

Program EVALPLOT  
(Version 2018-1)

by

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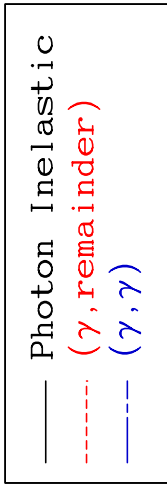
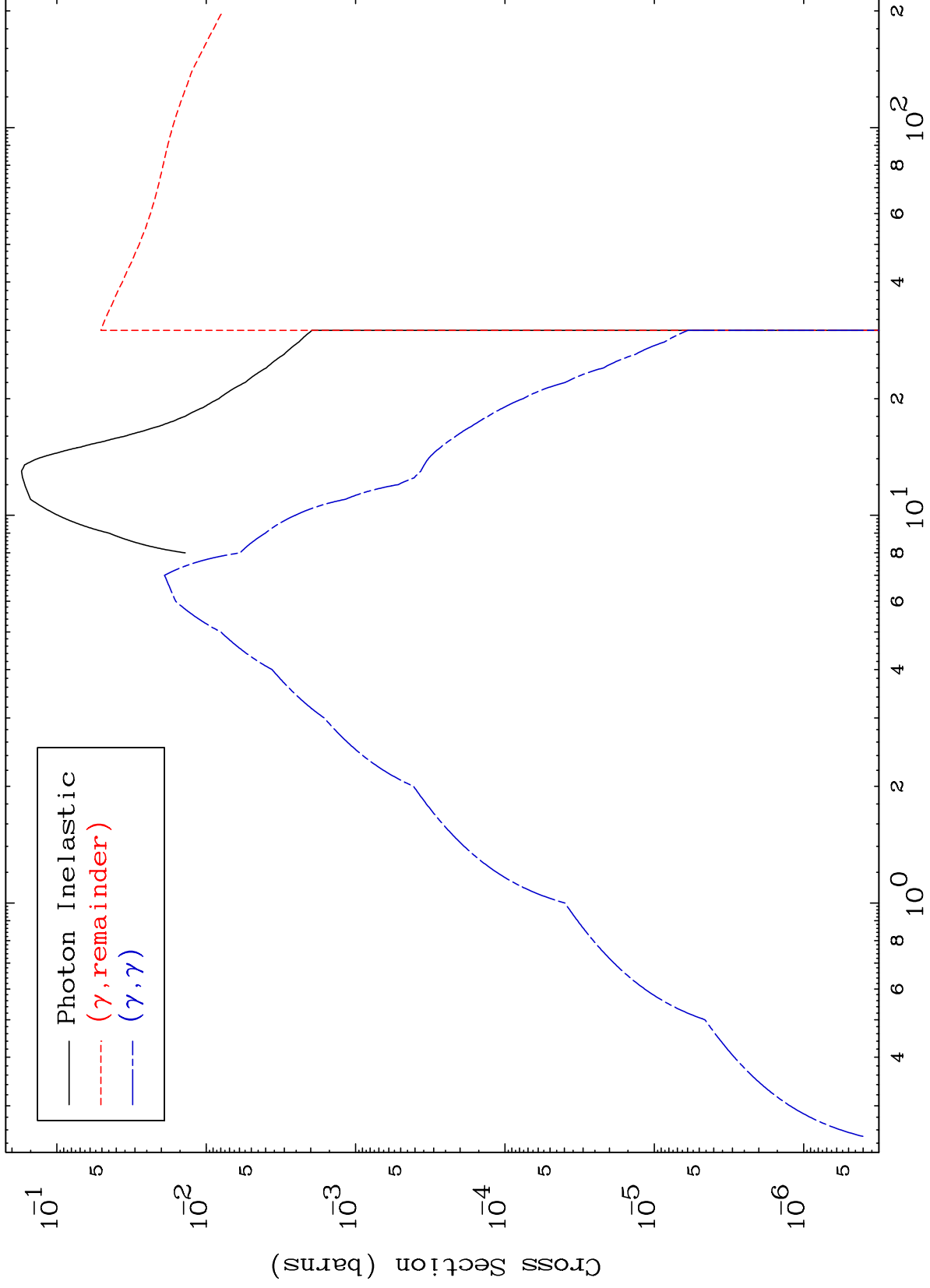
Press Mouse Button to Start

