

Program EVALPLOT  
(Version 2018-1)

by

Dermott E. Cullen  
(Present Contact Information)

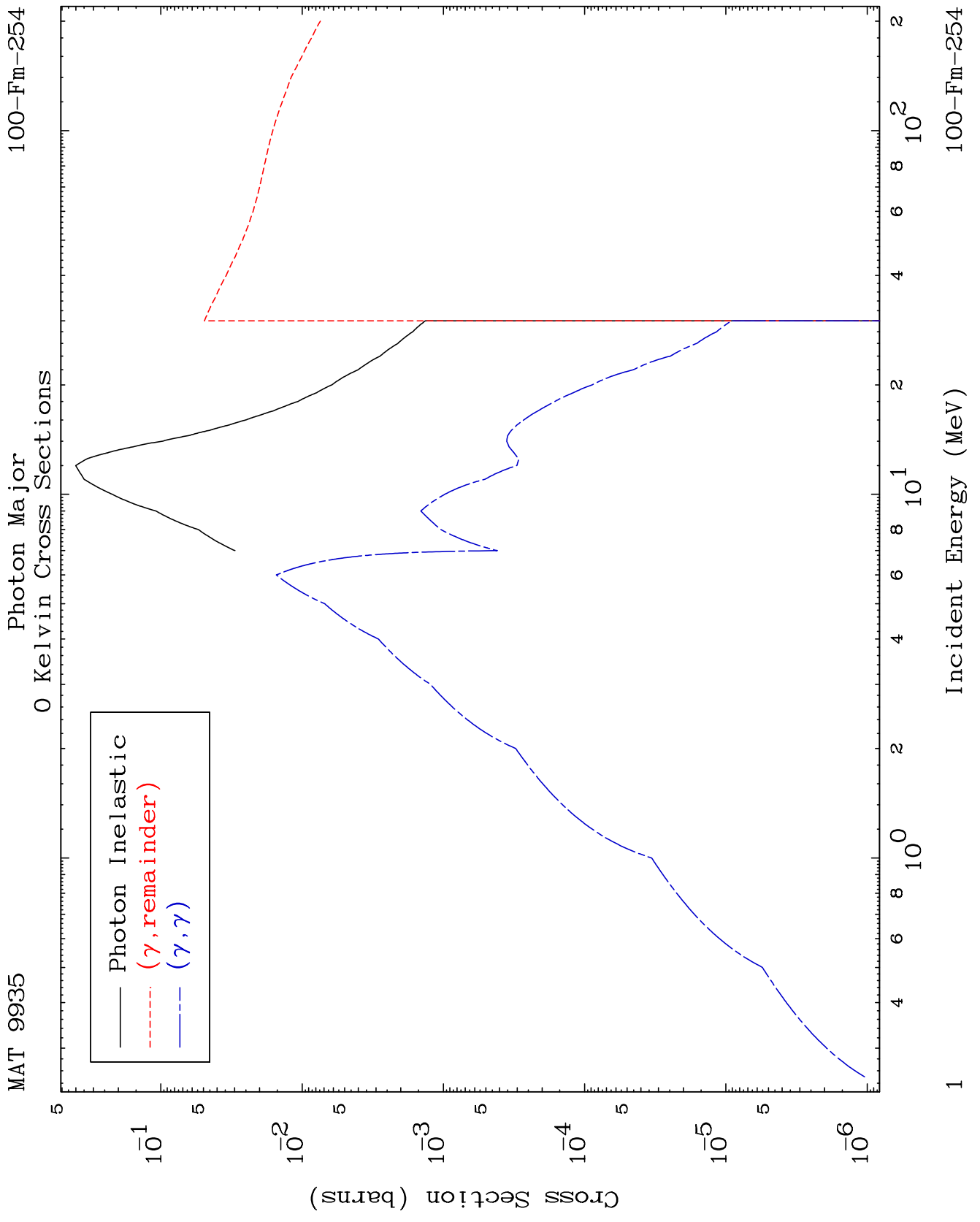
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Web:redcullen1.net/HOMEPAGE.NEW

