

Program EVALPLOT  
(Version 2018-1)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

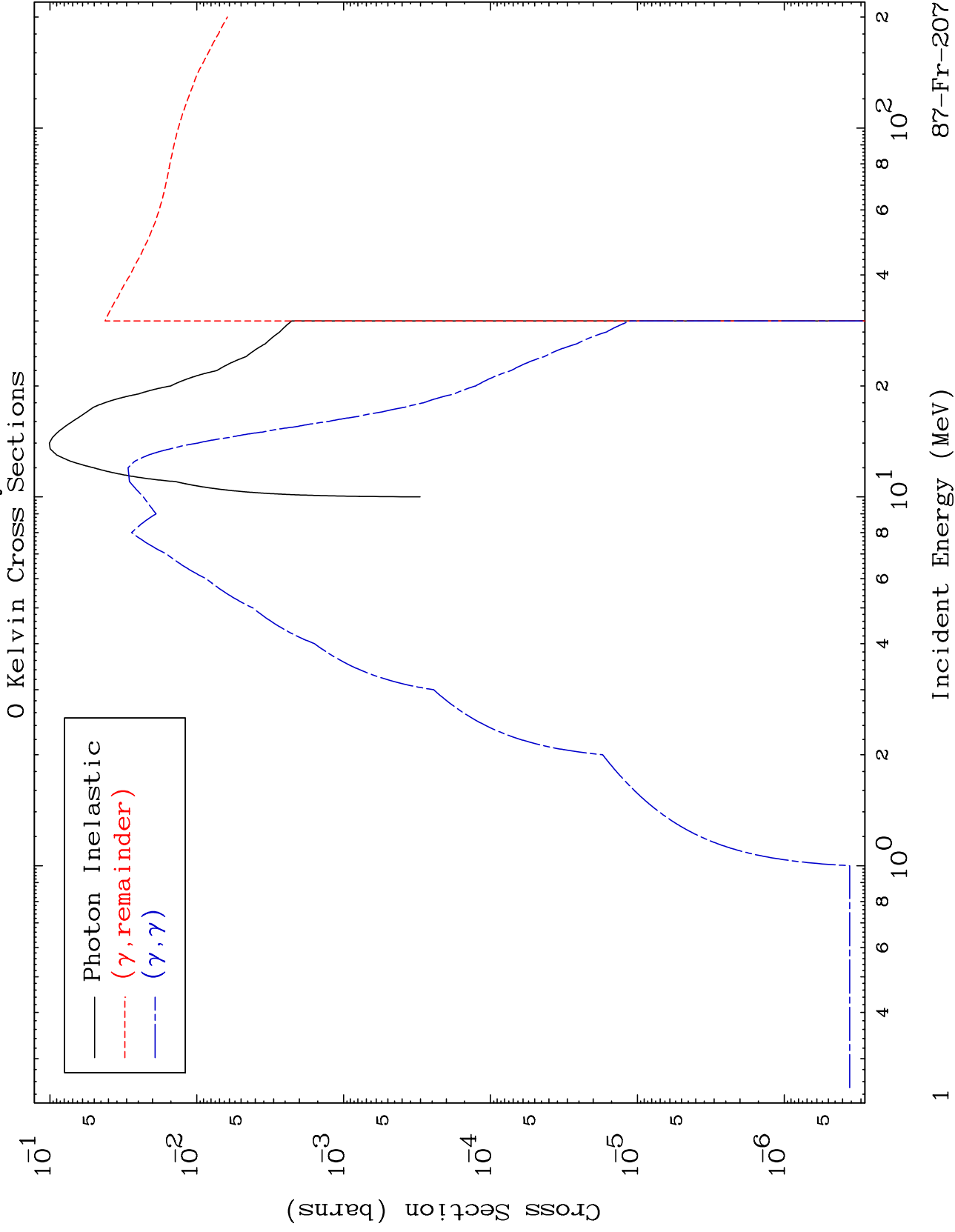
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 8710

Photon Major

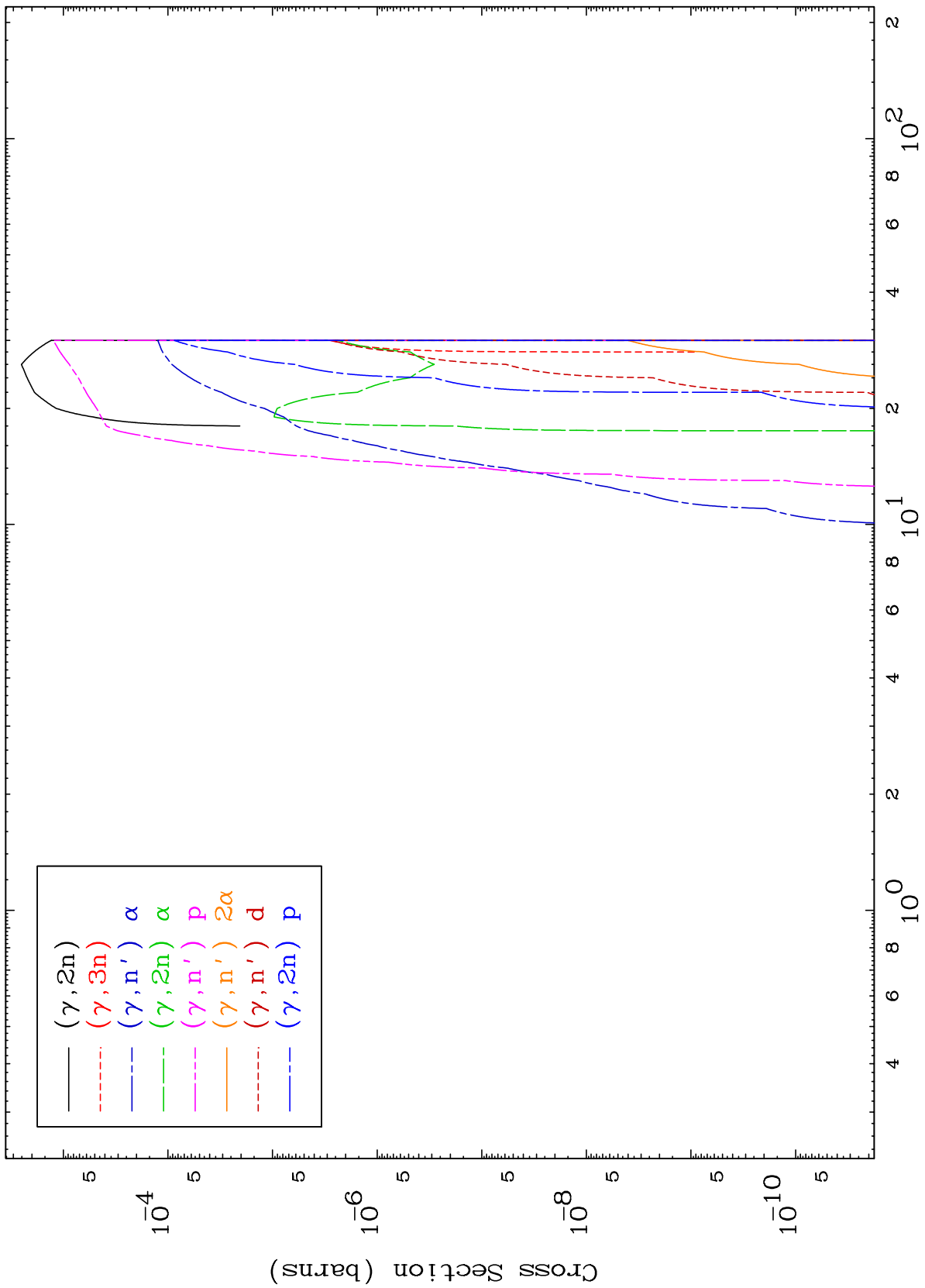
87-Fr-207



MAT 8710

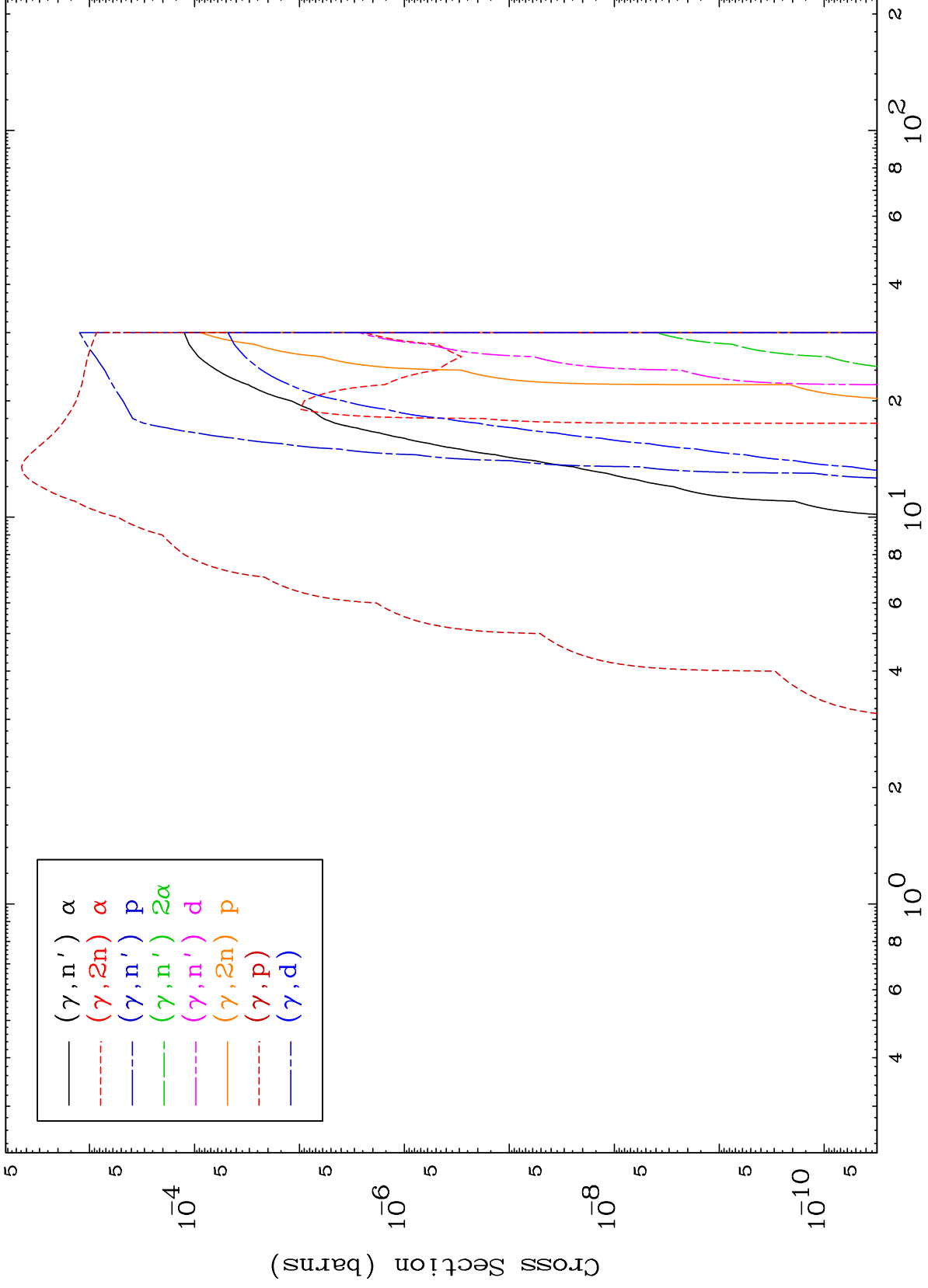
Photon Neutron Production  
0 Kelvin Cross Sections

