

Program EVALPLOT
(Version 2018-1)

by

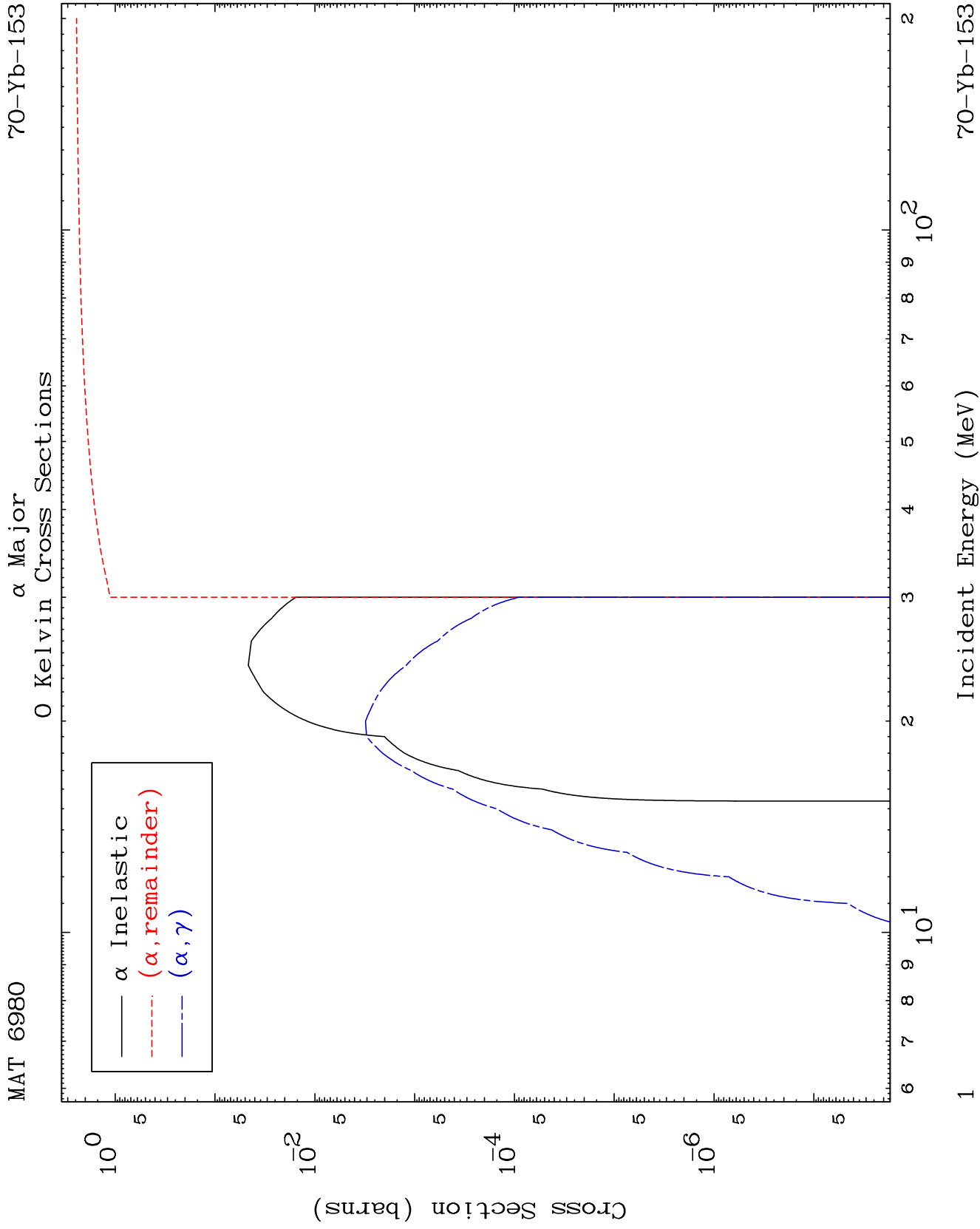
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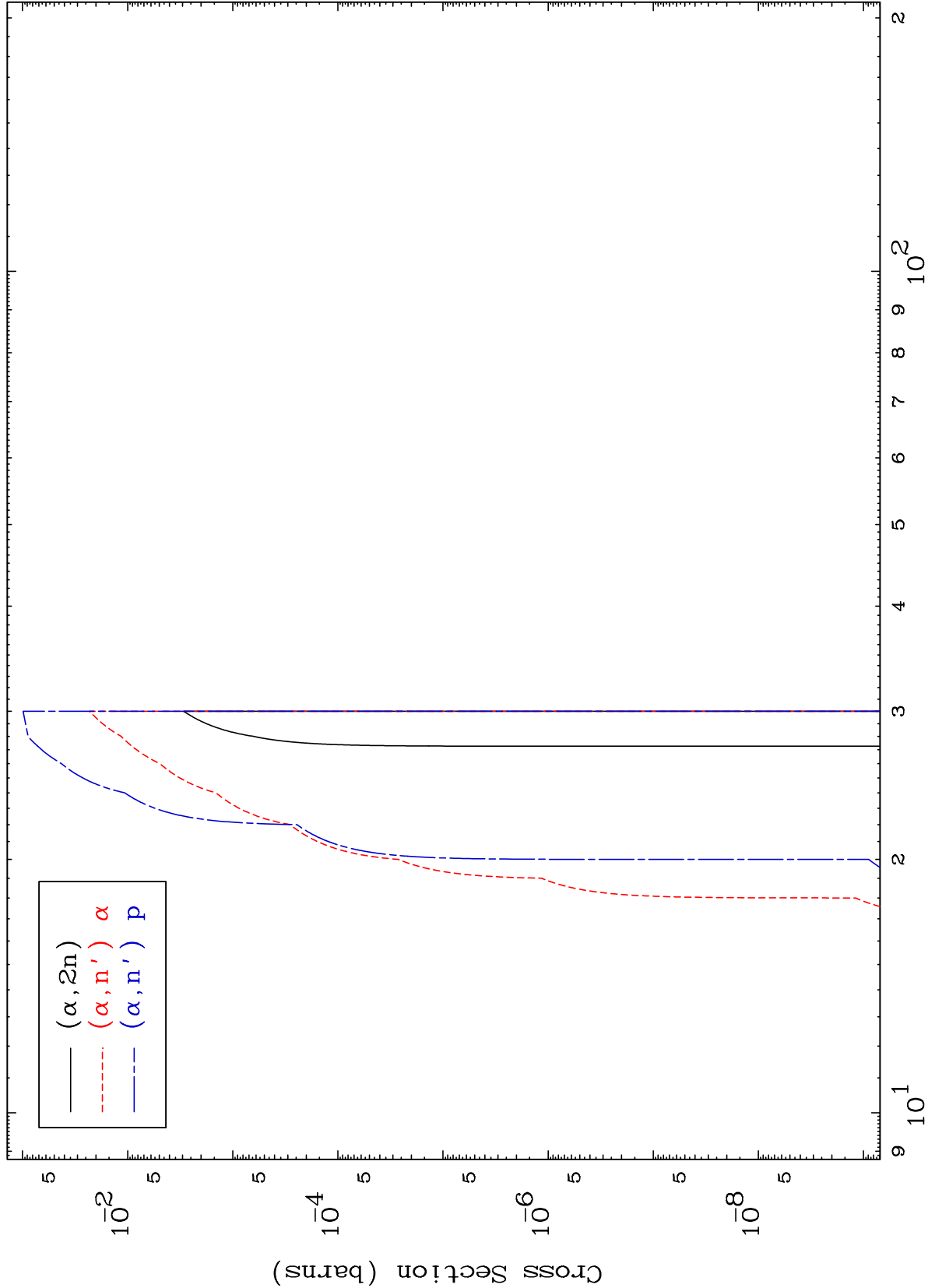
Press Mouse Button to Start