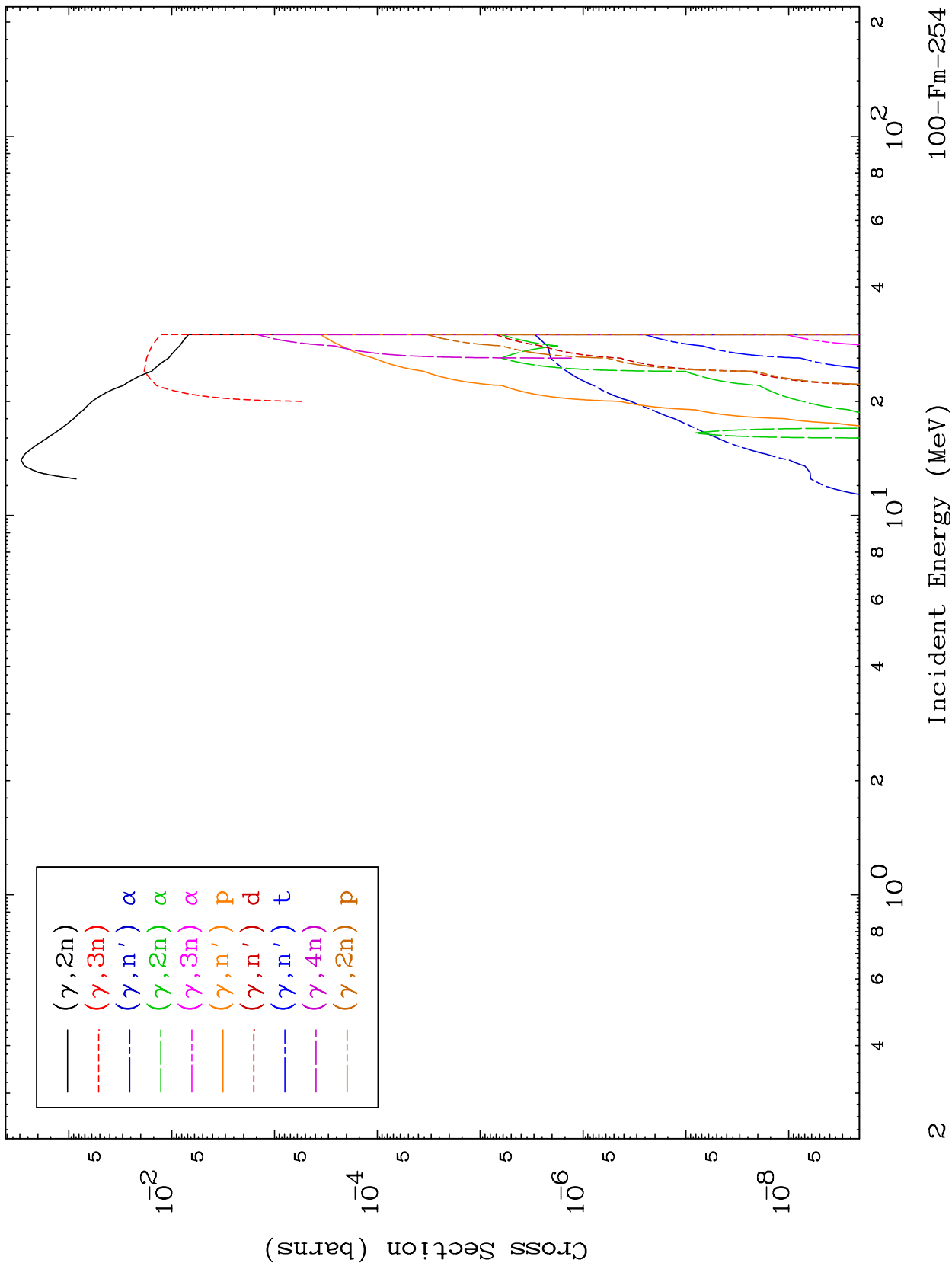
Press Mouse Button to Start



MAT 9935

Photon Neutron Production  
0 Kelvin Cross Sections

100-Fm-254



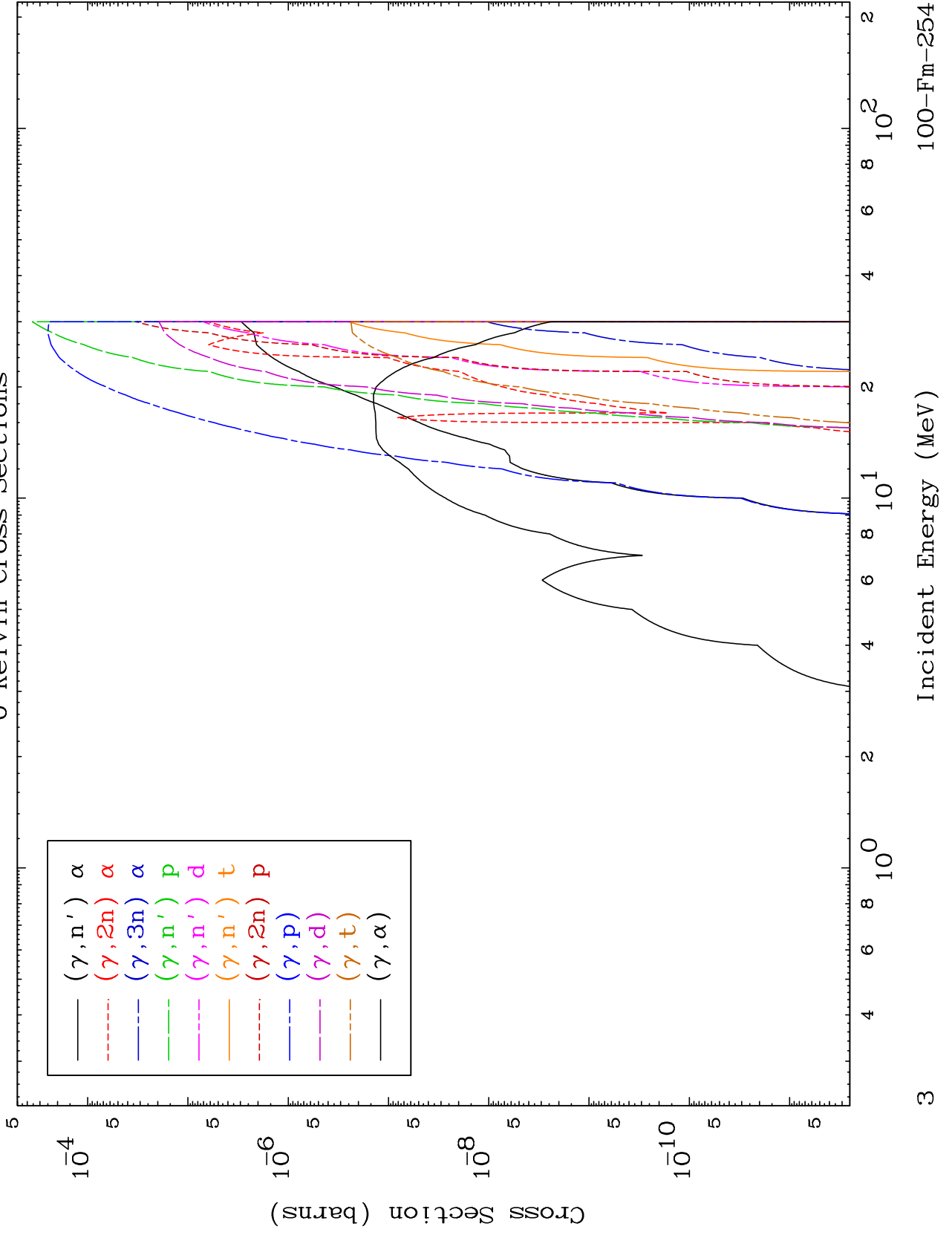
100-Fm-254

Incident Energy (MeV)