MAT 563

Photon Major

105-Db-263

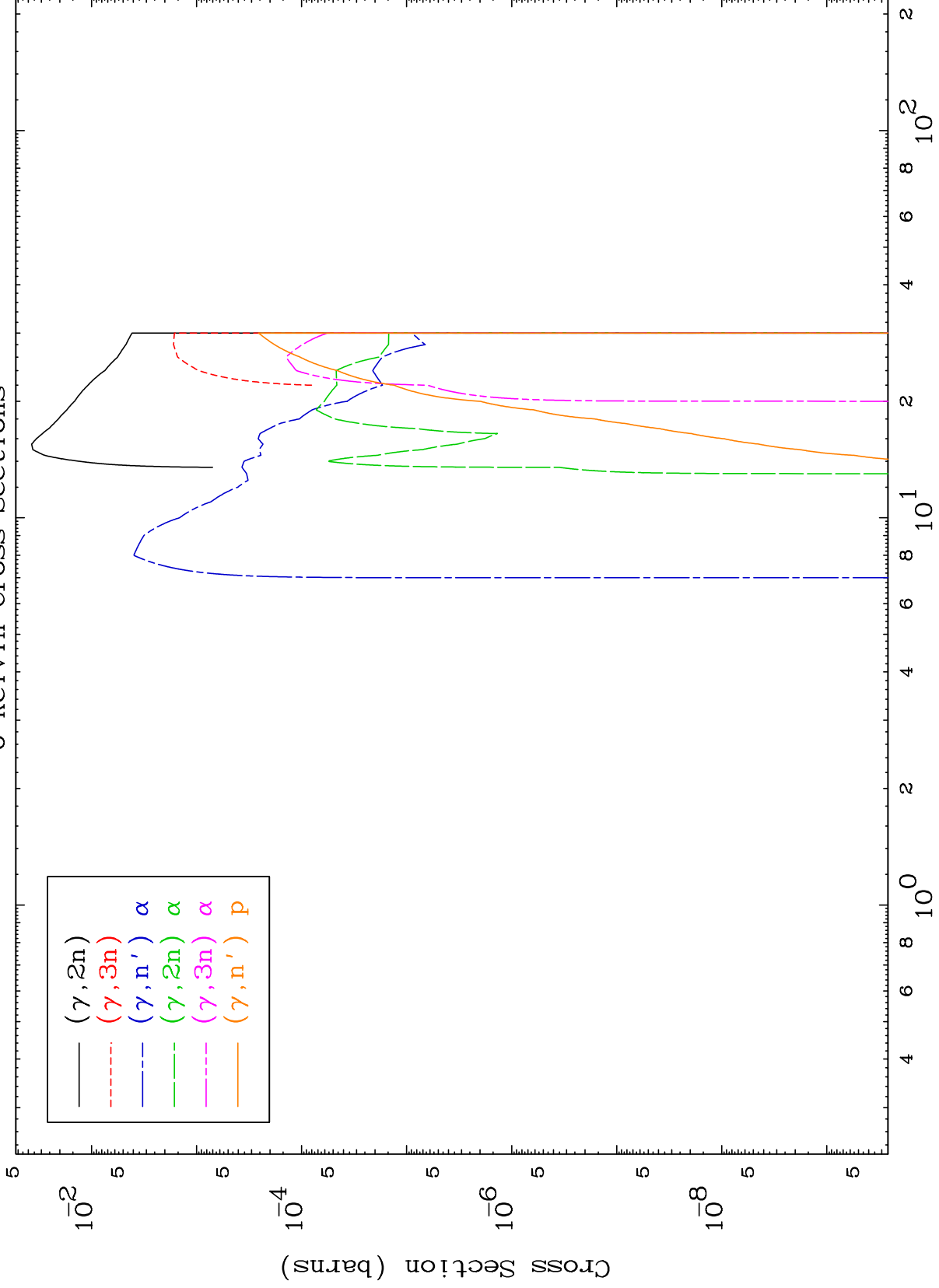
0 Kelvin Cross Sections



MAT 563

Photon Neutron Production  
0 Kelvin Cross Sections

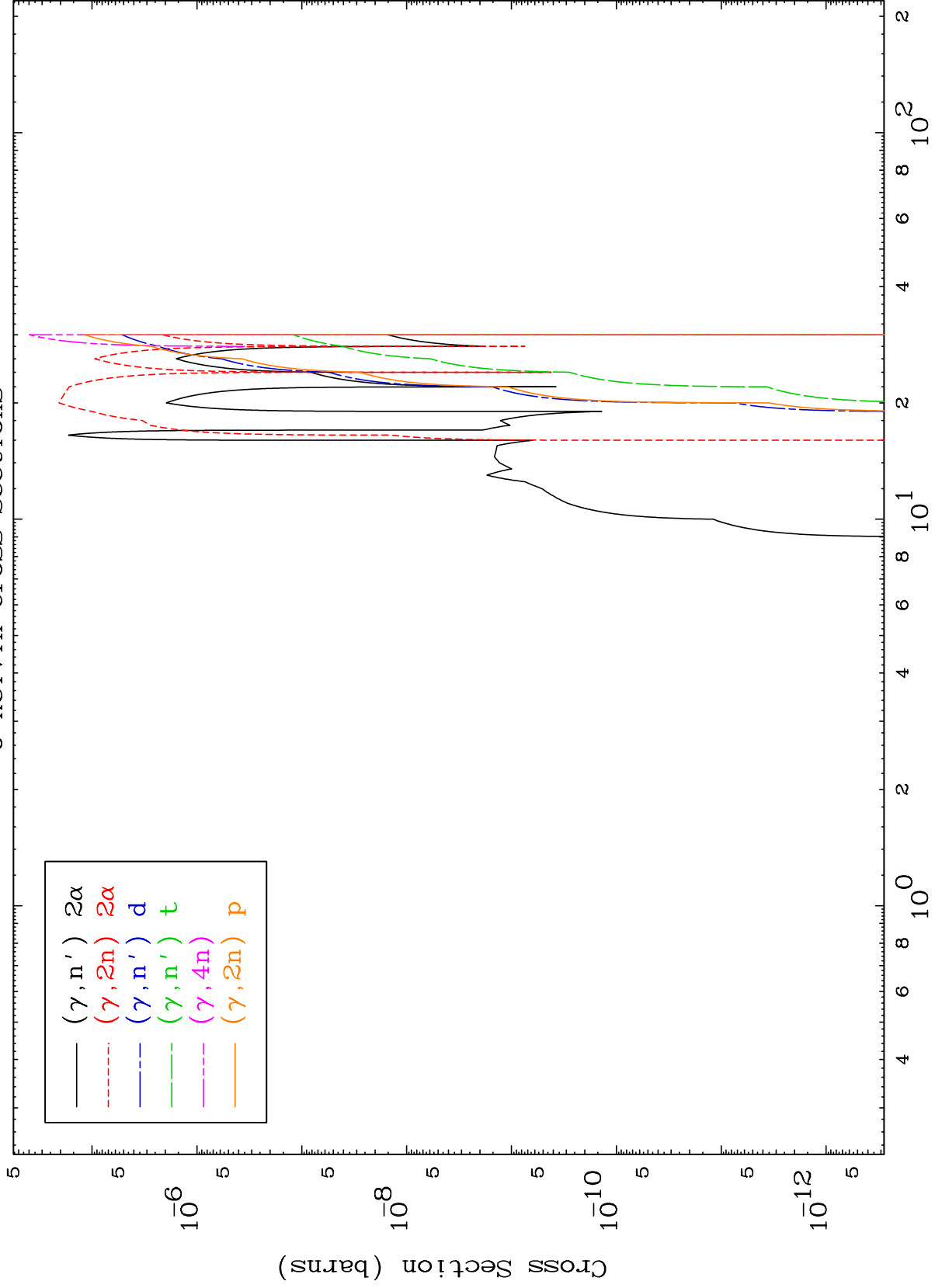
105-Db-263



MAT 563

Photon Neutron Production  
0 Kelvin Cross Sections

