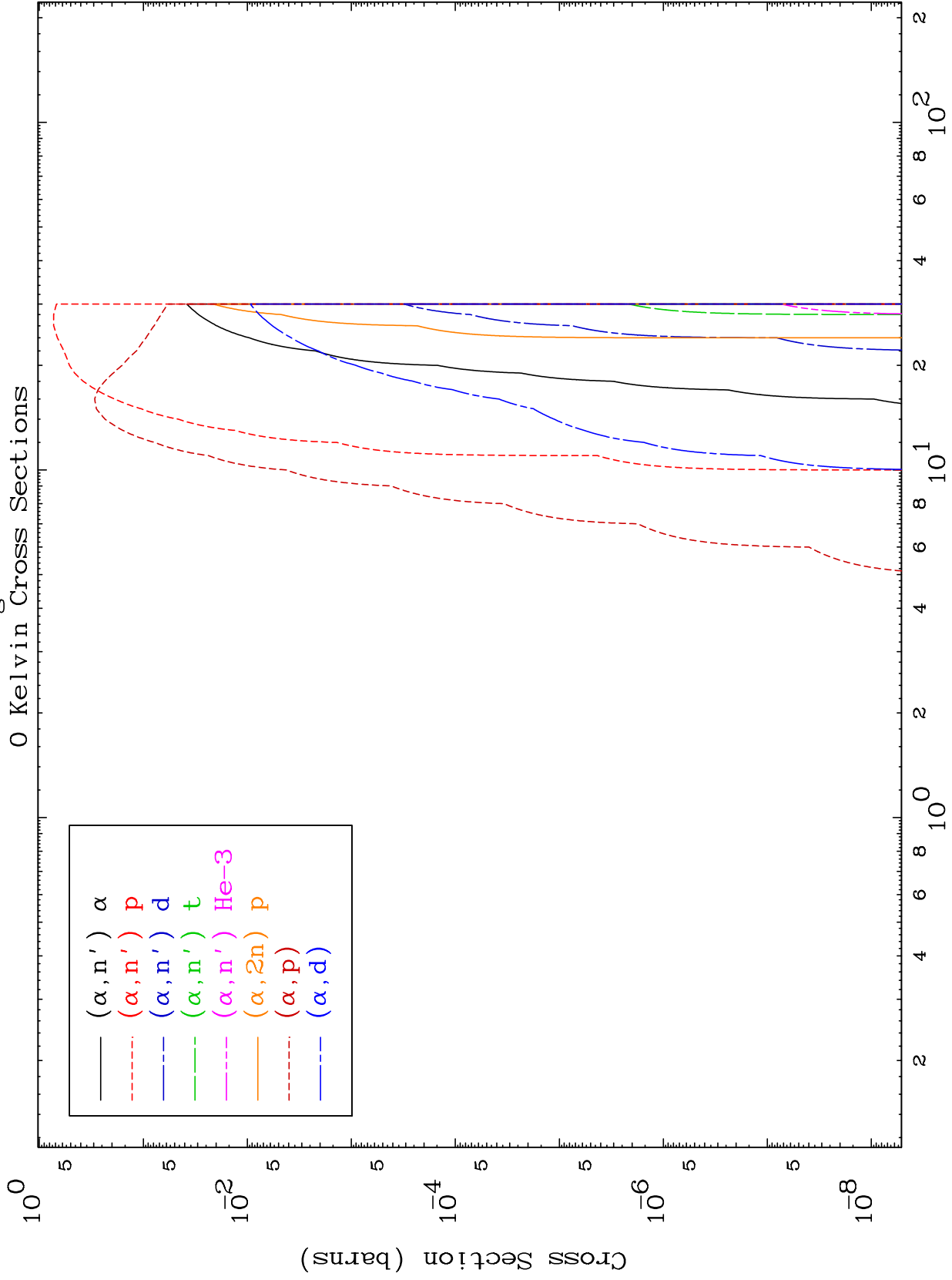
$\alpha$  Neutron Production  
0 Kelvin Cross Sections

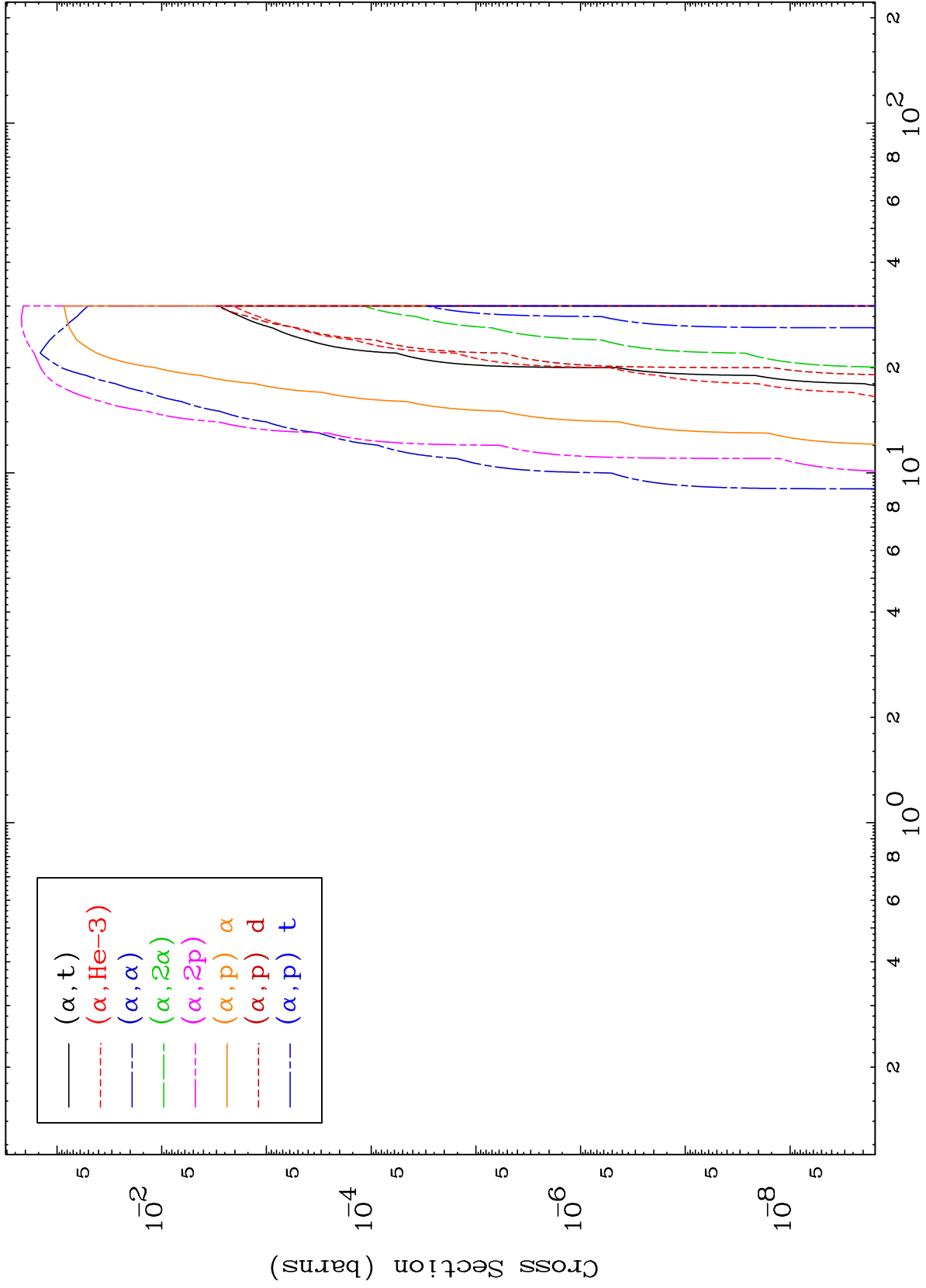
39-Y -84



Incident Energy (MeV)