87-Fr-207



87-Fr-207

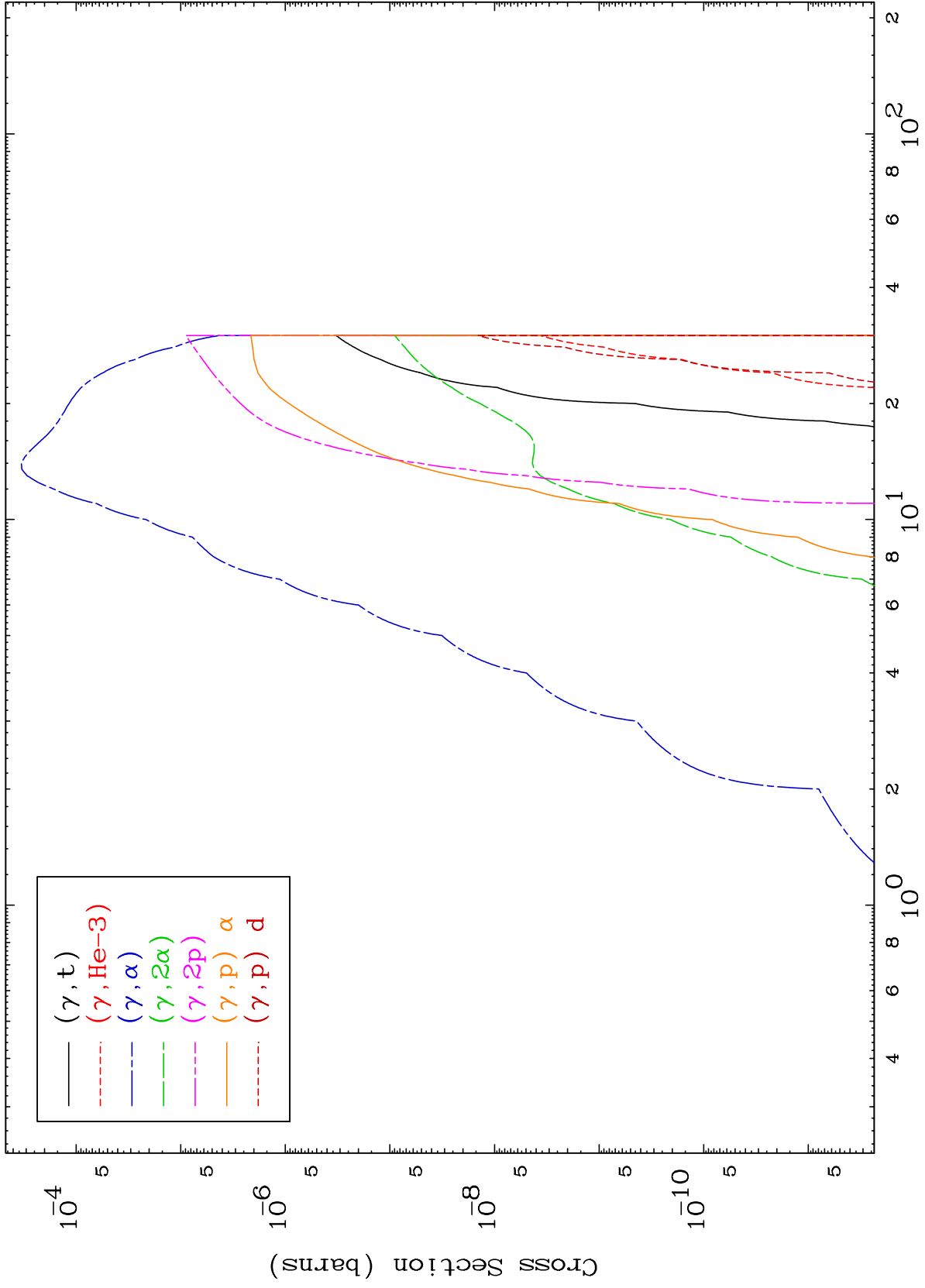
Incident Energy (MeV)