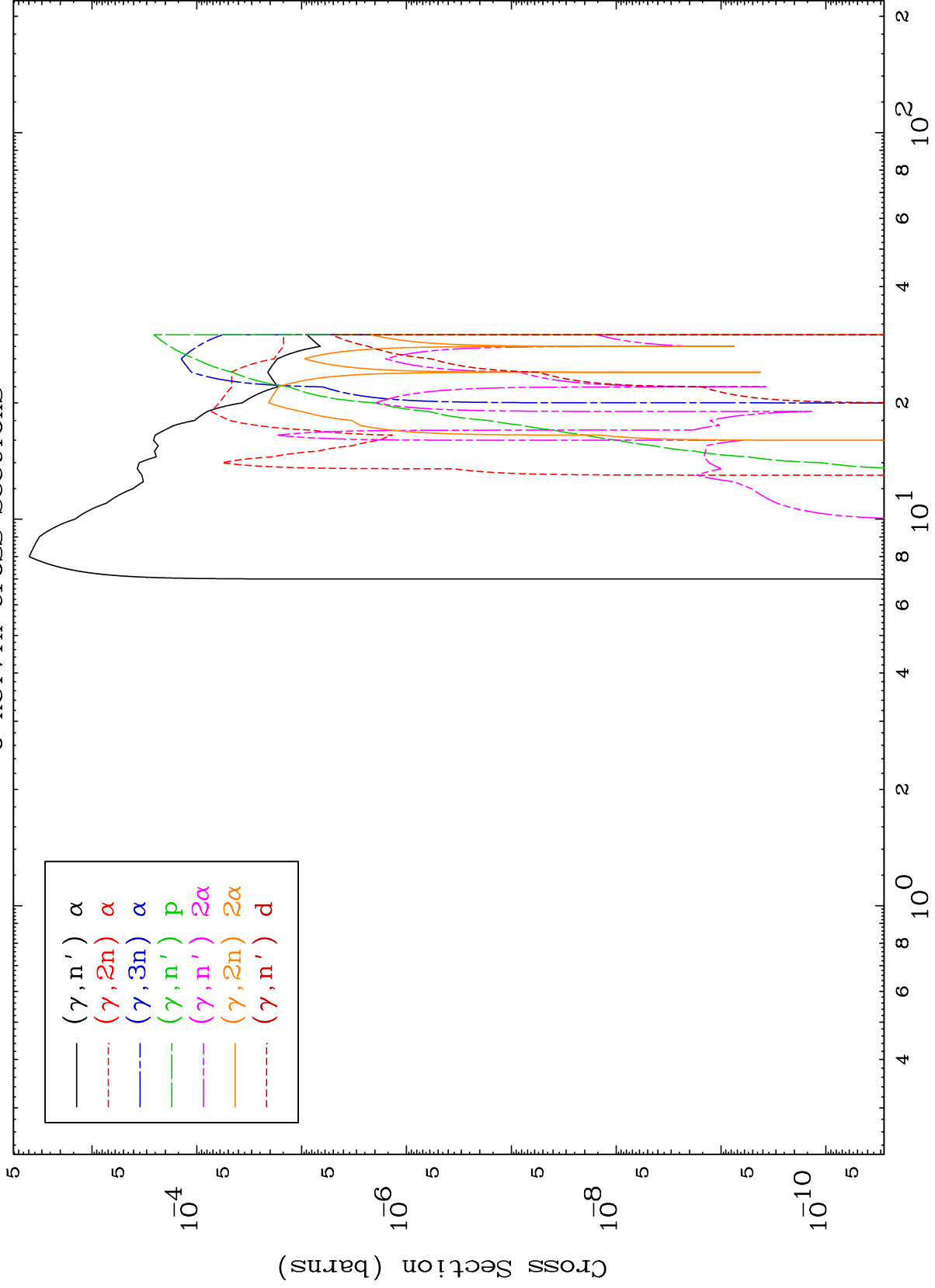
105-Db-263



MAT 563

Photon Charged Particle  
0 Kelvin Cross Sections

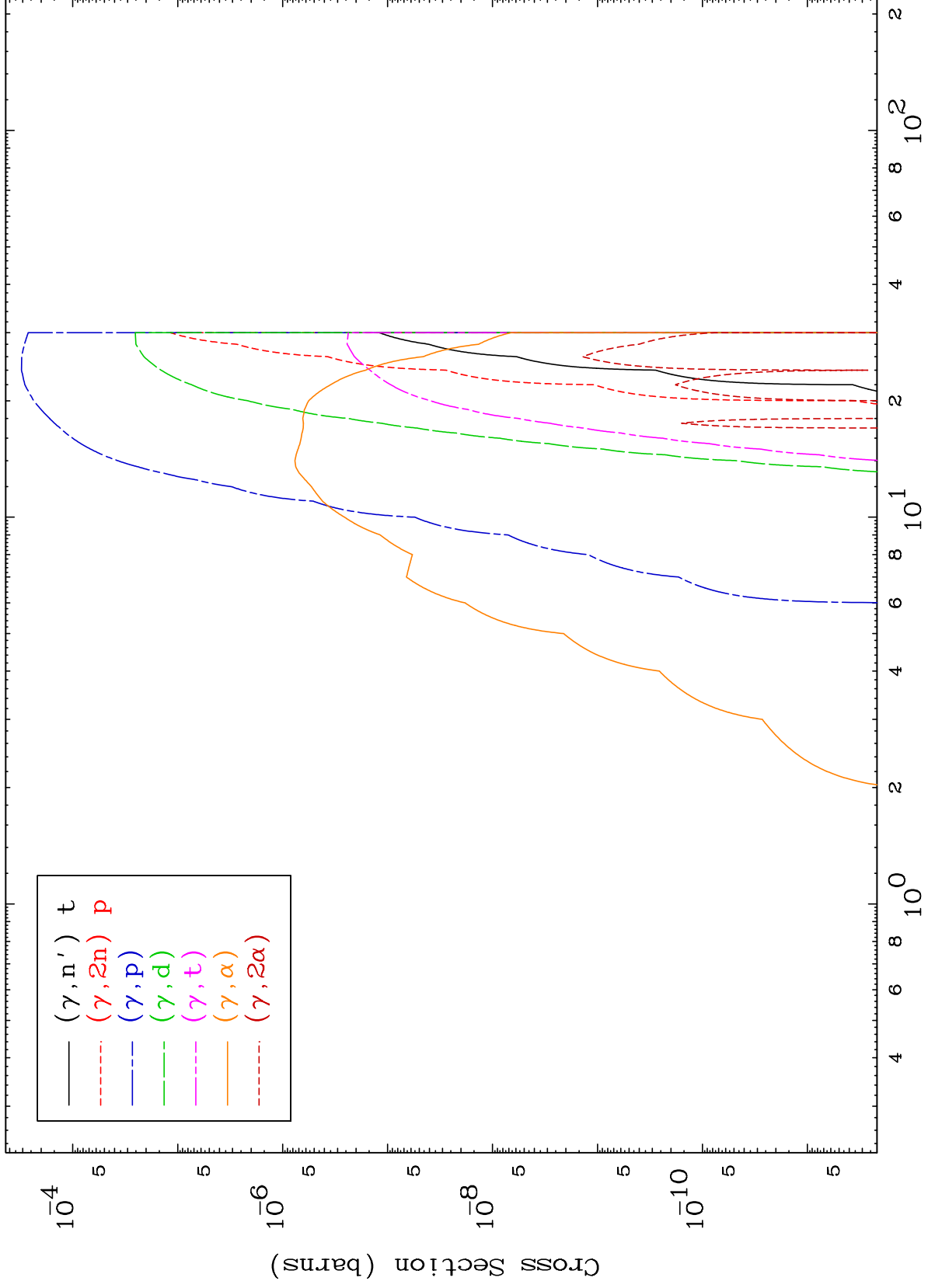
105-Db-263



MAT 563

Photon Charged Particle  
0 Kelvin Cross Sections

105-Db-263