39-Y -84



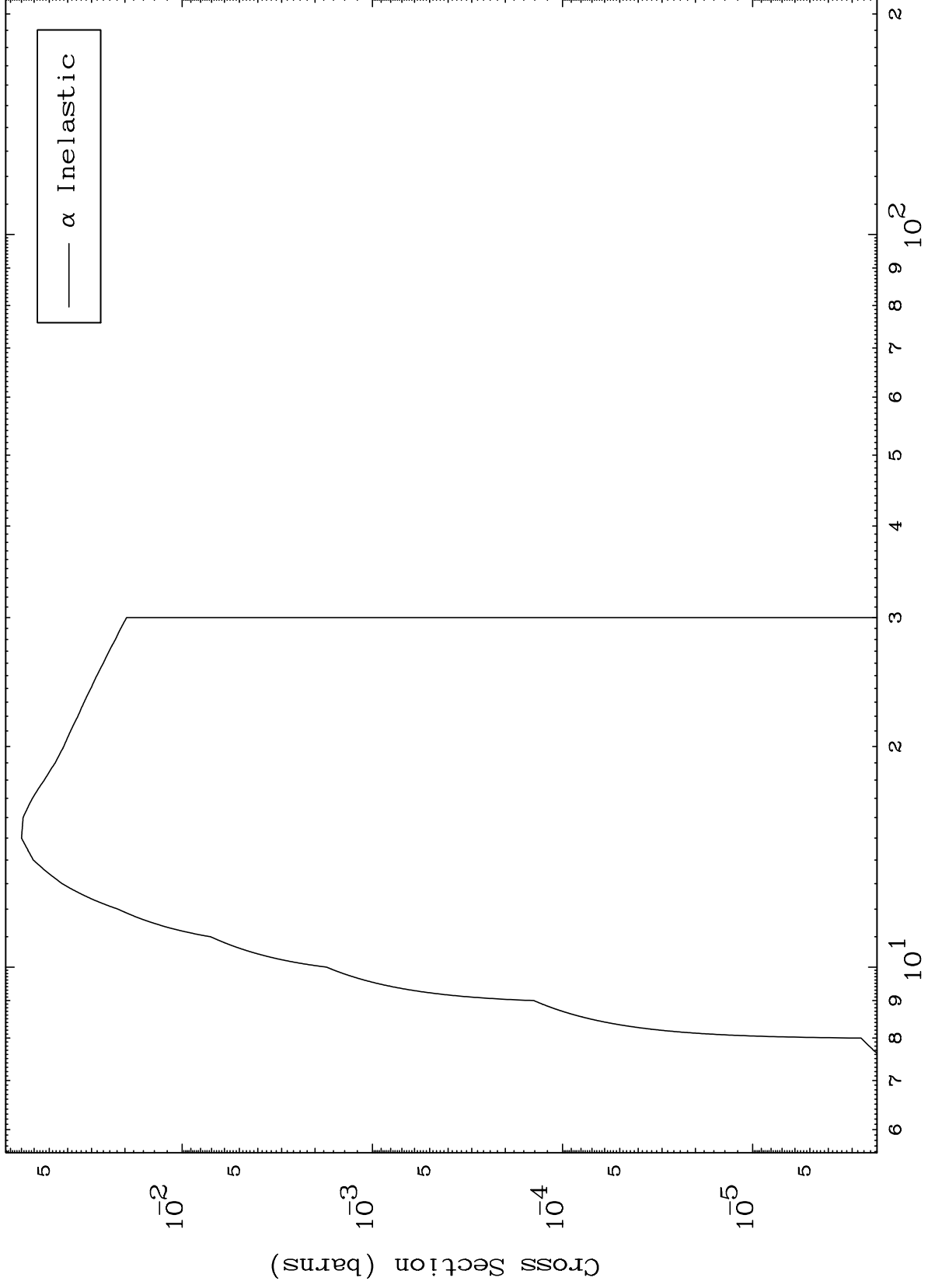


MAT 3911

( $\alpha, n'$ ) Level

39-Y -84

0 Kelvin Cross Sections

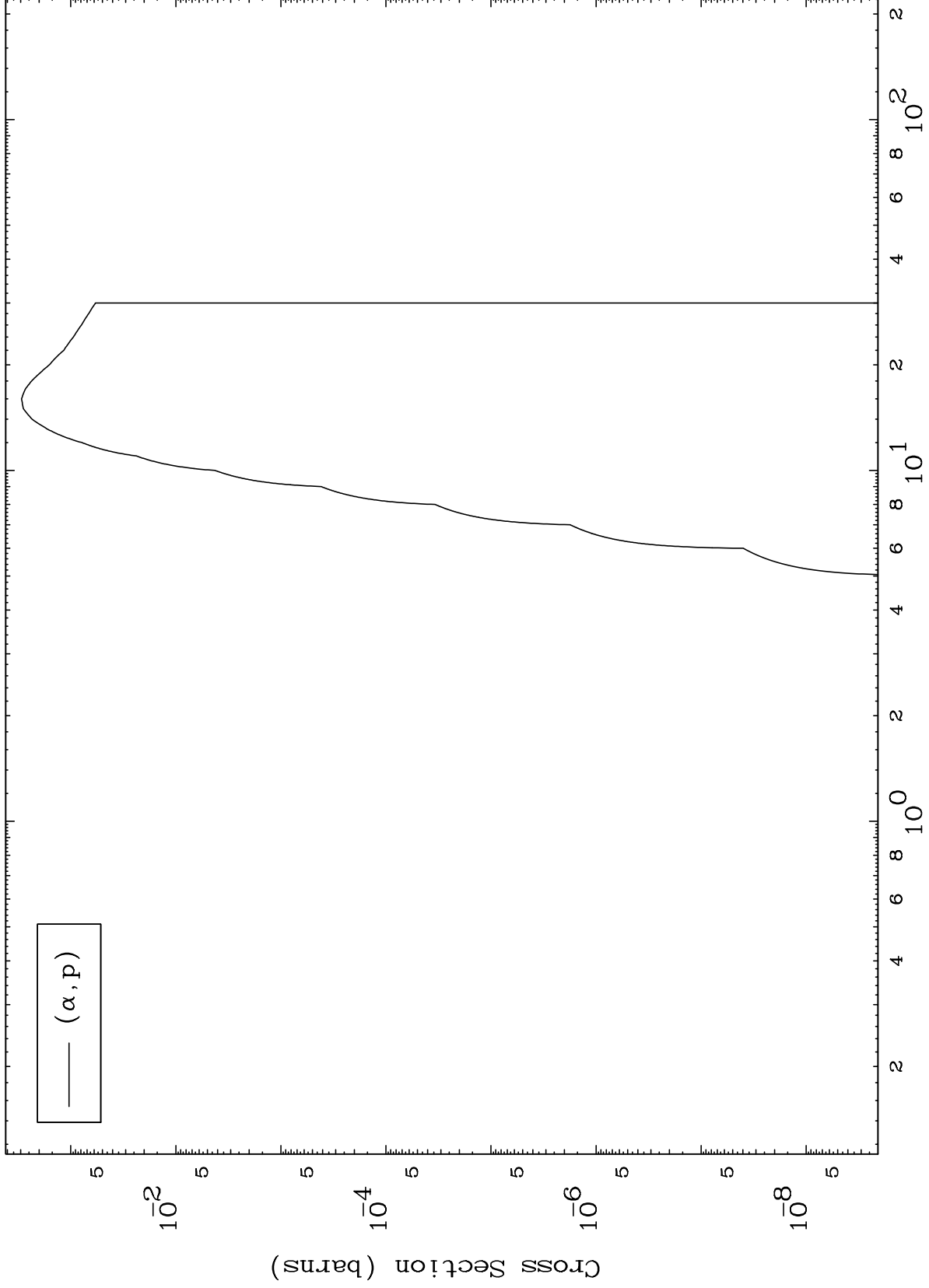


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Incident Energy (MeV)

39-Y -84

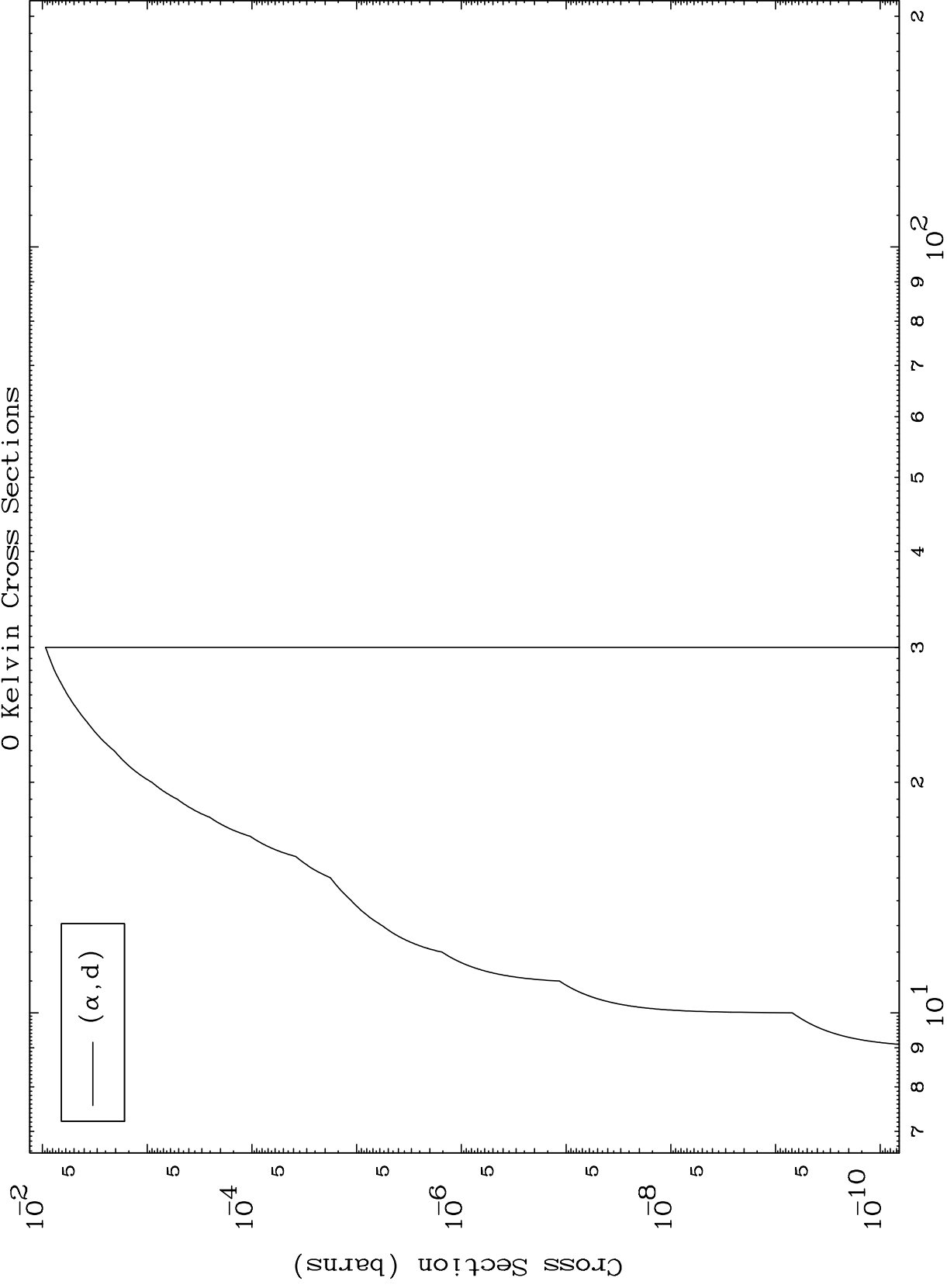
0 Kelvin Cross Sections



MAT 3911

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

39-Y -84

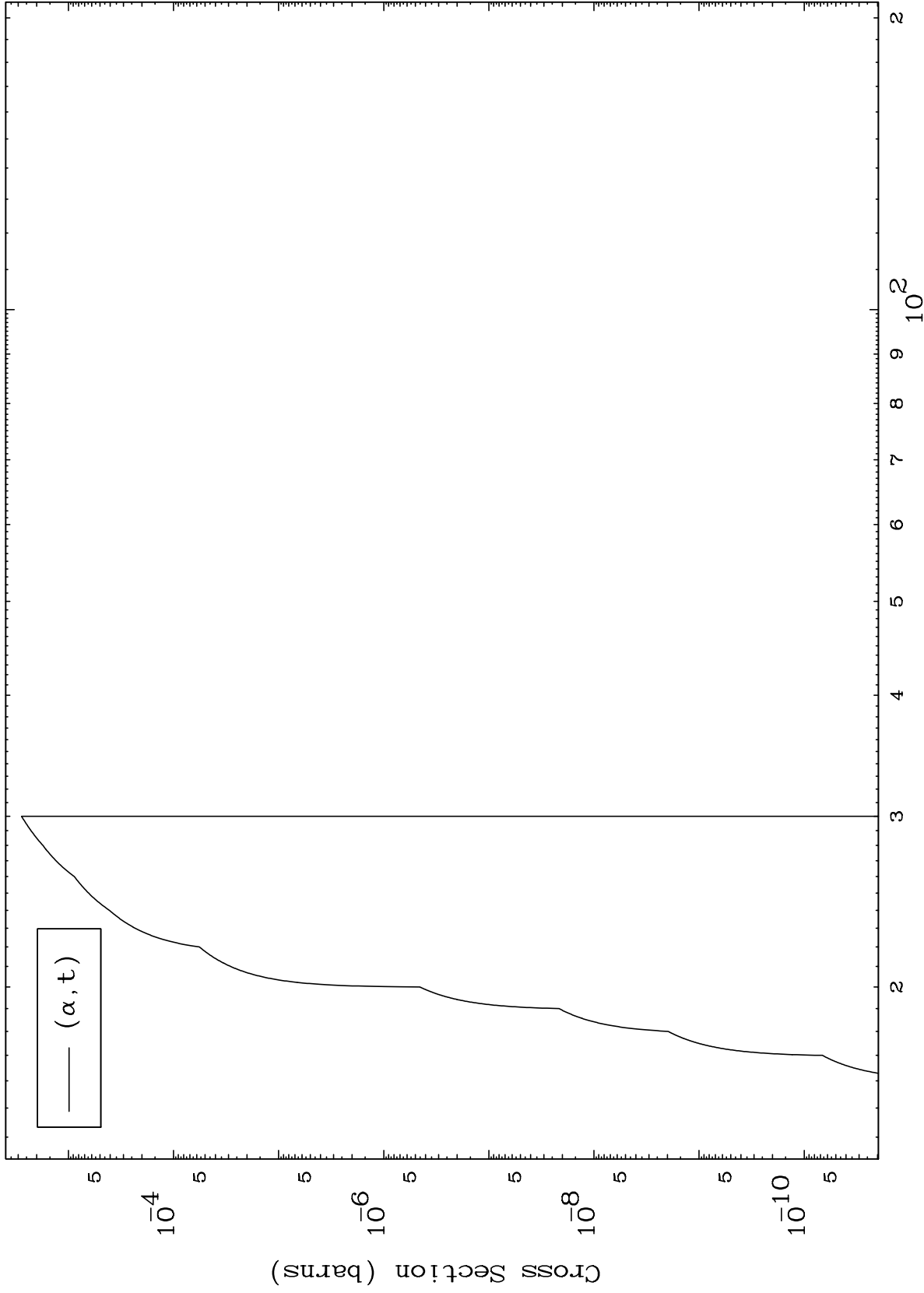


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Incident Energy (MeV)