MAT 9935

Photon Charged Particle  
0 Kelvin Cross Sections

100-Fm-254

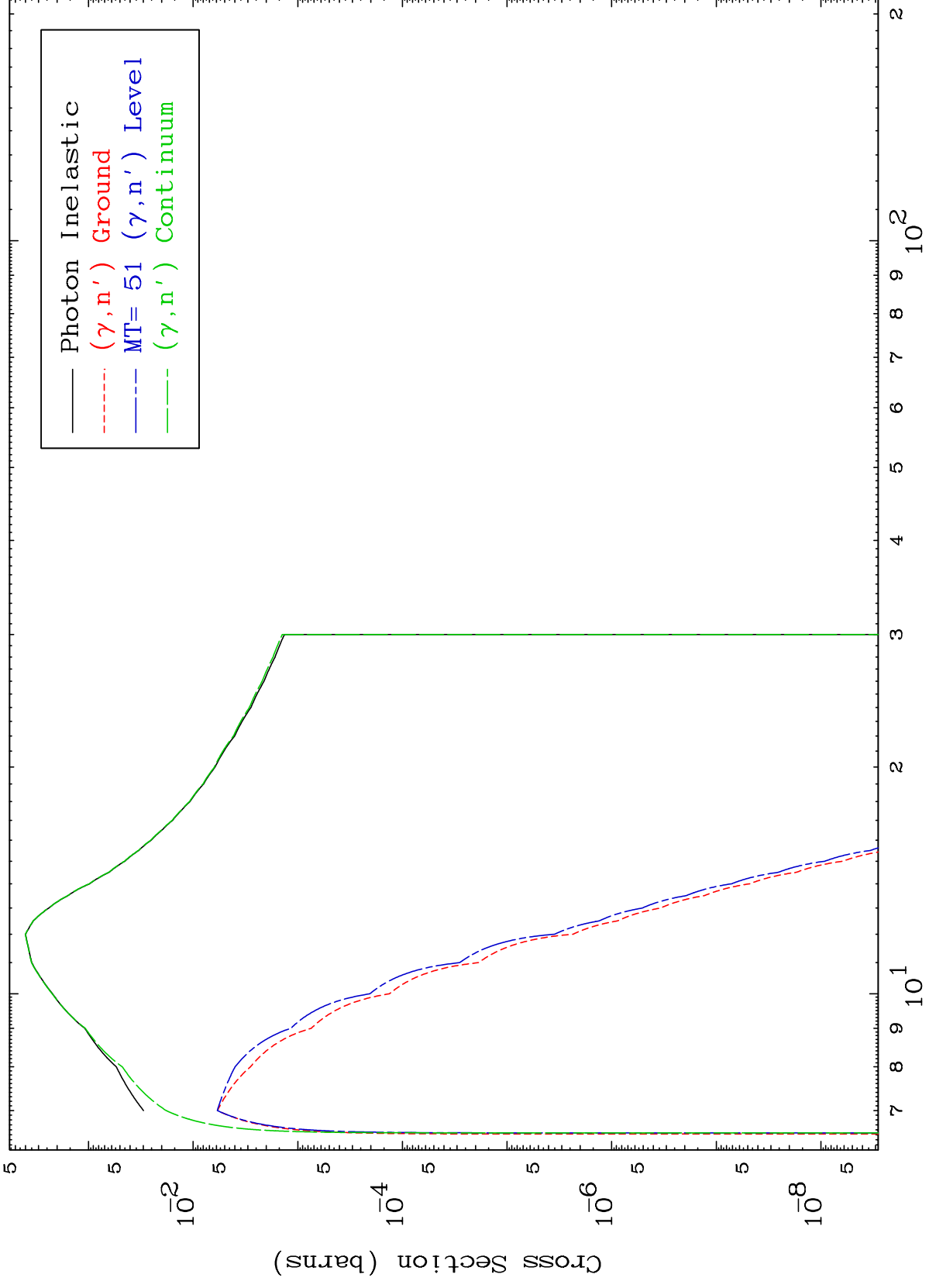


MAT 9935

( $\gamma, n'$ ) Level

100-Fm-254

0 Kelvin Cross Sections



Incident Energy (MeV)