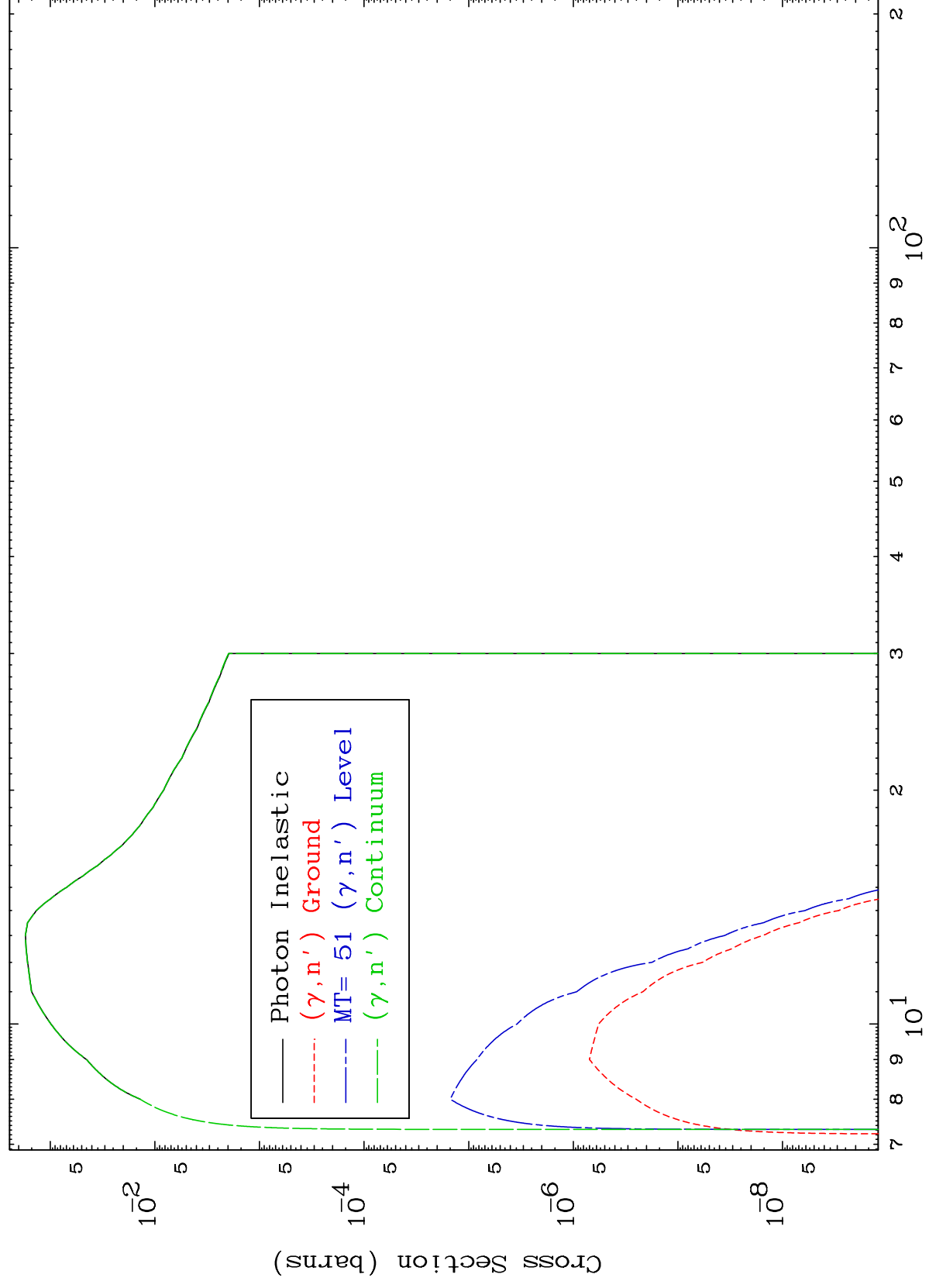


MAT 563

( $\gamma, n'$ ) Level

105-Db-263

0 Kelvin Cross Sections



Incident Energy (MeV)

105-Db-263

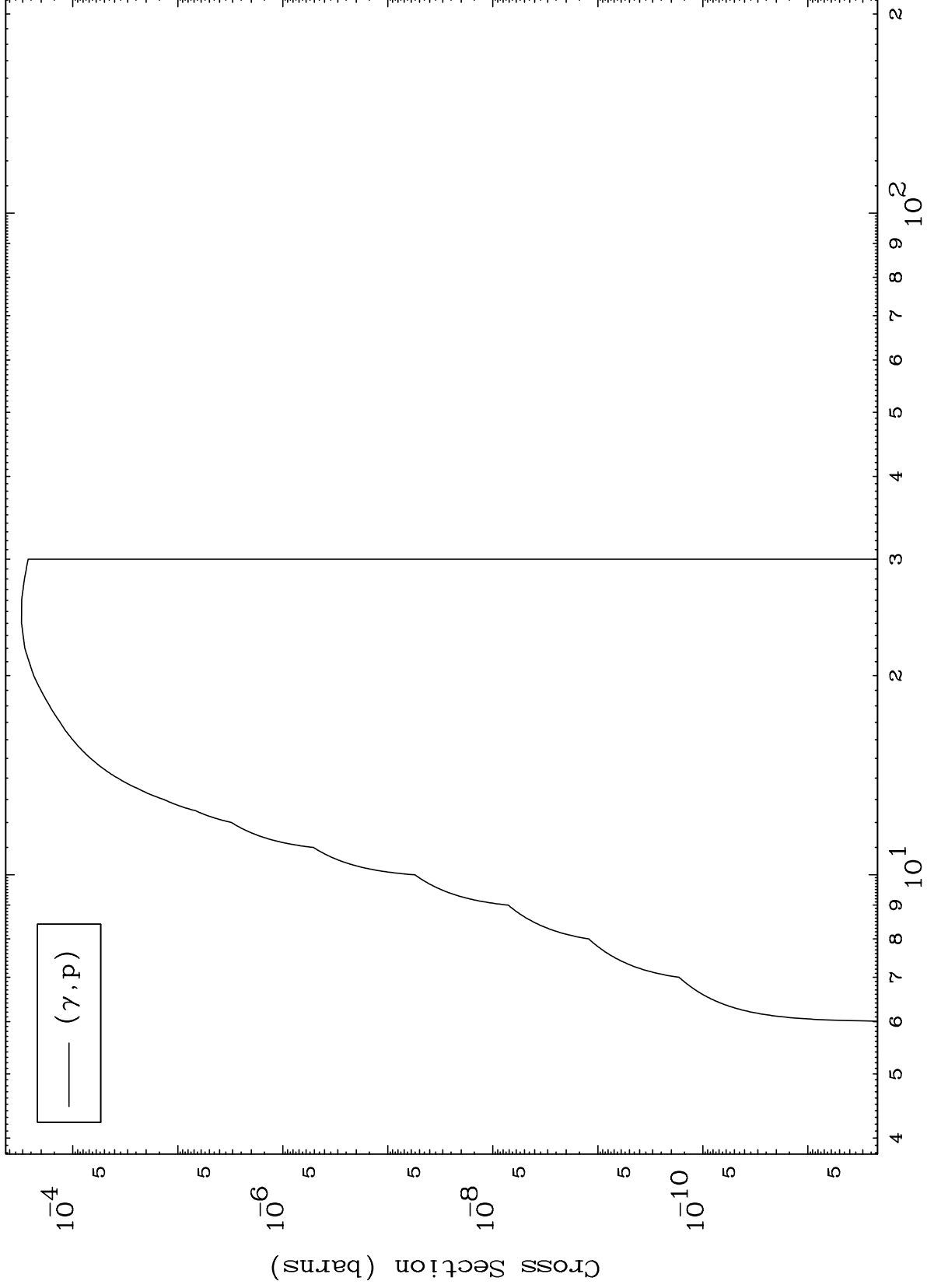
6

MAT 563

( $\gamma, p$ ) Levels

105-Db-263

0 Kelvin Cross Sections



7

Incident Energy (MeV)