39-Y -84



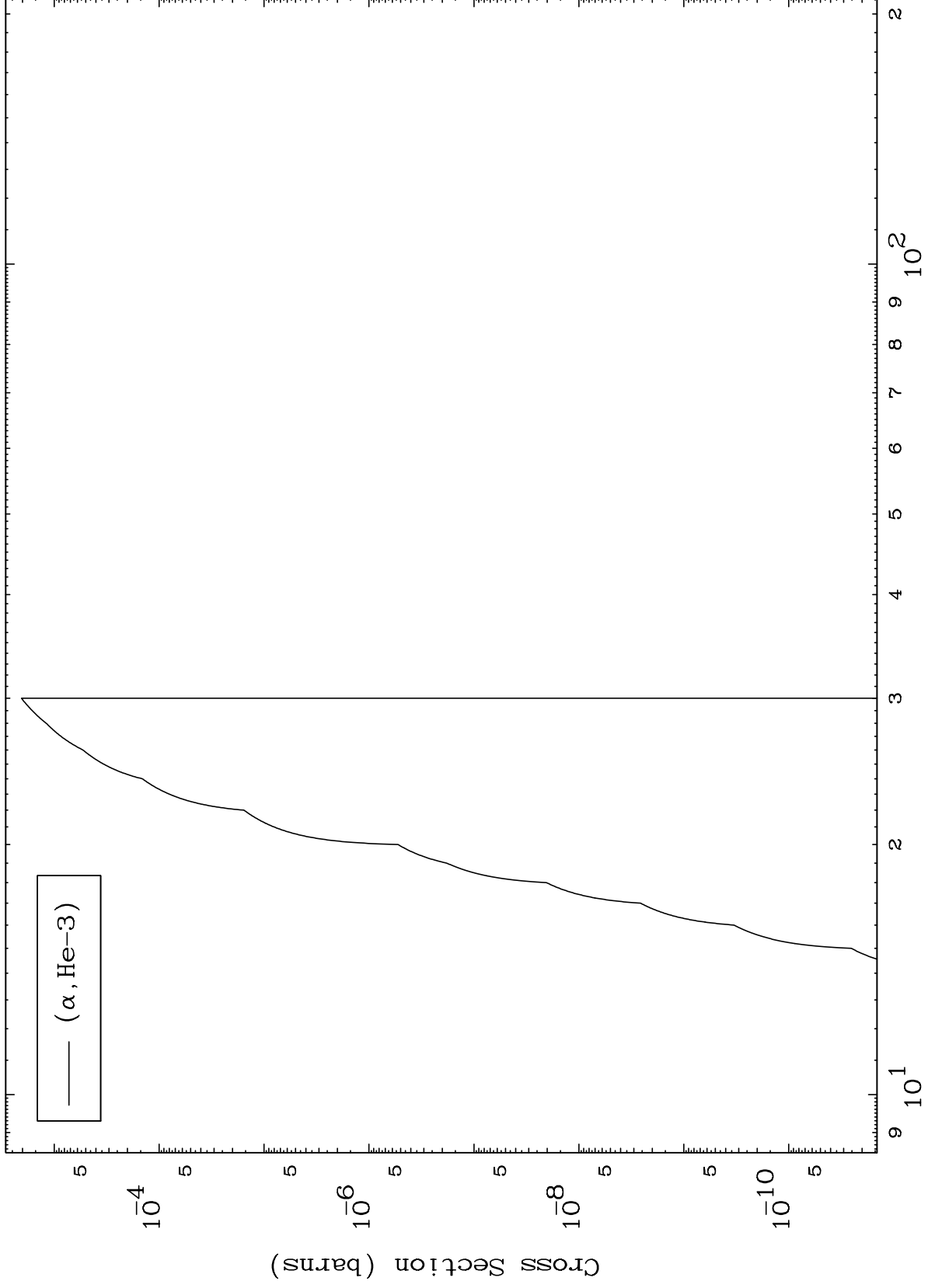


MAT 3911

( $\alpha$ , He3) Levels

39-Y -84

0 Kelvin Cross Sections



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Incident Energy (MeV)

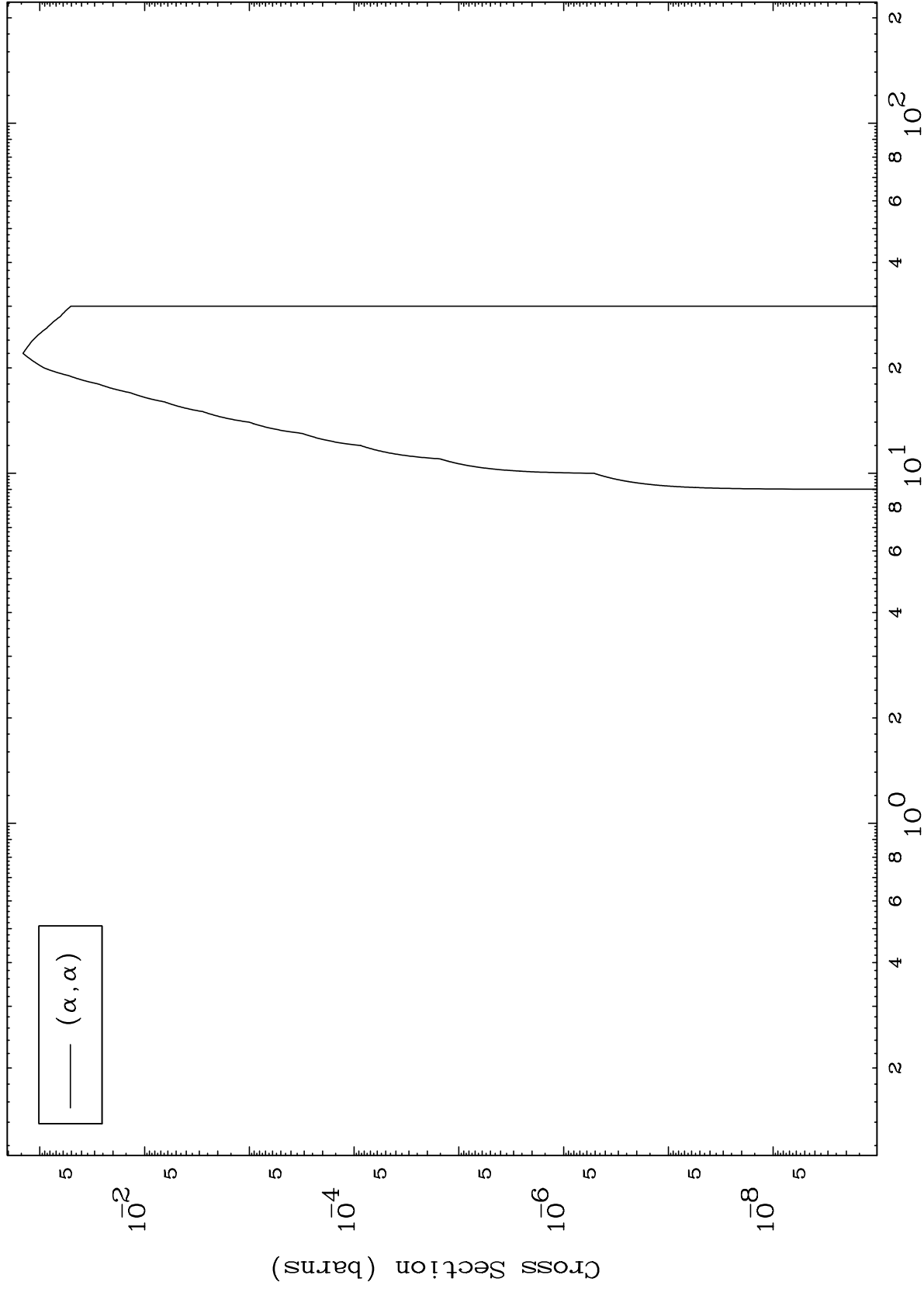
39-Y -84

MAT 3911

( $\alpha, \alpha$ ) Levels

39-Y -84

0 Kelvin Cross Sections



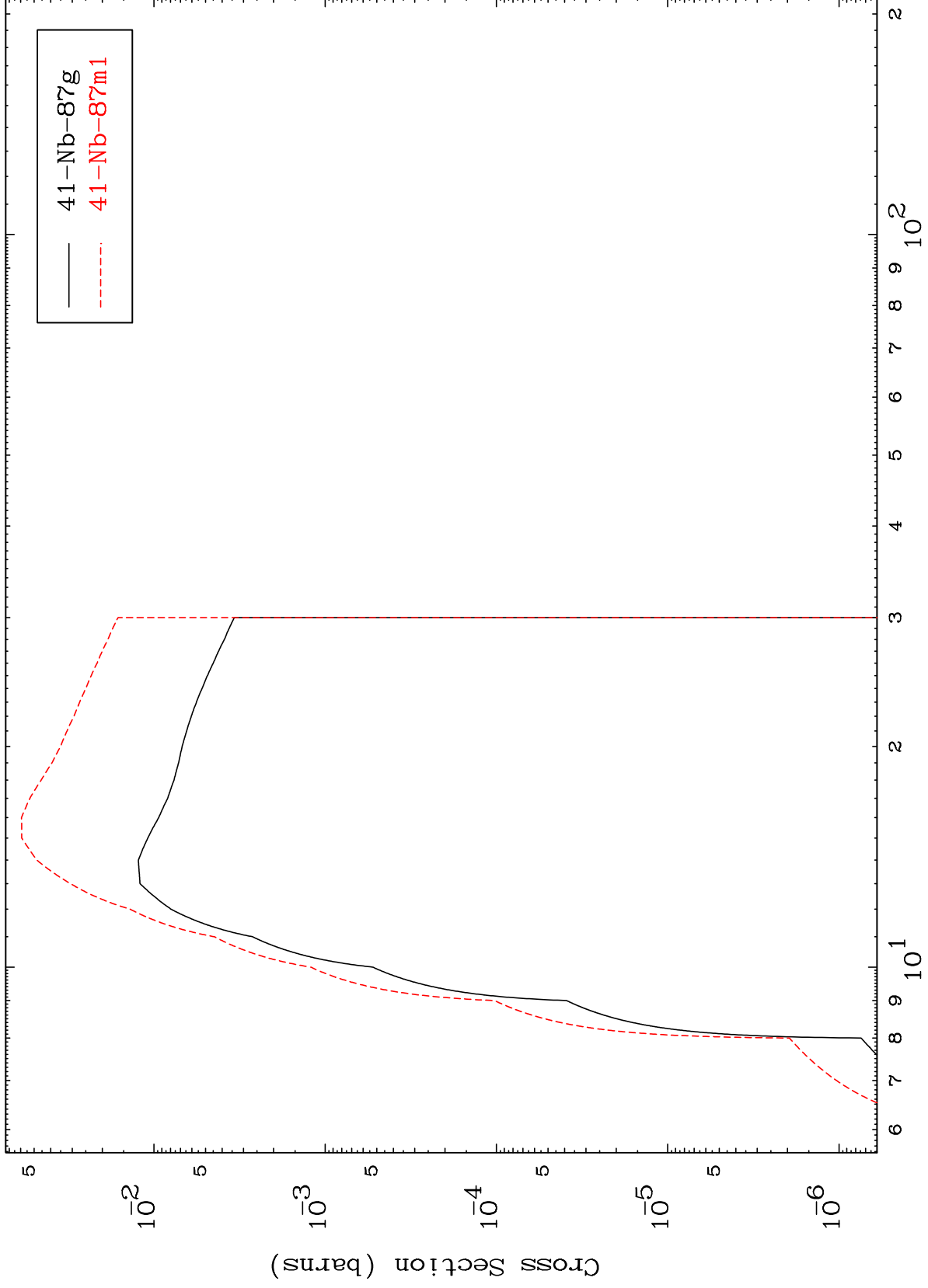
10

Incident Energy (MeV)