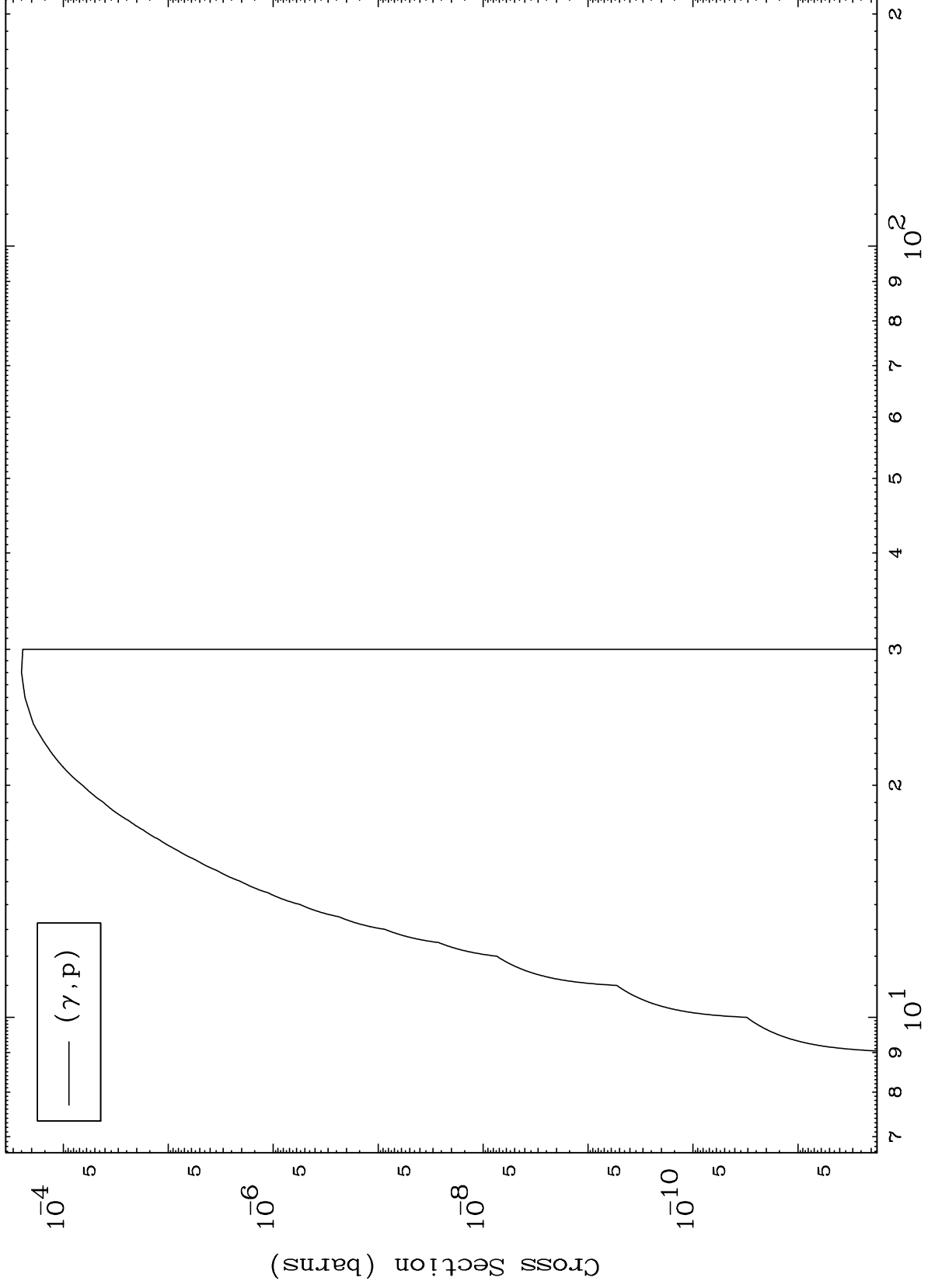
100-Fm-254

4

MAT 9935

( $\gamma, p$ ) Levels  
0 Kelvin Cross Sections

100-Fm-254



5

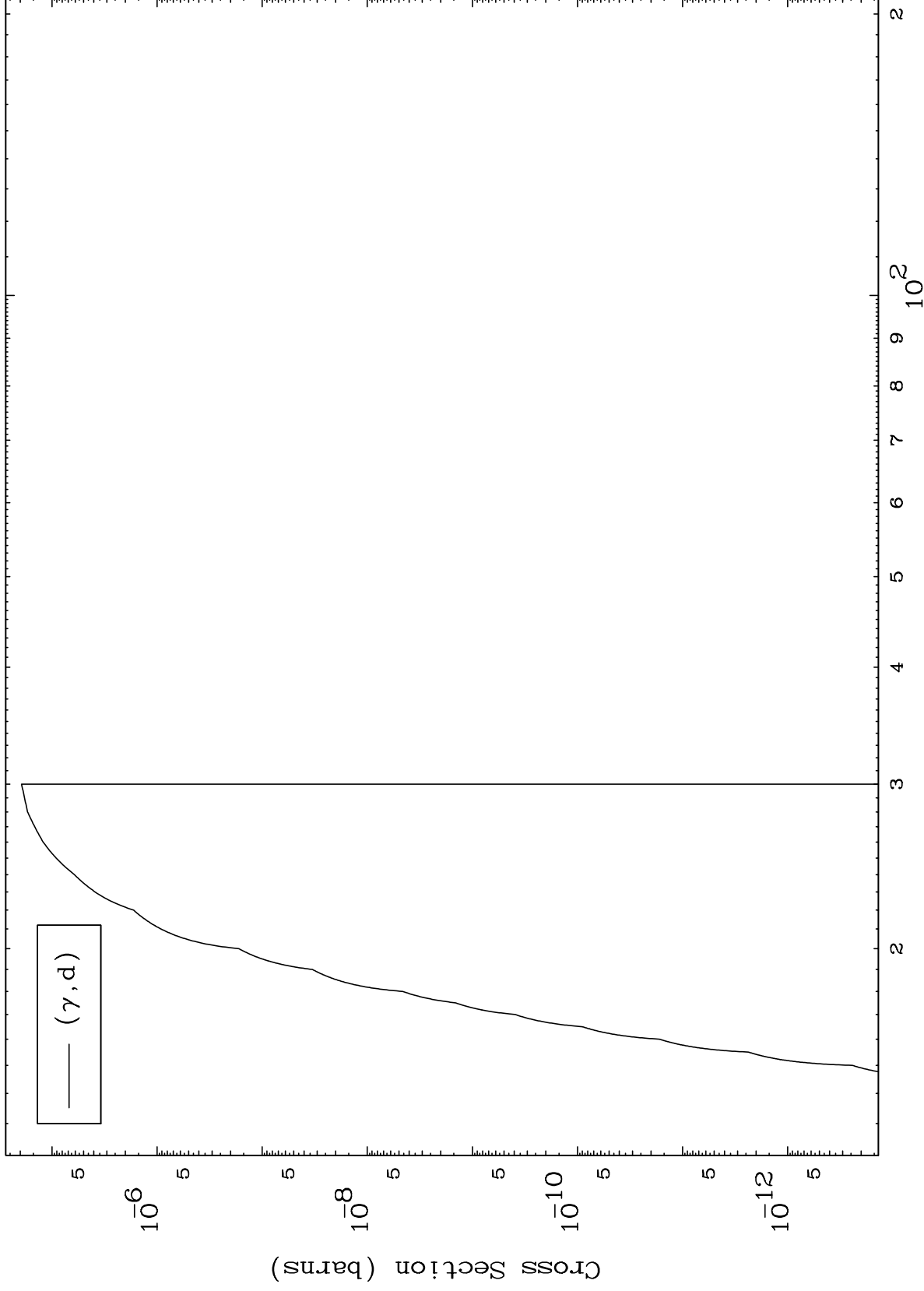
Incident Energy (MeV)