MAT 6980

α Neutron Production
0 Kelvin Cross Sections

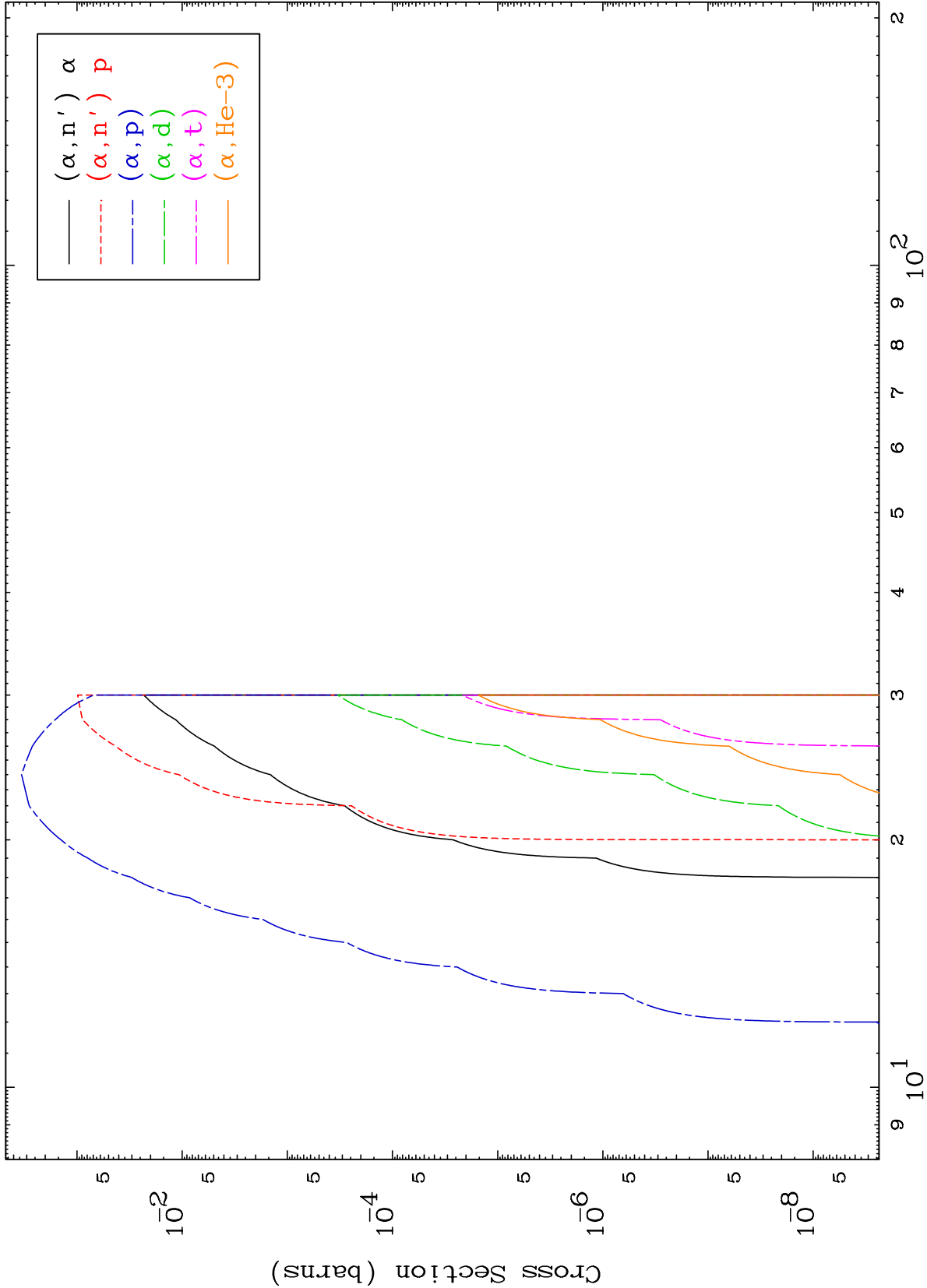
70-Yb-153

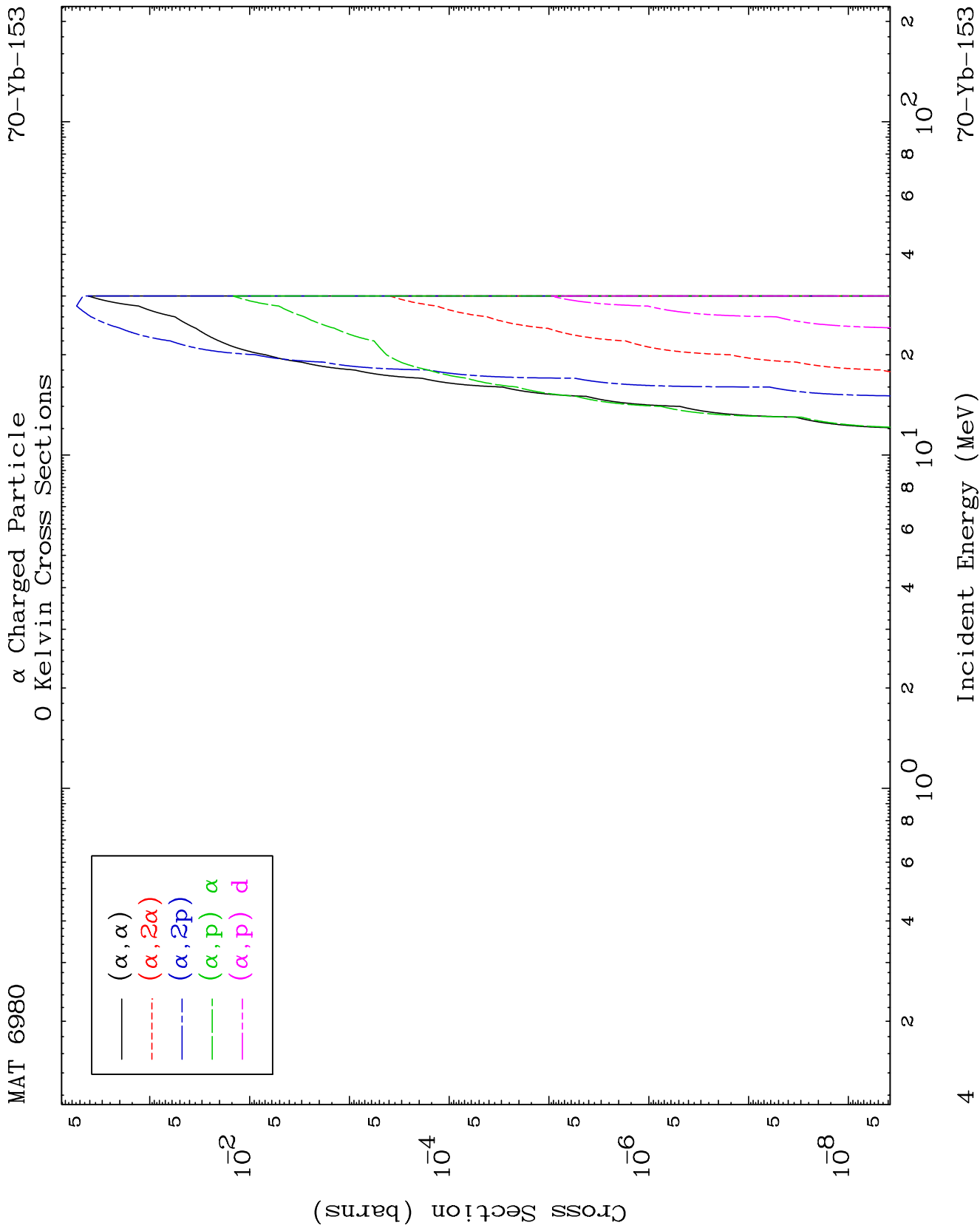


Incident Energy (MeV)