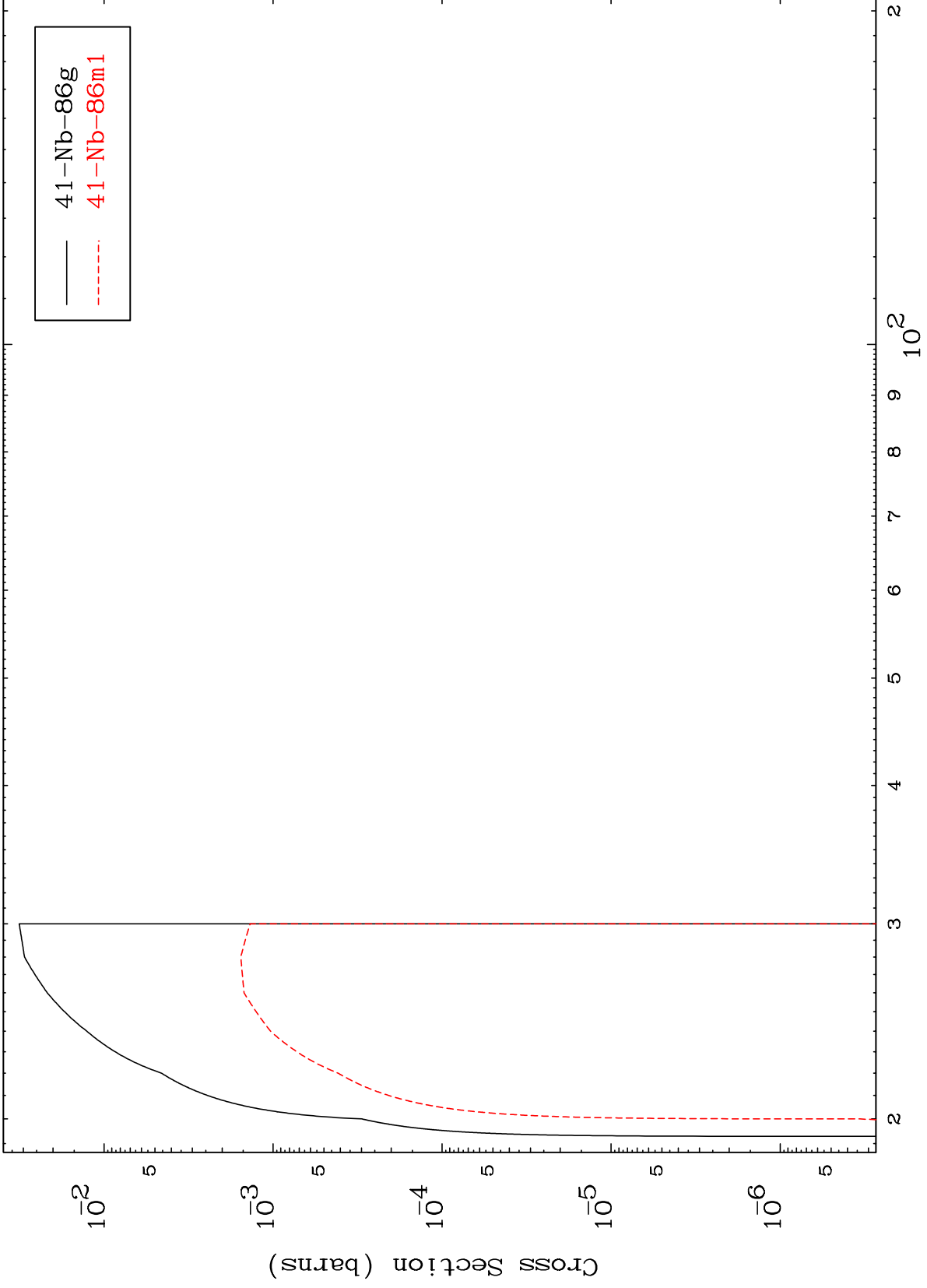
39-Y -84

Radionuclide Production Cross Section

$\alpha$  Inelastic



Radionuclide Production Cross Section  
( $\alpha, 2n$ )

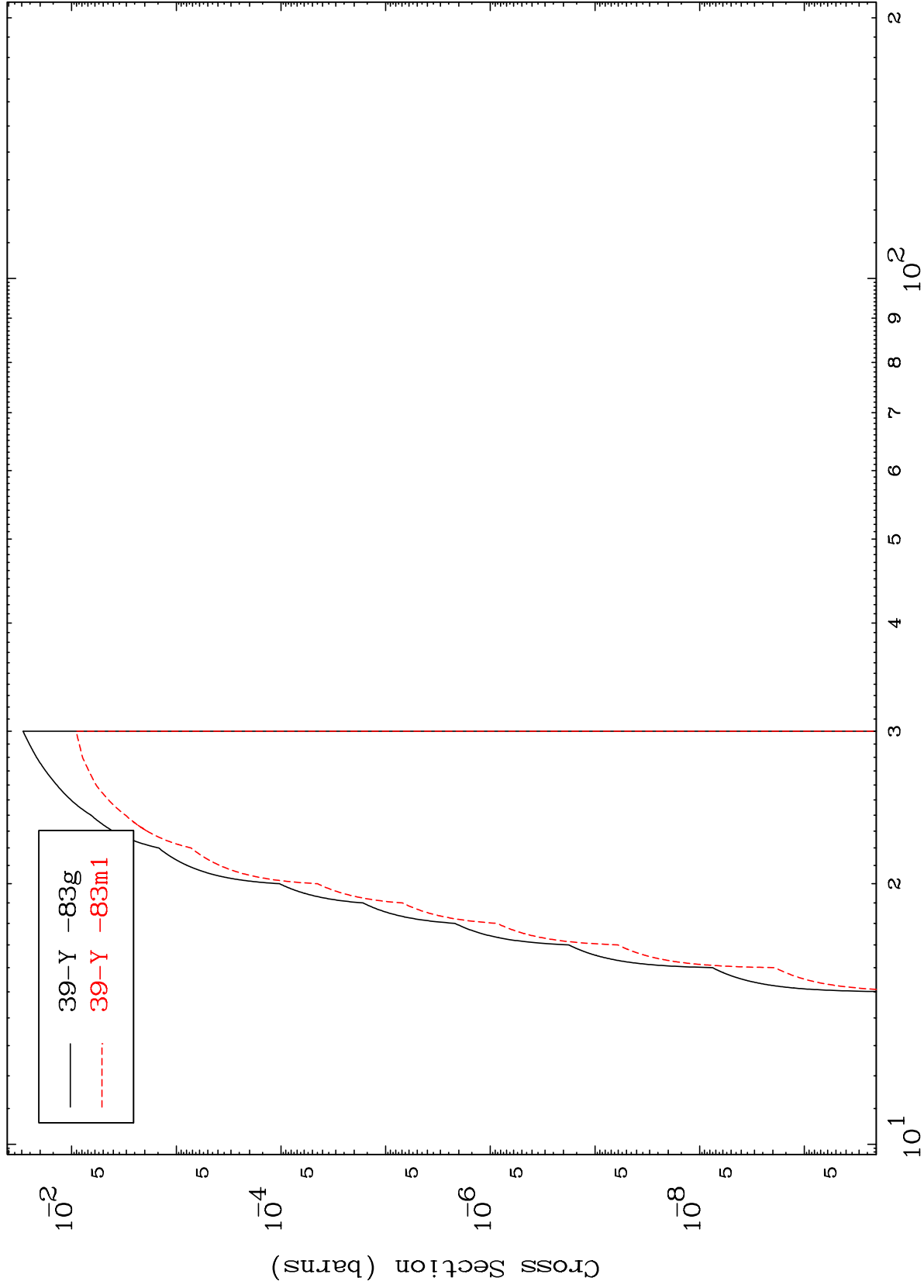


MAT 3911

( $\alpha, n'$ )  $\alpha$

39-Y -84

Radionuclide Production Cross Section



10<sup>1</sup>

Incident Energy (MeV)