100-Fm-254

MAT 9935

( $\gamma, d$ ) Levels  
0 Kelvin Cross Sections

100-Fm-254



6

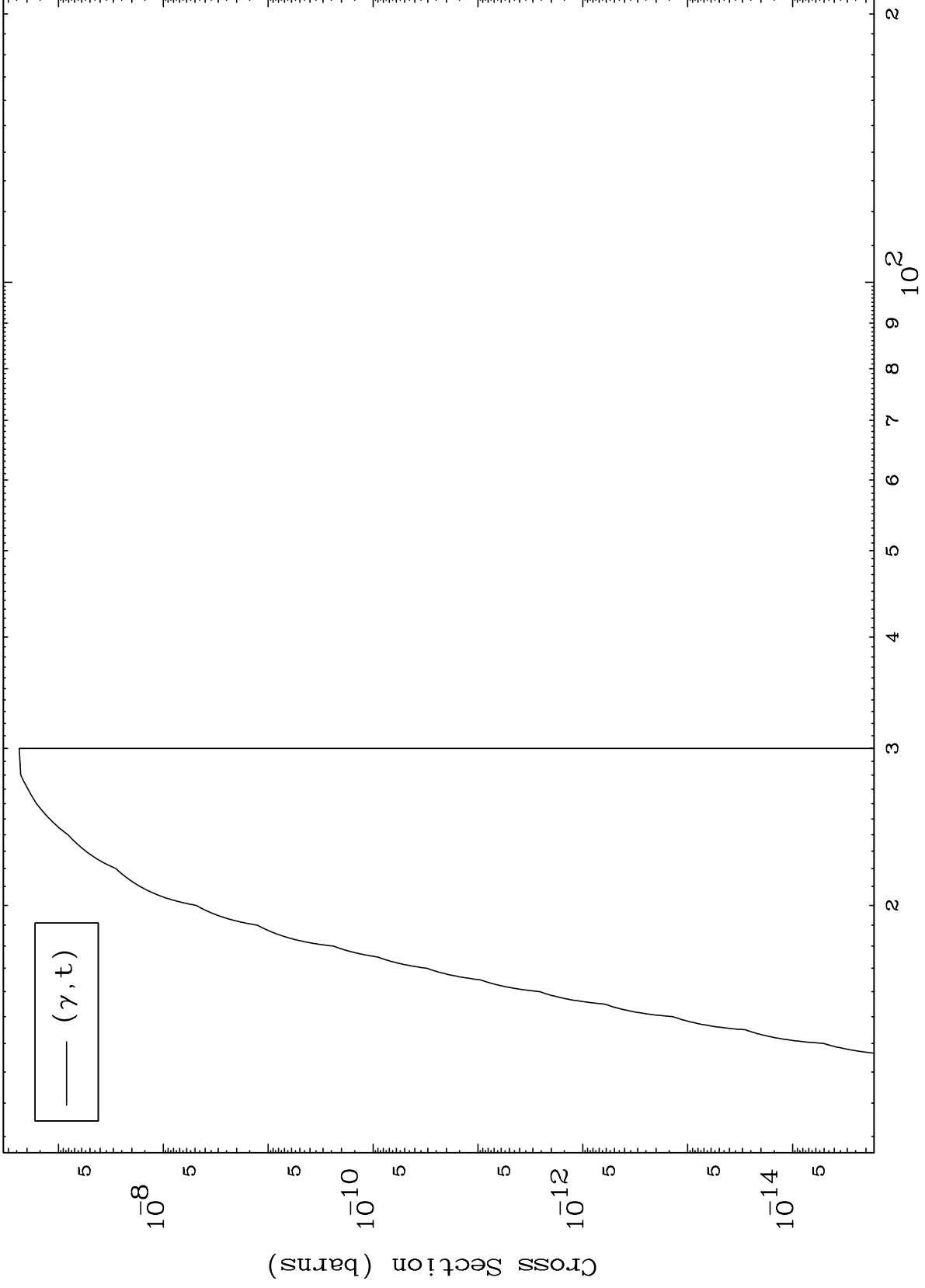
Incident Energy (MeV)

100-Fm-254

MAT 9935

( $\gamma, t$ ) Levels  
0 Kelvin Cross Sections

100-Fm-254



7

Incident Energy (MeV)