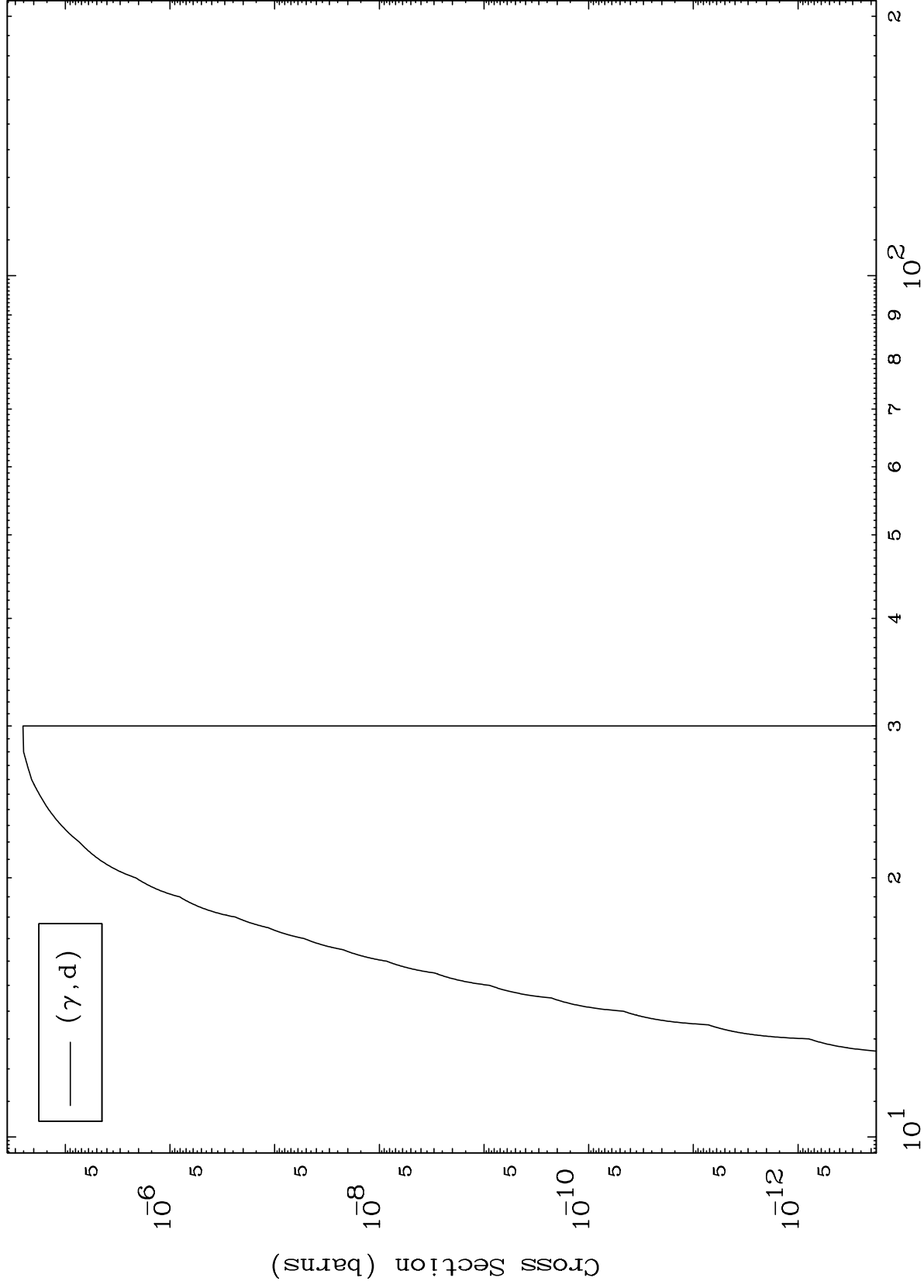
105-Db-263



MAT 563

( $\gamma, d$ ) Levels  
0 Kelvin Cross Sections

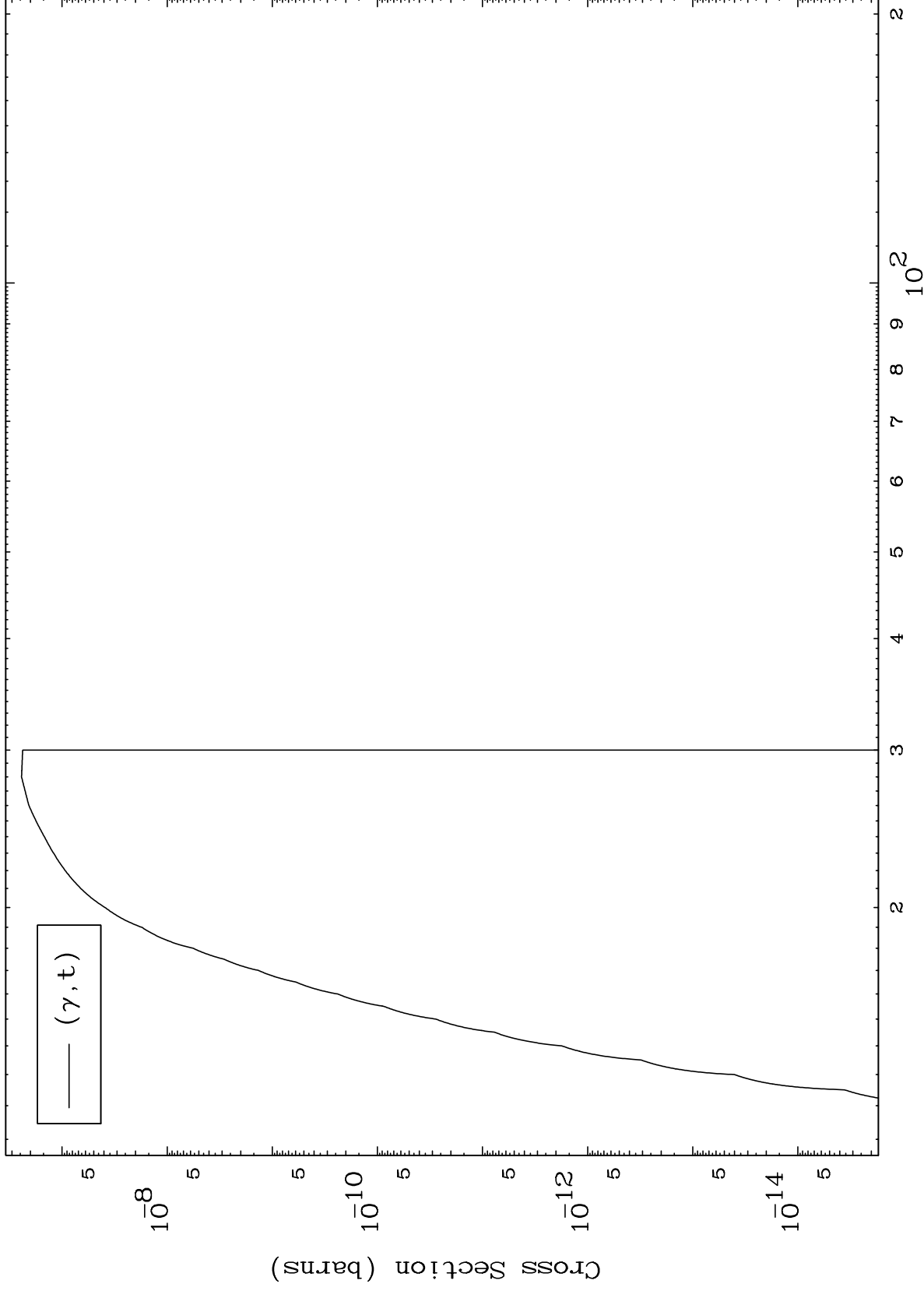
105-Db-263



Incident Energy (MeV)