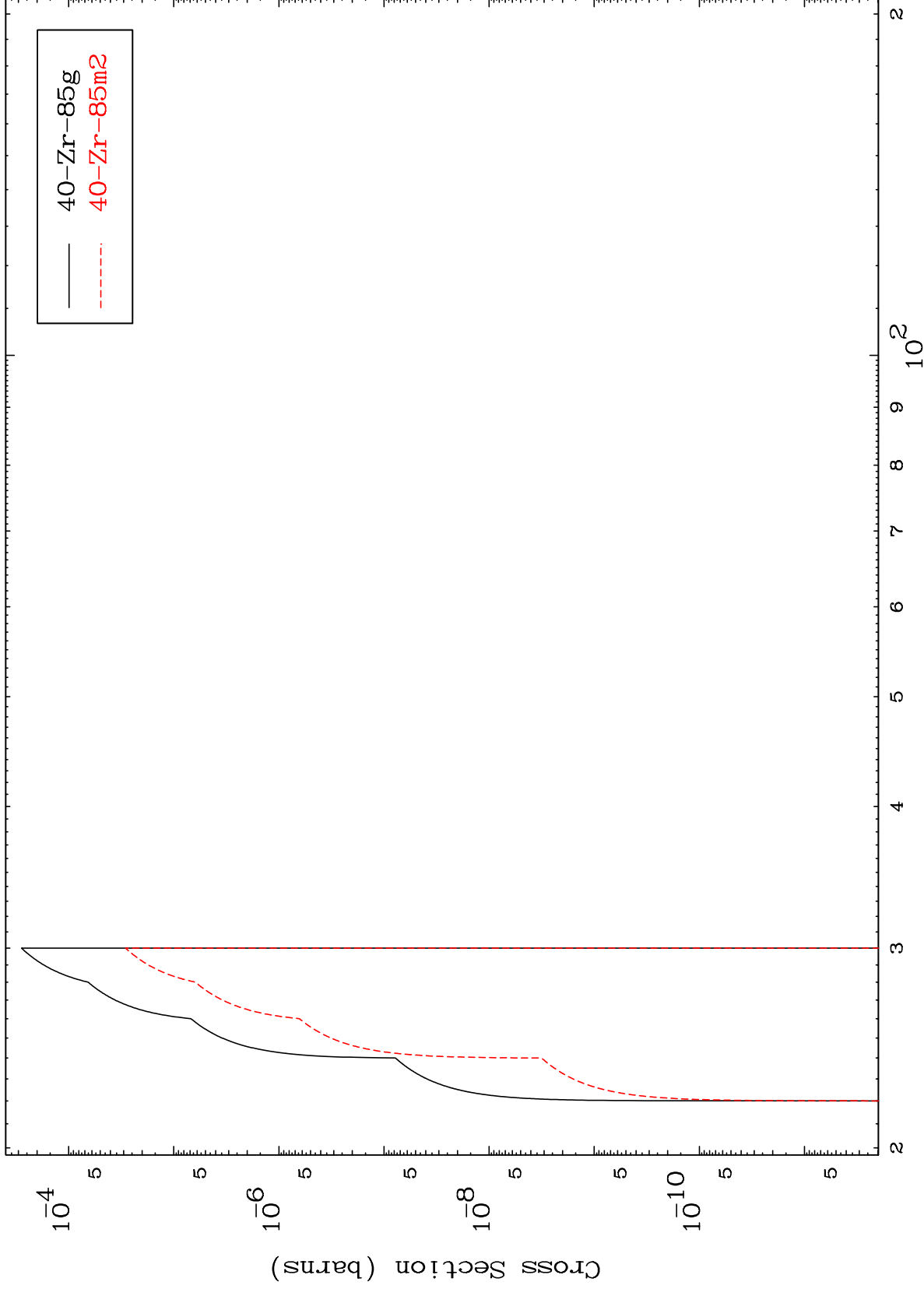
39-Y -84

MAT 3911

( $\alpha, n'$ ) d

39-Y -84

Radionuclide Production Cross Section

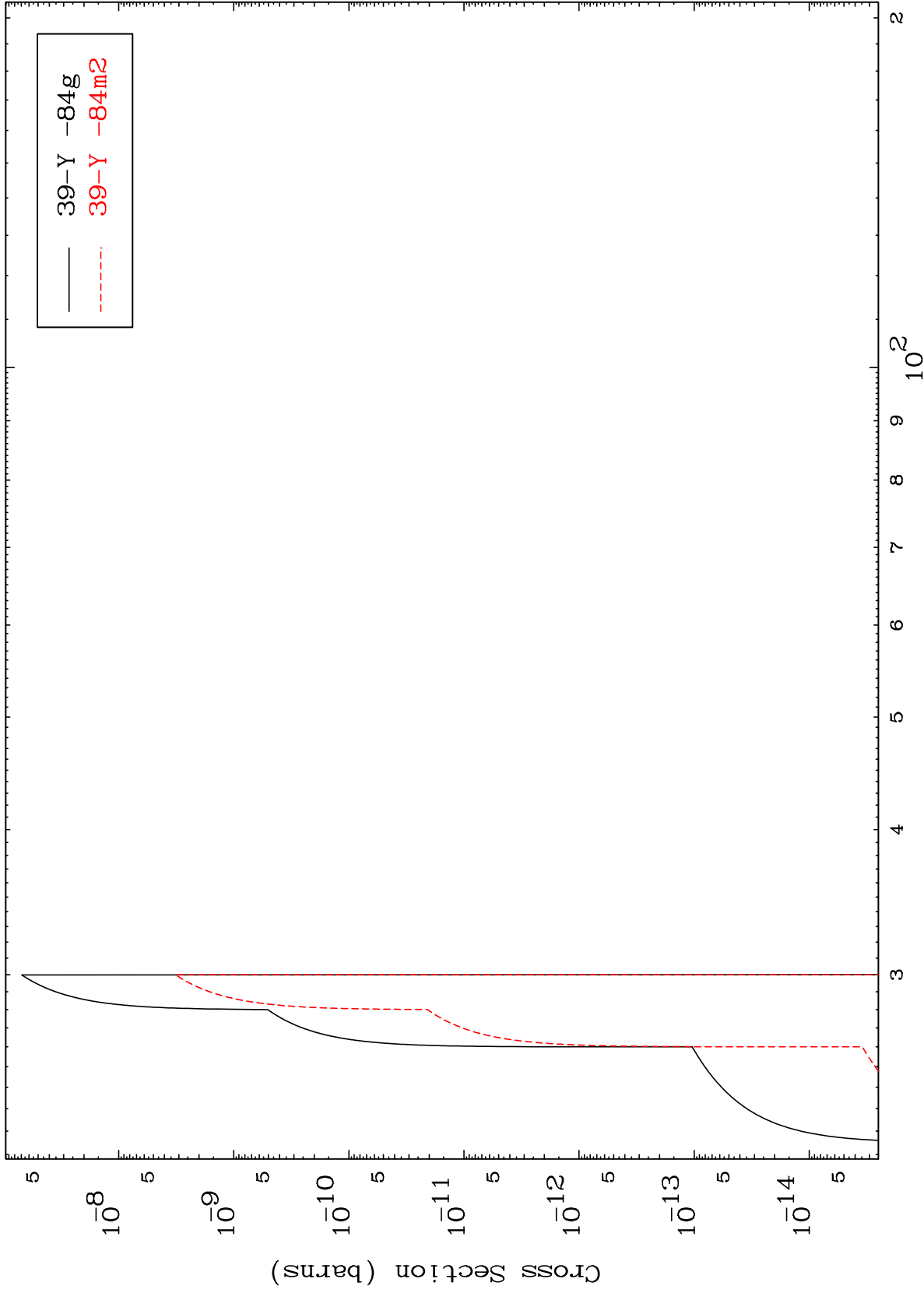


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Incident Energy (MeV)