MAT 8710

Photon Charged Particle  
0 Kelvin Cross Sections

87-Fr-207

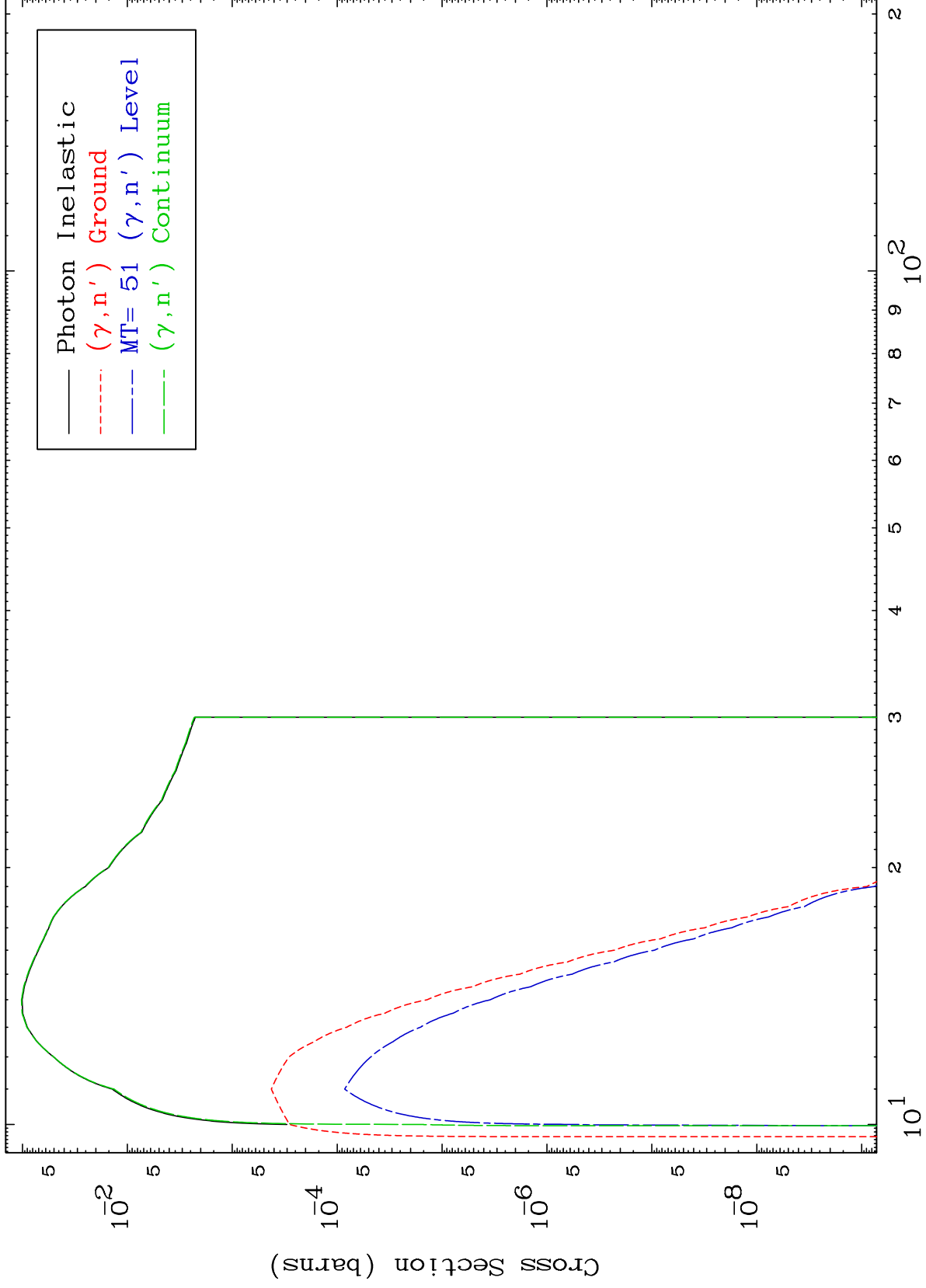


MAT 8710

$(\gamma, n')$  Level

87-Fr-207

0 Kelvin Cross Sections



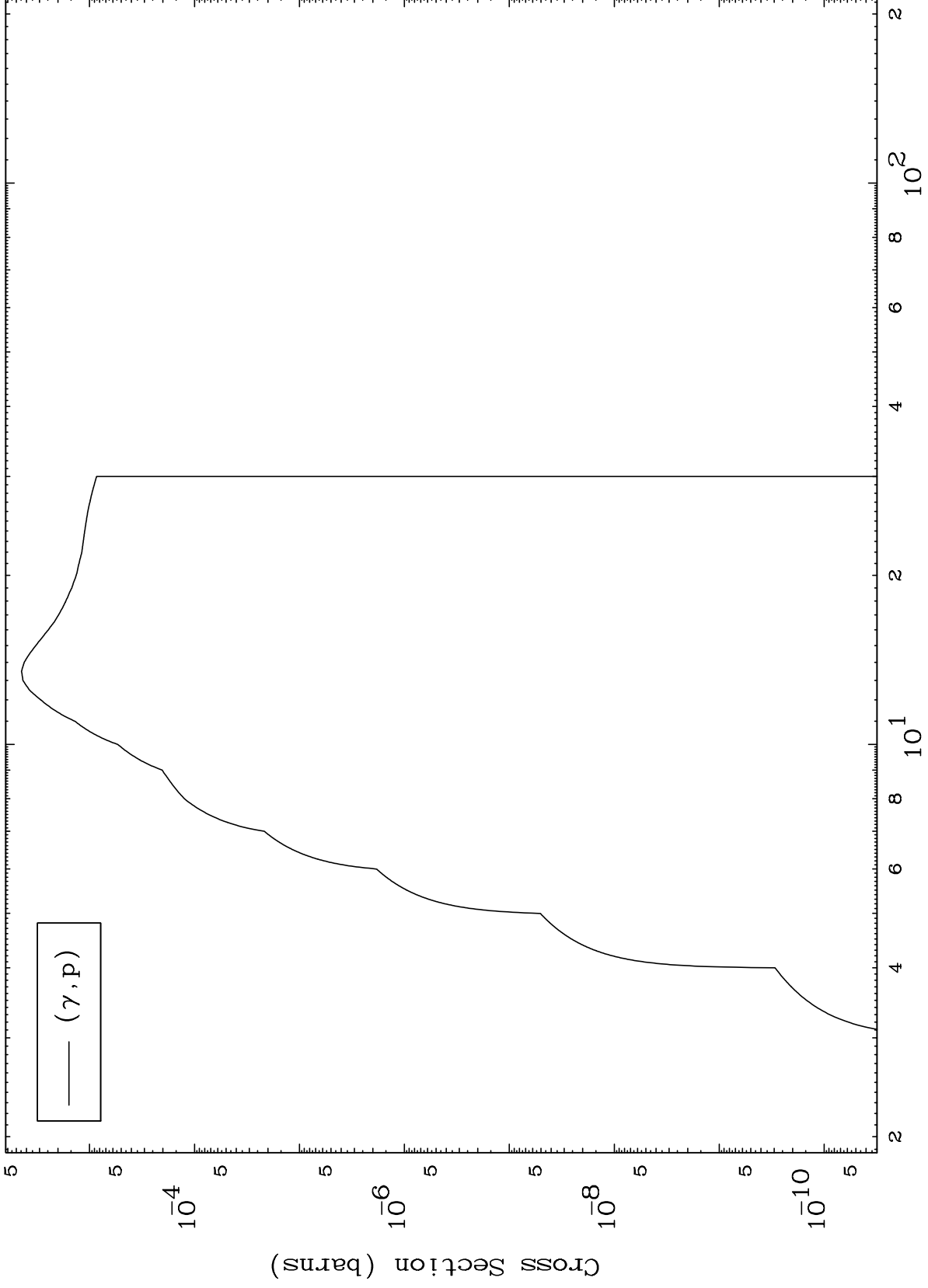
Incident Energy (MeV)

87-Fr-207

MAT 8710

( $\gamma, p$ ) Levels  
0 Kelvin Cross Sections

87-Fr-207



6

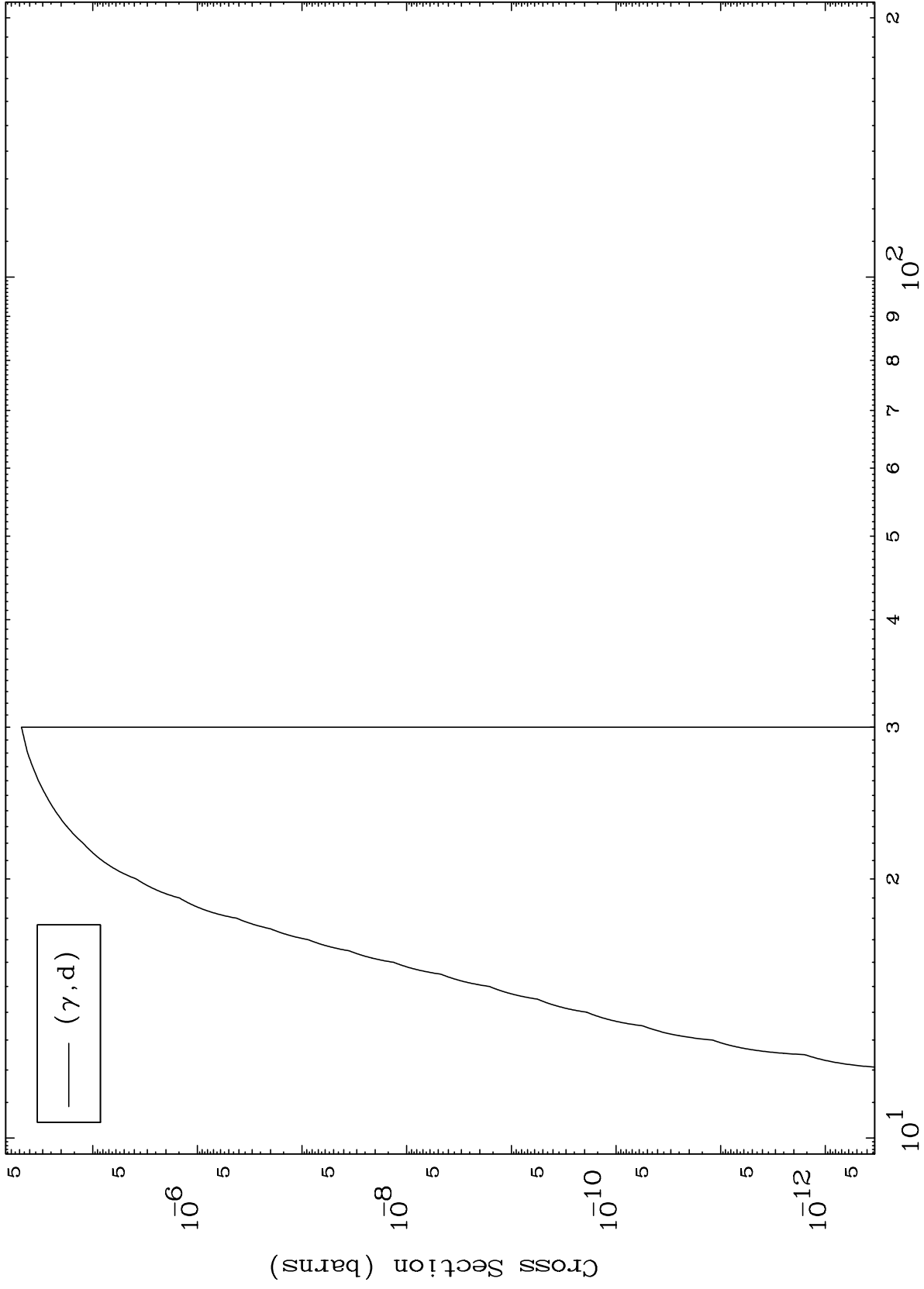
Incident Energy (MeV)