105-Db-263

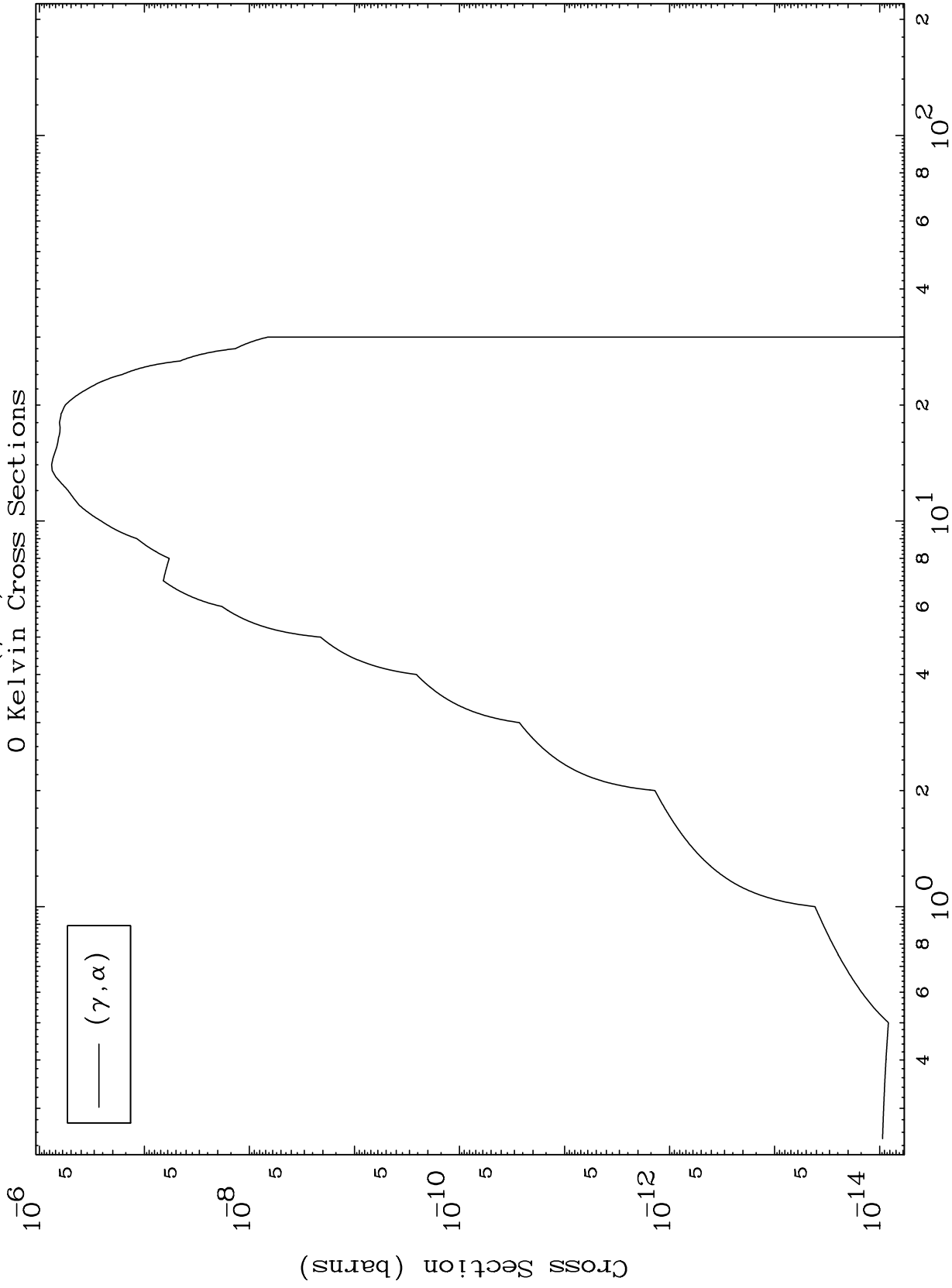
0 Kelvin Cross Sections



MAT 563

( $\gamma, \alpha$ ) Levels

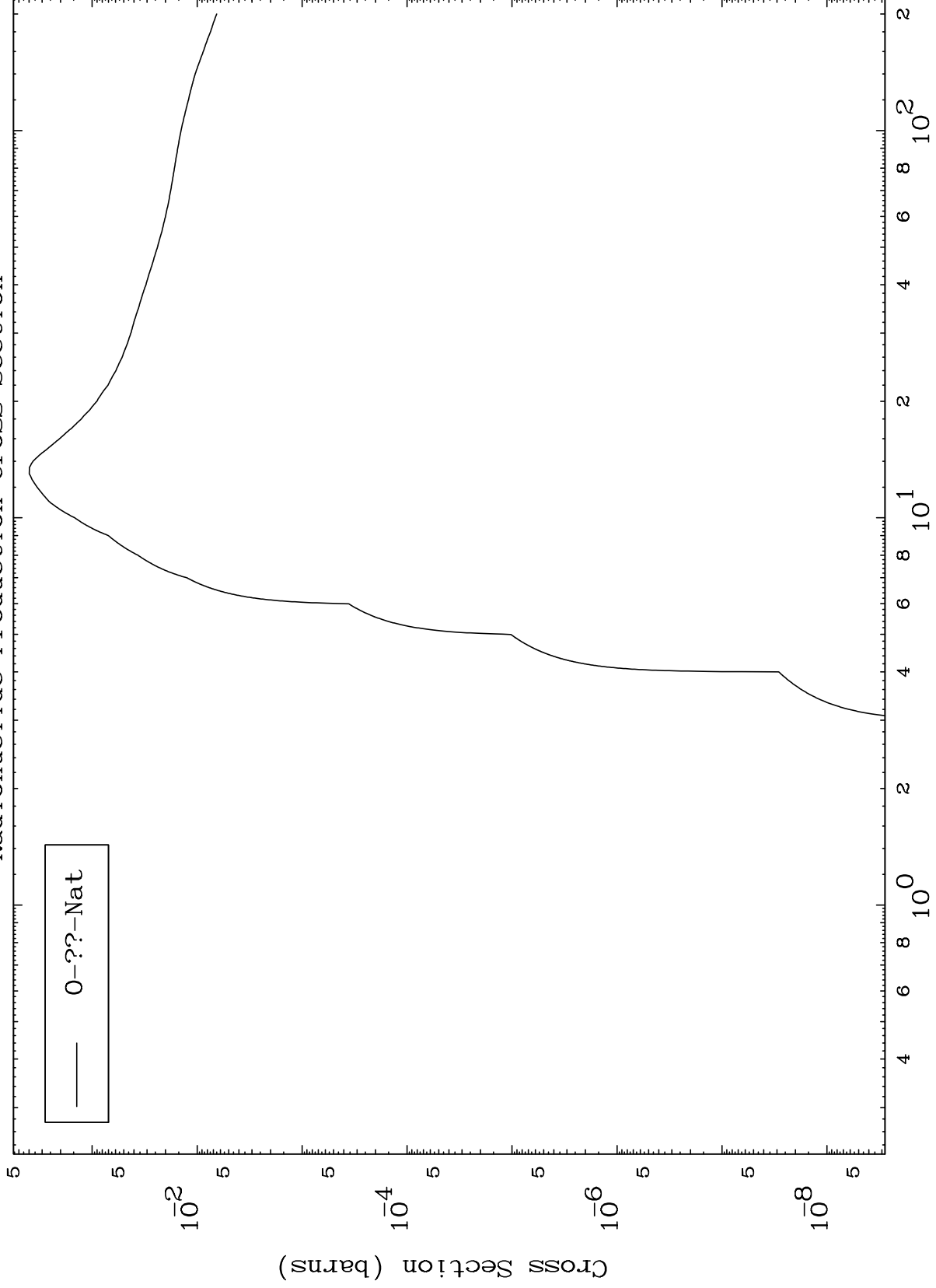
105-Db-263



MAT 563

Photon Fission  
Radionuclide Production Cross Section