100-Fm-254

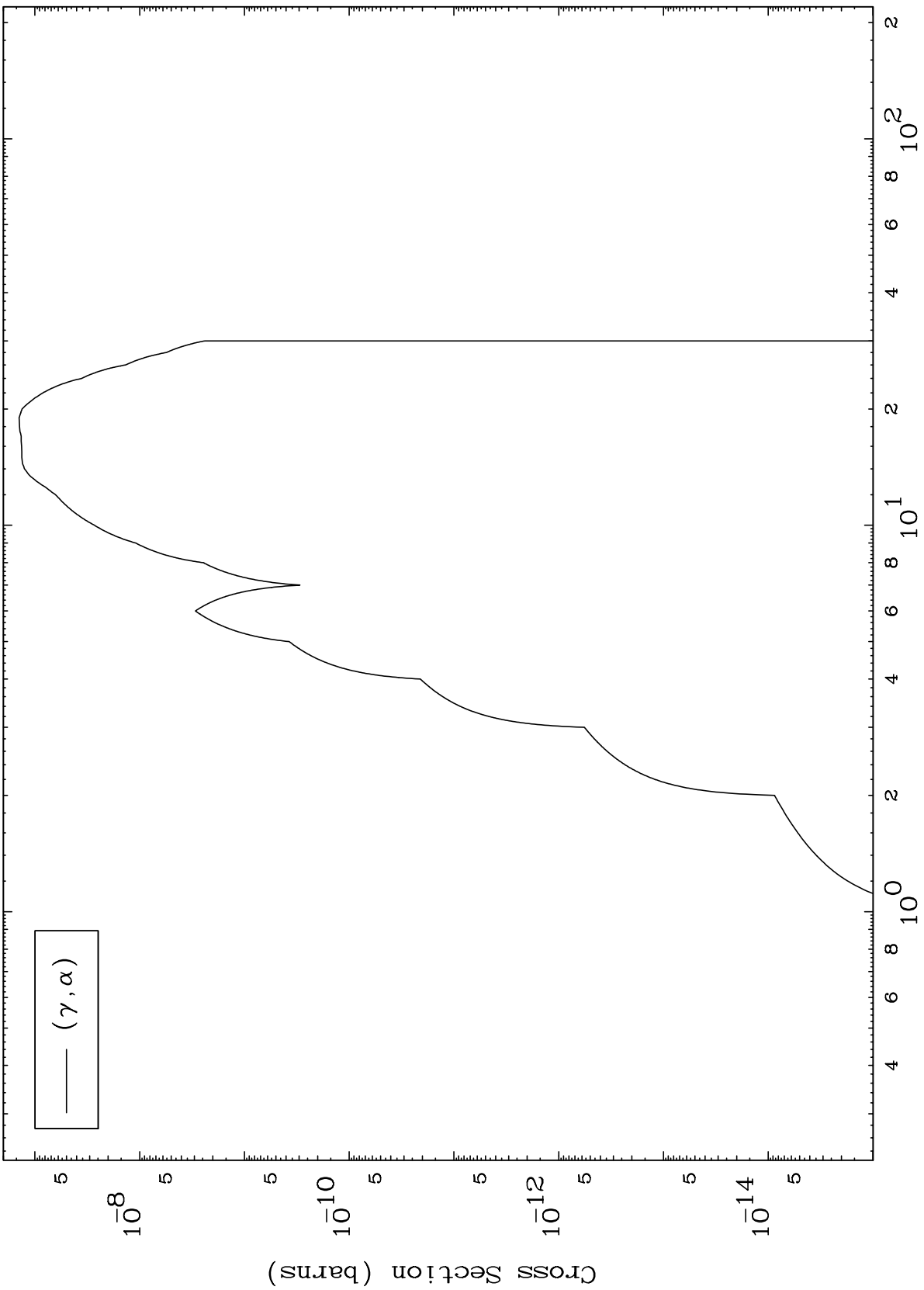


MAT 9935

( $\gamma, \alpha$ ) Levels

100-Fm-254

0 Kelvin Cross Sections

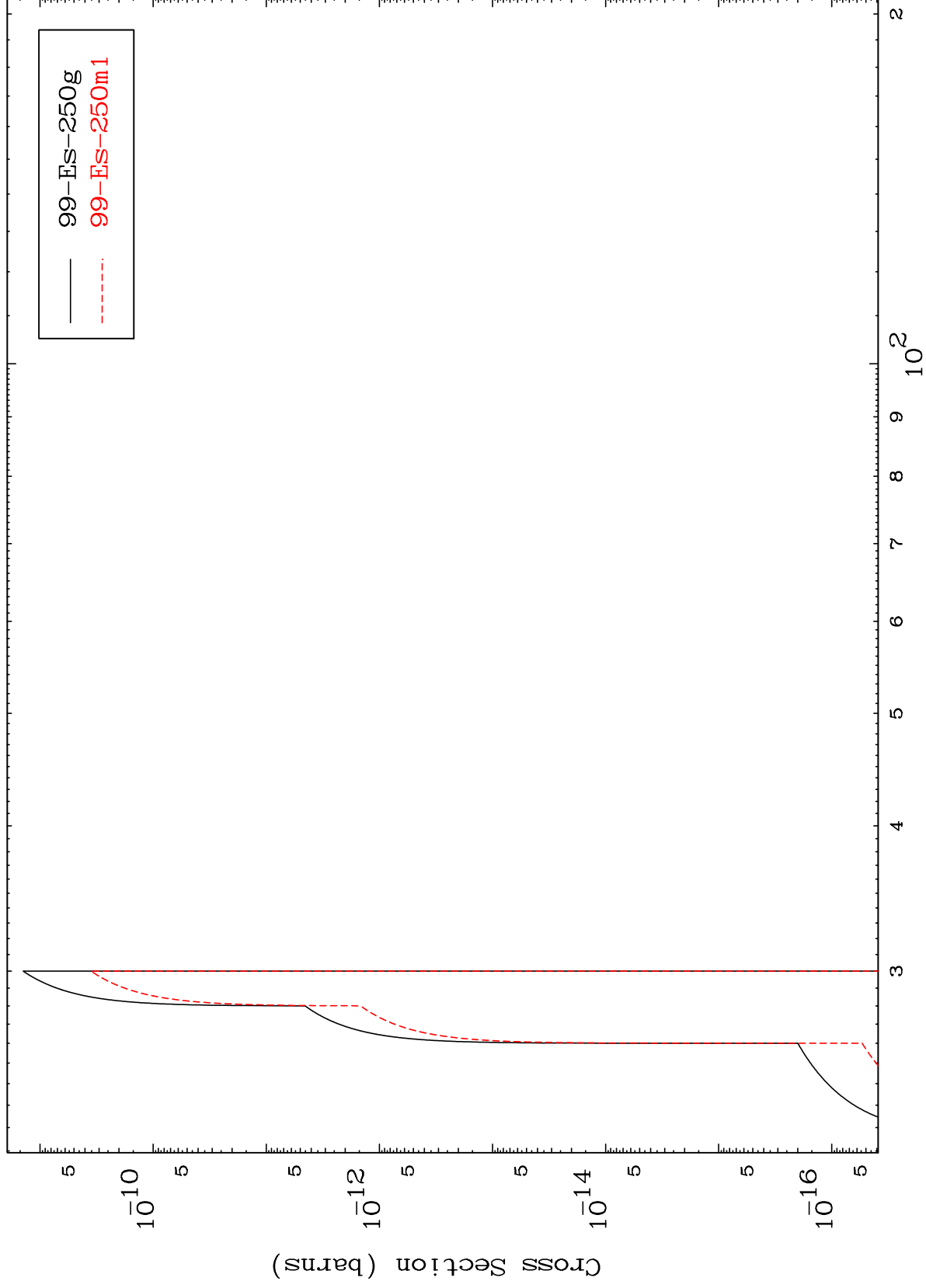


MAT 9935

( $\gamma, 2n$ ) d

100-Fm-254