87-Fr-207

MAT 8710

( $\gamma, d$ ) Levels  
0 Kelvin Cross Sections

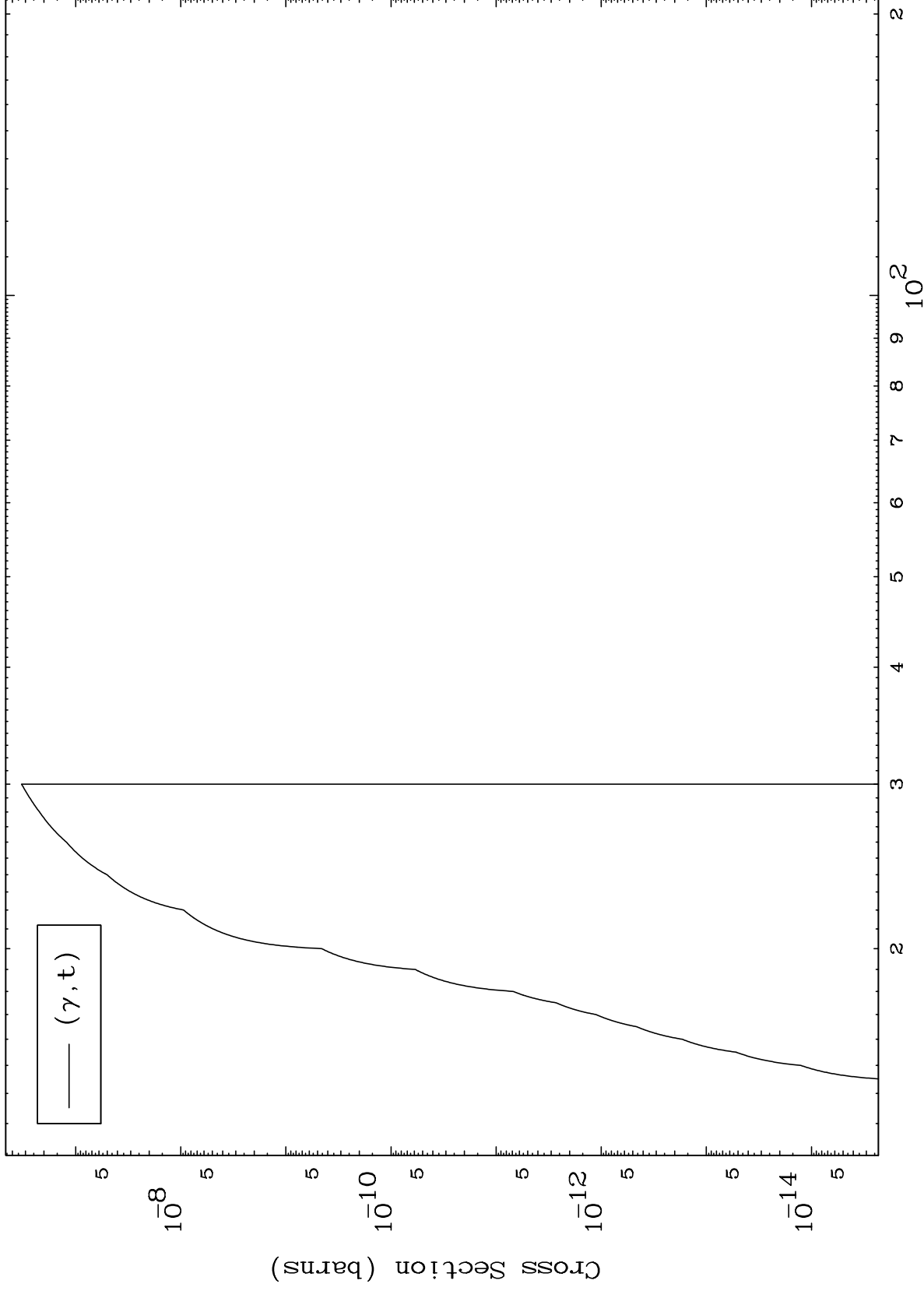
87-Fr-207



Incident Energy (MeV)

87-Fr-207

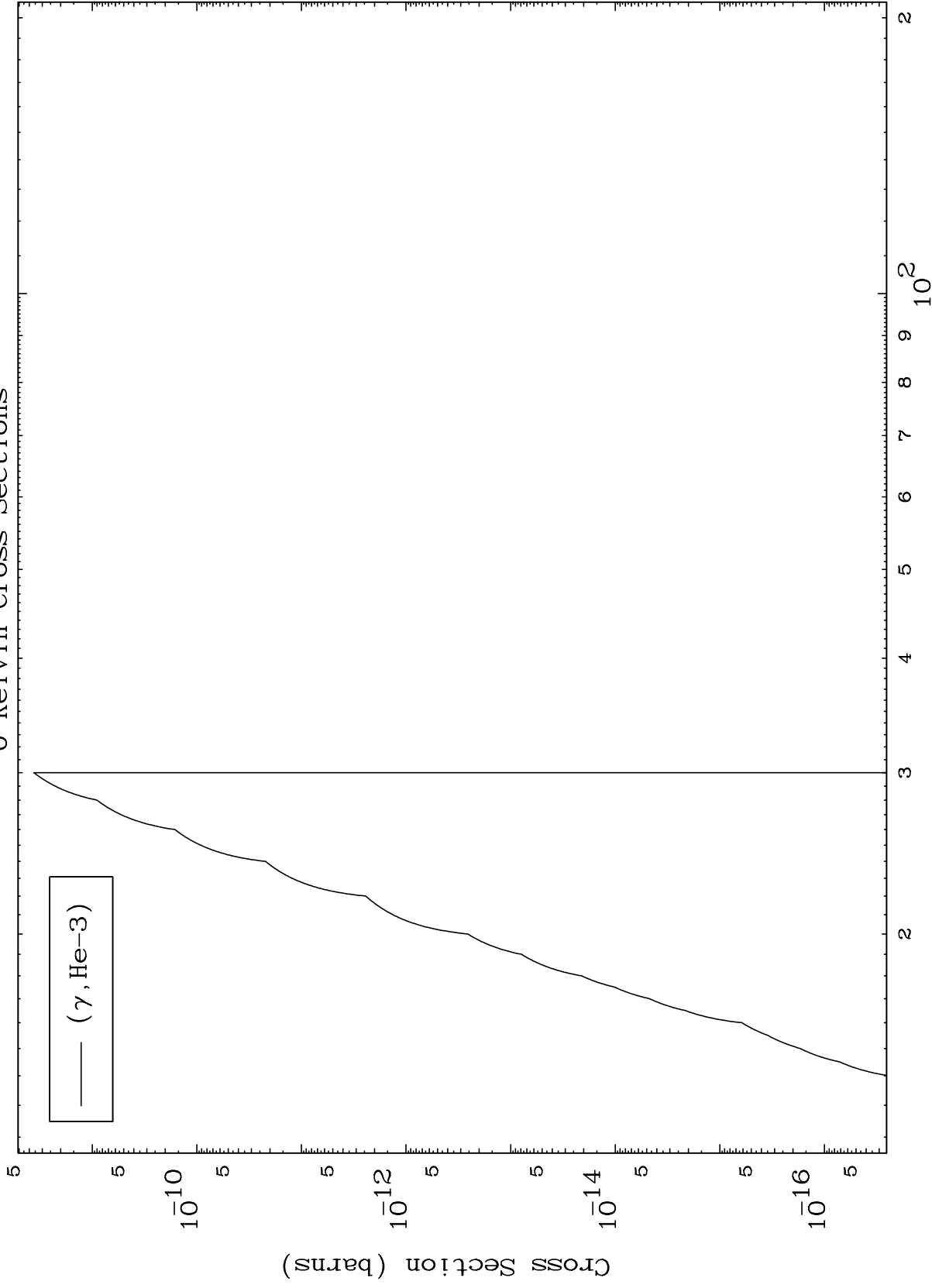




MAT 8710

( $\gamma$ ,He3) Levels  
0 Kelvin Cross Sections

87-Fr-207



Incident Energy (MeV)