105-Db-263

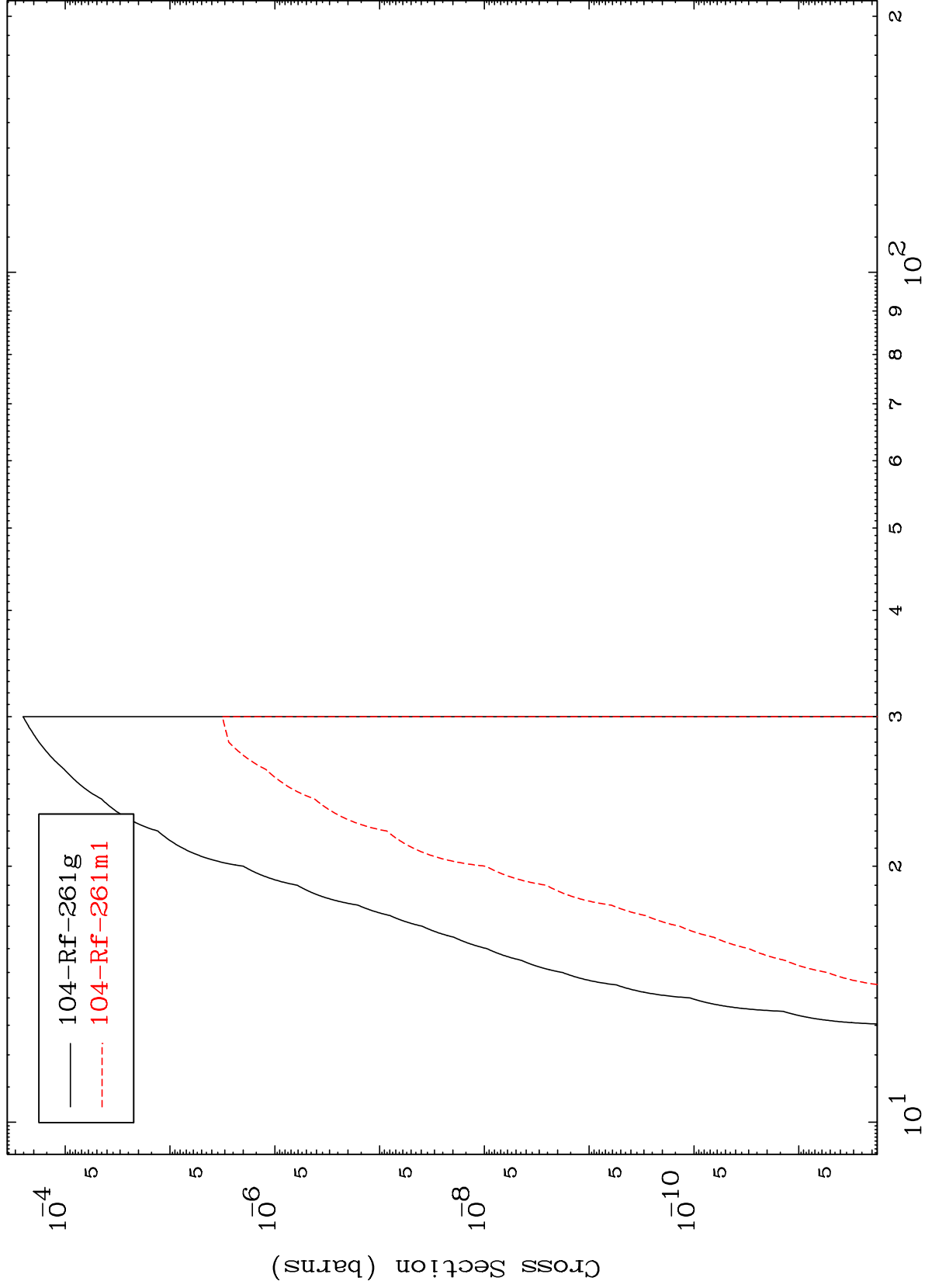


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$(\gamma, n')$  p

105-Db-263

Radionuclide Production Cross Section



Incident Energy (MeV)