Radionuclide Production Cross Section



9

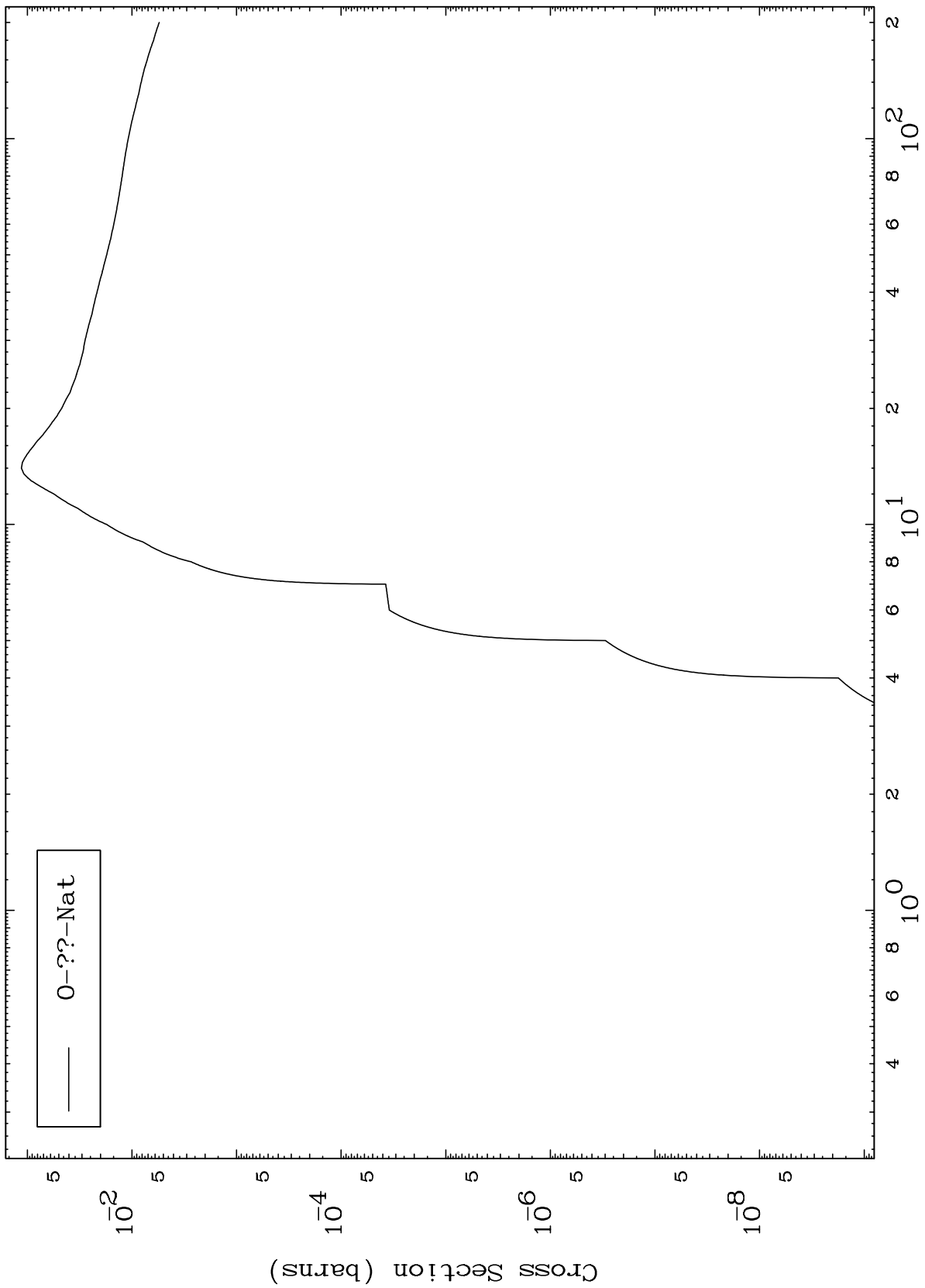
Incident Energy (MeV)

100-Fm-254

MAT 9935

Photon Fission  
Radionuclide Production Cross Section

100-Fm-254



10

Incident Energy (MeV)