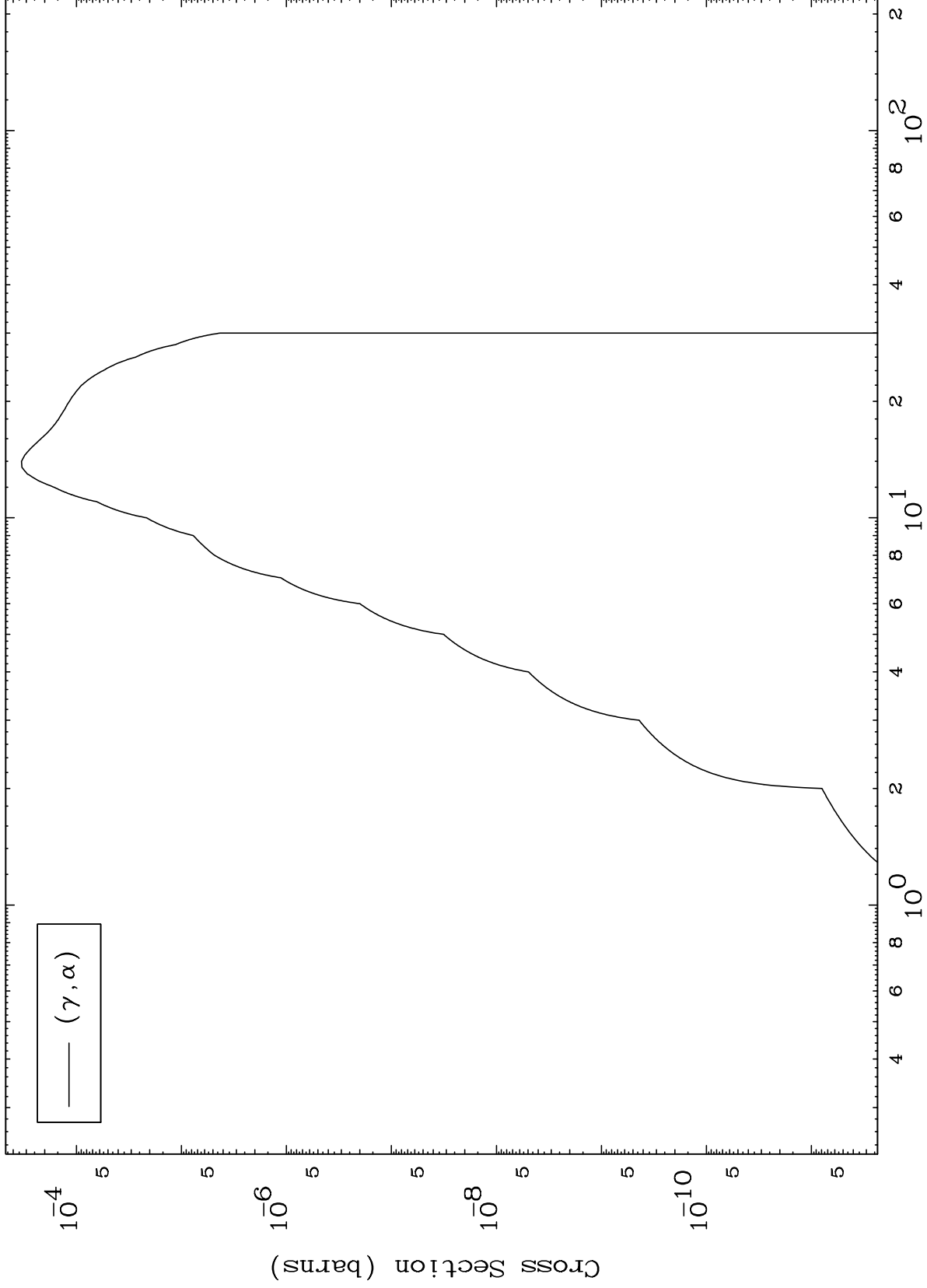
87-Fr-207

9

MAT 8710

( $\gamma, \alpha$ ) Levels  
0 Kelvin Cross Sections

87-Fr-207



10

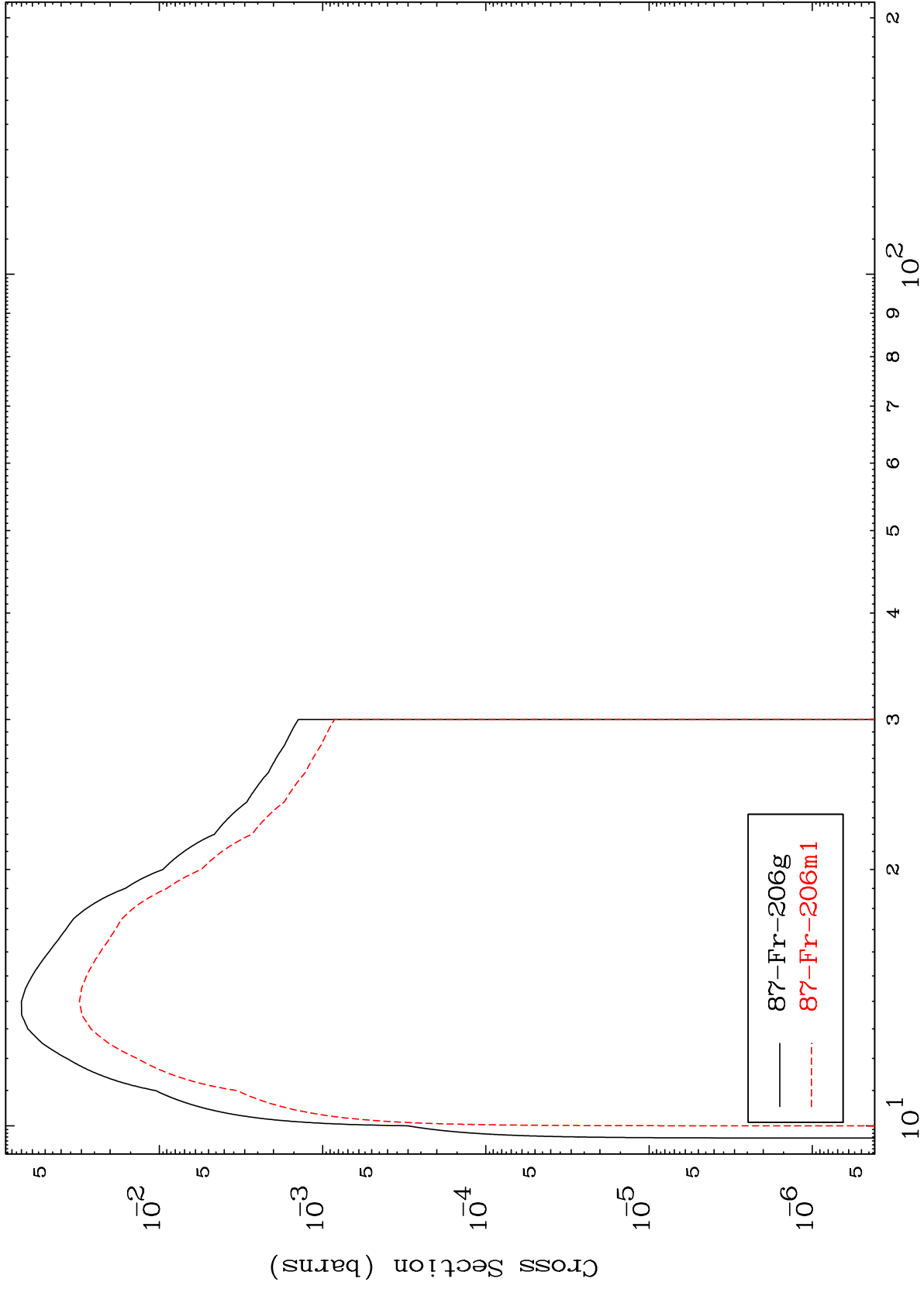
Incident Energy (MeV)

87-Fr-207

MAT 8710

Photon Inelastic  
Radionuclide Production Cross Section

87-Fr-207



11

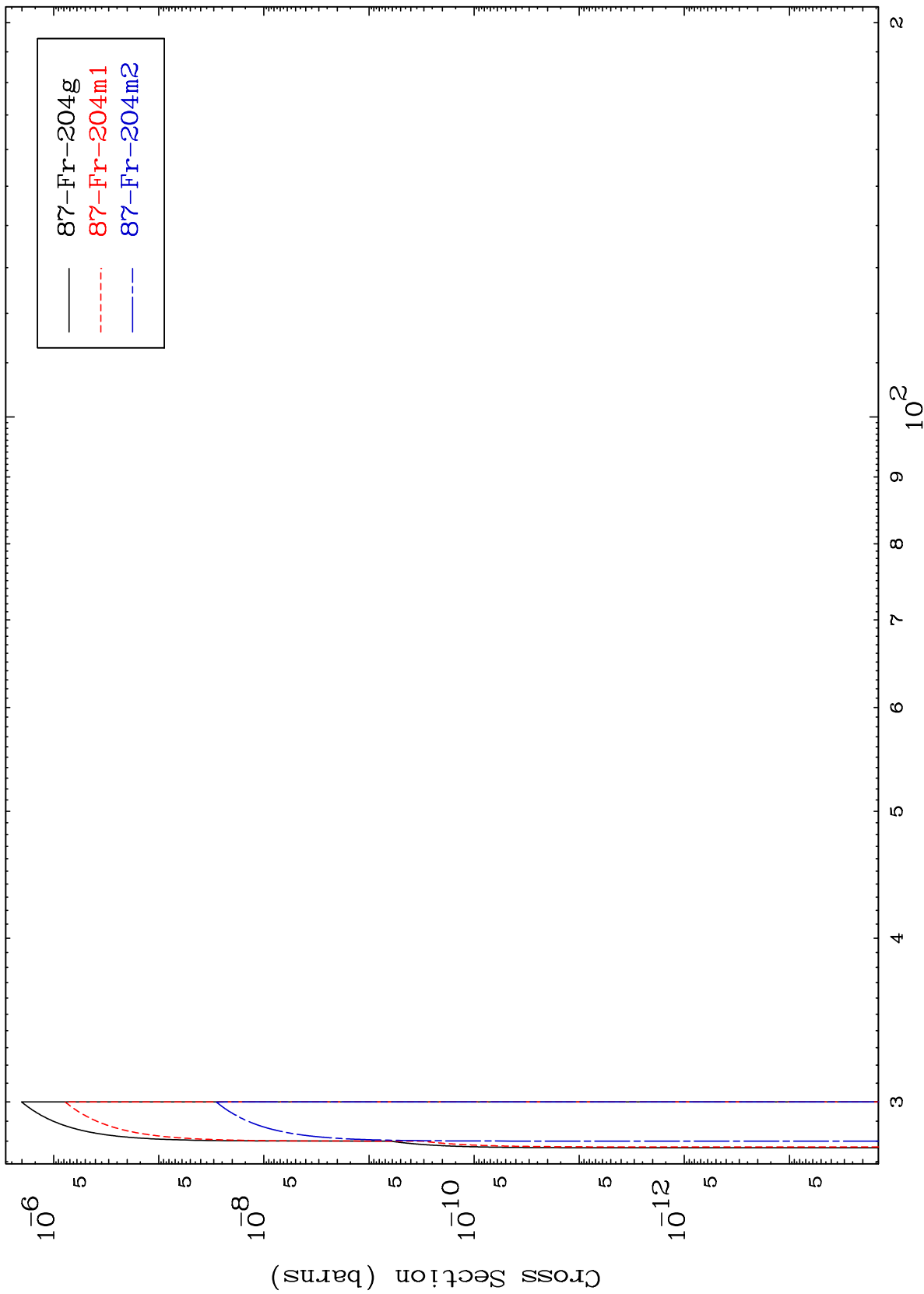
Incident Energy (MeV)

87-Fr-207

MAT 8710

87-Fr-207

( $\gamma, 3n$ )  
Radionuclide Production Cross Section



87-Fr-207

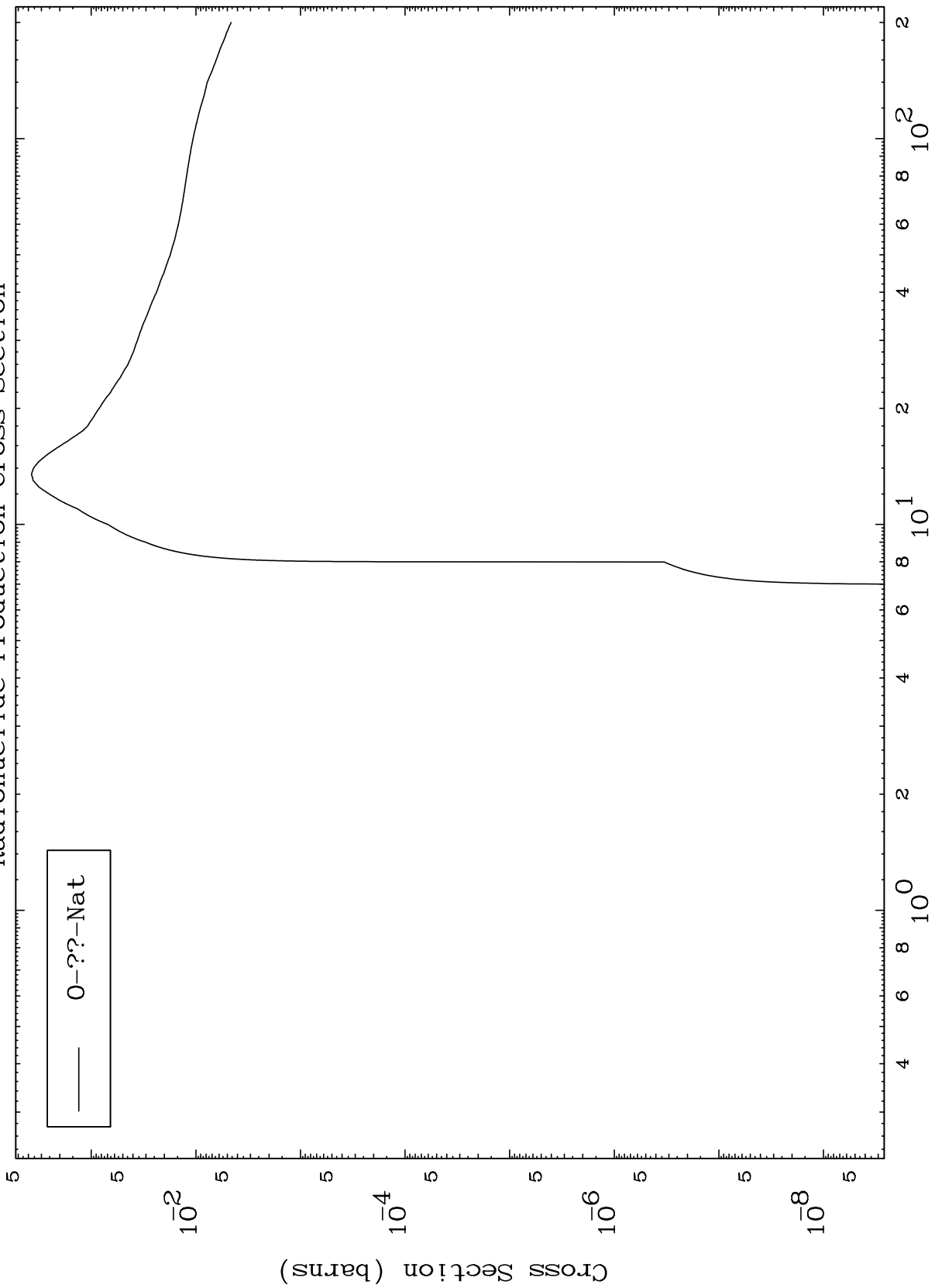
Incident Energy (MeV)