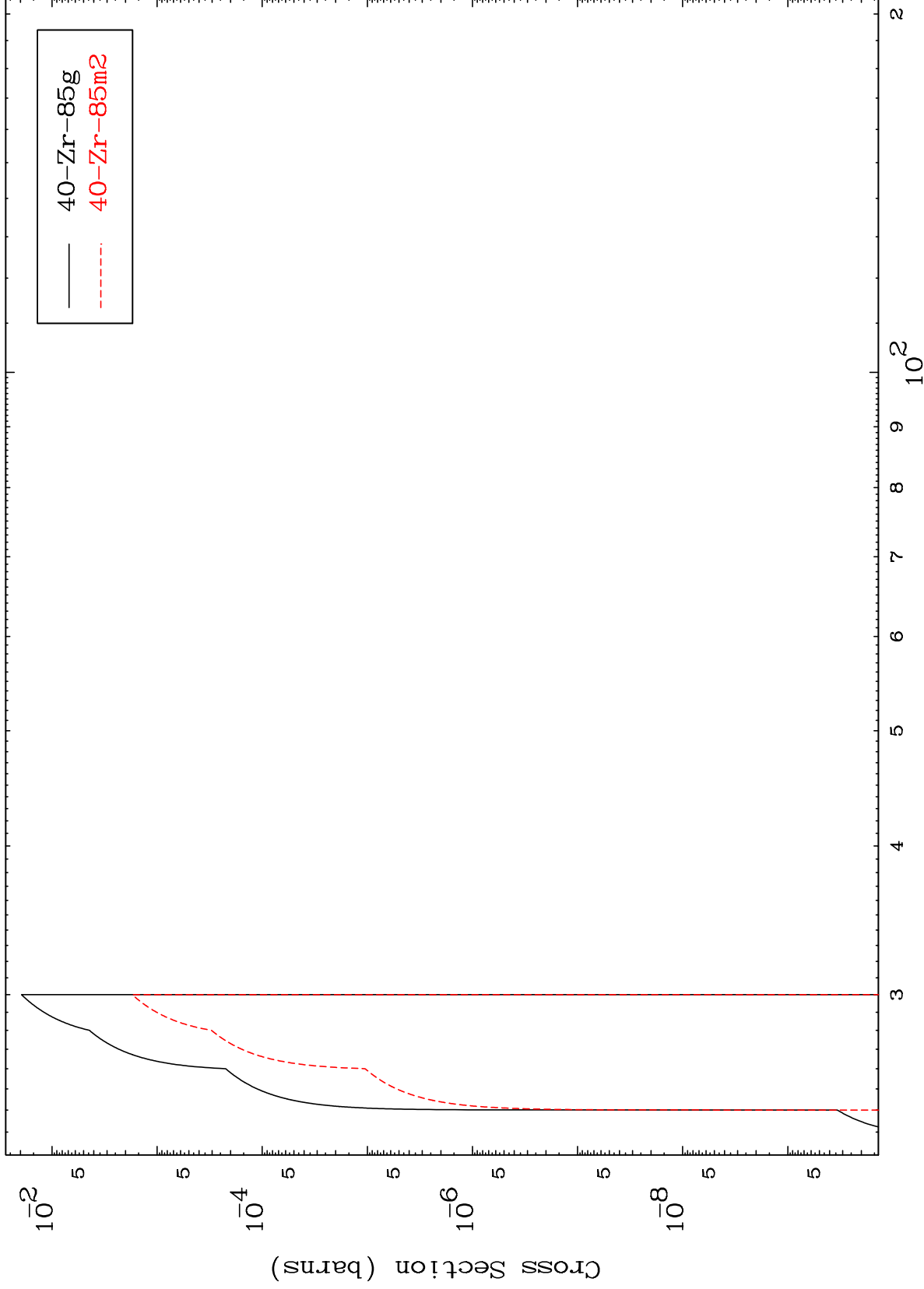
39-Y -84

Radionuclide Production Cross Section

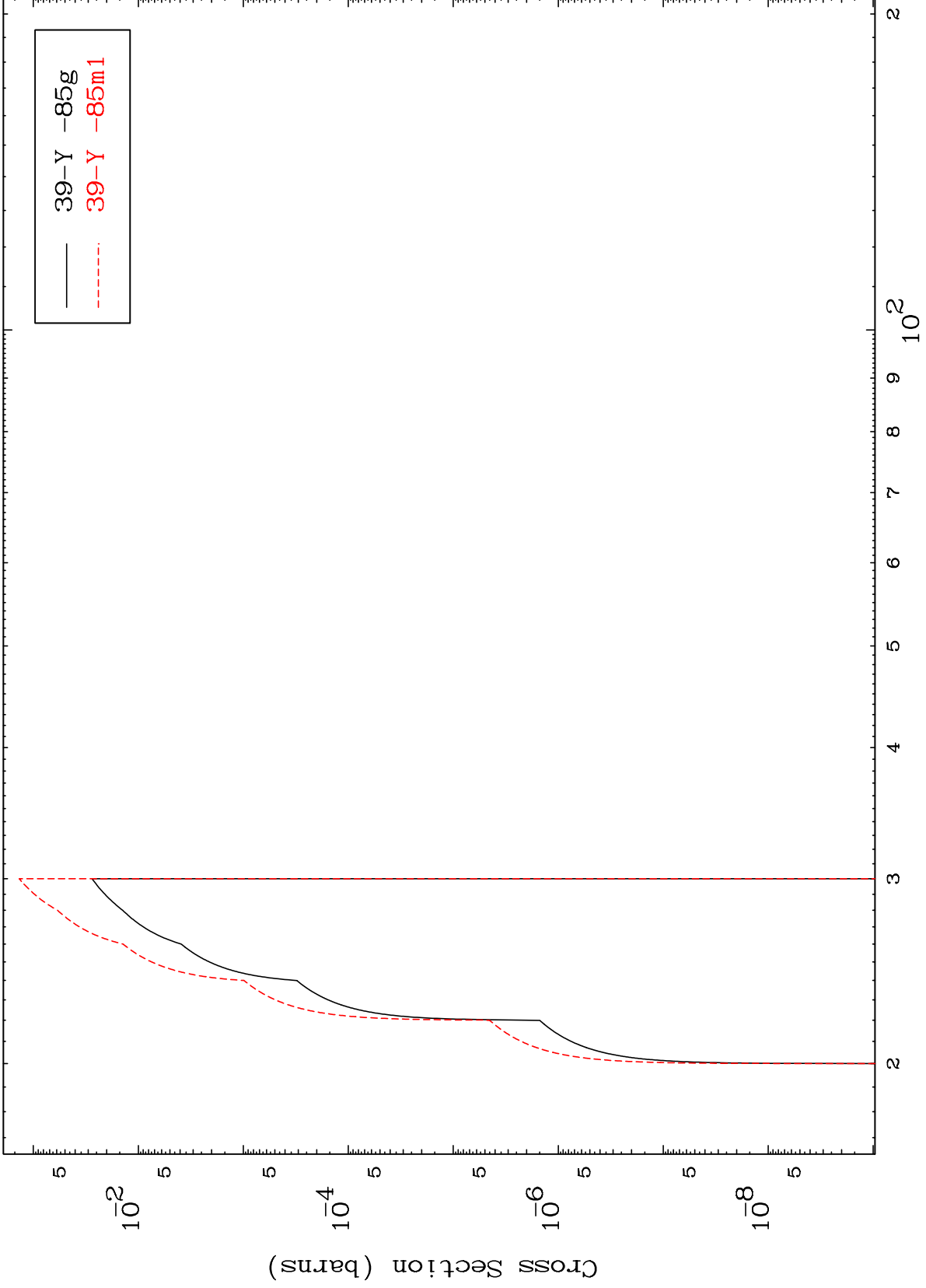




Radionuclide Production Cross Section



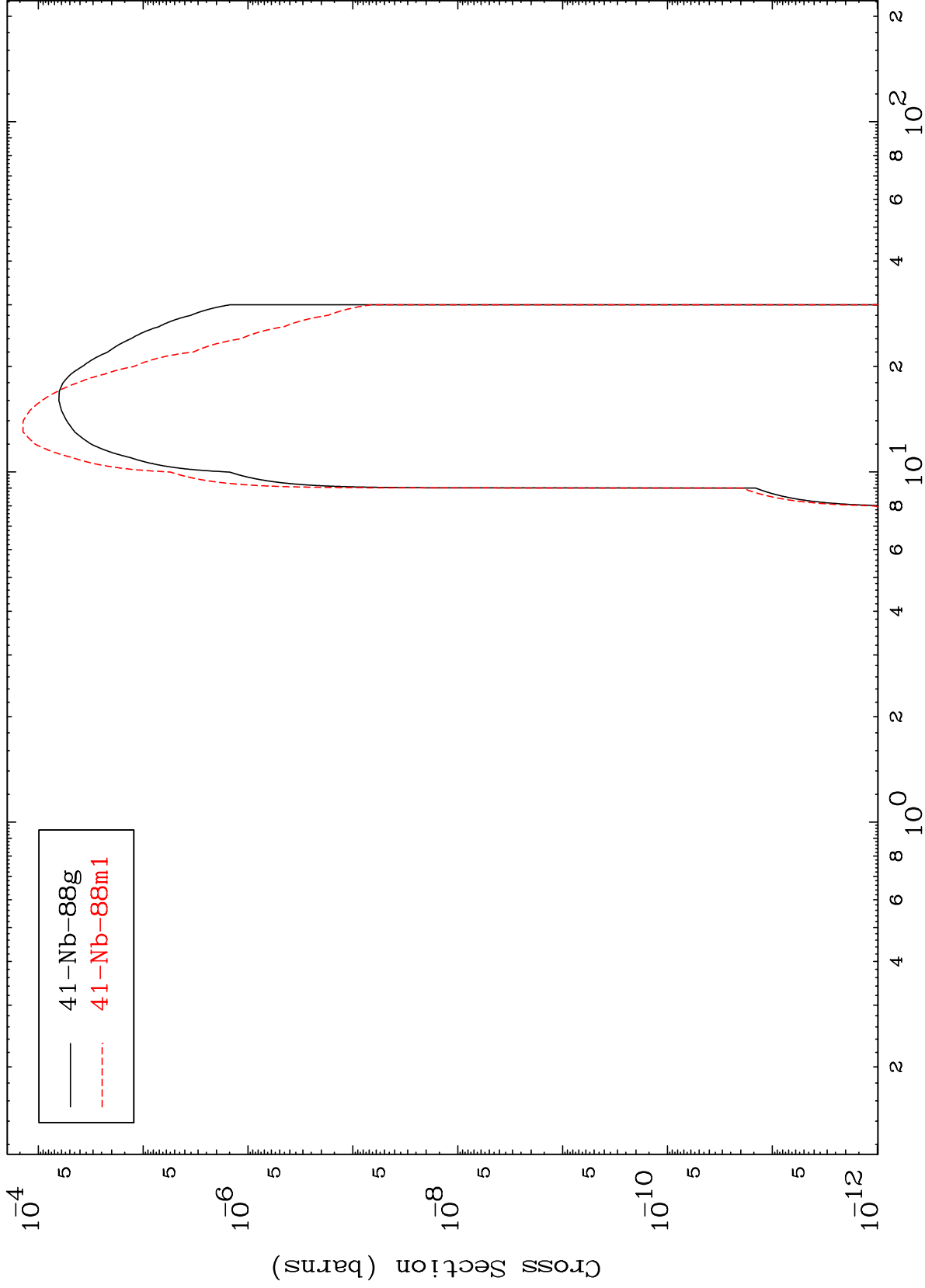
Radionuclide Production Cross Section



MAT 3911

39-Y -84

Radionuclide Production Cross Section  
( $\alpha, \gamma$ )



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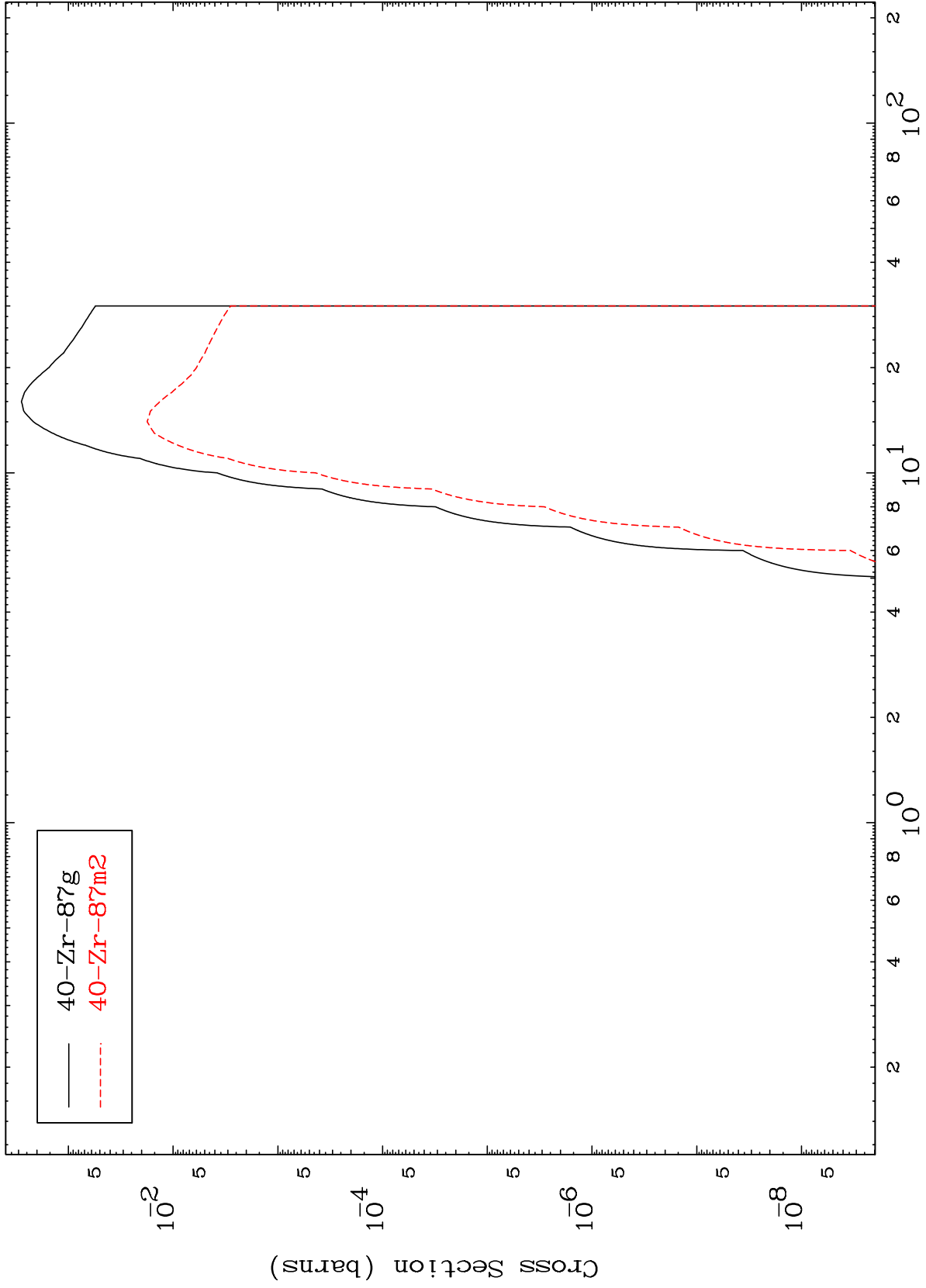
39-Y -84

Incident Energy (MeV)