70-Yb-153

0 Kelvin Cross Sections



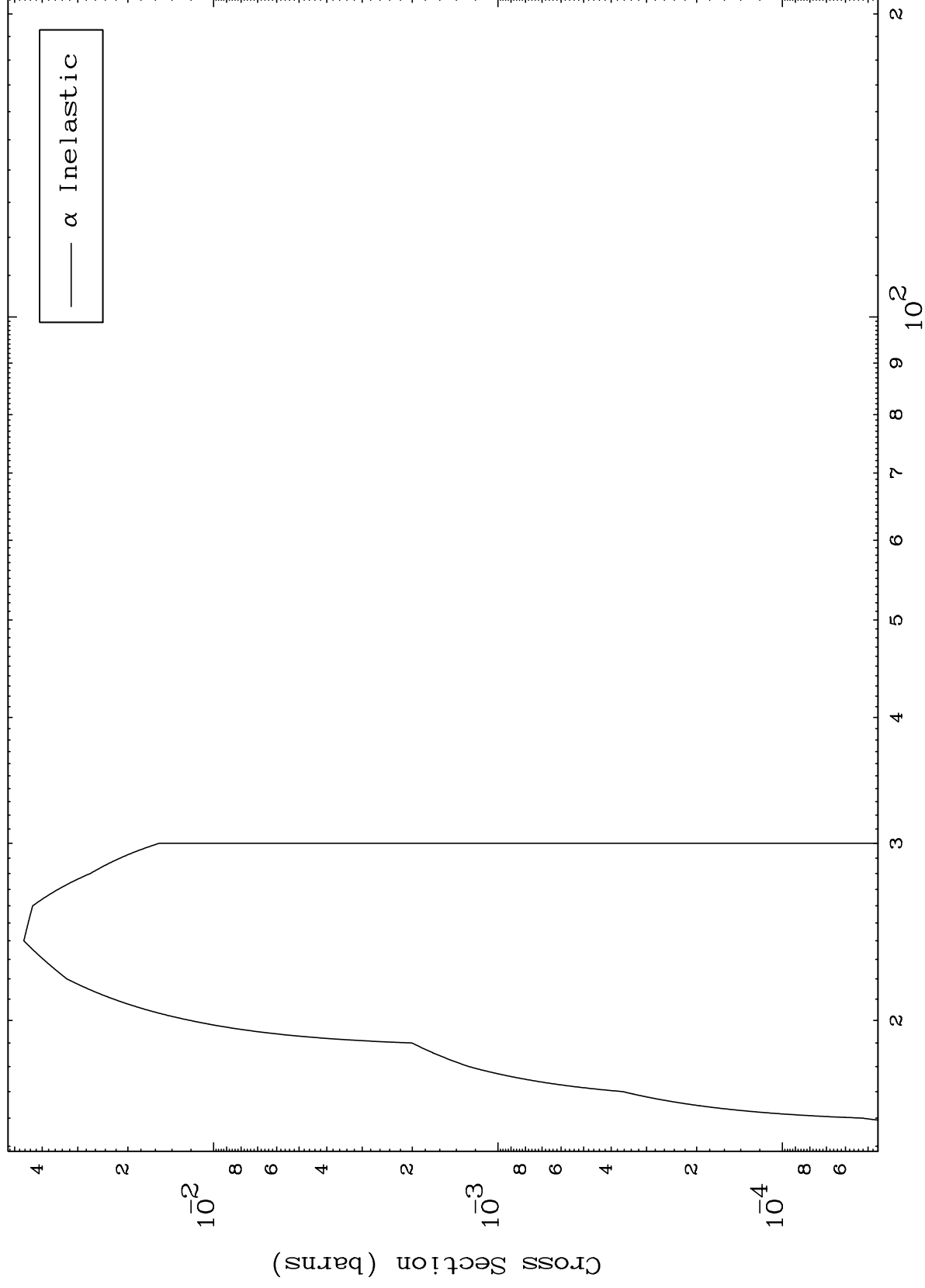


MAT 6980

(α, n') Level

70-Yb-153

0 Kelvin Cross Sections



5

Incident Energy (MeV)