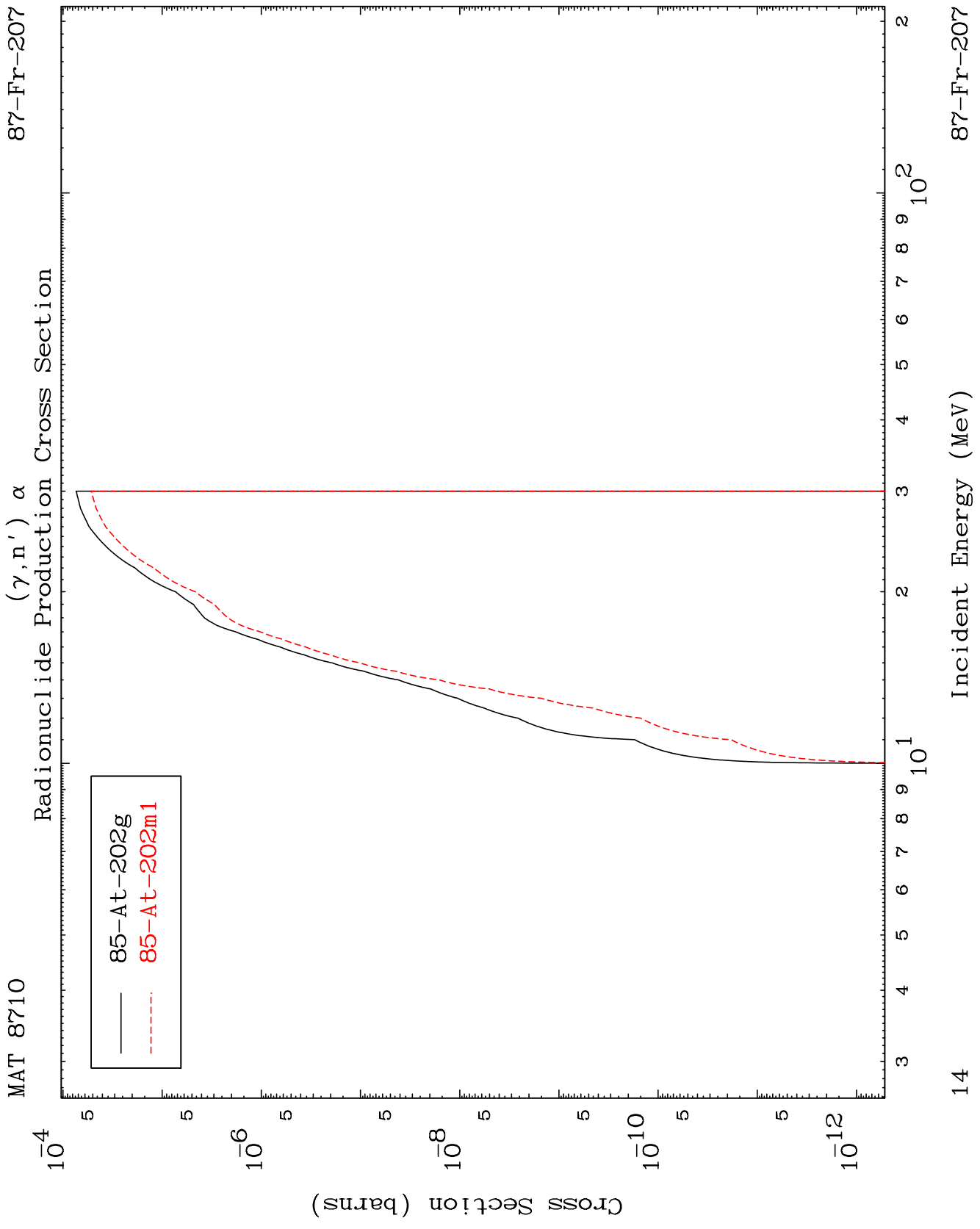
12

MAT 8710

Photon Fission  
Radionuclide Production Cross Section

87-Fr-207



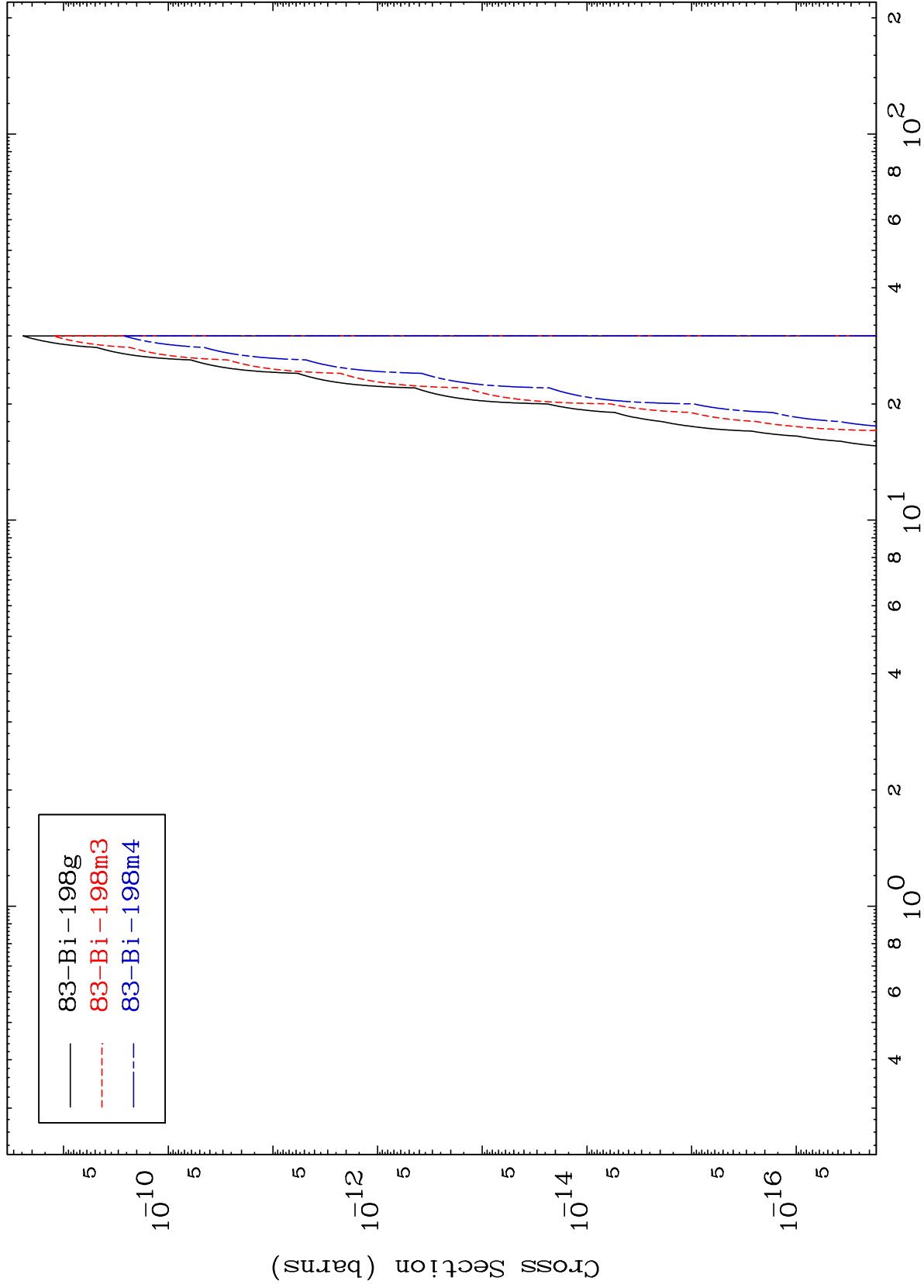


MAT 8710

$(\gamma, n')$   $2\alpha$

87-Fr-207

Radionuclide Production Cross Section



15

Incident Energy (MeV)