105-Db-263

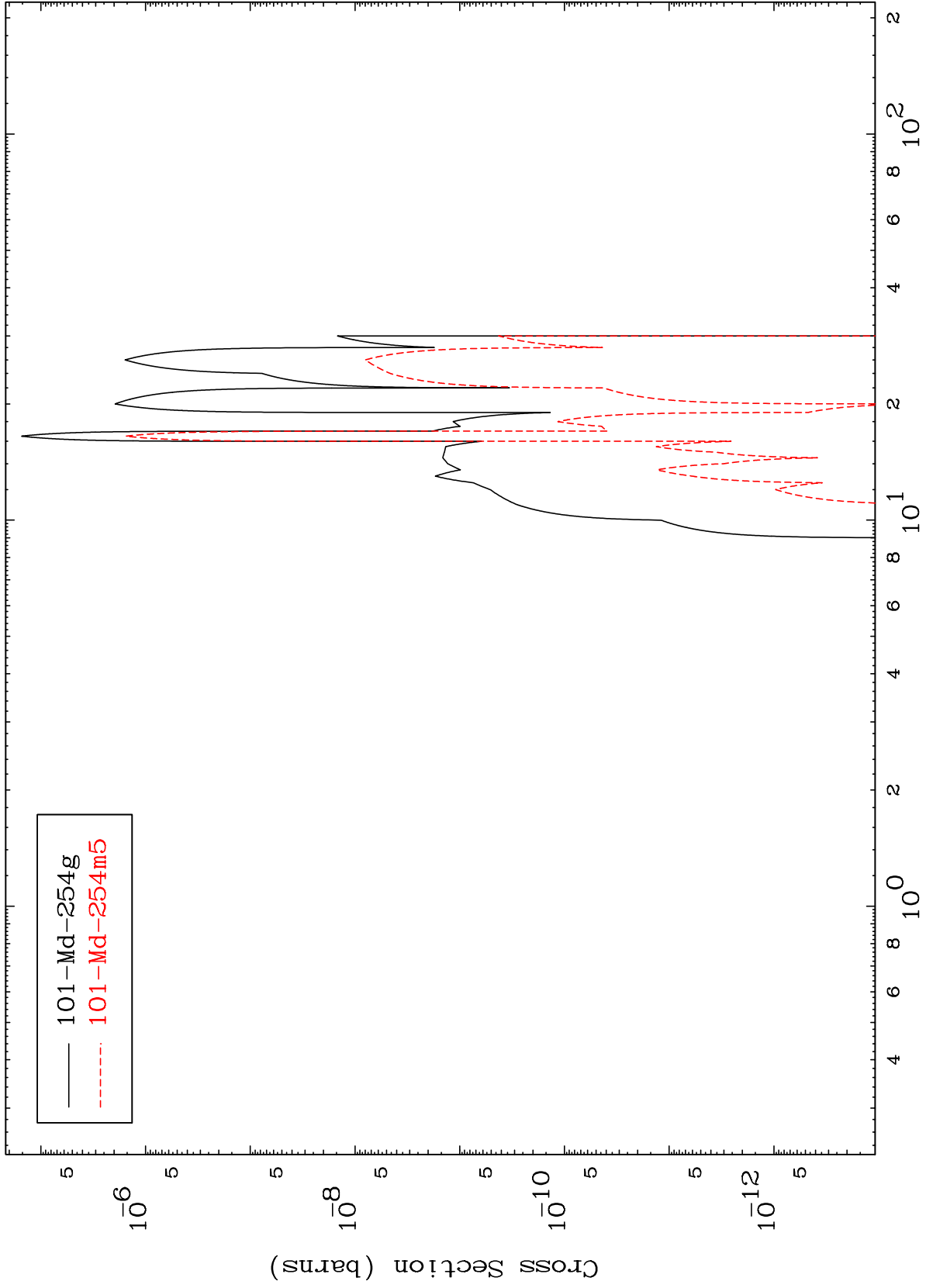
12

MAT 563

( $\gamma, n'$ )  $2\alpha$

105-Db-263

Radionuclide Production Cross Section



13

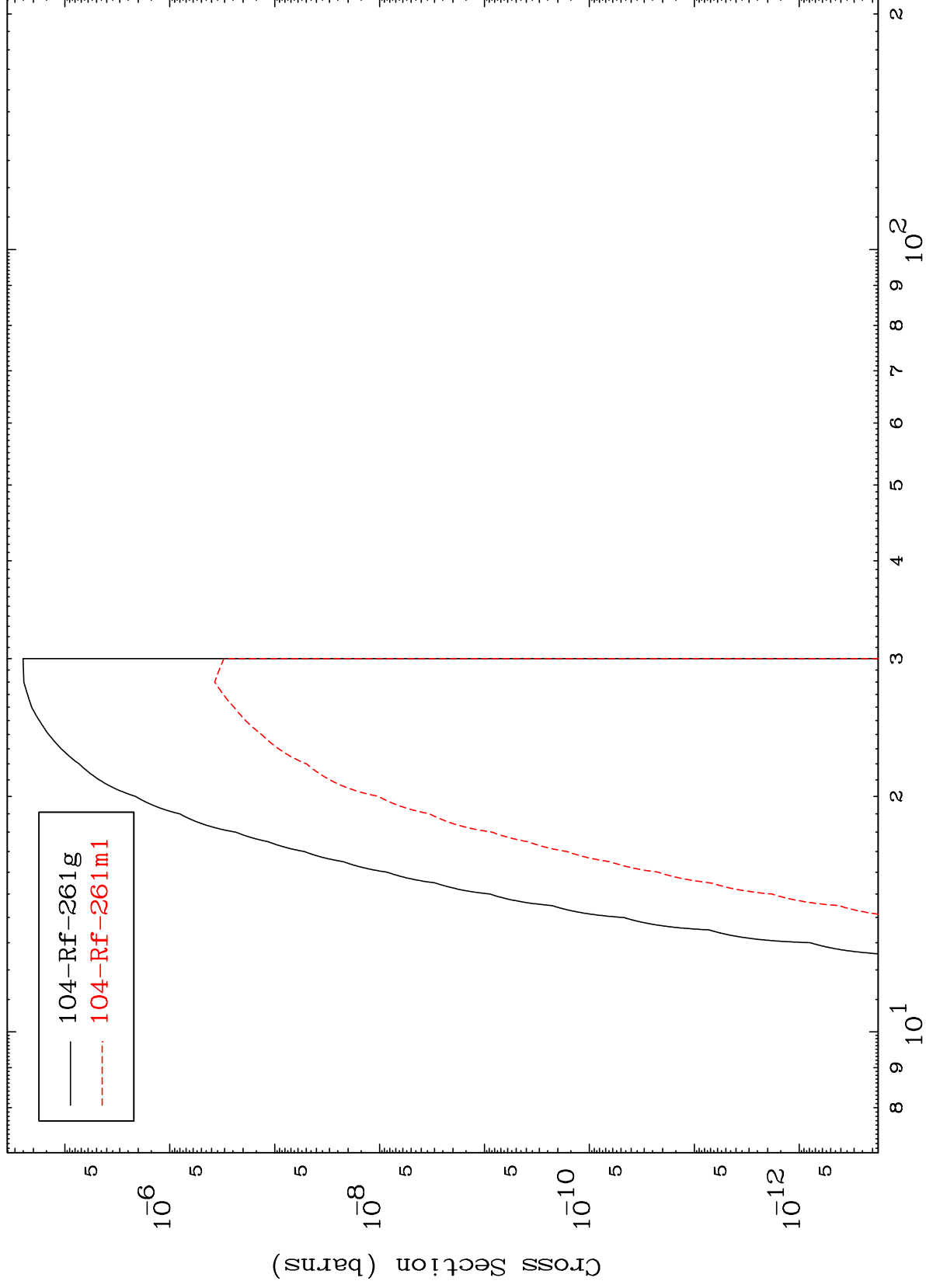
Incident Energy (MeV)

105-Db-263

MAT 563

105-Db-263

( $\gamma, d$ )  
Radionuclide Production Cross Section



14

Incident Energy (MeV)

105-Db-263