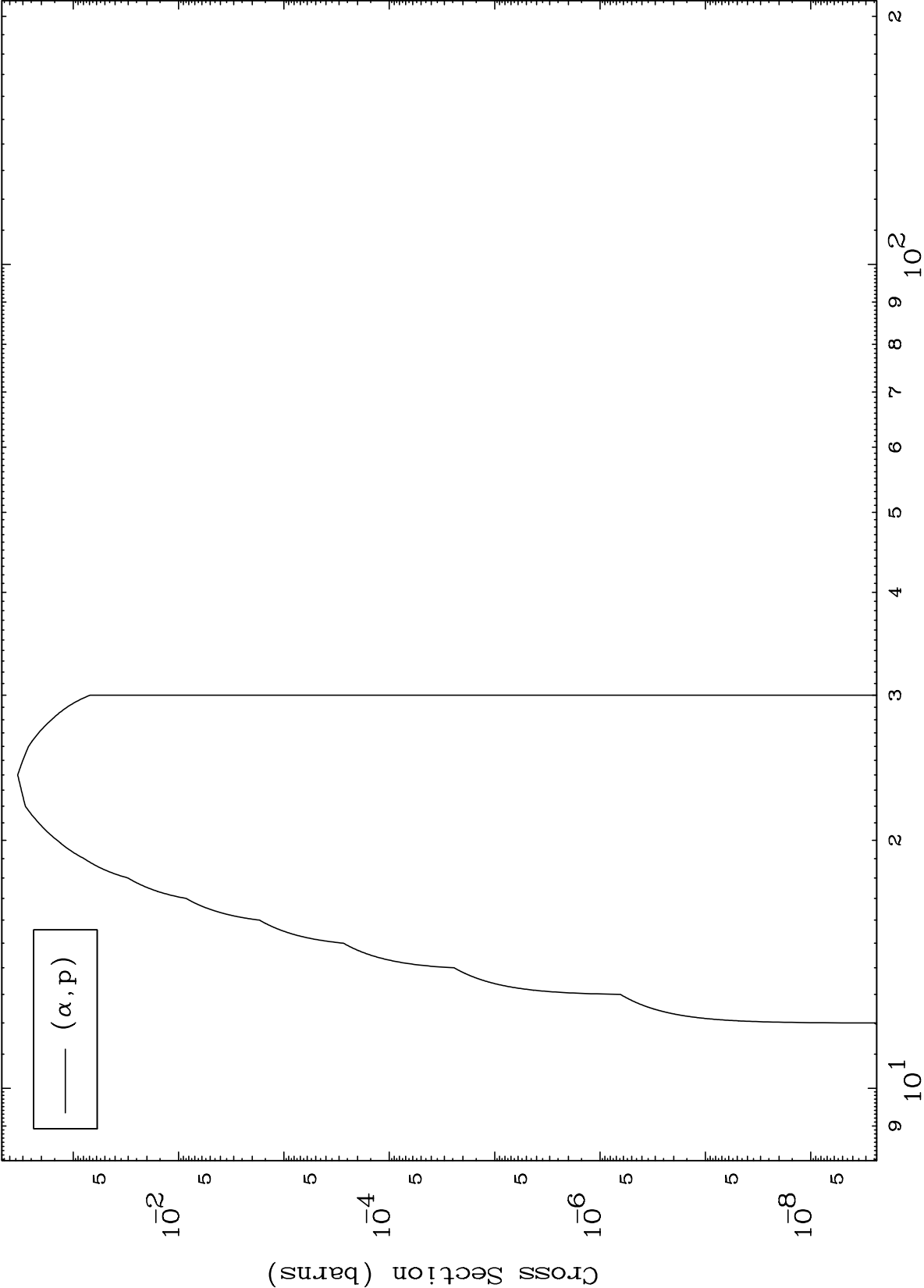
70-Yb-153

MAT 6980

(α ,p) Levels

$^{70}\text{Yb-153}$

0 Kelvin Cross Sections



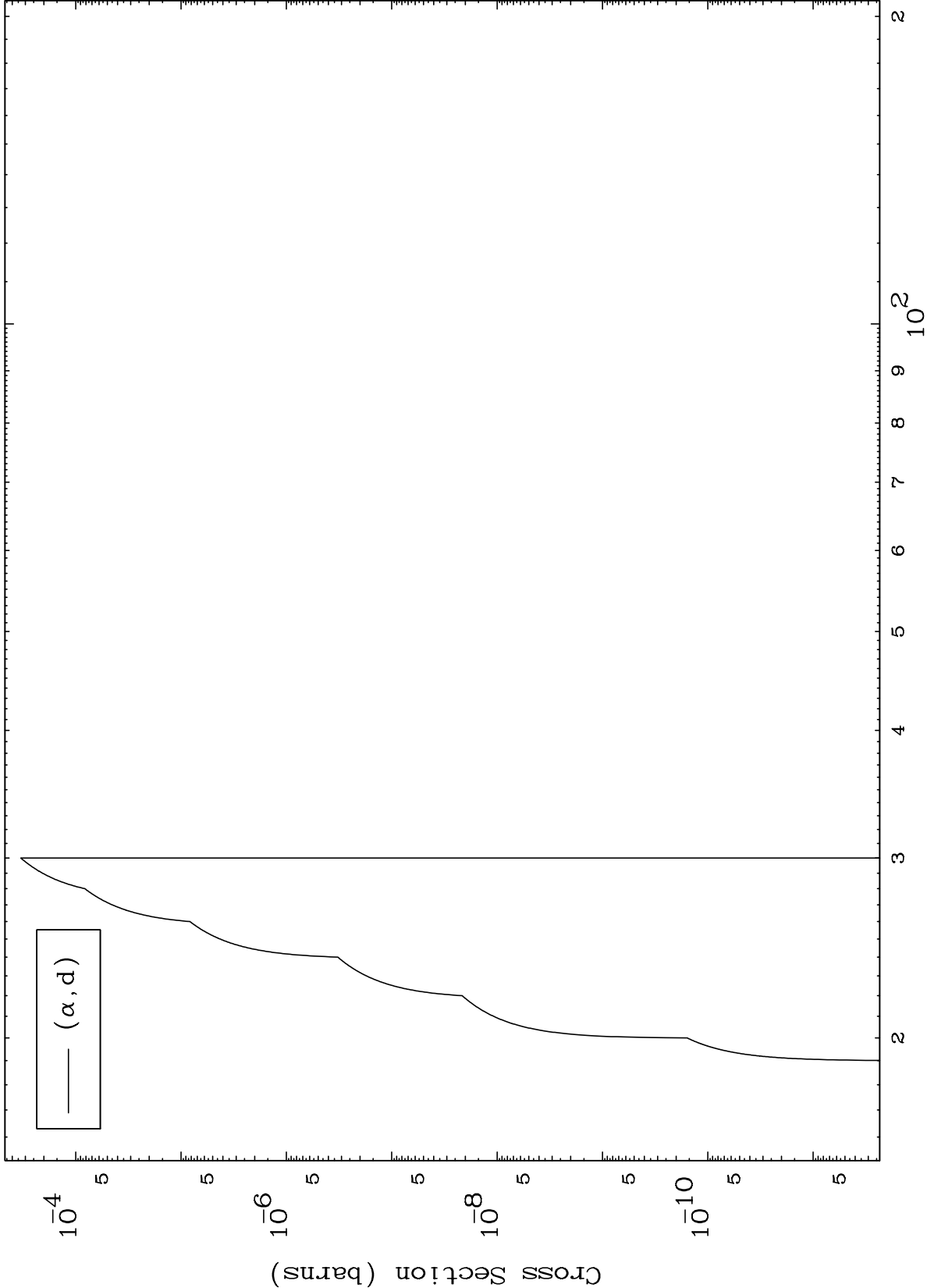
Incident Energy (MeV)

$^{70}\text{Yb-153}$

MAT 6980