87-Fr-207

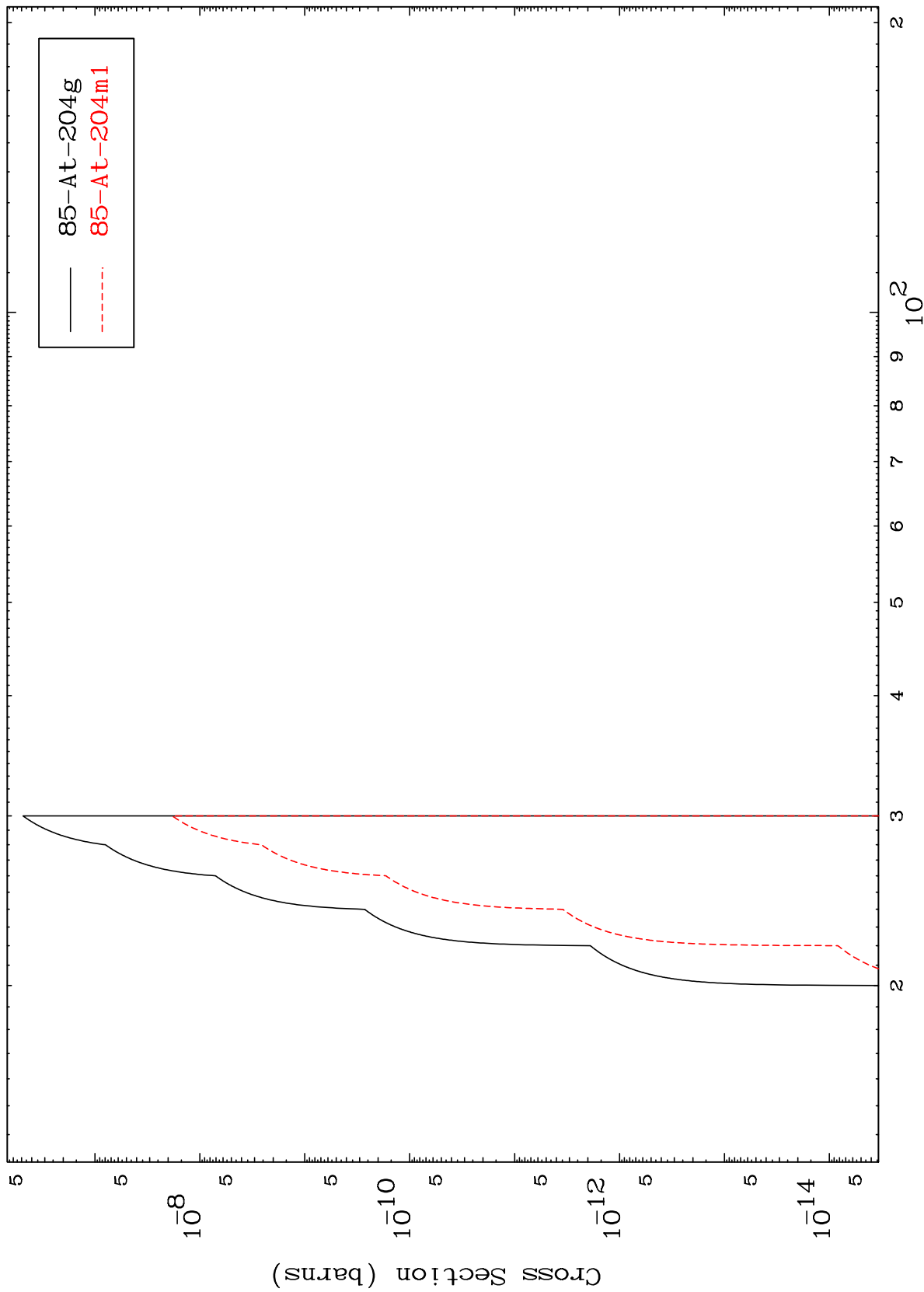


MAT 8710

( $\gamma, 2n$ ) p

87-Fr-207

Radionuclide Production Cross Section



16

Incident Energy (MeV)

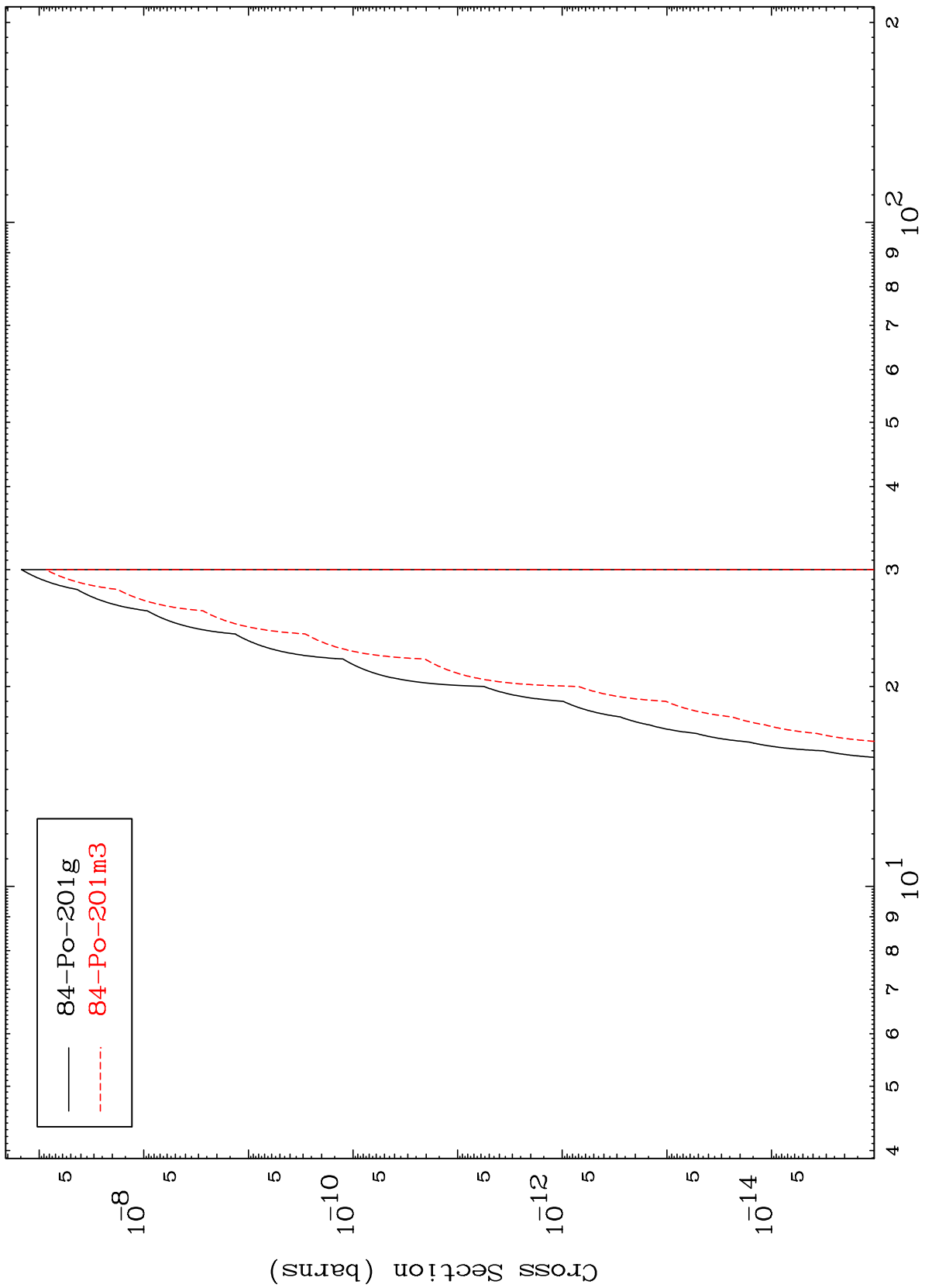
87-Fr-207

MAT 8710

( $\gamma, n'$ ) p  $\alpha$

87-Fr-207

Radionuclide Production Cross Section



84-Po-201g  
84-Po-201m3

17

Incident Energy (MeV)