MAT 3911

39-Y -84

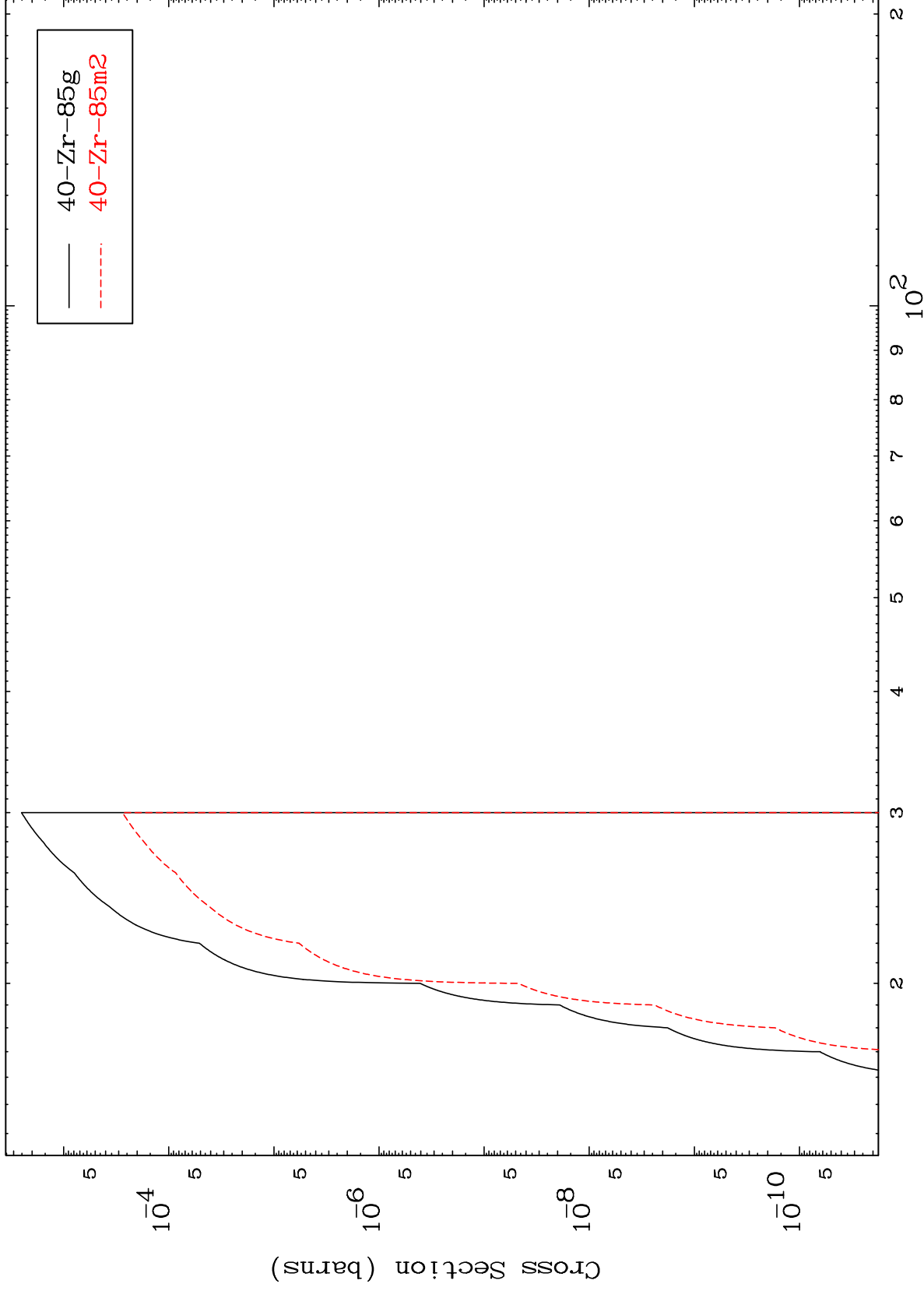
Radionuclide Production Cross Section  
( $\alpha, p$ )



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39-Y -84

Radionuclide Production Cross Section

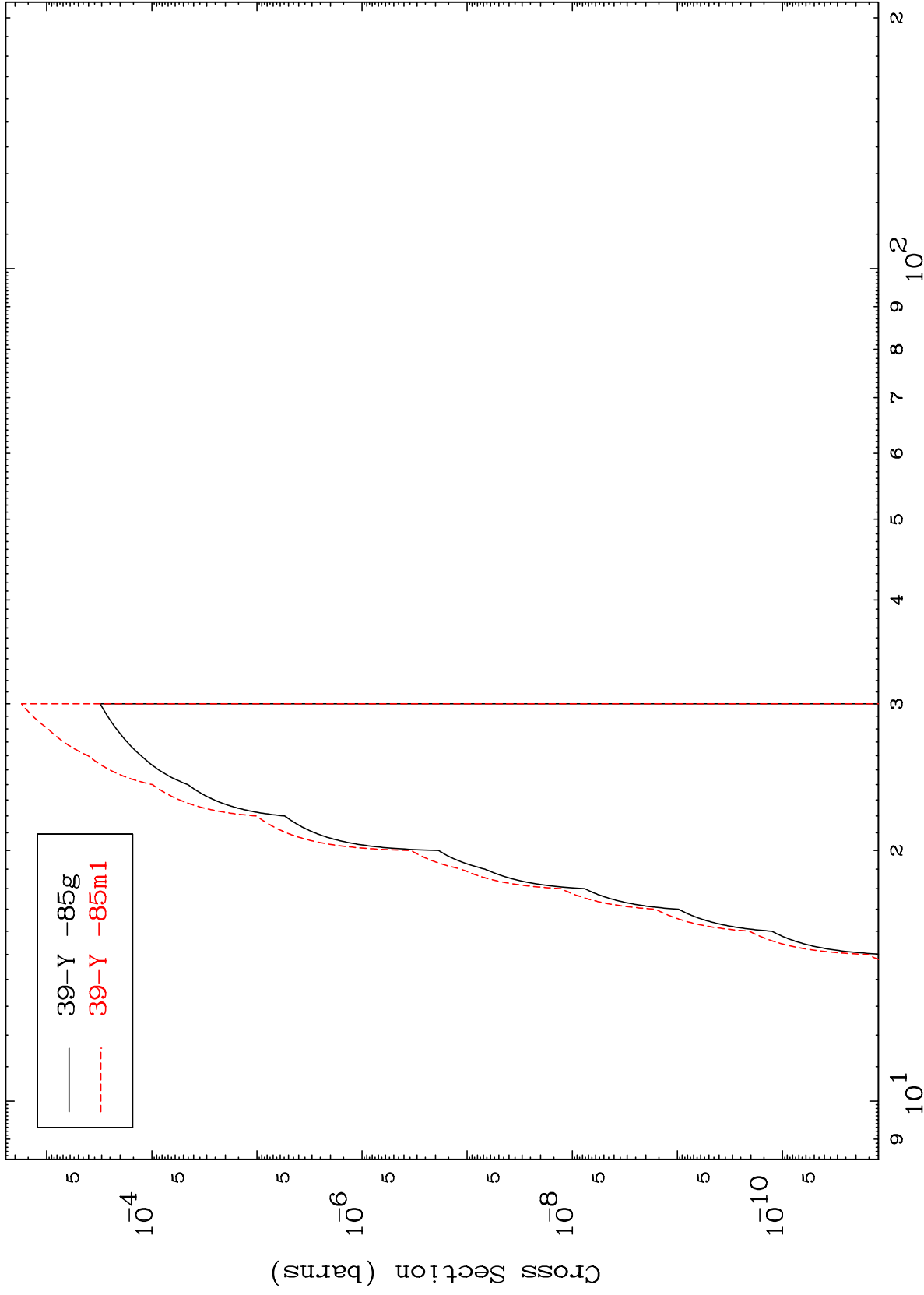


MAT 3911

( $\alpha$ ,He-3)

39-Y -84

Radionuclide Production Cross Section



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Incident Energy (MeV)

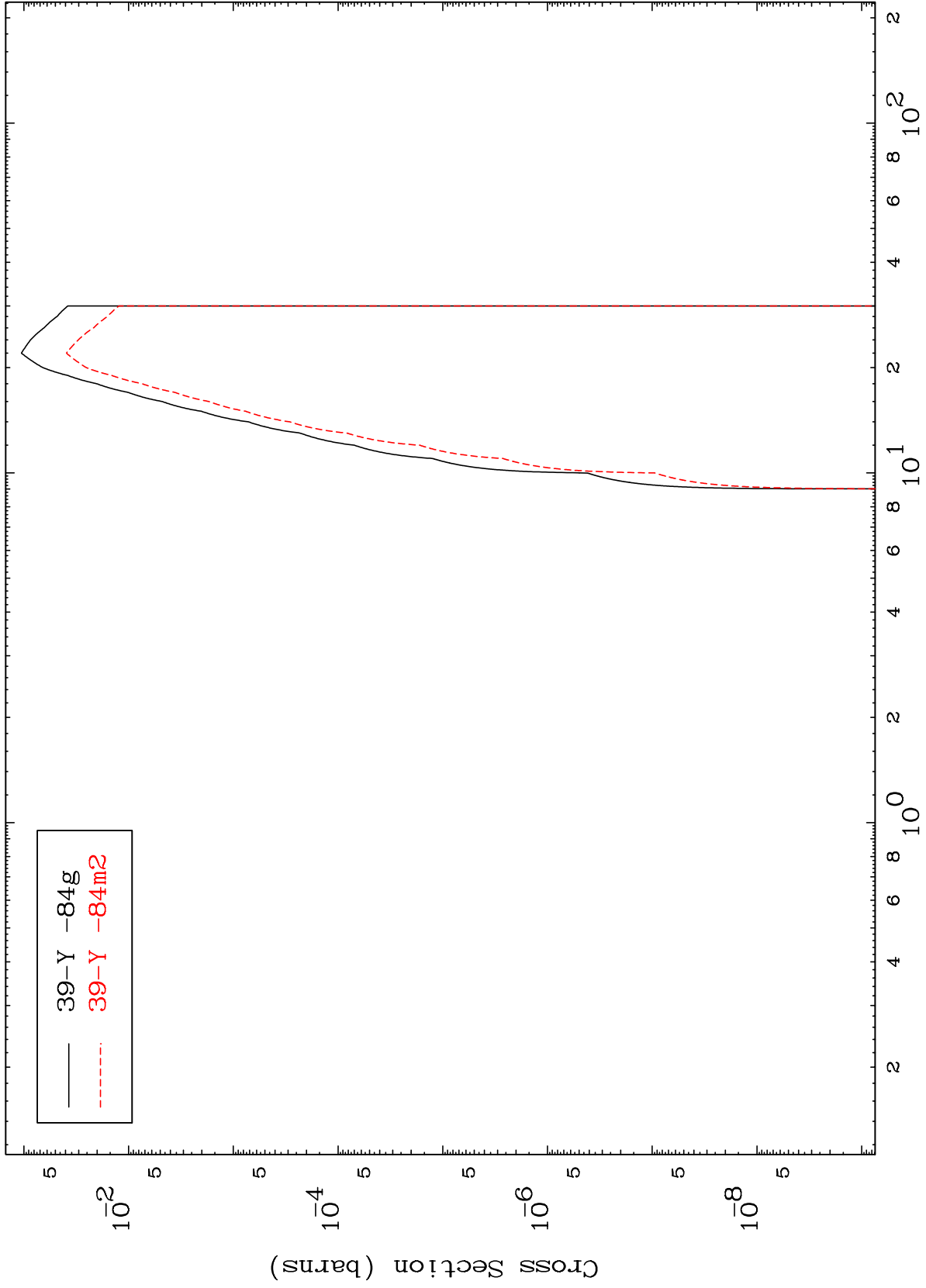
39-Y -84

MAT 3911

( $\alpha, \alpha$ )