(α ,d) Levels
0 Kelvin Cross Sections

70-Yb-153



7

Incident Energy (MeV)

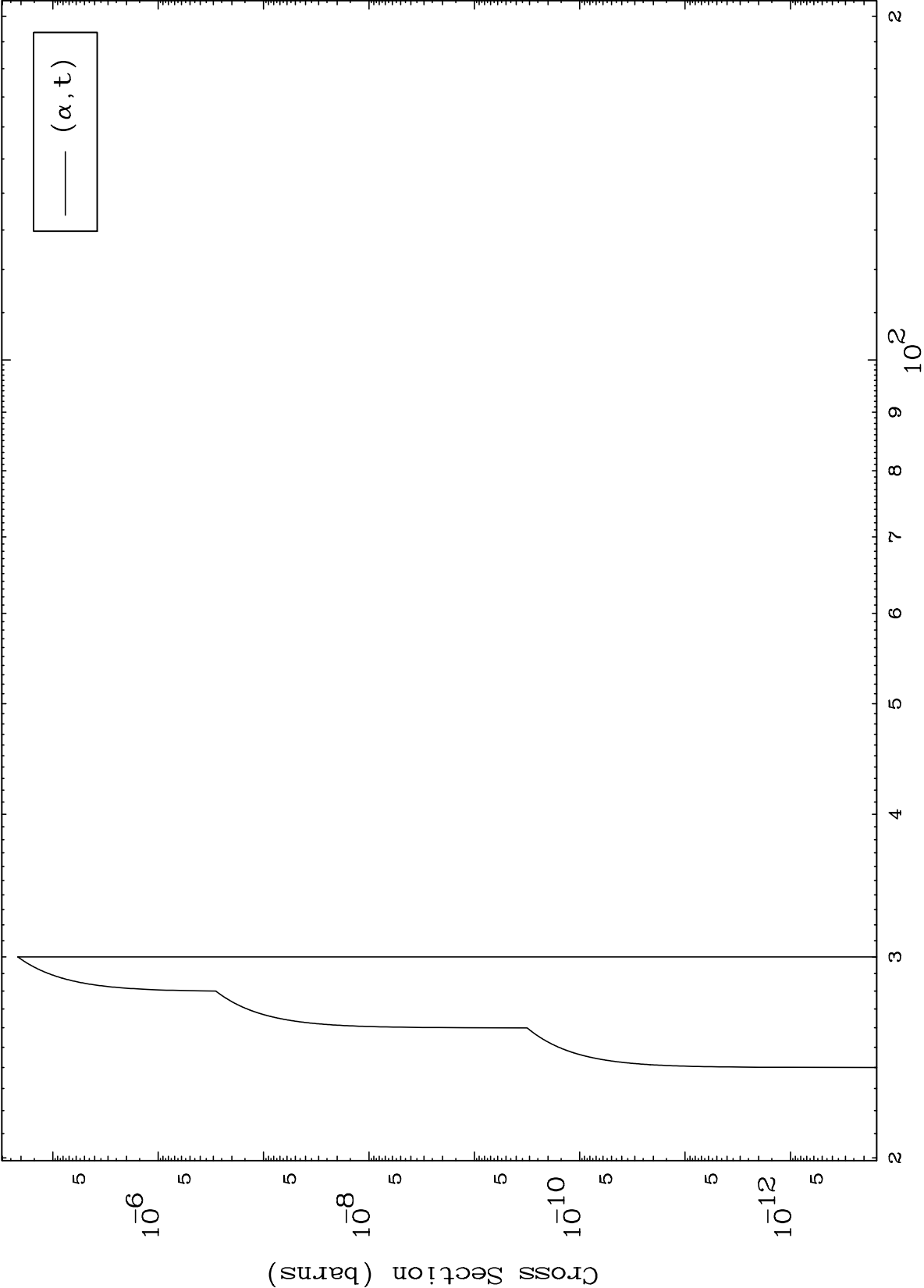
70-Yb-153

MAT 6980

(α , t) Levels

70-Yb-153

0 Kelvin Cross Sections



(α , t)

Incident Energy (MeV)