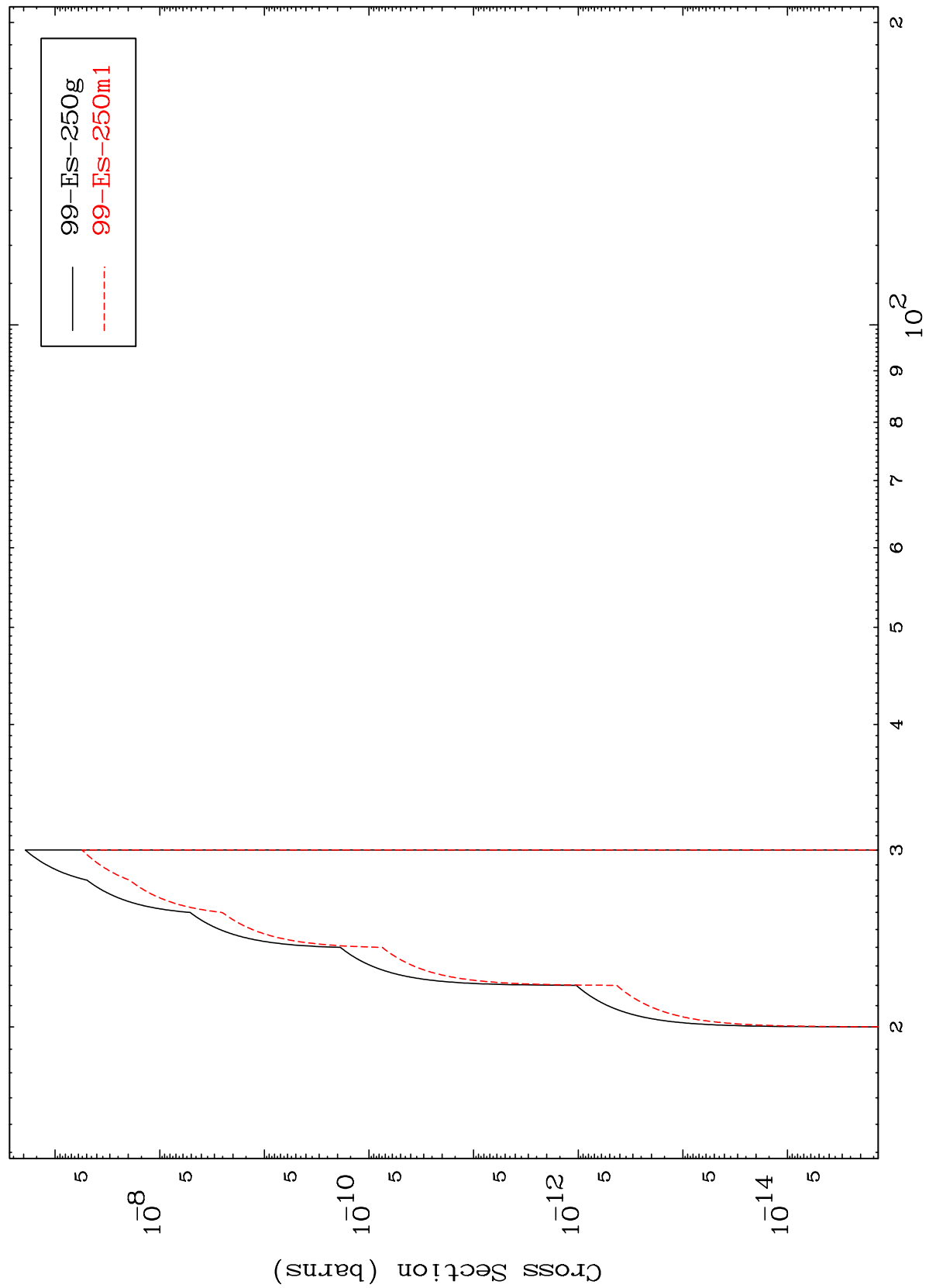
100-Fm-254

MAT 9935

( $\gamma, n'$ ) t

100-Fm-254

Radionuclide Production Cross Section



11

Incident Energy (MeV)

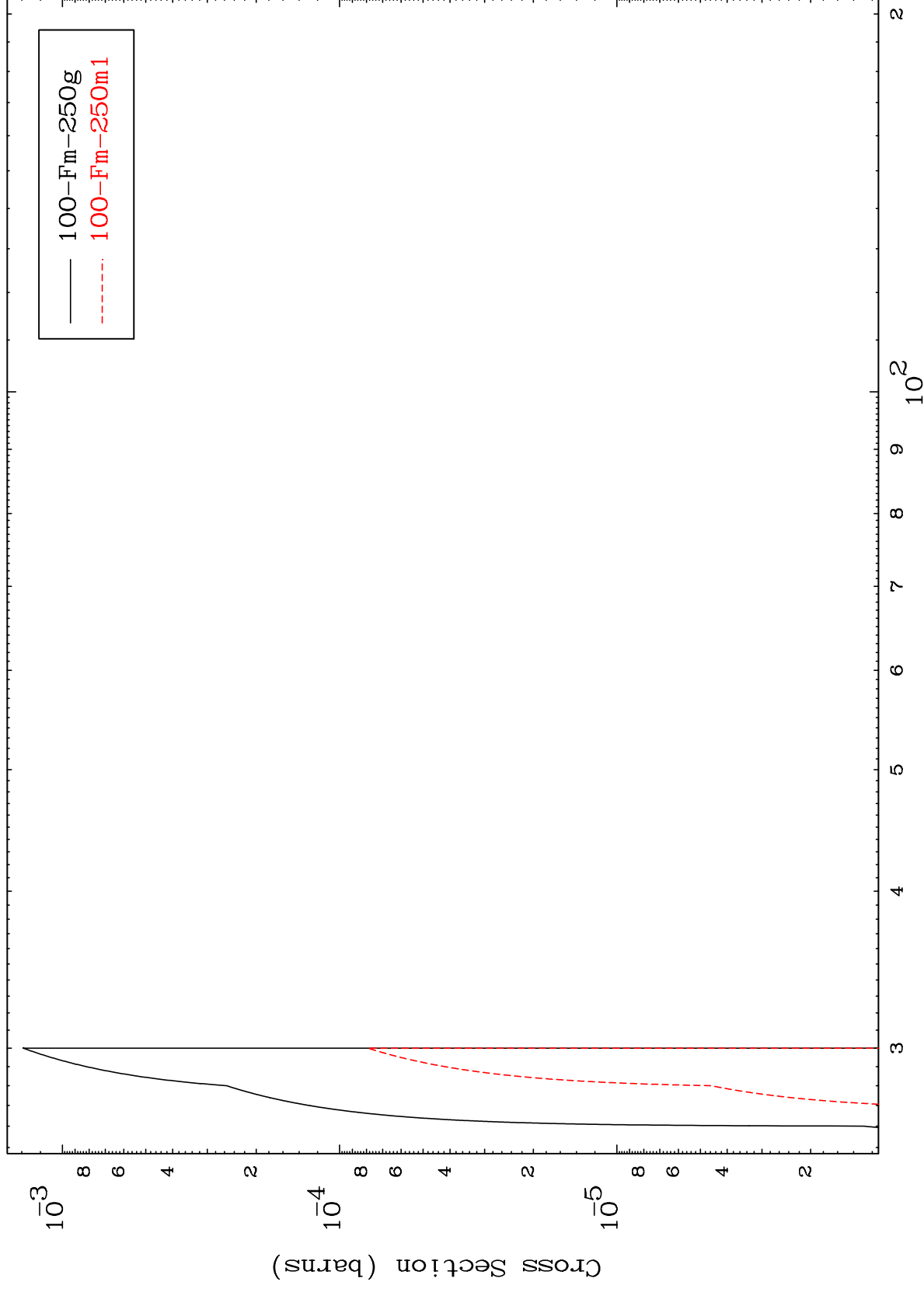
100-Fm-254

MAT 9935

100-Fm-254

( $\gamma, 4n$ )

Radionuclide Production Cross Section



12

100-Fm-254

Incident Energy (MeV)