39-Y -84

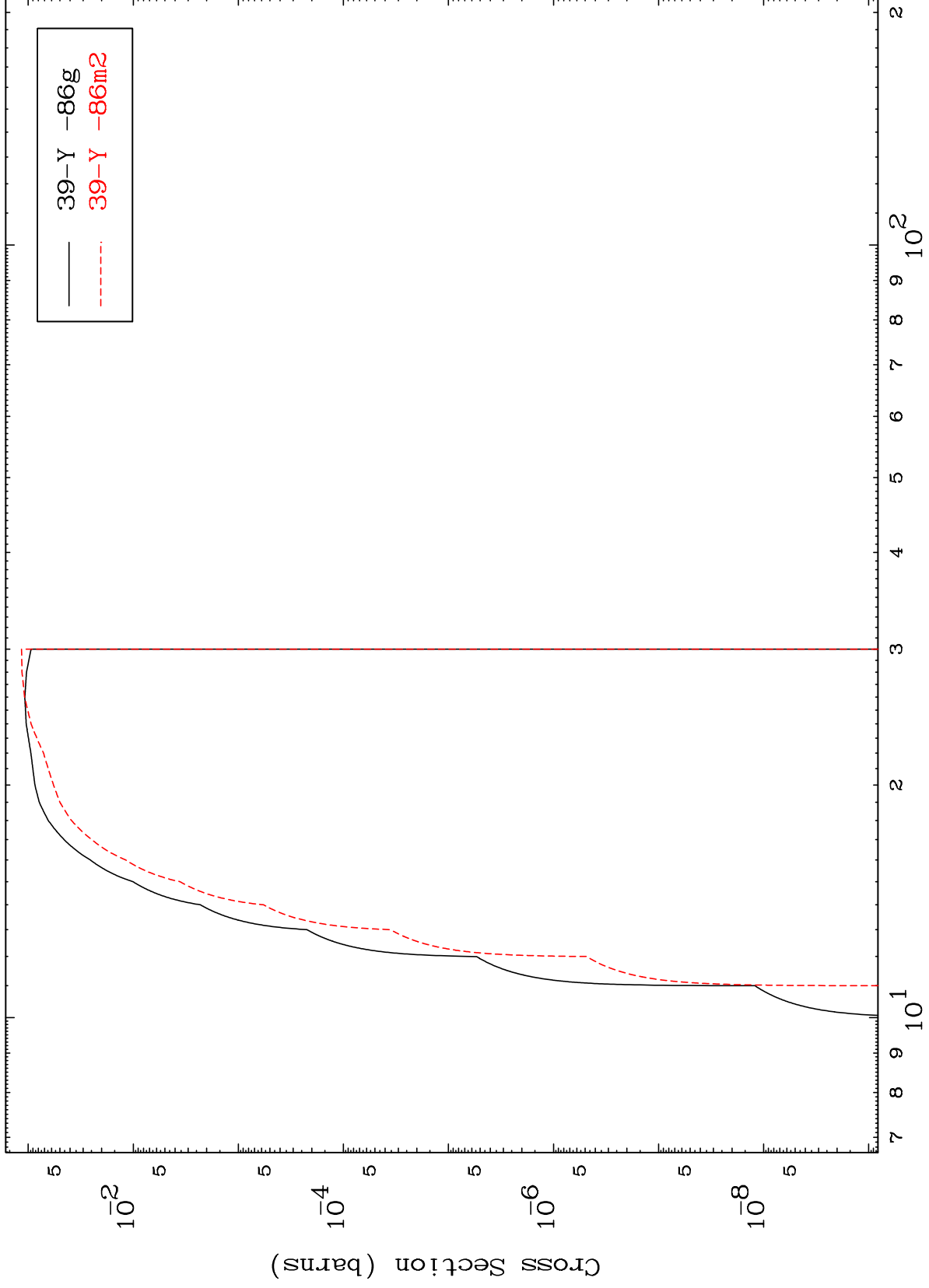
Radionuclide Production Cross Section



MAT 3911

39-Y -84

( $\alpha, 2p$ )  
Radionuclide Production Cross Section



23

Incident Energy (MeV)

39-Y -84

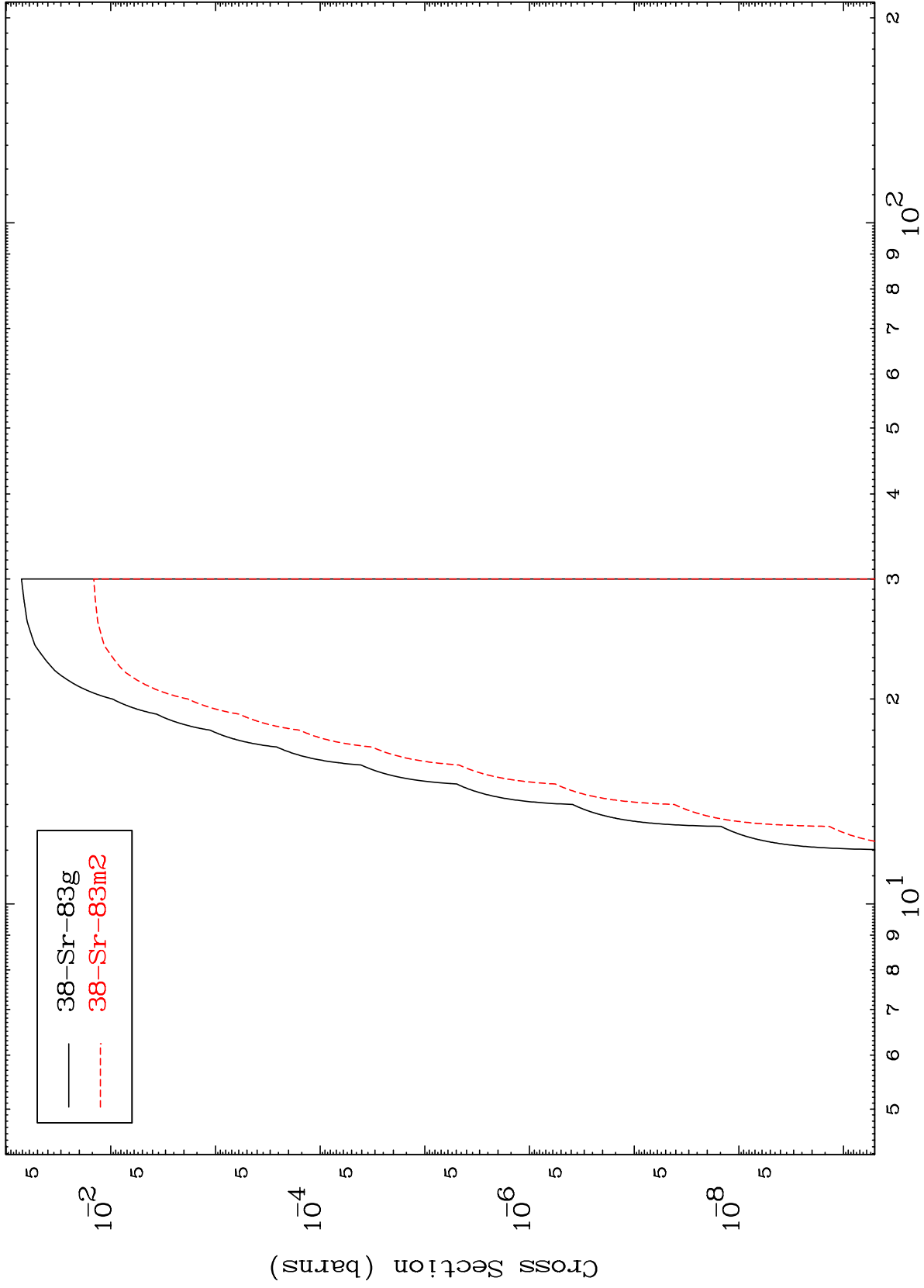


MAT 3911

( $\alpha, p$ )  $\alpha$

39-Y -84

Radionuclide Production Cross Section

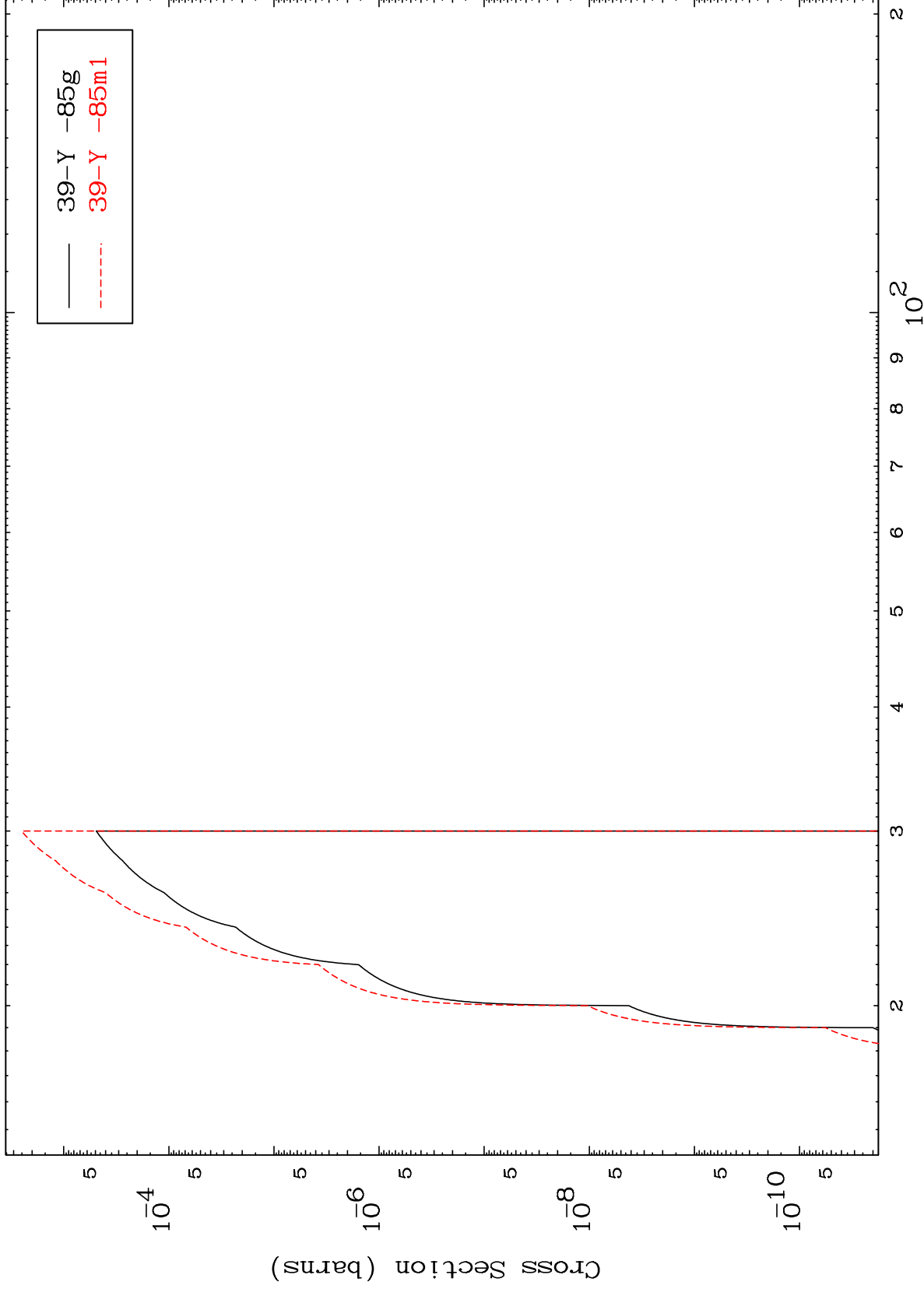


Incident Energy (MeV)

39-Y -84

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Radionuclide Production Cross Section



Radionuclide Production Cross Section