70-Yb-153

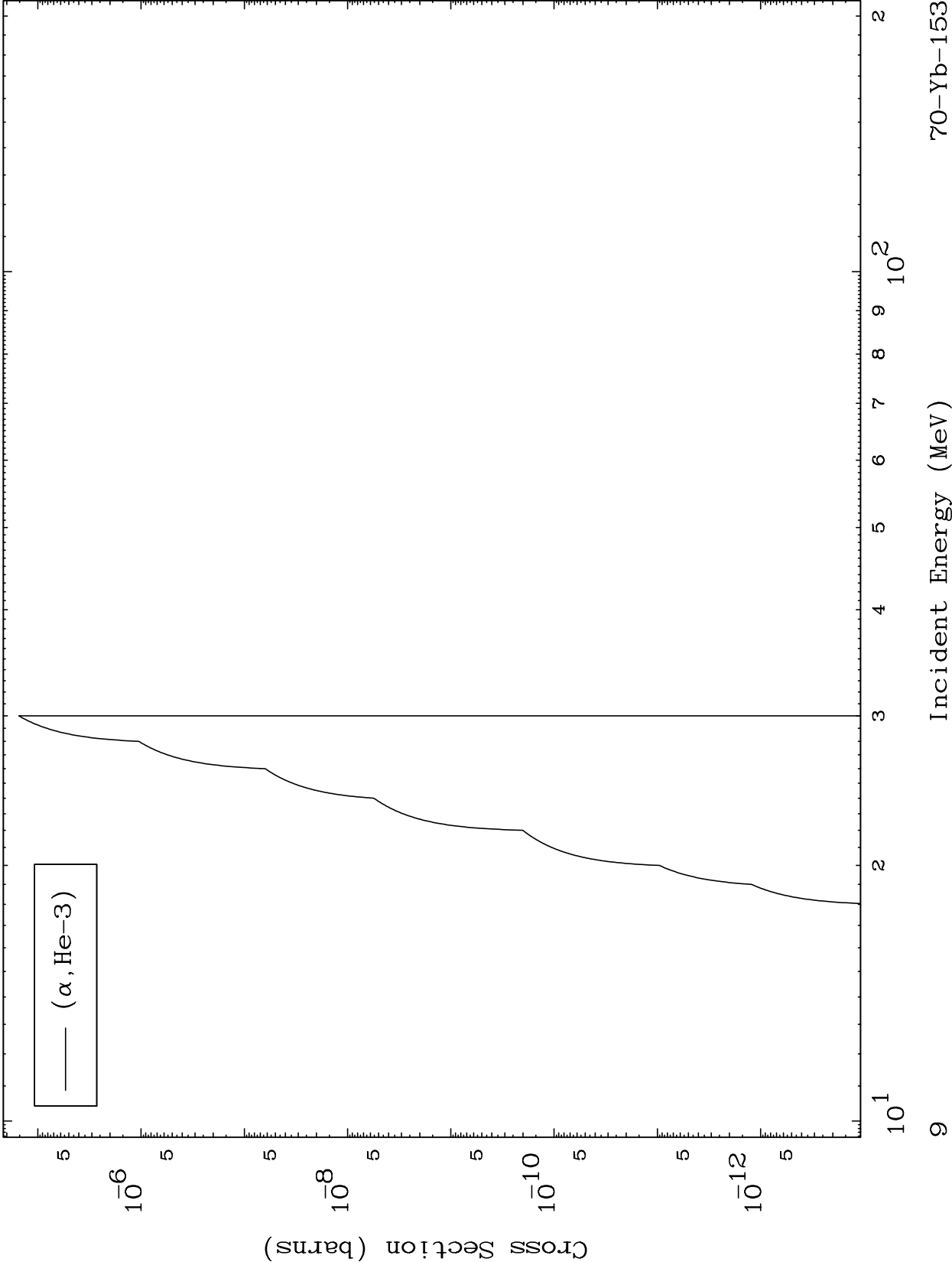
8

MAT 6980

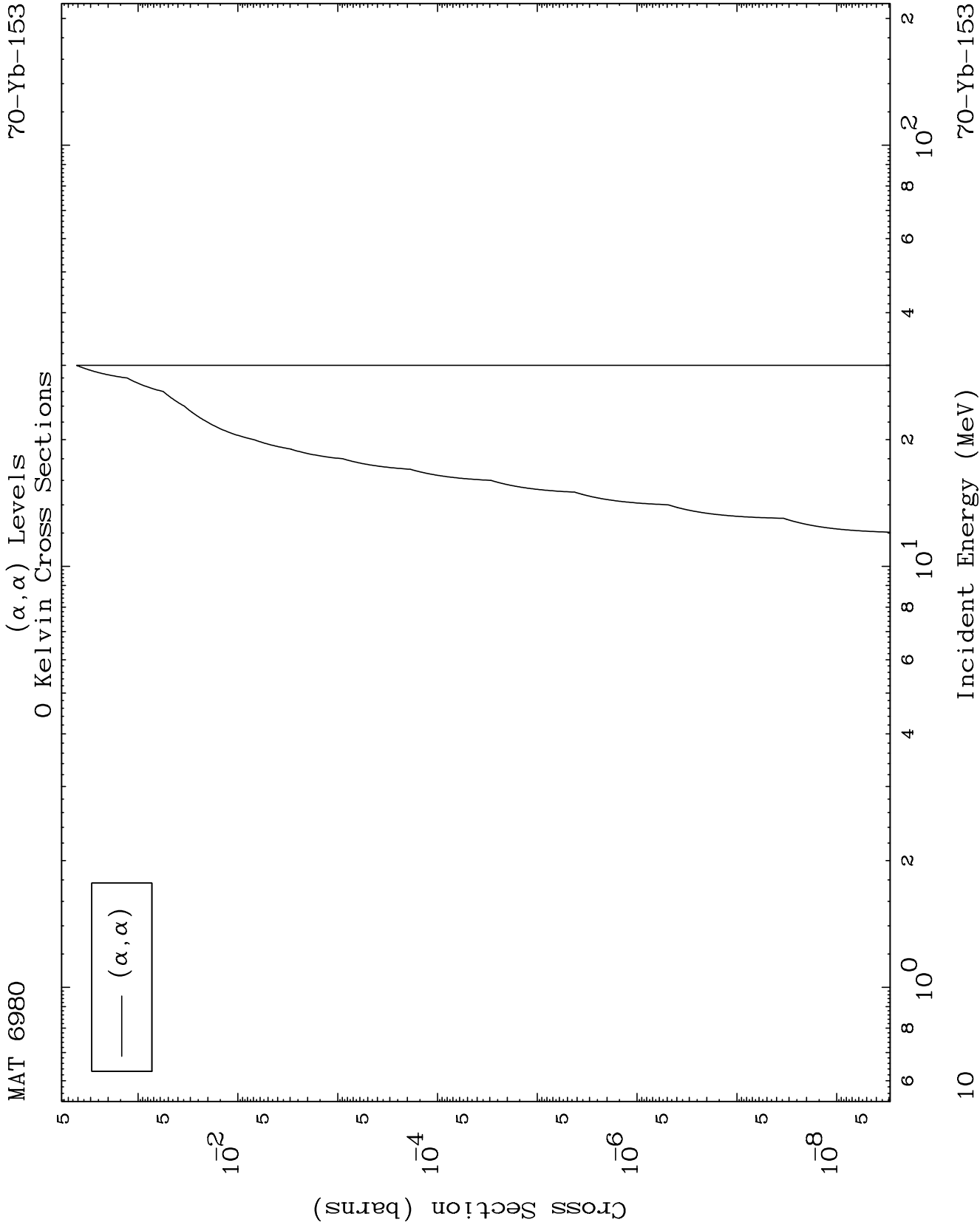
(α ,He3) Levels

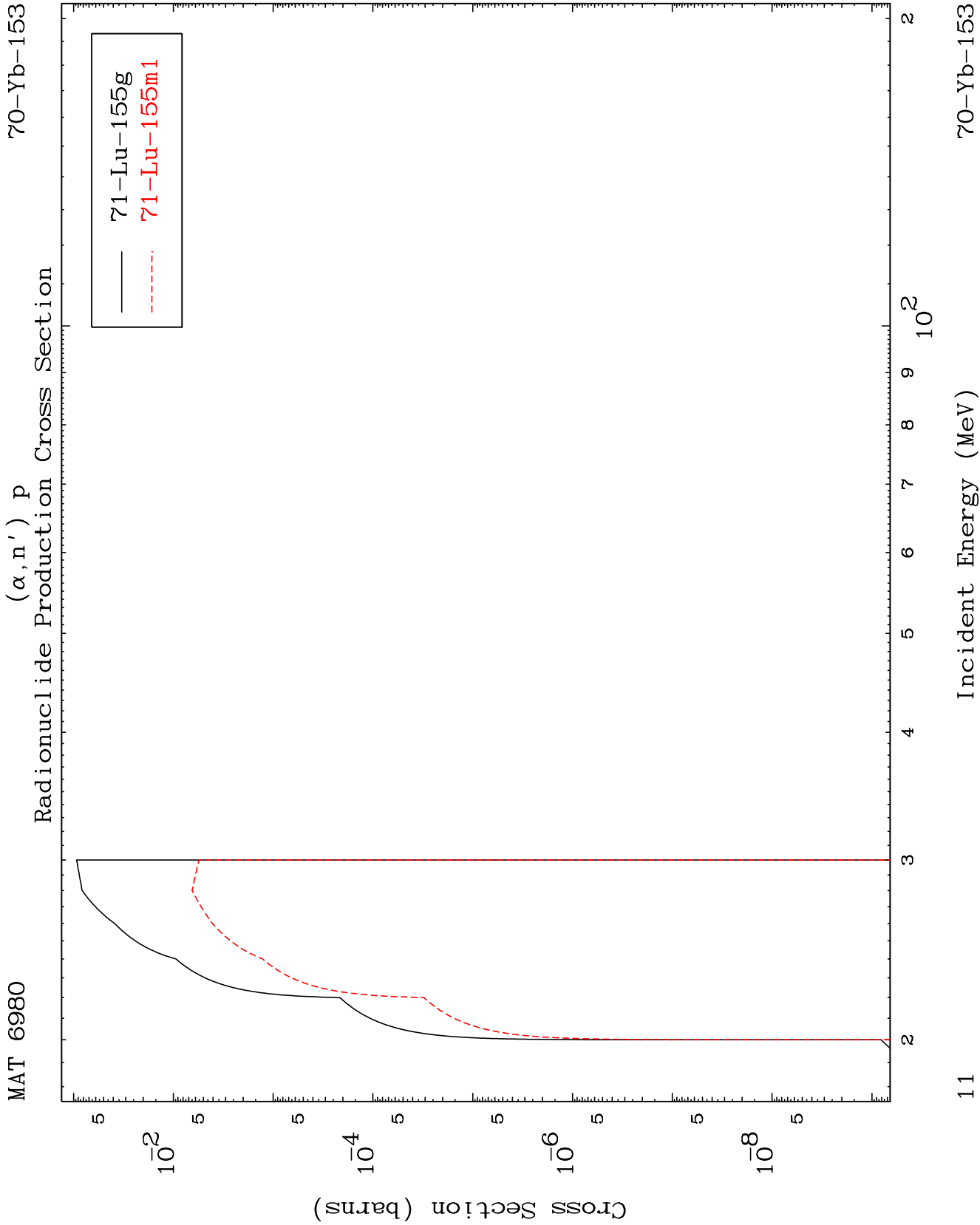