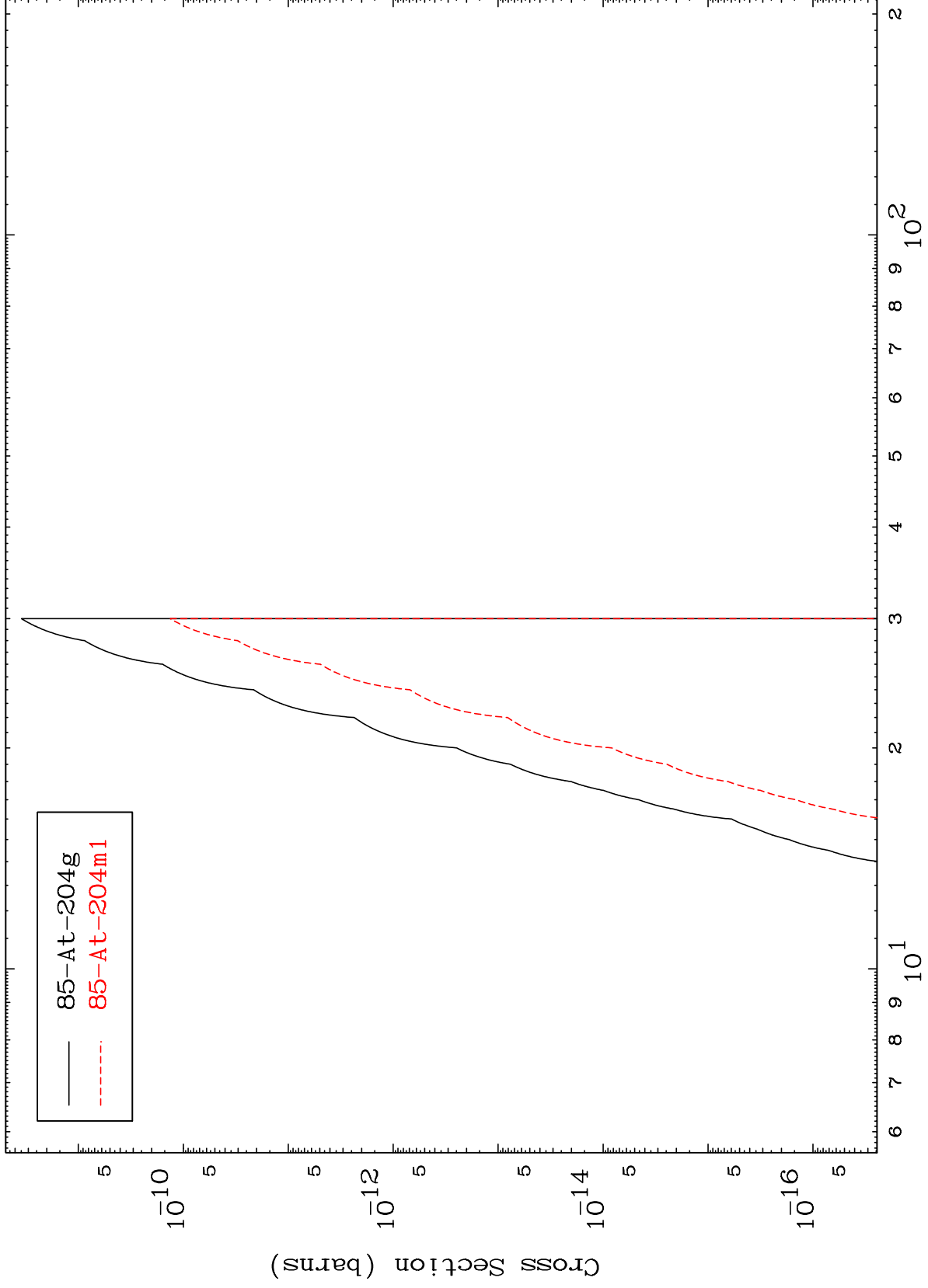
87-Fr-207

MAT 8710

( $\gamma, \text{He-3}$ )

87-Fr-207

Radionuclide Production Cross Section



18

Incident Energy (MeV)

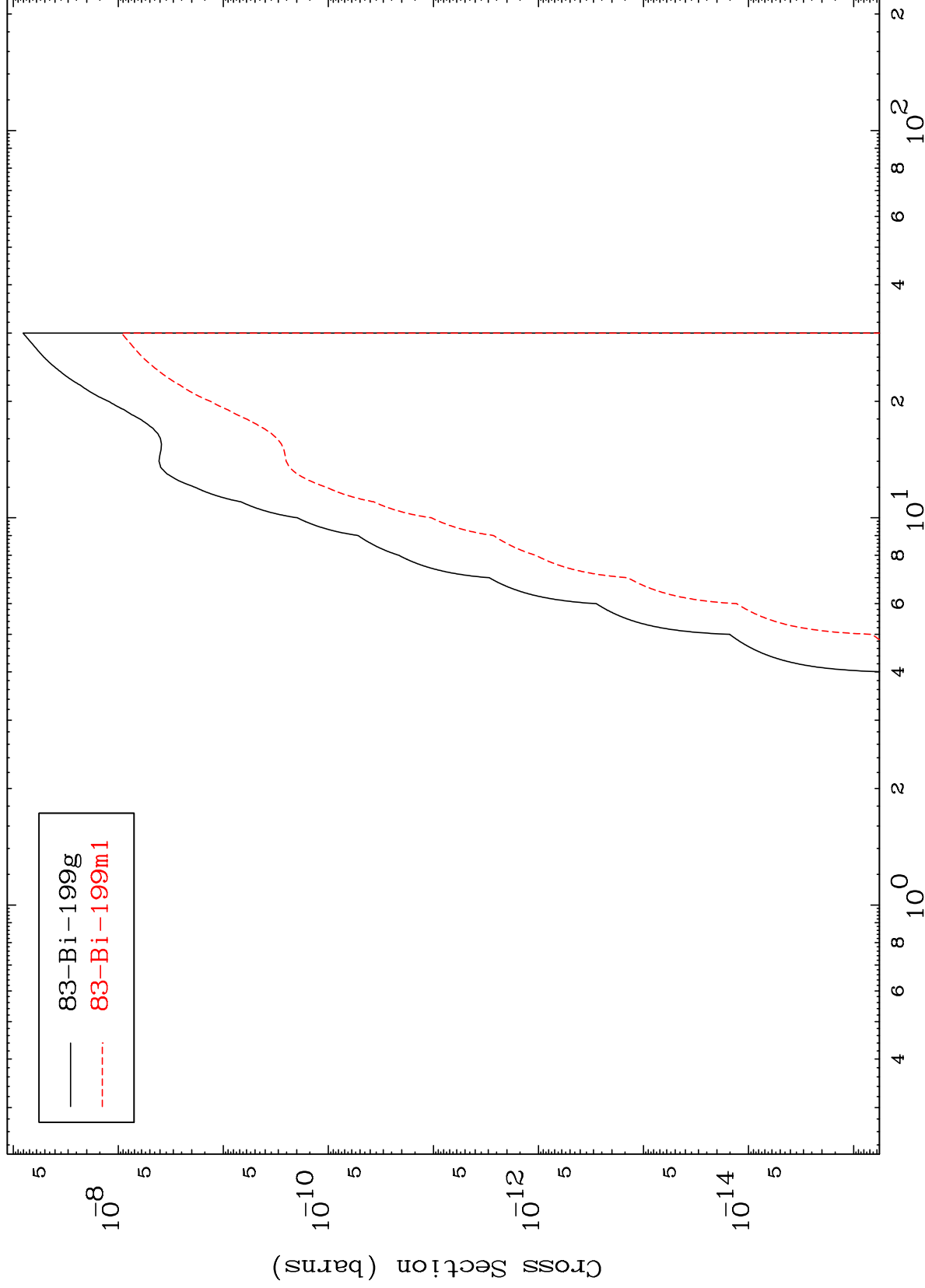
87-Fr-207

MAT 8710

( $\gamma, 2\alpha$ )

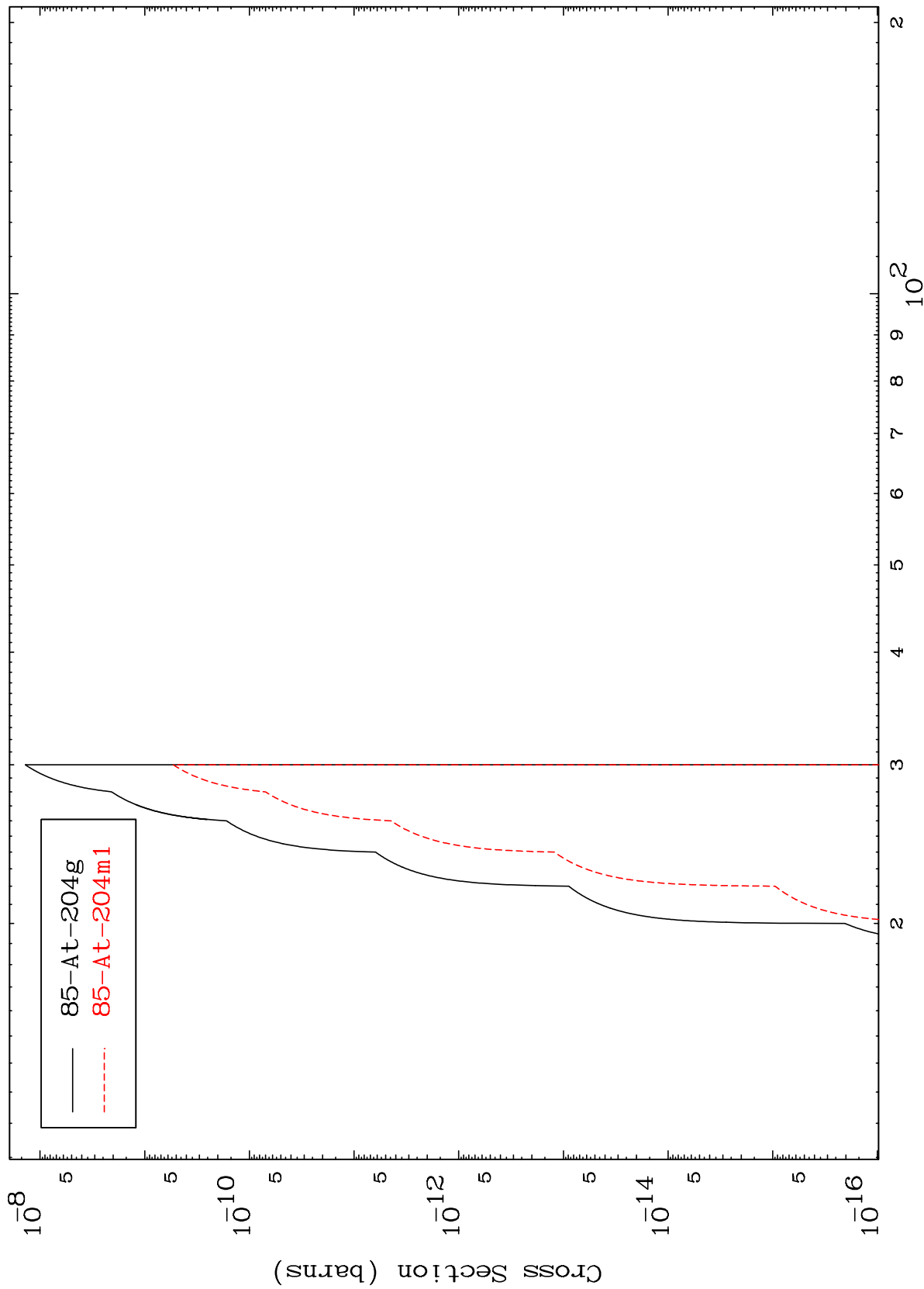
87-Fr-207

Radionuclide Production Cross Section



83-Bi-199g  
83-Bi-199m1

Radionuclide Production Cross Section



MAT 8710

( $\gamma, d$ )  $\alpha$

87-Fr-207

Radionuclide Production Cross Section