70-Yb-153

0 Kelvin Cross Sections

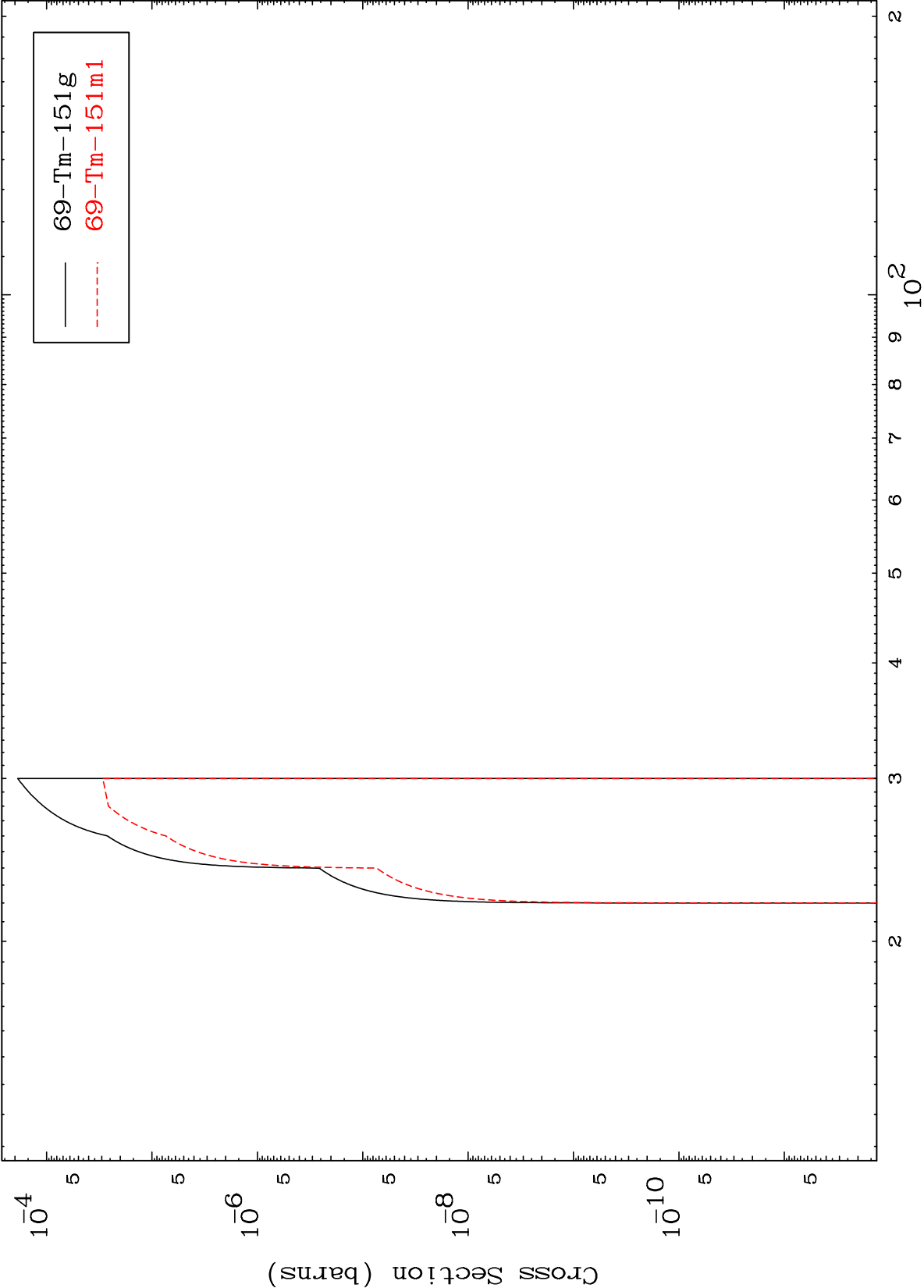


70-Yb-153





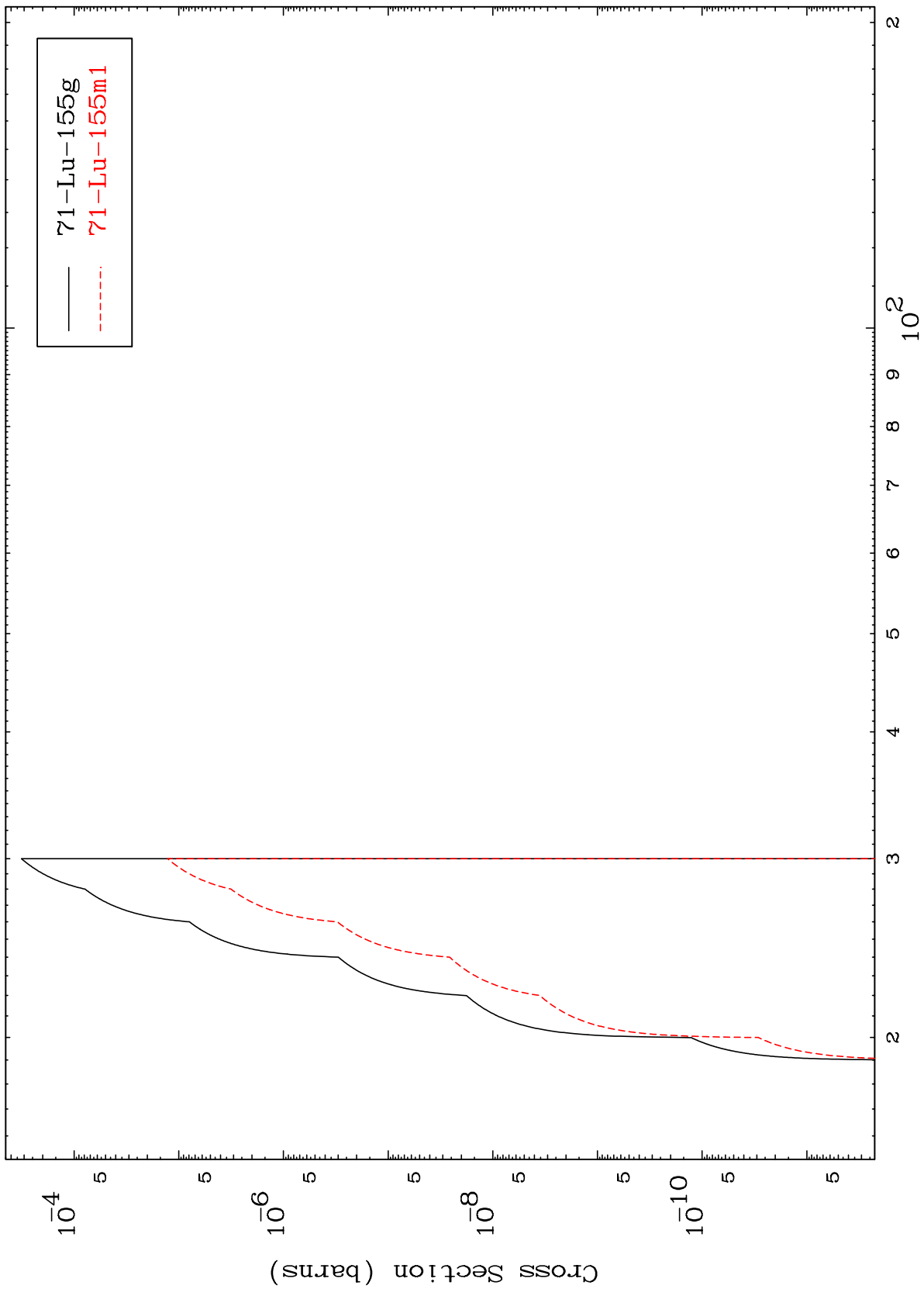
(α, n') p α
Radionuclide Production Cross Section



MAT 6980

70-Yb-153

(α, d)
Radionuclide Production Cross Section



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Incident Energy (MeV)

70-Yb-153

